

UNITED STATES NUCLEAR REGULATORY COMMISSION REGION II 101 MARIETTA STREET, N.W. ATLANTA, GEORGIA 30303

Report Nos.: 50-325/78-20 and 50-324/78-20

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Docket Nos.: 50-325 and 50-324

License Nos.: DPR-71 and DPR-62

License: Carolina Power and Light Company 336 Fayetteville Street Raleigh, North Carolina 27602

Facility Name: Brunswick 1 and 2

Inspection at: Southport, North Carolina

Inspection conducted: August 15-18, 1978

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Inspector: D. G. Hinckley

Reviewed by:

H. C. Dance, Chief Reactor Projects Section No. 1 Reactor Operations and Nuclear Support Branch

# Inspection Summary

Inspection on August 15-18, 1978 (Report Nos. 50-325/78-20 and 50-324/78-20) Areas Inspected: Routine, unannounced inspection of records, licensee event reports, IE bulletins and circulars, and follow up on open items. The inspection involved 28 inspectors-hours onsite by one NRC inspector. Results: In the five areas inspected, no items of noncompliance or deviations were identified.

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### 5. Records

The inspector reviewed the control of documents and records to verify that storage and retrieval requirements had been implemented as specified in Administrative Instructions AI-2, Revision 9, "Collection, Maintenance, Control and Storage of Plant Records" and AE-6, Revision 9, "Plant Filing Instructions." The inspector verified that the following records were retrievable:

- a. APRM recording charts for 1977.
- b. PT 3.1.7PC, Emergency Core Cooling System (Rev. 1), change review sign of sheet.
- System Description SD-2 Automatic Blowdown System, master file copy.
- d. RHR and HPCI room ventilation modification (MOD. 77-63)
- e. PT 45.2.6, Group I Valves Isolation Response Time, data sheets for 1977 test.
- f. Diesel generator trouble tickets for 1978.
- g. Foreign Print VE&C Drawing No. 9527-0-25018, Revision 10, Reactor Coolant Recirculation System, Unit 1, Sheet 1.
- h. Foreign Print VE&C Drawing No. 9527-LL-92037, Revision 6, MCC 1XB, Comp. 1-DP7, Ventilation RHR Heat Exchanger Area Fan 1B, Cable Diagram.

The inspector discussed the as-built drawing filing system with the document control custodian but did not verify the control and distribution of selected drawings. This verification will be done during a subsequent inspection. It was determined that a large backlog of needed revisions to existing drawings does exist. The as-built drawing revisions are necessary due to plant modifications. Many have been completed for over six months, and the associated as-built drawings have not been revised. This problem was discussed with plant management; they indicated their awareness and concern.

The inspector reviewed records of sixteen special tests. It was determined that safety evaluations were made as required by 10 CFR 50.59.b.

The inspector discussed, with a licensee representative, the techniques used to monitor plant operations and equipment performance to permit

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detection of adverse trends or long-term equipment degradation. It was determined that instrument and equipment failures are being entered into a computer file entitled, "Nuclear Plant Reliability Data System." These data are to be used to help identify generic problems.

#### 6. Licensee Event Report Followup

The below listed licensee event reports were reviewed in-office for potential generic problems, to detect trends, to determine if the information provided meets NRC reporting requirements and to consider whether corrective action discussed appeared appropriate. Selected reports, as denoted by asterisks, were further reviewed at the site to verify adequacy of information provided, corrective action taken and compliance with Technical Specifications and other regulatory requirements. The inspector examined internal reports, correspondence, operating records and discussed events with appropriate personnel.

LER NO.	Date	Event
1-78-52	6/7/78	Demineralized water storage tank low level
*1-78-54	5/15/78	Torus sample line solenoid valve stuck closed
*1-78-59	5/27/78	RCIC suction valve E51-F010 from CST stuck in closed position
*1-78-60	5/26/78	Control rod 12-23 (positon 48) reed switch failure
*1-78-61	6/7/78	Dry well to torus vacuum breaker CAC-X18G open
		position switch inoperable
*1-78-62	6/14/78	Diesel generator building fire detection system inoperable
*1-78-64	7/1/78	Oxygen analyzer CAC-AQH-1259 inoperable
*1-78-65	6/18/78	Oxygen mouitor CAC-ATH-1263 out of calibration
*1-78-66	7/2/78	Torus water temperature high
*1-78-67	6/22/78	Oxygen monitor CAC-AQH-1263 out of calibration
*2-78-45	6/1/78	Oxygen monitor CAC-AT-1263 power failure
*2-78-46	5/20/78	"Full-in" reed switch for control rod 50-19 inoperable

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LER No.	Date	Event
*2-78-50	6/1/78	Drywell oxygen content high
2-78-51	6/2/78	HPCI turbine control response time abonormal
*2-78-52	6/3/78	Personnel airlock door inoperable
*2-78-53	6/2/78	Diesel engine driven fire pump tripped on over-
		speed
*2-78-56	6/22/78	RBM channel "A" failure
2-78-57	6/27/78	RCIC turbine isolation signal on room cooler high differential temperture
*2-78-58	6/30/78	Main steam line low pressure switch B21-PS-N015B out of calibration

A supplemental response was requested for LER 2-78-52. This LER also addressed the inoperability of shutdown cooling isolation valve Ell-F009, but did not discuss the reason for the valve being inoperable. No other problems were identified.

7. IE Circular and Bulletin Followup

IEC 78-14, HPCI Turbine Reversing Chamber Hold Down Bolting. The inspection of the HPCI Turbine has been included in the 1979 outage list for both units. This circulatr is closed.

IEB-78-06, Defective Cutler Hammer, Type M Relays. With DC Coils. The licensee's investigation determined that relays of this type are not being used in safety-related systems or stocked in spare parts. This bulletin is closed.

IEB 78-07, Protection Affored By Air-Line Resperators and Supplied Air Hoods. The licensee stated that the respiratory protection program at the BSEP does not use air supplied hoods or air-line supplied respirators operated in the demand made. This bulletin is closed.

IEB 78-08, Radiation Levels from Fuel Element Transfer Tubes. The BSEP does not have a fuel transfer tube or high radiation areas adjacent to fuel storage. This bulletin is close.

The above listed circular and bulletins were discussed with a licensee representative. The licensee's investigation correspondence and

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findings were reviewed. It was also determined that the licensee's responses to the bulletins were made in a timely manner. No problems were identified.

## 8. Plant Tour

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The inspector made a tour of the Unit 1 and Unit 2 control room. A discussion was held with the control room operators concerning plant status. It was determined that the Unit 2 unidentified dry well leakage was in excess of 4 gpm (limit 5 gpm). The unit was scheduled for shutdown on August 19, 1978, to investigate