



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONNECTICUT YANKEE ATOMIC POWER COMPANY

DOCKET NO. 50-213

HADDAM NECK PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.132
License No. DPR-61

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Connecticut Yankee Atomic Power Company (the licensee), dated August 25, 1990, and as supplemented in a letter dated August 29, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9010260105 901022
PDR ADOCK 05000213
P FDC

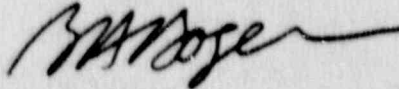
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-21 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 132, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Bruce A. Boger, Assistant Director
for Region I Reactors
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 22, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 132

FACILITY OPERATING LICENSE NO. DPR-61

DOCKET NO. 50-213

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-15

3/4 3-19

3/4 3-21

Insert

3/4 3-15

3/4 3-19

3/4 3-21

TABLE 3.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO ACTUATE</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
3. Auxiliary Feedwater*					
a. Wide Range Stm. Gen. Water Level-- Low					
1) Four Loops Operating	8	2 Channels in any one train	8	1(a)(b)	21
2) Three Loops Operating	6	2 channels in any one train	6	1(a)(b)	26
b. Trip of All Main Feedwater Pumps	1/pump	1 from each pump	1/pump	1(a)	26
4. Emergency Bus Undervoltage					
a. 4.16 kV Bus Under- voltage-level 1	3/bus	2/bus	2/bus	1, 2, 3, 4	24
b. 4.16 kV Bus Undervoltage- Level 2	3/bus	2/bus	2/bus	1, 2, 3, 4	24
c. 4.16 kV Bus Undervoltage- Level 3	3/bus	2/bus	2/bus	1, 2	24

- * For Cycle 16 operation only, OPERABILITY of automatic initiation of auxiliary feedwater (AFW) is defined as including (1) credit for operator action to adjust AFW to full required flow following automatic initiation and (2) reliance on the control air system to ensure successful automatic AFW initiation. Modifications will be implemented by the end of the Cycle 16 refueling outage, prior to startup for Cycle 17, to remove reliance on operator action and the control air system for successful automatic initiation of AFW.

TABLE 3.3-3

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. Safety Injection (Reactor Trip, Start Diesel Generators, Containment Isolation).		
a. Manual Initiation	N.A.	N.A.
b. Containment Pressure--High	≤ 4.7 psig	≤ 5.0 psig
c. Pressurizer Pressure--Low	≥ 1720 psig	≥ 1700 psig
2. Steam Line Isolation		
a. Steam Flow in Two Steam Lines--High	$\leq 108\%$ of full* steam flow.	$\leq 110\%$ of full* steam flow.
3. Auxiliary Feedwater**		
a. Steam Generator Water Level--Low	$\geq 47\%$ of wide range instrument span.	$\geq 45\%$ of wide range instrument span.
b. Trip of All Main Feedwater Pumps	N.A.	N.A.

** For Cycle 16 operation only, OPERABILITY of automatic initiation of auxiliary feedwater (AFW) is defined as including (1) credit for operator action to adjust AFW to full required flow following automatic initiation and (2) reliance on the control air system to ensure successful automatic AFW initiation. Modifications will be implemented by the end of the Cycle 16 refueling outage, prior to startup for Cycle 17, to remove reliance on operator action and the control air system for successful automatic initiation of AFW.

TABLE 4.3-2

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

<u>CHANNEL FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Safety Injection (Reactor Trip, Start Diesel Generators, Containment Isolation)					
a. Manual Initiation	N.A.	N.A.	N.A.	R	1, 2, 3
b. Containment Pressure- high	D	R	SW	N.A.	1, 2, 3
c. Pressurizer Pressure- Low	S	R	SW	N.A.	1, 2, 3
2. Steam Line Isolation					
a. Steam Flow in Two Steam Lines-High	S	R	R	N.A.	1, 2, 3
3. Auxiliary Feedwater*					
a. Steam Generator Water Level-Low	S	R	M	N.A.	1
b. Trip of All Main Feedwater Pumps	N.A.	N.A.	N.A.	R	1

* For Cycle 16 operation only, OPERABILITY of automatic initiation of auxiliary feedwater (AFW) is defined as including (1) credit for operator action to adjust AFW to full required flow following automatic initiation and (2) reliance on the control air system to ensure successful automatic AFW initiation. Modifications will be implemented by the end of the Cycle 16 refueling outage, prior to startup for Cycle 17, to remove reliance on operator action and the control air system for successful automatic initiation of AFW.