UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Detroit Edison Company (the licensee) dated February 1, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

   Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 60 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert C. Pierson, Director
Project Directorate III-1
Division of Reactor Projects - III,
IV, V & Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 12, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 3-15	3/4 3-15
3/4 3-16	3/4 3-16
3/4 3-17	3/4 3-17
3/4 3-17a	3/4 3-17a

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Low Water Level		
1) Level 3	$\geq$ 173.4 inches*	$\geq$ 171.9 inches
2) Level 2	$\geq$ 110.8 inches*	$\geq$ 103.8 inches
3) Level 1	$\geq$ 31.8 inches*	$\geq$ 24.8 inches
b. Drywell Pressure - High	$\leq$ 1.68 psig	$\leq$ 1.88 psig
c. Main Steam Line		
1) Radiation - High	$\leq$ 3.0 x full power background	$\leq$ 3.6 x full power background
2) Pressure - Low	$\geq$ 756 psig	$\geq$ 736 psig
3) Flow - High	$\leq$ 137.9% of rated flow/109.0 psid	$\leq$ 139.5% of rated flow/112.0 psid
d. Main Steam Line Tunnel Temperature - High	$\leq$ 200°F	$\leq$ 206°F
e. Condenser Pressure - High	$\leq$ 6.85 psia	$\leq$ 7.05 psia
f. Turbine Bldg. Area Temperature - High	$\leq$ 200°F	$\leq$ 206°F
g. Deleted		
h. Manual Initiation	NA	NA

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
2. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High	≤ 55.1 gpm	≤ 63.4 gpm
b. Heat Exchanger/Pump/High Energy Piping Area Temperature - High	≤ 175°F	≤ 183°F
c. Heat Exchanger/Pump/Phase Separator Area Ventilation Δ Temperature - High	≤ 50°F	≤ 53°F
d. SLCS Initiation	NA	NA
e. Reactor Vessel Low Water Level - Level 2	≥ 110.8 inches*	≥ 103.8 inches
f. Deleted		
g. Manual Initiation	NA	NA
3. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION		
a. RCIC Steam Line Flow - High		
1. Differential Pressure	≤ 87.0 inches H_2O	≤ 95.0 inches H_2O
2. Time Delay	3 seconds	3±2 seconds
b. RCIC Steam Supply Pressure - Low	≥ 62 psig	≥ 53 psig
c. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10 psig	≤ 20 psig
d. RCIC Equipment Room Temperature - High	≤ 154°F	≤ 162°F
e. Manual Initiation	NA	NA

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
4. HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION		
a. HPCI Steam Line Flow - High		
1. Differential Pressure	$\leq$ 395.0 inches H_2O	$\leq$ 410.0 inches H_2O
2. Time Delay	3 seconds	3±2 seconds
b. HPCI Steam Supply Pressure - Low	$\geq$ 100 psig	$\geq$ 90 psig
c. HPCI Turbine Exhaust Diaphragm Pressure - High	$\leq$ 10 psig	$\leq$ 20 psig
d. HPCI Equipment Room Temperature - High	$\leq$ 154°F	$\leq$ 162°F
e. Manual Initiation	NA	NA
5. RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION		
a. Reactor Vessel Low Water Level - Level 3	$\geq$ 173.4 inches*	$\geq$ 171.9 inches
b. Reactor Vessel (Shutdown Cooling Cut-in Permissive Interlock) Pressure - High	$\leq$ 89.5 psig***	$\leq$ 95.5 psig***
c. Manual Initiation	NA	NA

*Above TAF. See Bases Figure B 3/4 3-1.

***Represents steam dome pressure; actual trip setpoint is corrected for cold water head with reactor vessel flooded.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
6. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Vessel Low Water Level - Level 2	$\geq$ 110.8 inches*	$\geq$ 103.8 inches
b. Drywell Pressure-High	$\leq$ 1.68 psig	$\leq$ 1.88 psig
c. Fuel Pool Ventilation Exhaust Radiation-High	$\leq$ 5 mR/hr	$\leq$ 6 mR/hr
d. Manual Initiation	NA	NA

*Above TAF. See Bases Figure B 3/4 3-1.