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Prepared for the U.S. Nuclear Regulatory Commission
Office of Nuclear Regulatory Research
Under Interagency Agreements DOE 40-551-75 and 40-552-75

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ABSTRACT

Fission-product decay energy-release rates have been measured for thermal-neutron fission of ^{235}U. Samples of mass 1 to 10 μgm were irradiated for 1 to 100 sec using the fast pneumatic-tube facility at the Oak Ridge Research Reactor. The resulting beta- and gamma-ray emissions were counted for times-after-fission between 2 and 14,000 secs. The data were obtained for beta and gamma rays separately as spectral distributions, $N(E_\gamma)$ vs E_γ and $N(E_\beta)$ vs E_β. For the gamma-ray data the spectra were obtained using a NaI detector, while for the beta-ray data the spectra were obtained using an NE-110 detector with an anticoincidence mantle. The raw data were unfolded to provide spectral distributions of moderate resolution. These distributions are given in graphical and tabular form as differential cross-section values of $d\sigma/dE$/fission for gamma-ray energy intervals ranging from 10 keV for $E_\gamma < 0.18$ MeV to 100 keV for $E_\gamma > 6.8$ MeV, and beta-ray energy intervals ranging from 20 keV for $E_\beta < 0.25$ MeV to 160 keV for $E_\beta > 6.4$ MeV. Counting-time intervals range from 1 sec for times-after-fission (t_w) < 6 sec to 4000 sec for $t_w = 10^4$ sec. The graphical representations also include calculated spectra using summation methods and the ENDF/B-IV fission yield and decay scheme data base.

I. INTRODUCTION

In a recently issued report (hereinafter referred to as "Ref. 1") data were reported on measurements of the total beta and gamma energy release rate from fission-product decay as a function of time following thermal-neutron fission of the element ^{235}U. The total energy-release rate was obtained by measuring separately the energy-release rates for each component. That is, one set of data was obtained for gamma energy-release using a gamma-ray detector, and another set of data was obtained for beta decay energy-release using a beta-ray detector. Data were obtained by detecting and measuring individual events for several irradiation times (t_{irrad}), waiting times following the end of irradiation (t_{wait}), counting times starting at the end of the waiting time (t_{count}), and particle energies (E_β or E_γ). The resulting energy spectra were integrated over particle energy to obtain the energy release for each component for every combination of t_{irrad}, t_{wait}, and t_{count}, and the data were reported in this form in Ref. 1.

A total of 86 differential data sets were measured, 43 for beta-ray energy release each containing 186 data, and 43 for gamma-ray energy release each containing 350 data, the data consisting of the differential yield, $N(E_\beta)$ or $N(E_\gamma)$, and uncertainties, $\Delta N(E_\beta)$ or $\Delta N(E_\gamma)$, as a function of E_β or E_γ. In this report these 86 sets of differential data are presented in tabular and graphical form. Comparisons are also made with summation calculations using CINDER[2] for the beta-ray spectra and ORIGEN[3] for the gamma-ray spectra, both calculations using the ENDF/B-IV data base[4] with the "^{98}Zr" correction.[5] In Ref. 1 we observed that the summation method,

> "..., a large calculational problem, has the merit that once it is proven to reproduce measurements satisfactorily, it could be applied to more complex situations in a reactor, taking into account for example reactor power variations or the evolution of fissionable isotopes as a function of reactor operation."

In our opinion the most important use of the data given in this report will be to guide the calculational efforts to a point where they can be used to compute correctly the fission-product decay heat for any operation history.

II. EXPERIMENTAL METHOD

The experimental method has been presented in complete detail in Ref. 1. We present in this report only that information required to understand the format of the present data; the verification of the techniques are given in Ref. 1 and will not be presented here.

A schematic representation of the experimental arrangement is shown in Fig. 1. Small samples (1, 5 and 10 μgm) of ^{235}U were fabricated. These samples were put into pneumatic carriers known as rabbits, and transported to the Oak Ridge Research Reactor (ORR) for irradiation by thermal neutrons. The periods of irradiation were 1, 10, and 100 sec. The samples were rapidly recovered following irradiation, so that measurements could begin in less than 2 sec following the end of irradiation. All rabbit movements and data accumulation were controlled by the PDP-15 computer shown in Fig. 1. Following the irradiation and spectral measurements the samples were cooled (allowed to decay) for varying periods and then counted for characteristic gamma rays associated with decay of ^{97}Nb

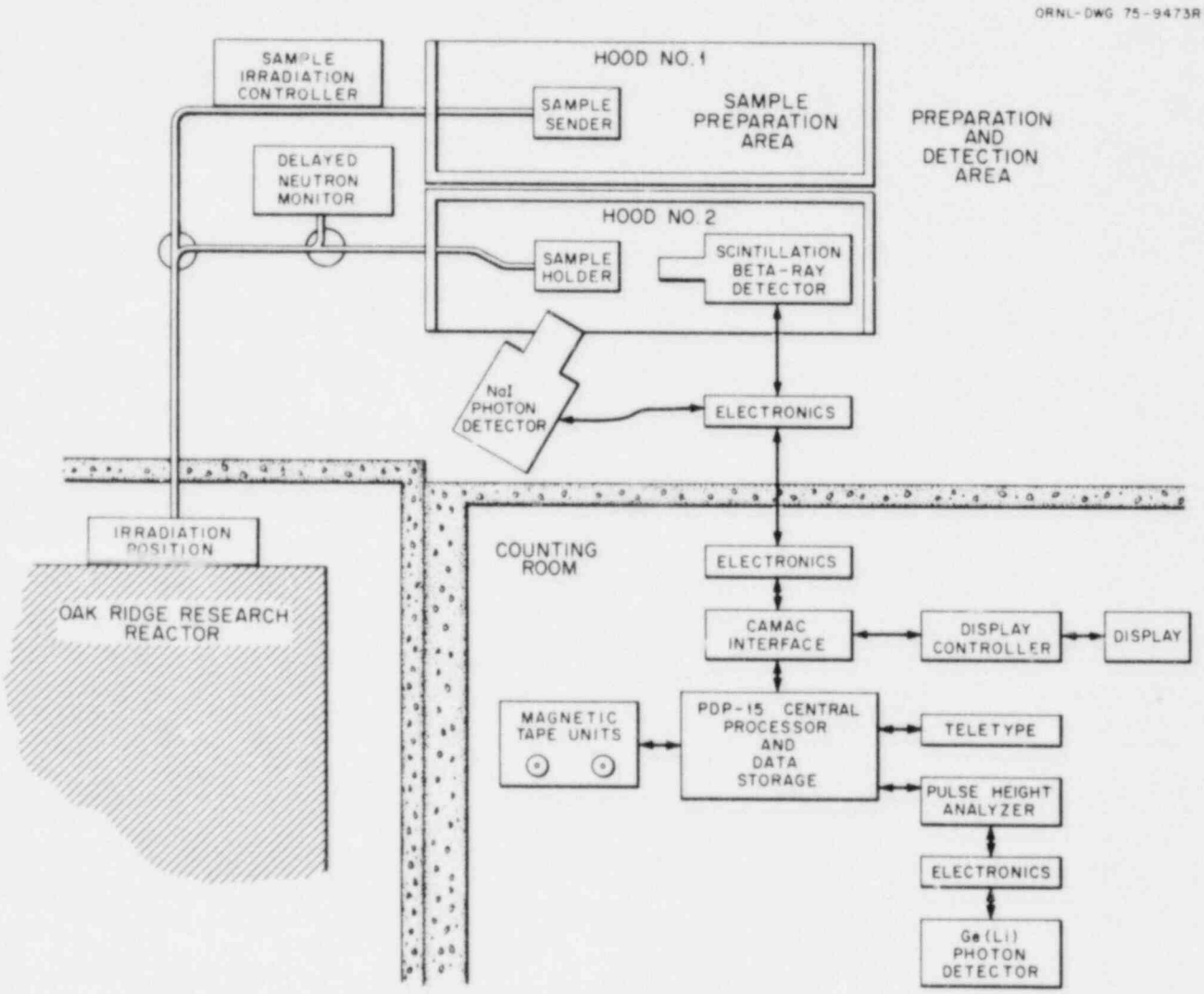

Fig. 1. Schematic Representation of Experimental Arrangement for Fission-product Decay-heat Measurements. Samples of ^{235}U are placed in a rabbit and put in the Sample Sender. The Central Processor controls the movement of the rabbit to and from the Irradiation Position, then to the Sample Holder. After a specified cooling time, either photons or beta rays are counted, and the data are stored in the computer. At the completion of data accumulation the data are stored on magnetic tape for offline reduction.

(from ^{97}Zr), ^{99}Mo, and ^{132}Te to obtain the number of fissions (n_f) that were created in the sample by the irradiation. The measurements of n_f are believed accurate to ±1.5% (Ref. 1).

The experiment required measurements of both beta and gamma radiation, and Fig. 1 shows the positions of both detectors. However, only one can be used at a time. The gamma-ray detector is shown in Fig. 2 and the beta-ray detector is shown in Fig. 3. For the gamma-ray detector background was measured by running a blank sample. For the beta-ray detector two samples had to be run. One sample was measured using magnetic deflection (see caption of Fig. 3 for position of magnetic field) measuring gamma rays (γ) only, and a second sample was measured without magnetic deflection measuring both betas and gamma ($\beta + \gamma$). From these data the contribution of the beta rays was determined by subtracting the "magnet-up" data from the "magnet-down" data. This operation is equivalent to

$$(\beta) + (\beta + \gamma) - (\gamma) \qquad (1)$$

During the course of this experiment many hundreds of irradiations were required (almost 200 for the final data-taking runs). The software for the PDP-15 was designed to provide an efficient and reproducible method of controlling and monitoring each irradiation and subsequent beta- or gamma-ray counting, and also be simple to use. A complete description and listing of the computer program is available in a separate report.[6] The primary purpose of the electronics, shown in Fig. 4, is to process information related to the event which occurred in the detector and to send the processed data to the computer. In addition, the system is designed to have methods for verifying its own working order which do

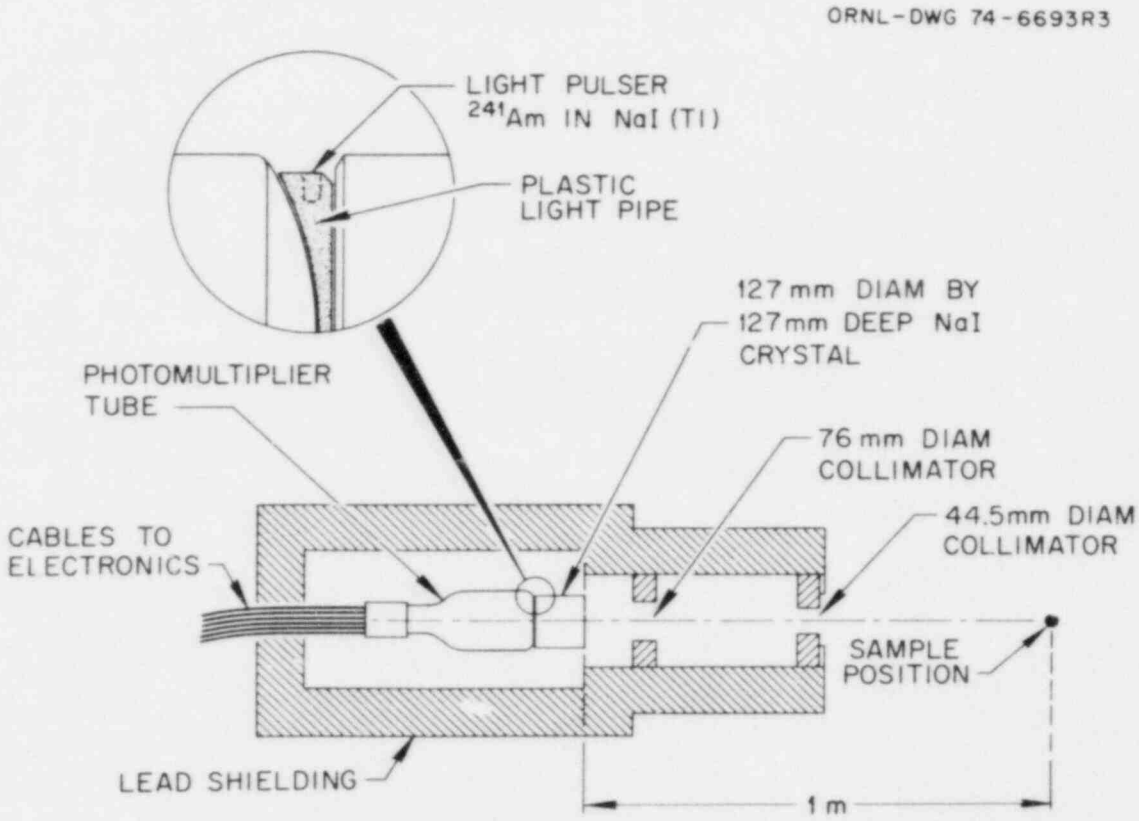

Fig. 2. Gamma-ray Detector Arrangement. The detector is enclosed in a lead cave, 0.1 m thick on the top, sides, and bottom. The inset shows the position of the alpha source used as a light pulser to monitor possible gain shift. For beta-ray deflection a permanent magnet was positioned between the 44.5 mm diam collimator and the sample. The ^{241}Am "Light Pulser" is used to monitor the gain of this detector during the measurements.

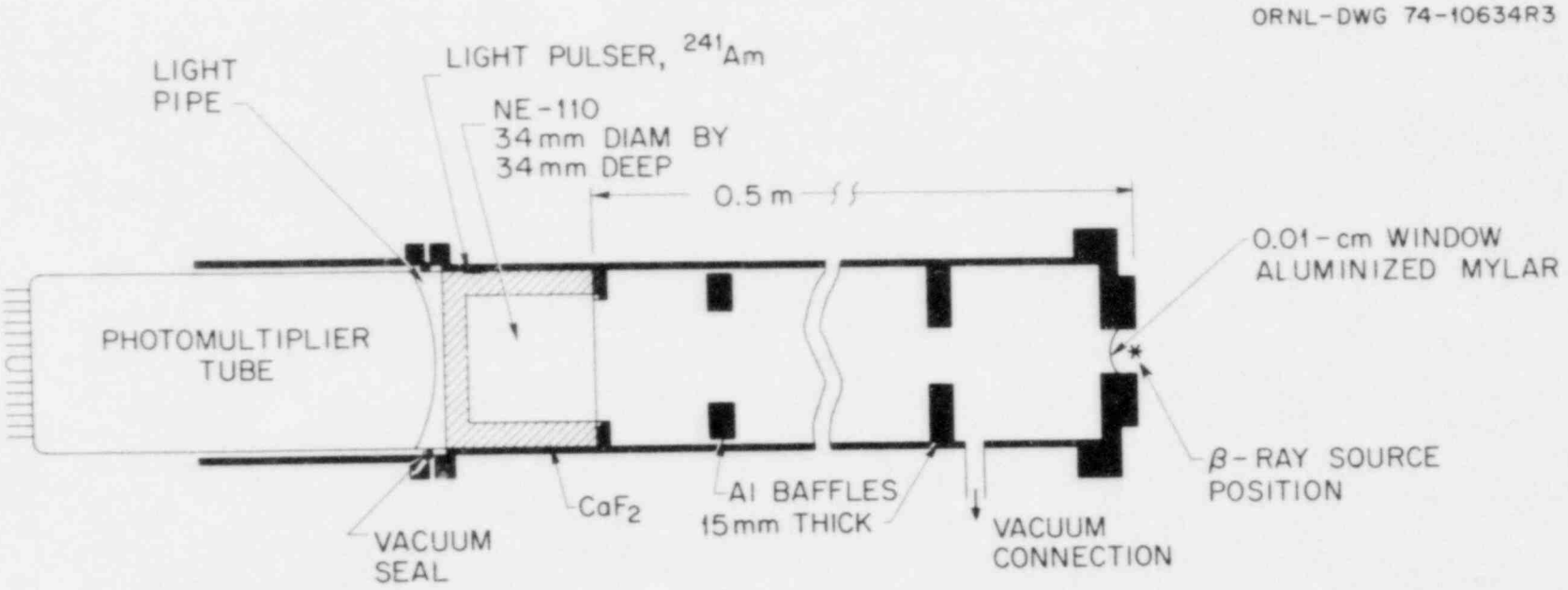

Fig. 3. Beta-ray Scintillation Spectrometer. The beta-ray source position is about 8 mm from the entrance foil. For some measurements a strong magnetic field (~2700 gauss) is placed between the entrance foil and the first collimator perpendicular to the path between source and detector. The "Light Pulser, ^{241}Am" is used to monitor possible gain shifts during data taking.

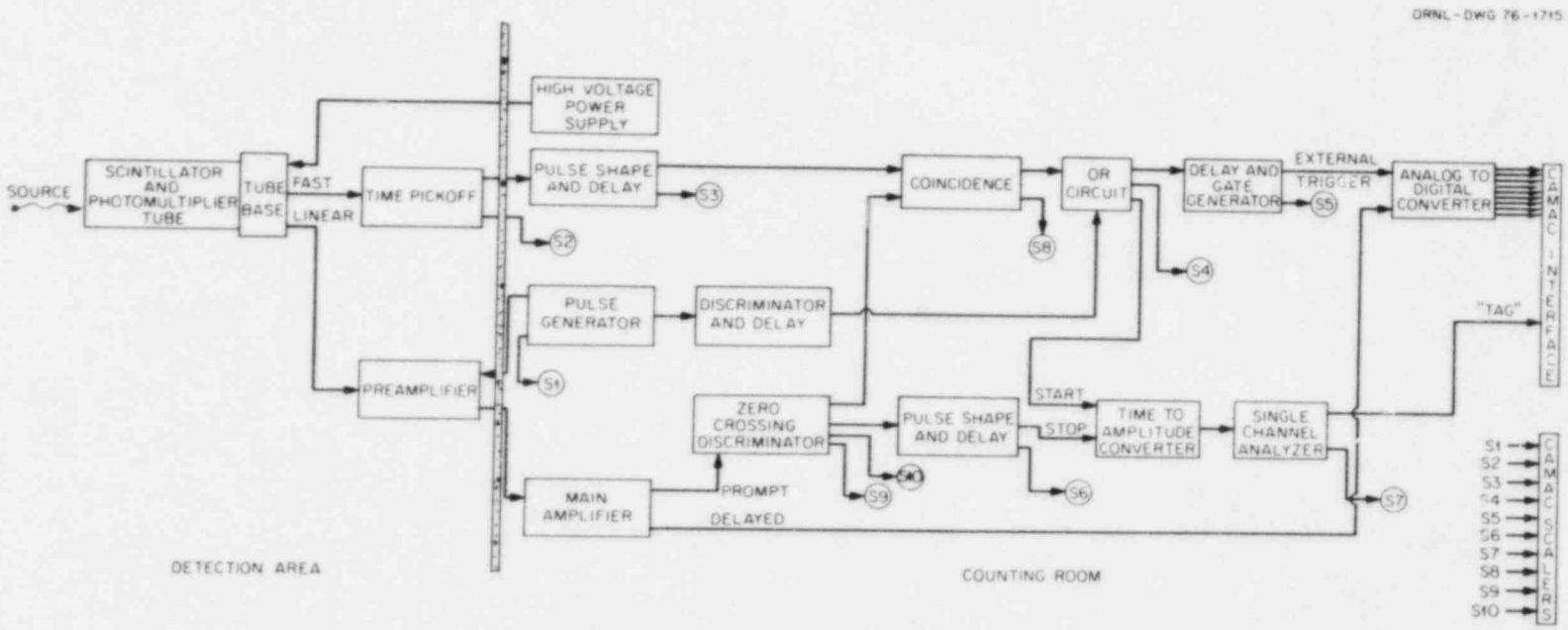

Fig. 4. Electronics Block Diagram. This circuitry is set up for pulse amplification and pulse-shape discrimination. Data are accumulated as two 512-channel spectra, with the "TAG" bit used to differentiate data corresponding to desired events in the detector from data corresponding to monitoring events in the detector.

not interfere with the measurements.

The Main Amplifier (see Fig. 4) was used at two gain settings, a Low setting corresponding to ~8 MeV full scale (and ~0.2 MeV lower-level cutoff), and a High setting corresponding to ~2 MeV full scale (and ~0.05 MeV lower-level cutoff). For each gain setting, and for each t_{irrad}, t_{wait}, and t_{count}, and detector configuration, more than one measurement had to be made to obtain enough data for statistical accuracy. Thus, prior to unfolding, the addition of data from several equivalent runs was required. This "addition" included background subtraction in the case of gamma-ray measurements, and subtraction of "magnet-up" data from "magnet-down" data in the case of beta-ray measurements. Then the data for the High-gain setting were combined with the data for the Low-gain settings to provide one set of data for each t_{irrad}, t_{wait}, t_{count}, and pulse height.

The raw spectral data, corrected for background and electronic dead time, were than binned by energy (E_β or E_γ) and unfolded using the FERD[7] code. This code required a response matrix for the detector on which the measurements were made. Some of the responses in the gamma-ray response matrix are shown in Fig. 5; these were determined from careful study of gamma-ray sources having $0.06 \leq E_\gamma \leq 7.1$ MeV. Note that the full-energy peak, characterized by a Gaussian width σ given by

$$\sigma = 0.01\, E_\gamma (1.352 + 5.064/\sqrt{E_\gamma})/2.35482 \qquad (2)$$

is the primary response for $E_\gamma \leq 3$ MeV, and the plurality of the total response for $E_\gamma \leq 6$ MeV. This feature of the response matrix arose from the choice of collimators shown in Fig. 2.

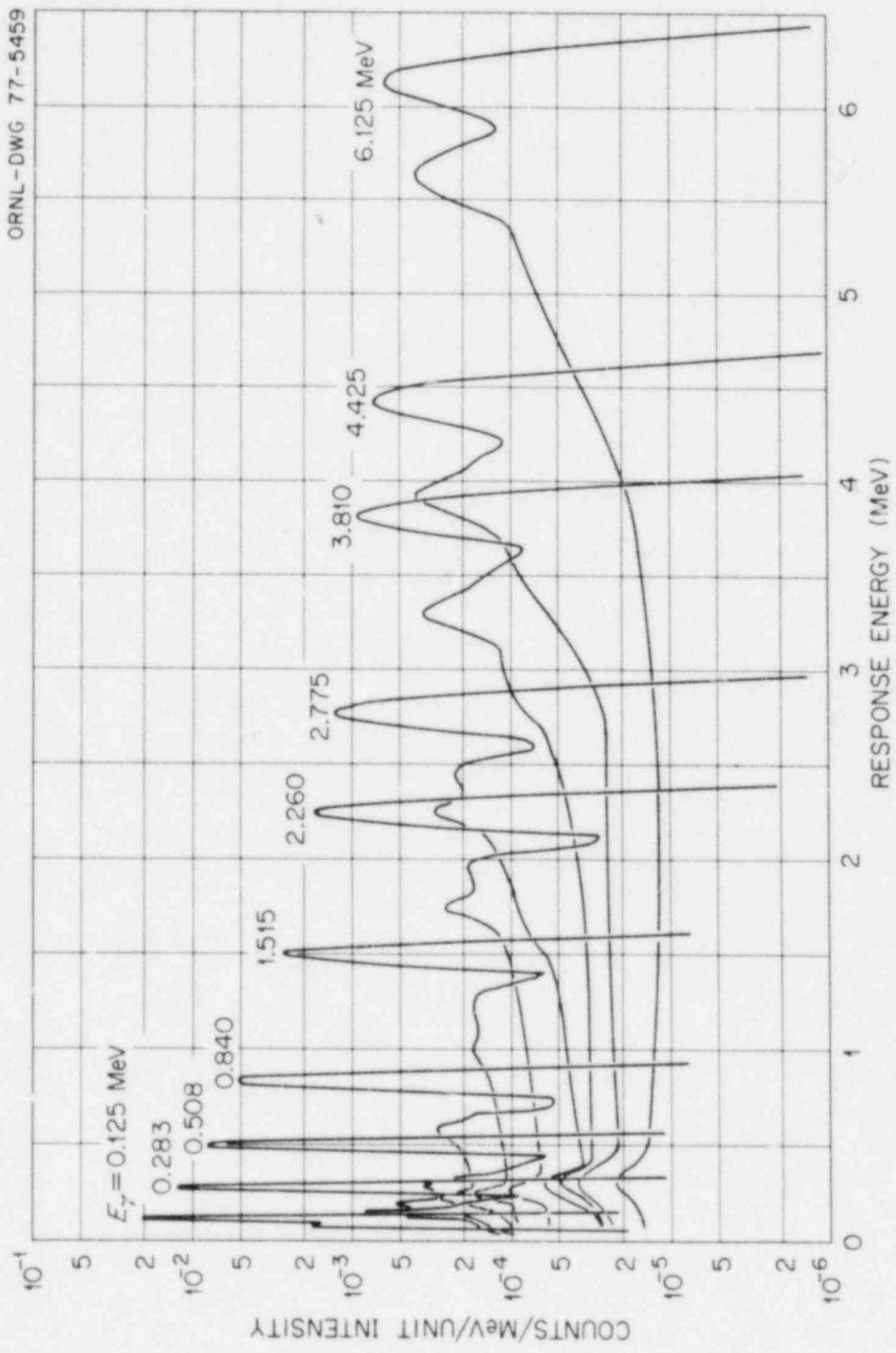

Fig. 5. Examples of Responses of the Gamma-ray Detector to Monoenergetic Gamma Rays.

Some of the responses in the beta-ray spectrometer response matrix shown in Fig. 6. Those for $E_\beta \leq 1$ MeV were determined from study of conversion-electron (monoenergetic) sources having $0.13 \leq E_\beta < 0.98$ MeV. For $E_\beta > 1$ MeV we had to rely on beta-decay distributions. What was done was to extrapolate for $E_\beta > 1$ MeV such parameters as resolution, peak-to-total, and efficiency determined for $E_\beta < 1$ MeV, and then to iterate on these parameters to obtain a response matrix which yielded measured beta-decay distributions (e.g. from decay of ^{106}Ru, ^{144}Pr, ^{20}F, etc.) after unfolding which agreed with calculated spectra. The full-energy peak is characterized by a Gaussian width, σ, given by

$$\sigma = 0.01\, E_\beta \sqrt{25.0 + 90.0/E_\beta}/2.35482 \qquad (3)$$

These responses (of Fig. 6) can be compared with those obtained by Wohn et al.[8] for a 65 mm diam by 58 mm deep cylindrical Pilot B detector. Despite some geometrical differences in the experiments the responses are quite similar, each response being primarily a Gaussian peak and a low-energy tail. The major difference is that the low-energy tail of the Wohn system remains finite for zero pulse height.

III. UNCERTAINTIES

The final spectral distributions are similar to spectra expected if the detector response were a pure Gaussian distribution with a characteristic width σ. If the detector possessed this ideal Gaussian response each datum in the spectrum would have an associated uncertainty, but since the detector response is not ideal, there is an added uncertainty incurred in the process of the transformation (i.e., "unfolding") from measured data to final spectrum.

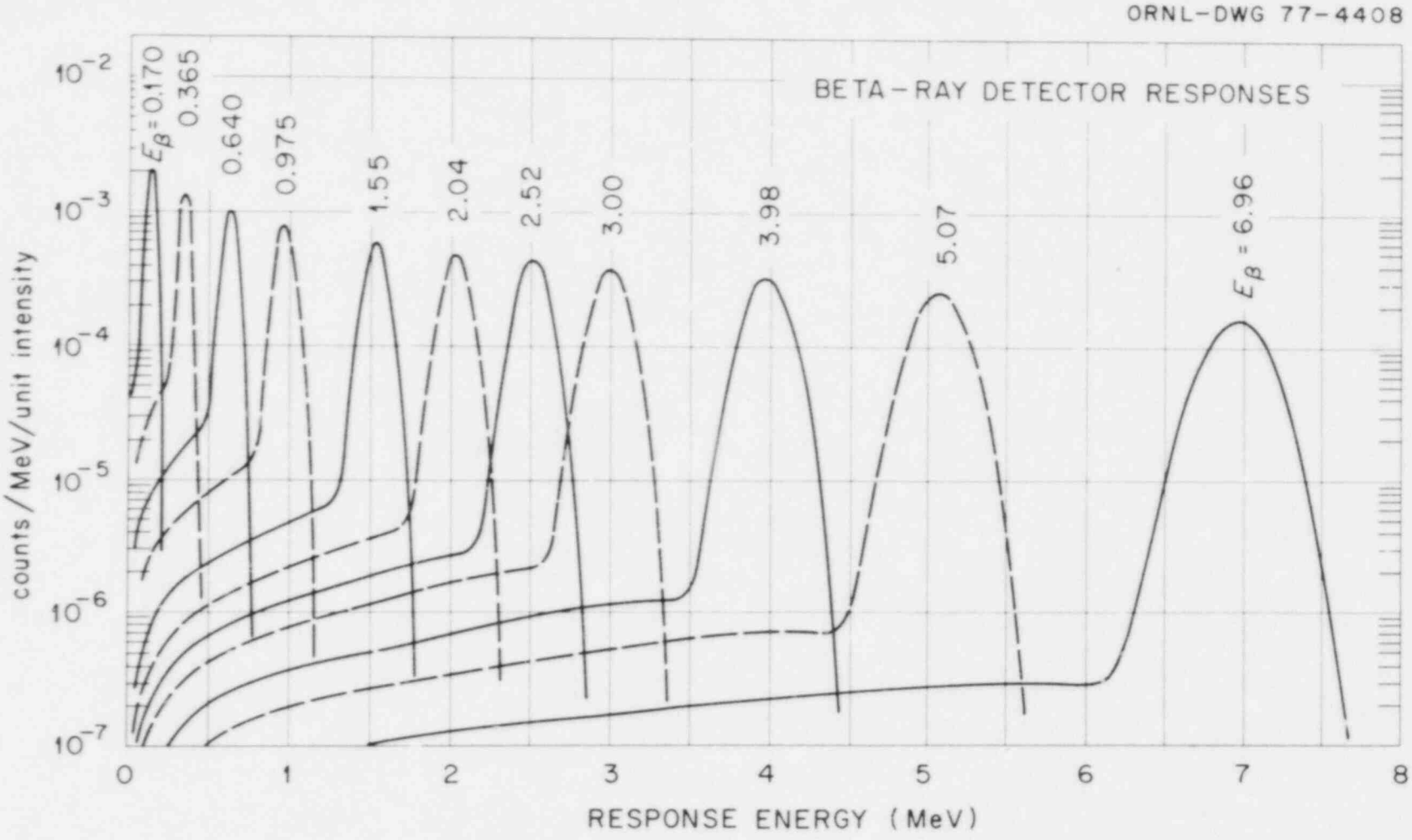

Fig. 6. Examples of Responses of the Beta-ray Detector to Monoenergetic Beta Rays.

The total uncertainties thus derived (the "confidence interval" as output from the FERD[7] unfolding code) are almost independent, having only a small, and short-range correlation, on the order of σ in energy.

The spectra are corrected for dead time and normalized to the number of fissions, n_f; however uncertainties for these operations are not included in the uncertainty assigned to each individual datum because these uncertainties are fully correlated. There are also corrections due to fission-gas losses which are included in the integral results of Ref. 1 but not added into the spectral data nor are these corrections to the individual uncertainties due to fission-gas loss. The lack of further correction to the one-standard deviation confidence intervals means that the uncertainty assigned to each datum in this report does not represent a true standard deviation but somewhat less than a standard deviation.

GRAPHICAL AND TABULAR PRESENTATION OF THE SPECTRA

Following the acknowledgments are 43 figures showing our measured and unfolded beta-ray energy release spectra (Figs. 7 to 49), and following these figures are 43 figures showing our measured and unfolded gamma-ray energy release spectra (Figs. 50 to 92). The calculated beta-ray spectra shown on Figs. 7 to 49 are courtesy of England and Stamatelatos.[9] Some of these results have already been presented in a report by England, et al.[10]; the exact use of the data in the file is given in that report as follows:

> "The spectral calculations rely on the spectral data for the 180 fission products (among a total of 824) that are available in the ENDF/B files. The calculated spectra were normalized so that energy integration over the spectrum produces the total calculated energy release (beta or gamma) from all 824 fission products in

ENDF/B-IV. In other words, the spectral shapes are determined by 180 fission products while their magnitudes are determined by all 824 fission products. The comparisons with experiment are absolute.

Three figures of our beta-ray spectra were taken from Ref. 1, and they show in addition data from previous measurement of beta-ray spectra obtained by Tsoulfanidis, et al.[11] The agreement between the two sets of experimental data is quite good for the data shown in Figs. 30 and 41, not so good for the data shown in Fig. 12. The agreement between calculated spectra and the present measurements is very good for $E_\beta > 3$ MeV, but the calculations tend to overpredict for $1.5 < E_\beta < 3$ MeV, and underpredict for $E_\beta < 1.5$ MeV. One possible explanation for some of the underprediction at low energies is the lack of conversion electron contributions to the calculated spectra. (See, for example, a calculated ^{137}Cs beta-decay spectrum[12] compared with our measurements in Ref. 1).

The calculated gamma-ray spectra shown in Figs. 50 to 92 were obtained using the Oak Ridge summation code ORIGEN and gamma-ray data in the ENDF/B-IV data file (with the "^{98}Zr" correction) containing data for ≈800 fission-product nuclides. However, the contributions from the ≈600 nuclides lacking experimental data are not correctly treated in these calculations. For these nuclides only an average gamma ray energy is given in the file; the calculation treats this <u>average</u> gamma-ray energy as if it were the energy of a <u>single</u> gamma ray due to decay of the nuclide. Particularly for short t_{wait}, the calculated spectra do not agree well with the experimental spectra.

Therefore, for t_{wait} <100 sec, two calculated curves are shown in the figures, one (the line) representing an ORIGEN calculation using only data from the 180 fission-product nuclides in the file containing spectroscopic

data, and another (the histogram) representing data from the ≈600 nuclides having only an average gamma-ray energy in the file. These spectra (Figs. 50 to 71) are also plotted as Gamma-Ray Energy Times Yield vs Gamma-Ray Energy on linear scales to emphasize the contributions to the gamma-ray energy release. For t_{wait} <50 sec the gamma-ray energy-integral contributions from the ≈600 "unknown" nuclides are greater than the gamma-ray energy-integral contributions from the 180 known nuclides. In Table 1 we give the contributions from the 180 known and ≈600 unknown nuclides to the calculated gamma-ray energy release. For t_{wait} >100 sec the figures show the data as yield vs gamma-ray energy on semi-logarithmic scale with one solid-line curve representing the sum of the yields calculated from the 180 known nuclides and yields calculated for the ≈600 unknown nuclides in the file. The important "false peaks", that is prominent peaks containing significant contributions from the ≈600 unknown nuclides, are indicated in the figure captions.

We point out a useful aspect of these comparisons, and that is that although the resolution of the NaI detector is much less than Ge(Li) spectroscopy, the resolution is sufficient so that the comparisons shown in these figures should be very helpful in locating entries in the data file needing improvements.

Following the figures of gamma-ray spectra are the tabular data. The energy (E_β or E_γ) is in MeV, and the yield and uncertainty are in units of particles/MeV/fission on a pointwise basis (not histogram basis). The data are available from the authors on punched cards.

Table 1. Gamma-ray Energy Release from Fission Created by Thermal-Neutron Fission of $^{235}U^{a}$ (in MeV/fission)

Irradiation Time (sec)	Waiting Time (sec)	Counting Time (sec)	Experiment	Calculation		
				180[b]	≈600[c]	Total
1.0	1.7	1	0.188	0.028	0.122	0.150
	2.7	1	0.146	0.024	0.096	0.120
	3.7	1	0.119	0.021	0.078	0.099
	4.7	2	0.188	0.036	0.124	0.160
	6.7	3	0.210	0.045	0.140	0.180
	9.7	5	0.248	0.063	0.166	0.229
	14.7	5	0.182	0.054	0.120	0.174
	19.7	5	0.143	0.047	0.092	0.139
	24.7	10	0.223	0.084	0.134	0.218
	34.7	10	0.170	0.074	0.093	0.167
	44.7	15	0.199	0.097	0.097	0.194
	59.7	15	0.154	0.084	0.066	0.150
	75	15	0.124	0.074	0.047	0.121
	90	20	0.131	0.086	0.044	0.130
10.0	10.7	6	0.216	0.064	0.138	0.202
	16.7	8	0.211	0.073	0.132	0.204
	24.7	10	0.198	0.079	0.114	0.193
	34.7	10	0.156	0.070	0.081	0.151
	44.7	10	0.127	0.063	0.061	0.124
	54.7	20	0.197	0.111	0.084	0.195
	74.7	20	0.149	0.094	0.053	0.147
	95	20	0.117	0.080	0.038	0.118

[a] See Table 12 of Ref. 1.

[b] For the 180 fission-product nuclides in the ENDF/B-IV file having spectroscopic information.

[c] For the ≈600 nuclides having only an average gamma-ray energy datum in the ENDF/B-IV data file.

IV. ACKNOWLEDGMENTS

This project succeeded because of the gracious contributions of time and talent by many individuals. We are very grateful for the contributions of all of the persons acknowledged in Ref. 1, and in particular for the present report, to T. England and M. Stamatelatos (LASL) for providing the calculations shown in Figs. 7 to 49, to O. W. Hermann (UC-CSD) for assistance with the ORIGEN code, to J. H. Bratten and W. C. Colwell for assistance in the preparation of the many figures, and to P. L. McNutt for manuscript preparation.

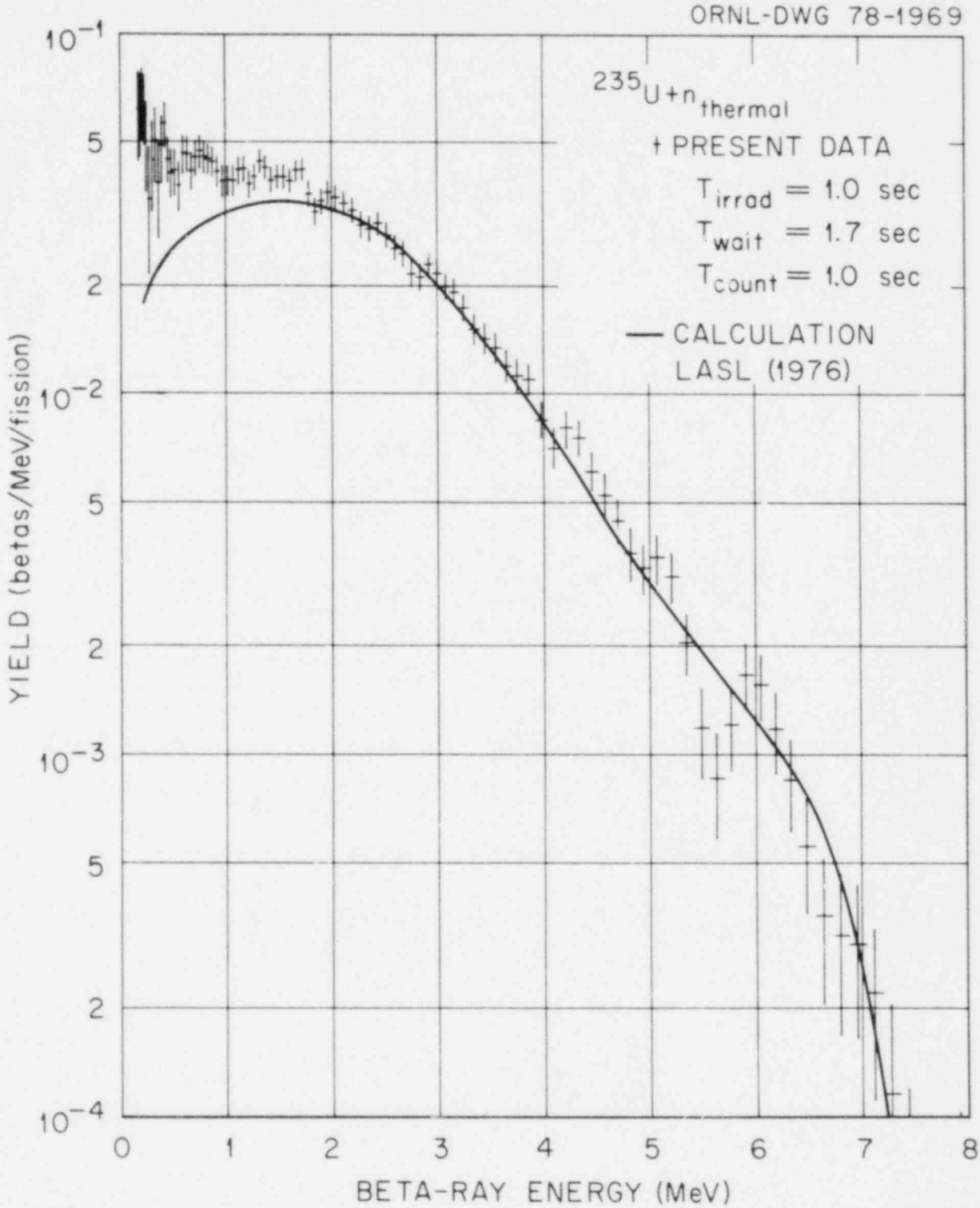

Fig. 7. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

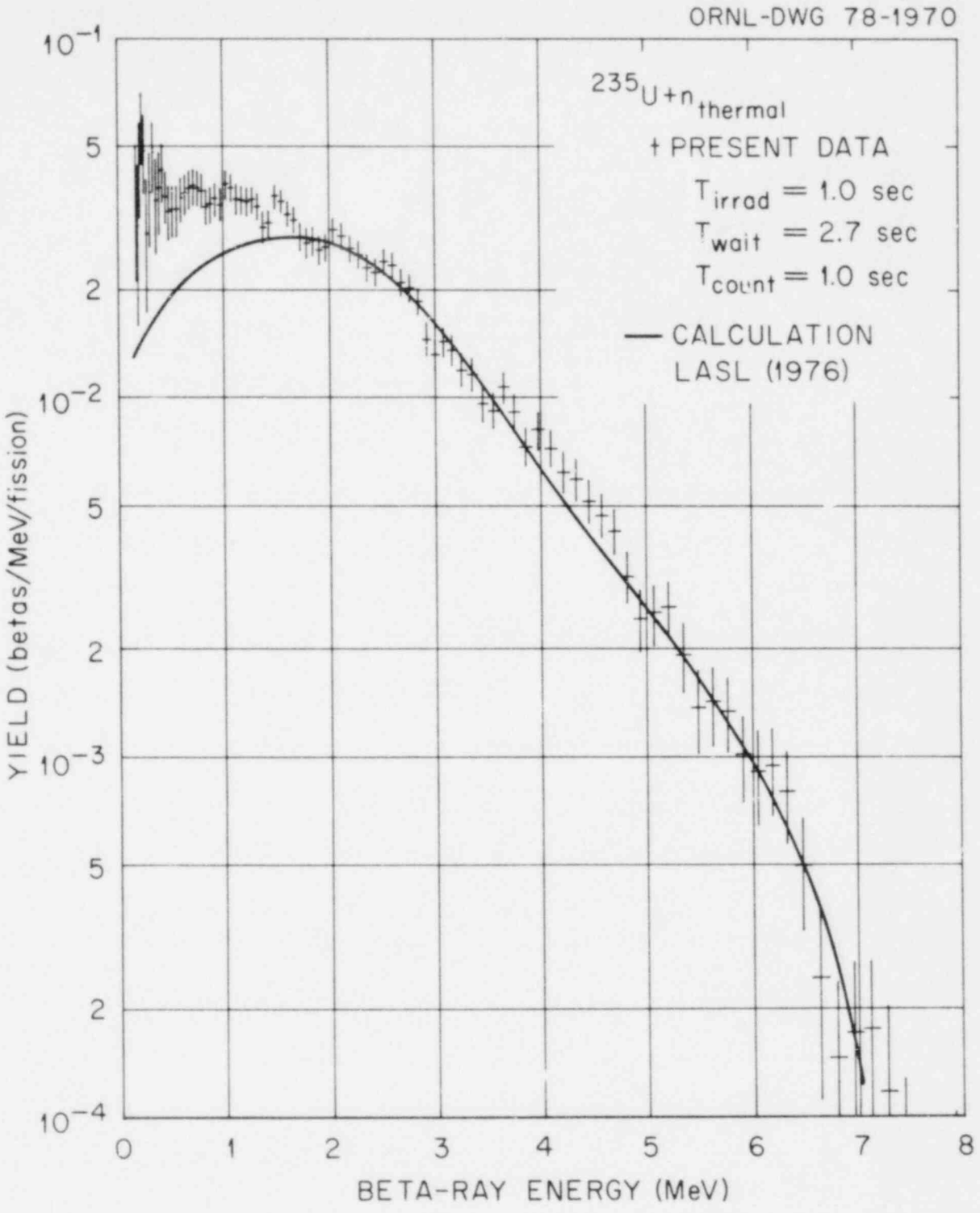

Fig. 8. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

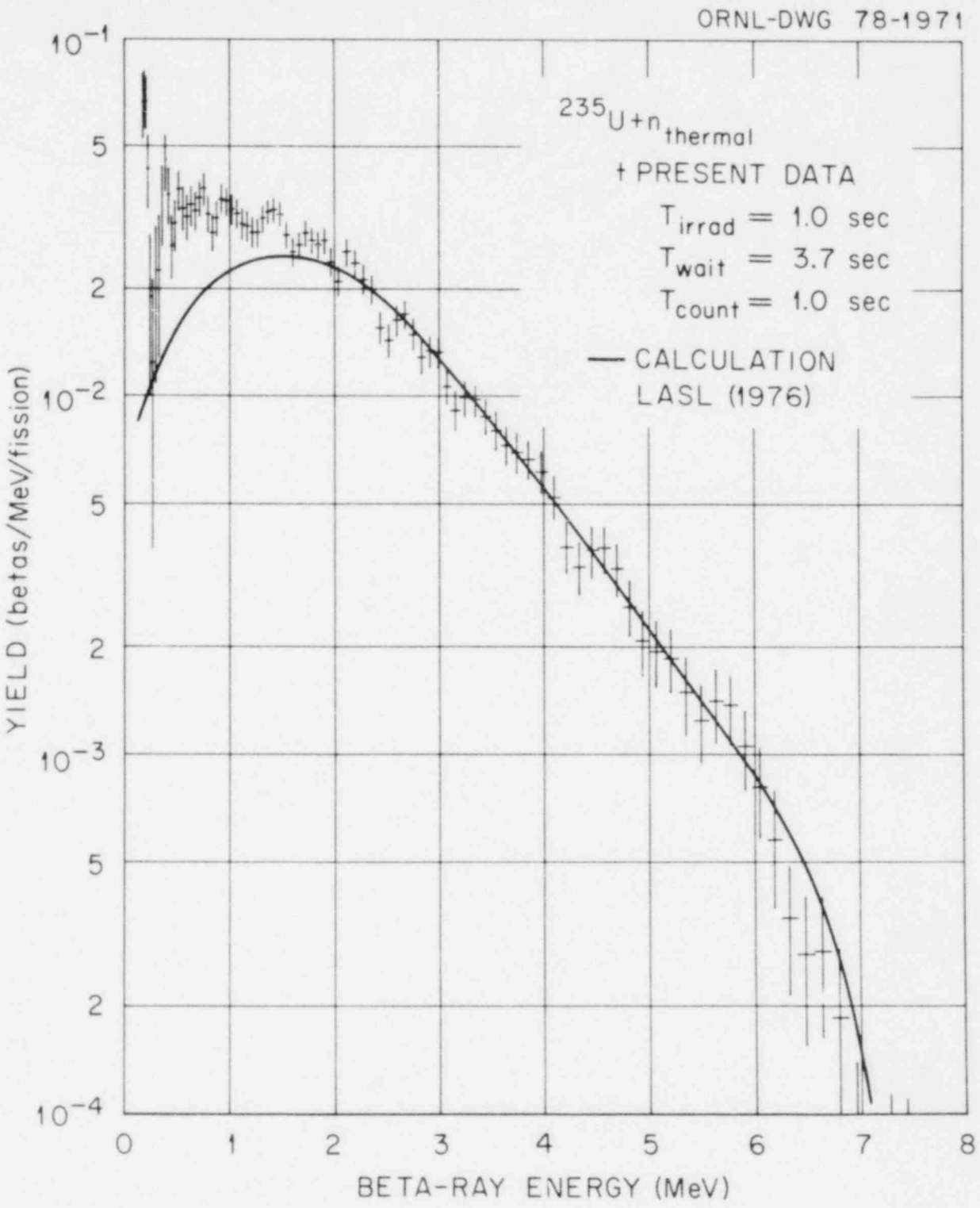

Fig. 9. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

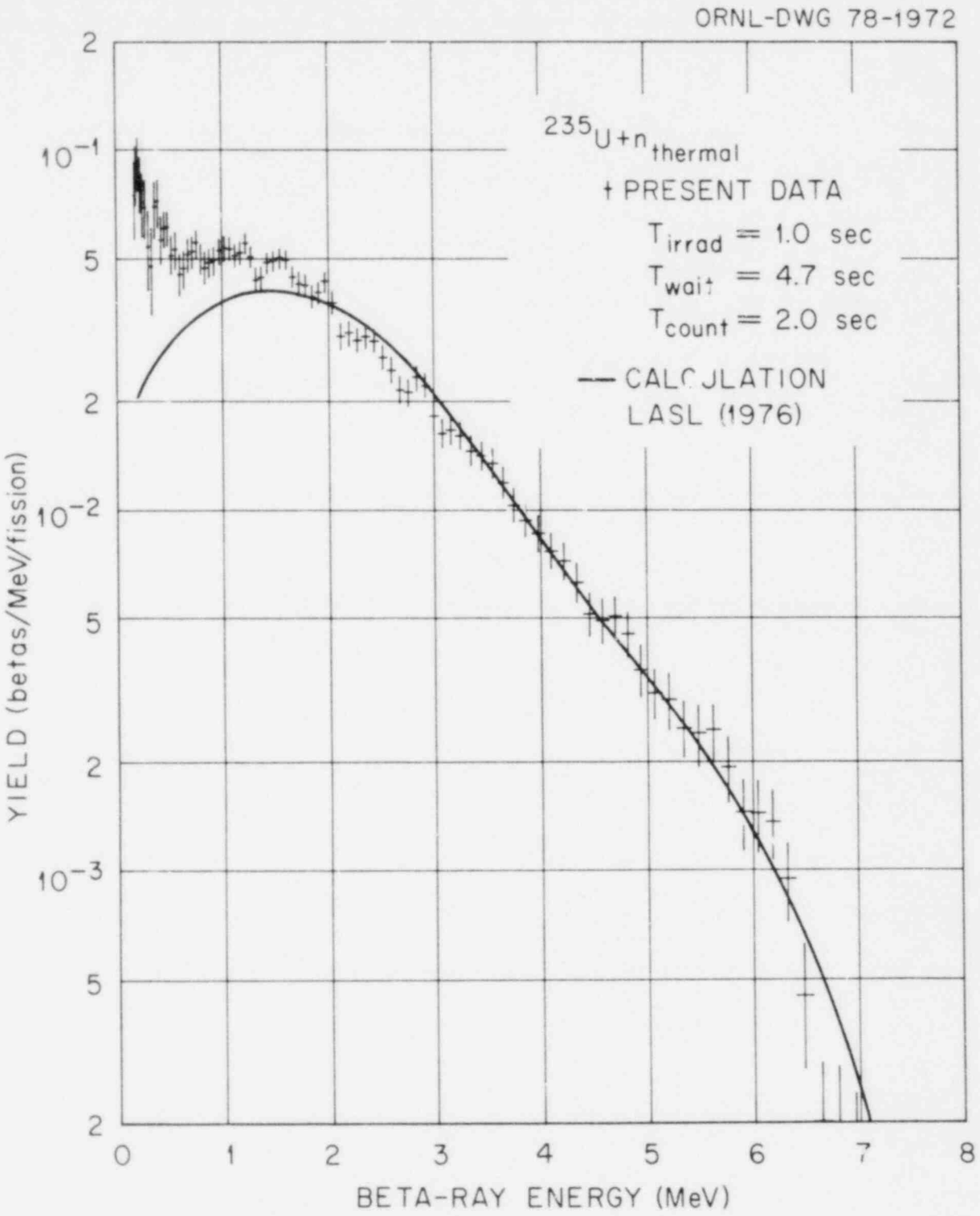

Fig. 10. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

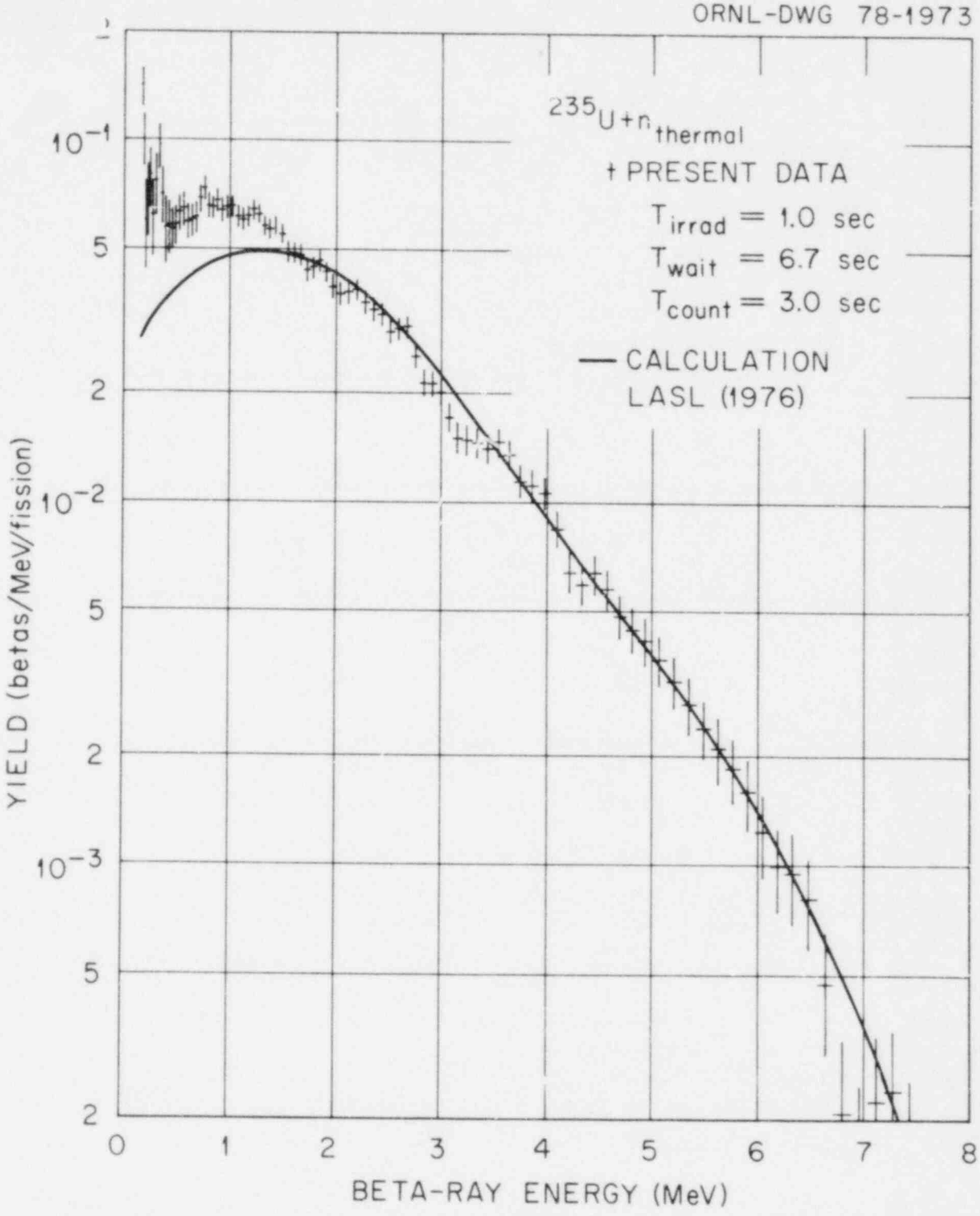

Fig. 11. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

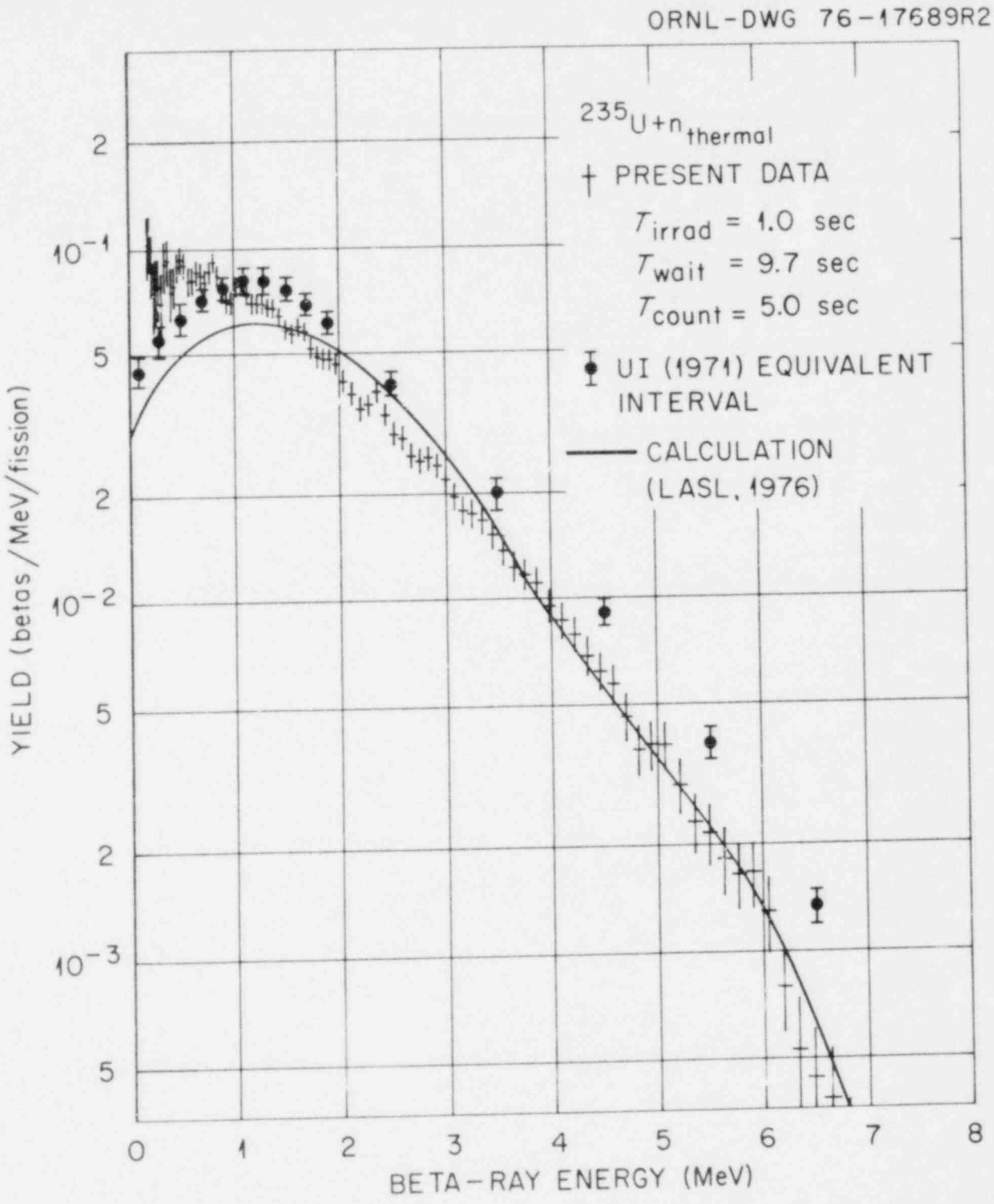

Fig. 12. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The solid points are the data of Tsoulfanidis et al. (Ref. 11) and the calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

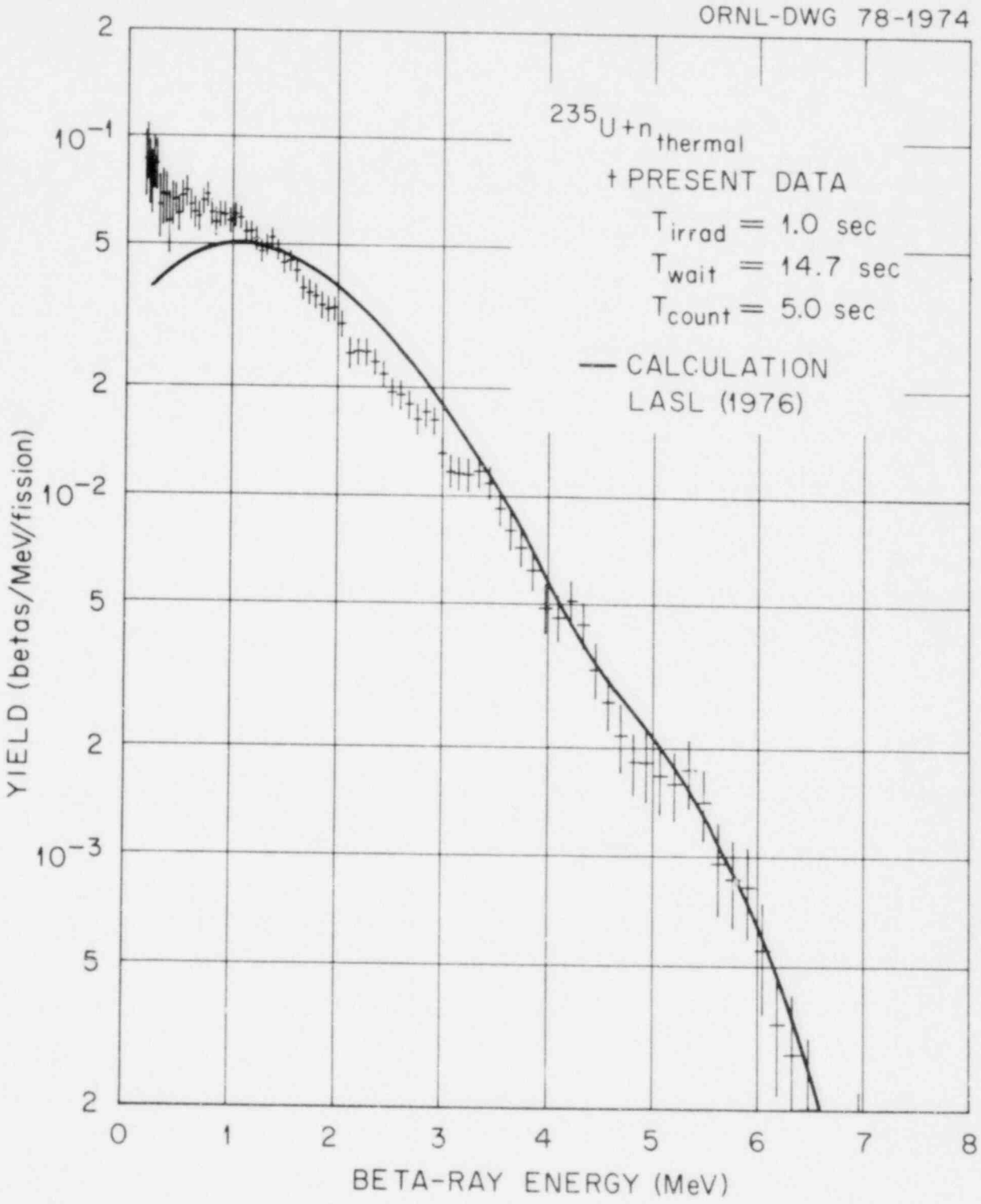

Fig. 13. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

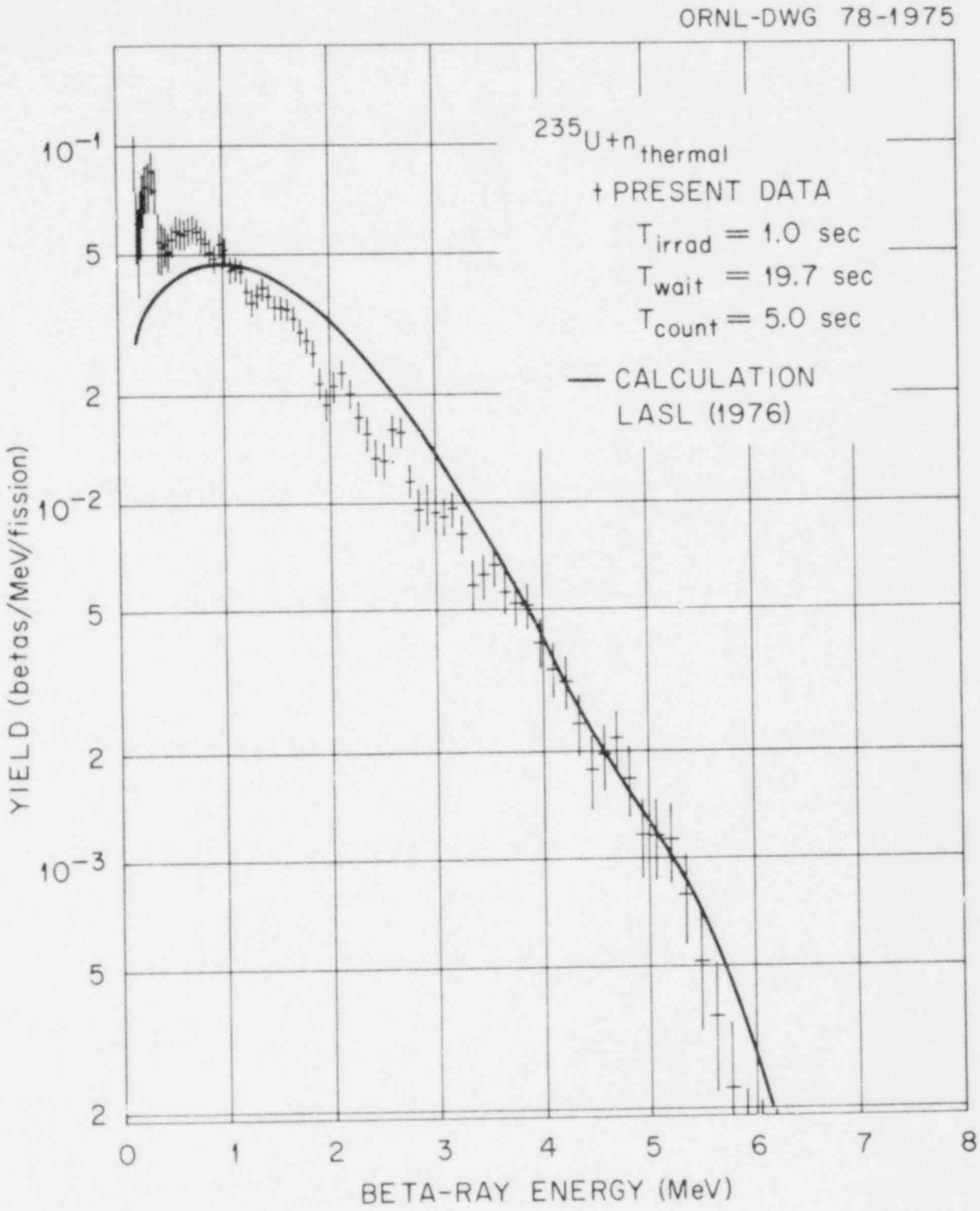

Fig. 14. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

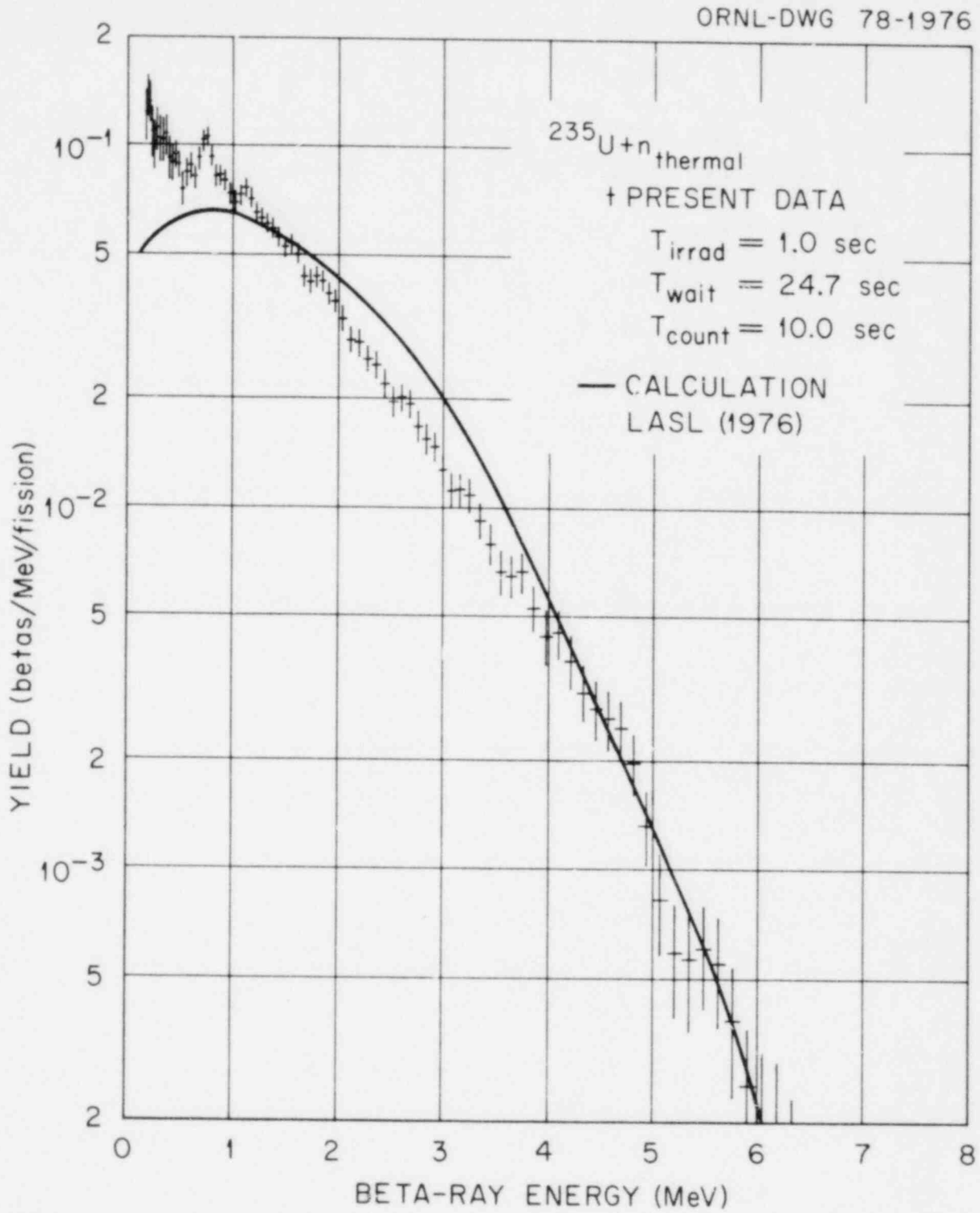

Fig. 15. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

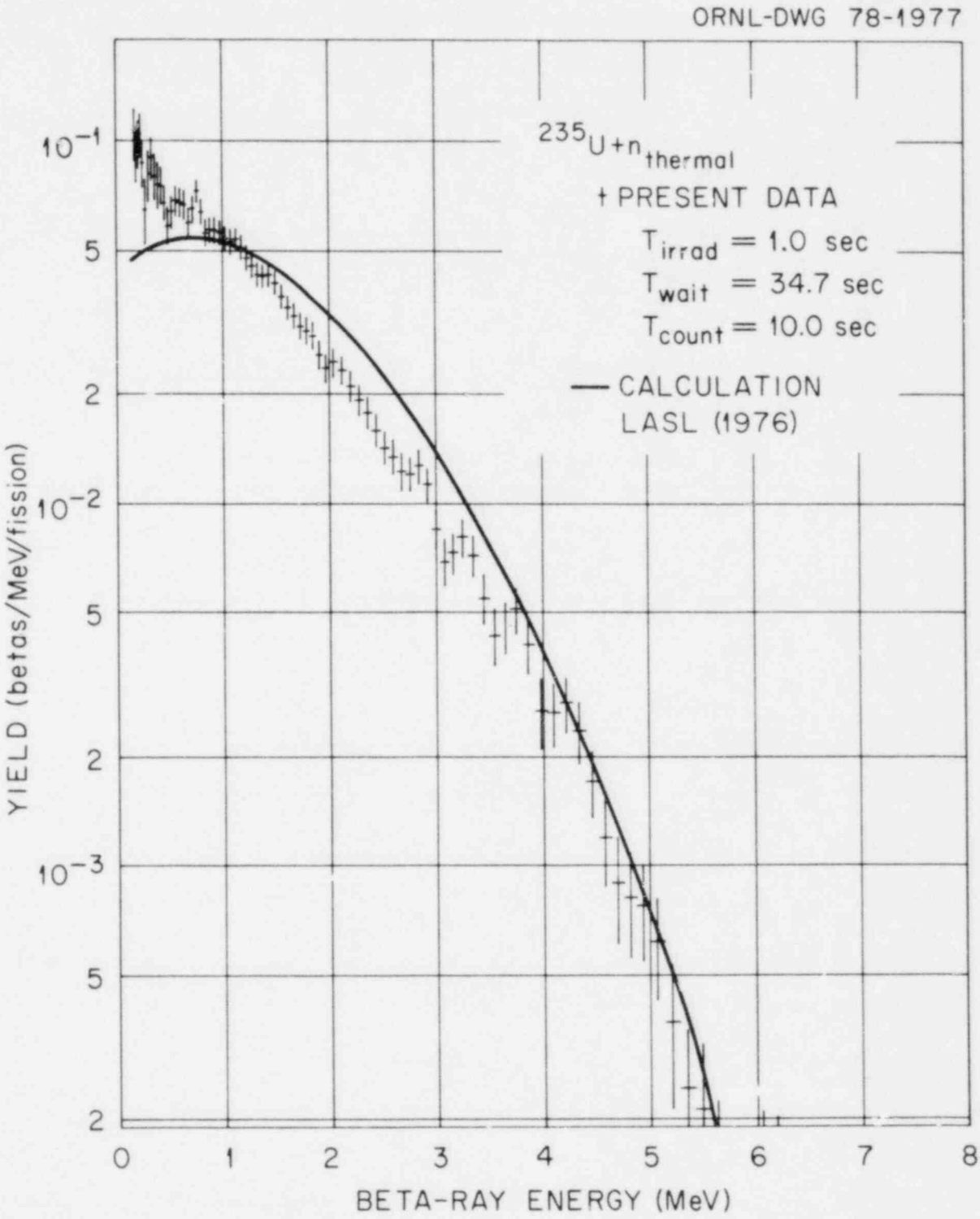

Fig. 16. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

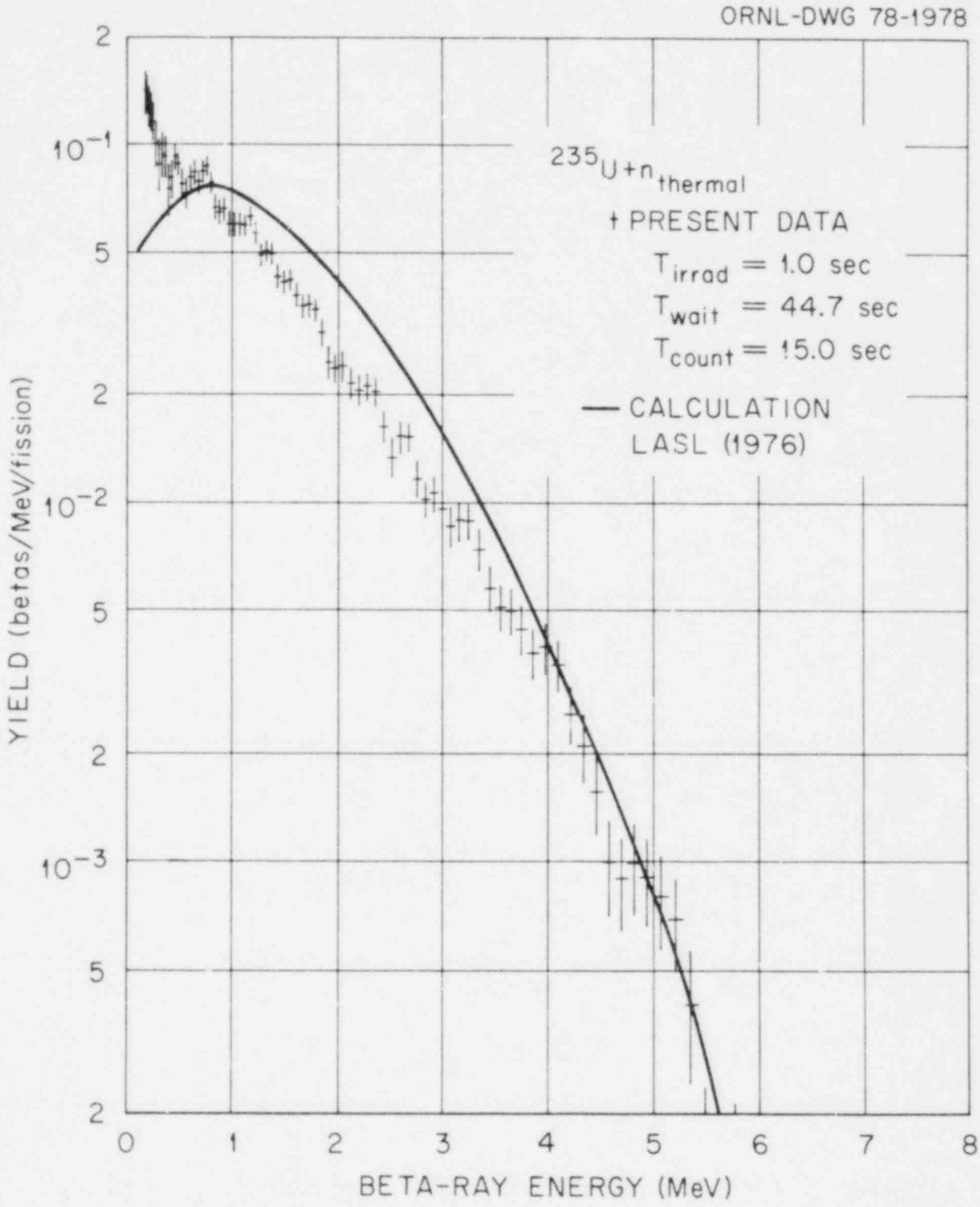

Fig. 17. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

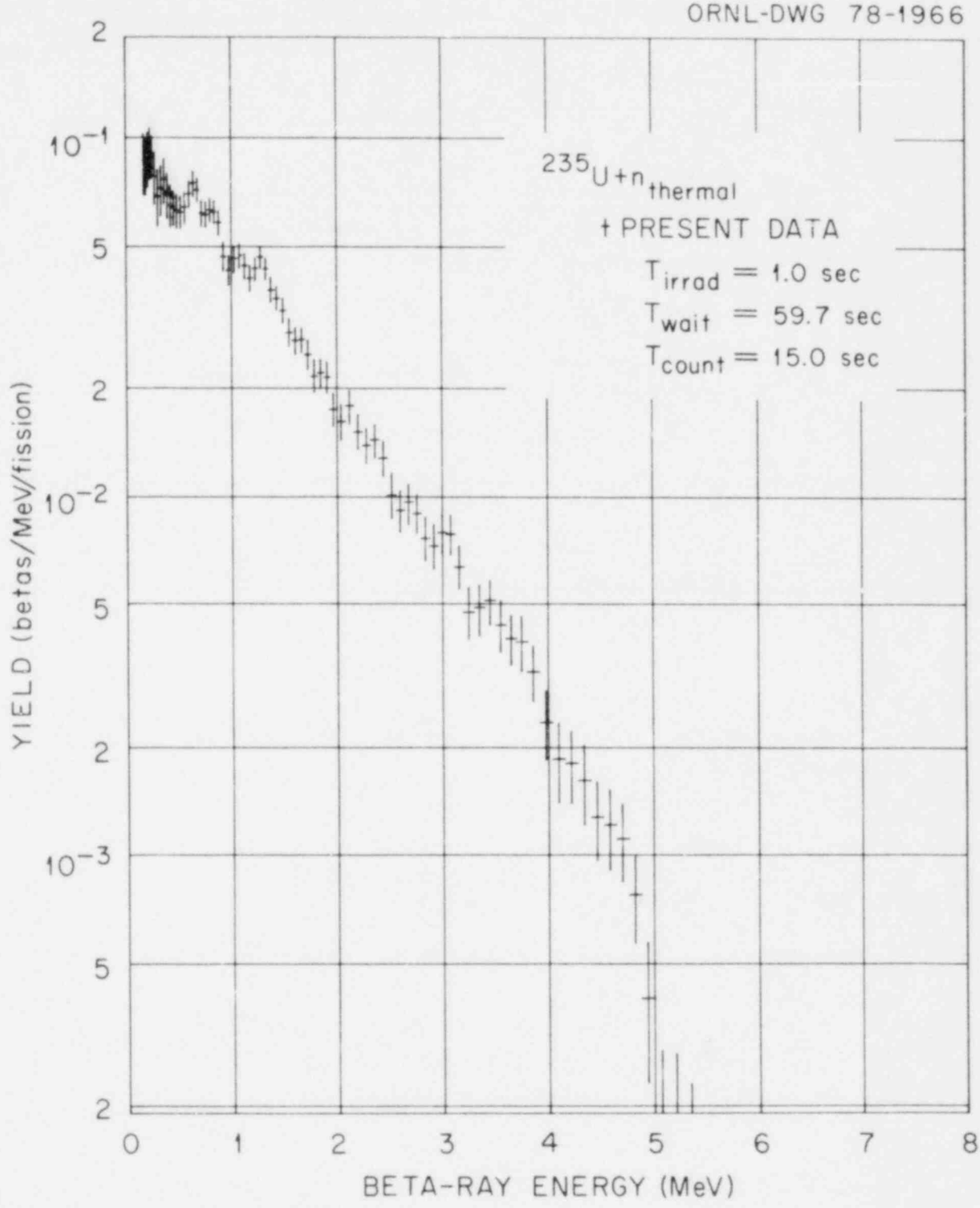

Fig. 18. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The irradiation time, waiting time, and counting time intervals are given in the legend.

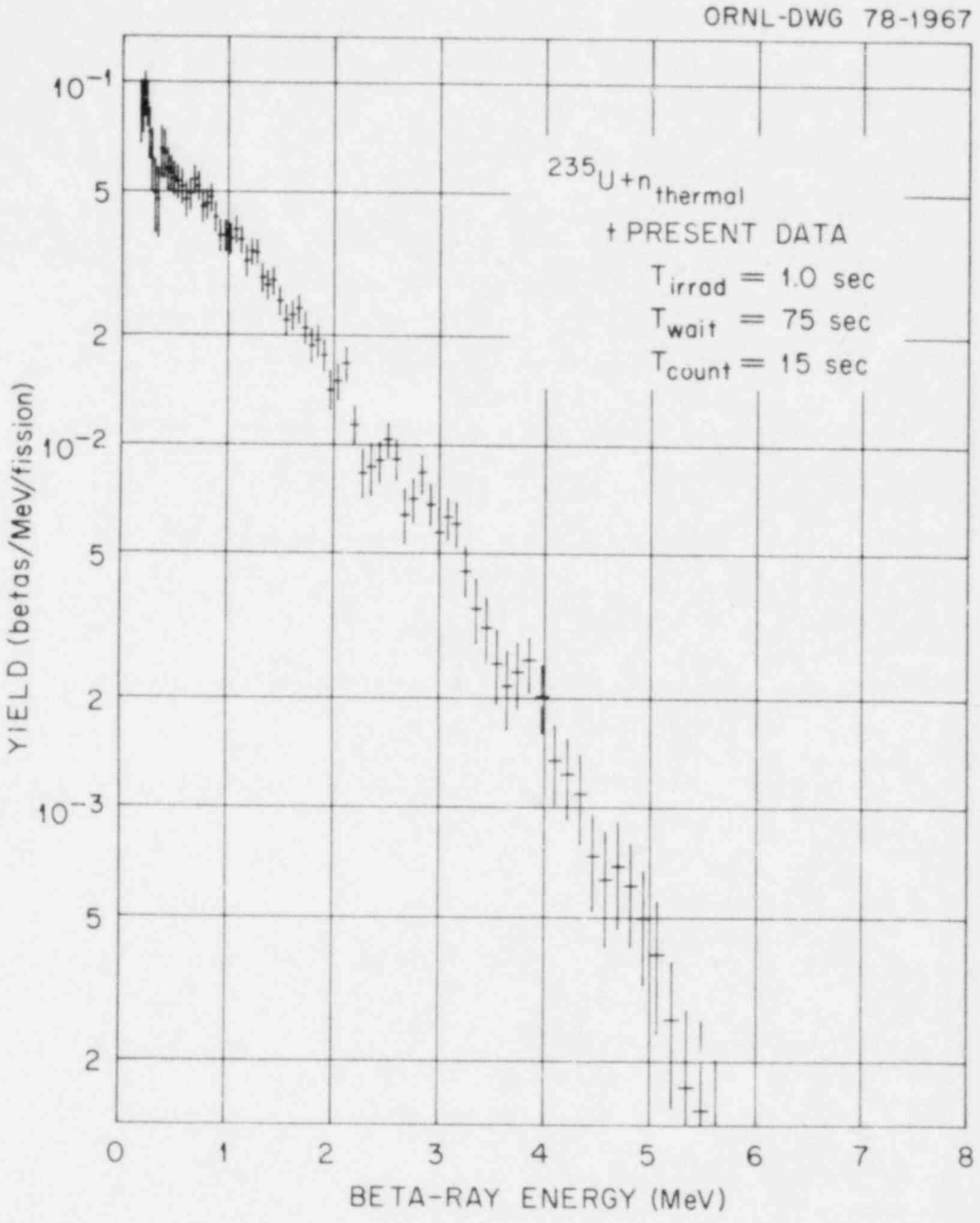

Fig. 19. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The irradiation time, waiting time, and counting time intervals are given in the legend.

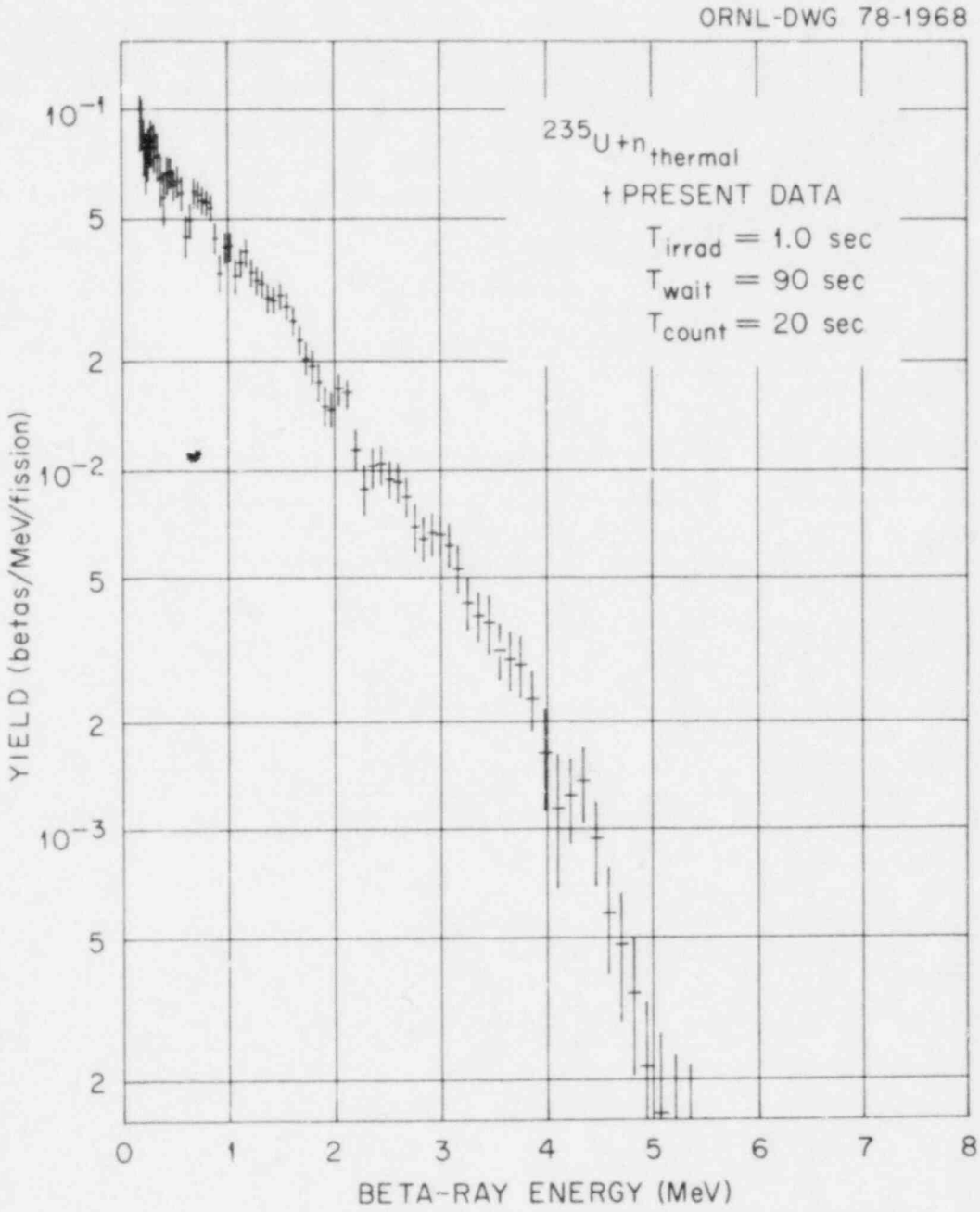

Fig. 20. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The irradiation time, waiting time, and counting time intervals are given in the legend.

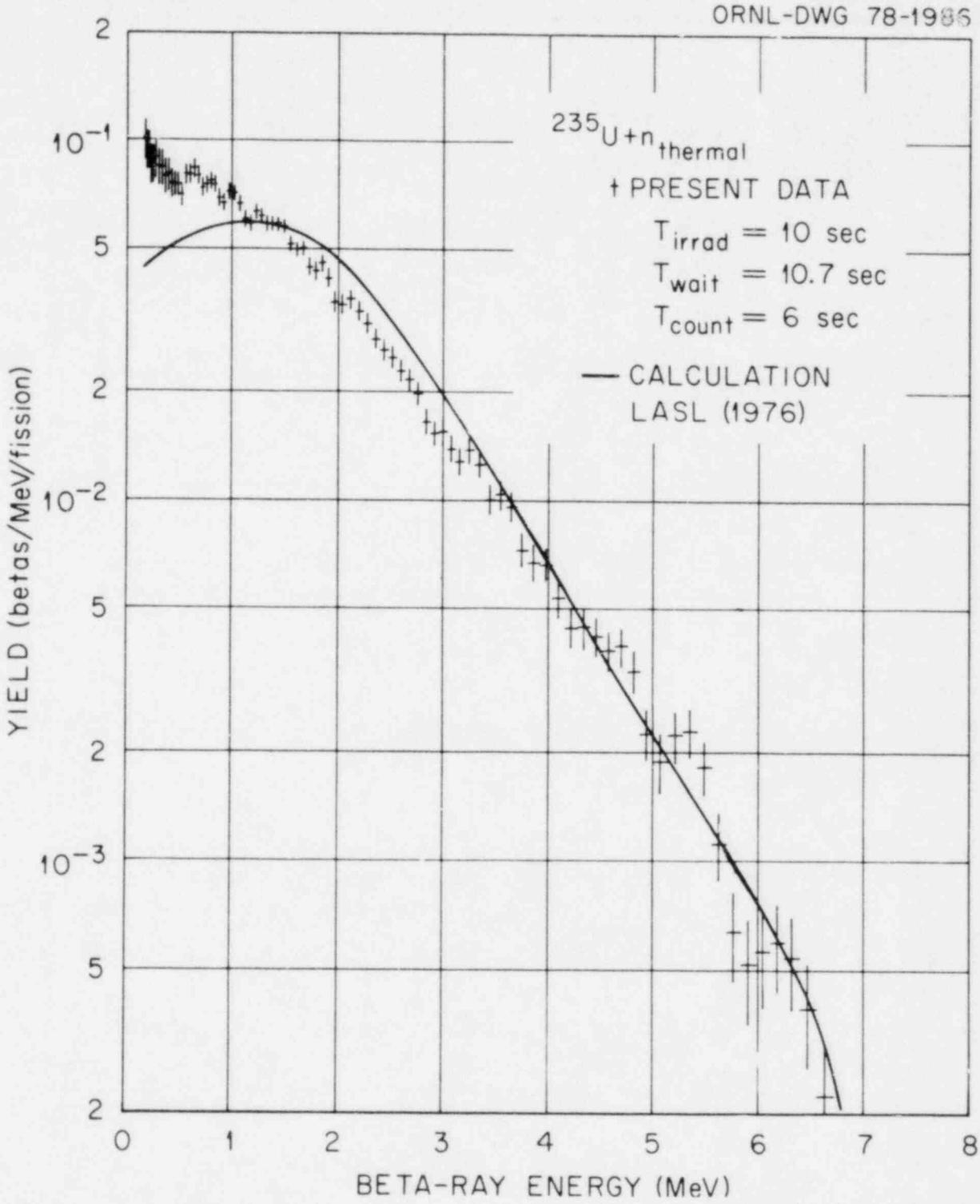

Fig. 21. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

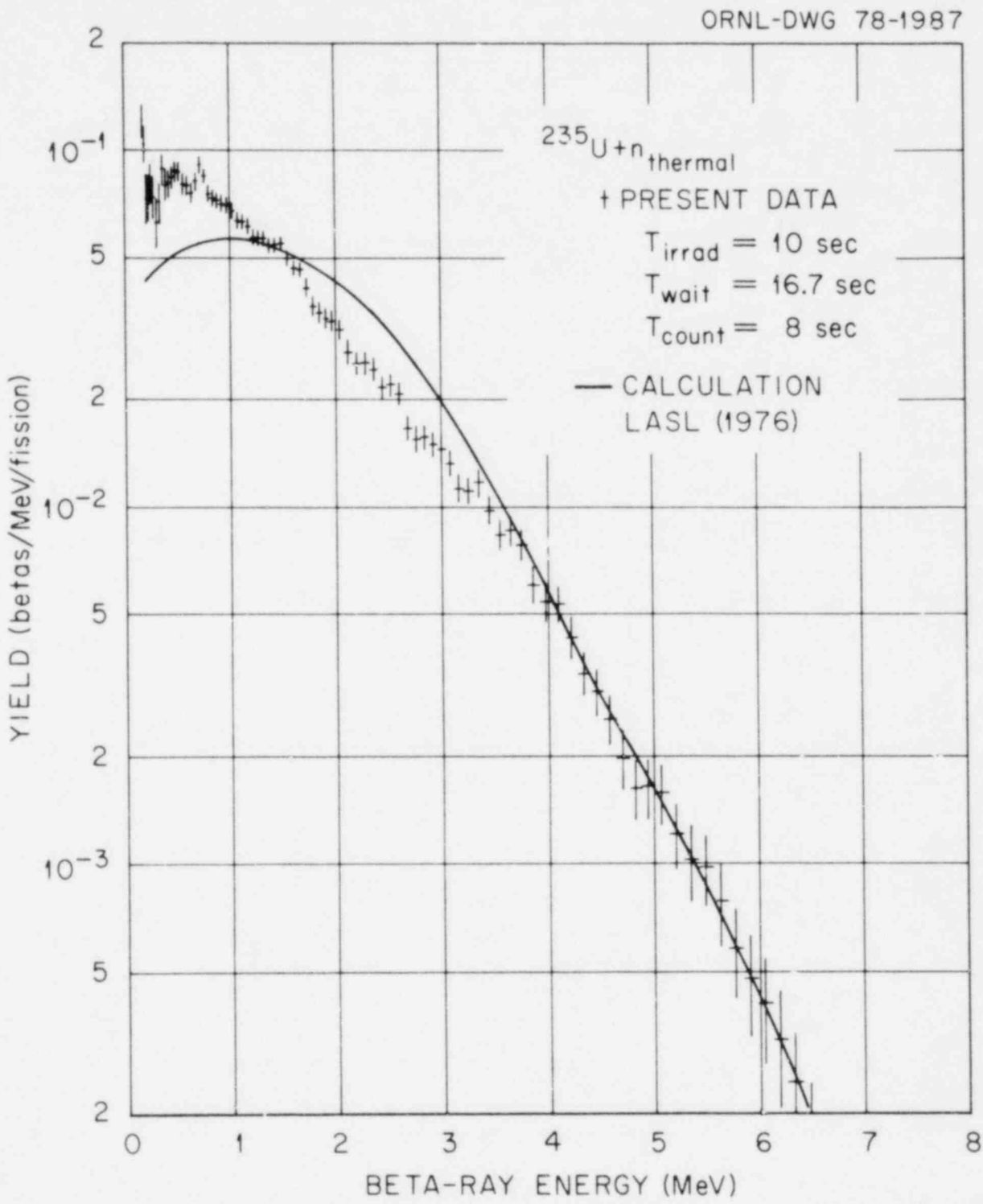

Fig. 22. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

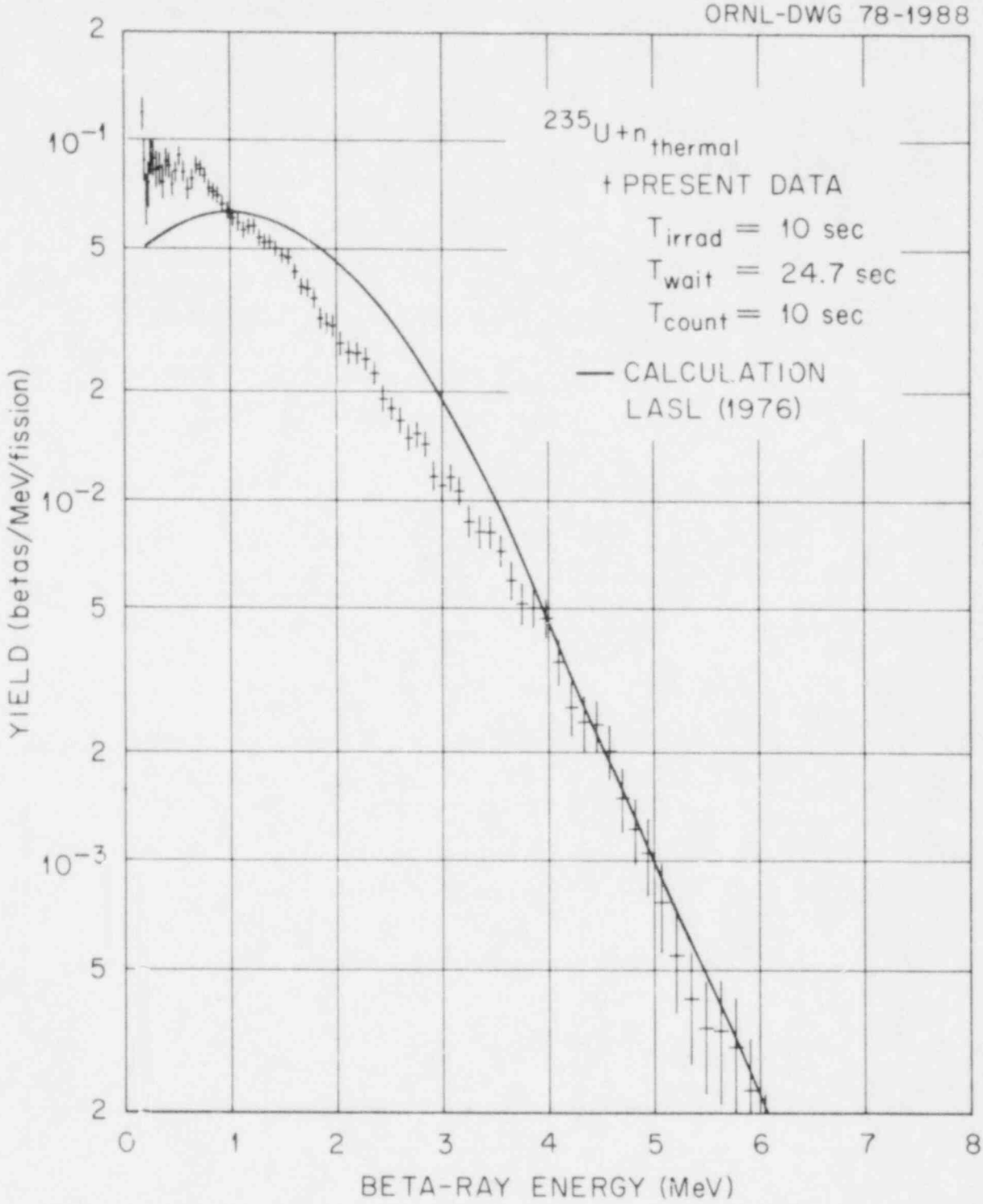

Fig. 23. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

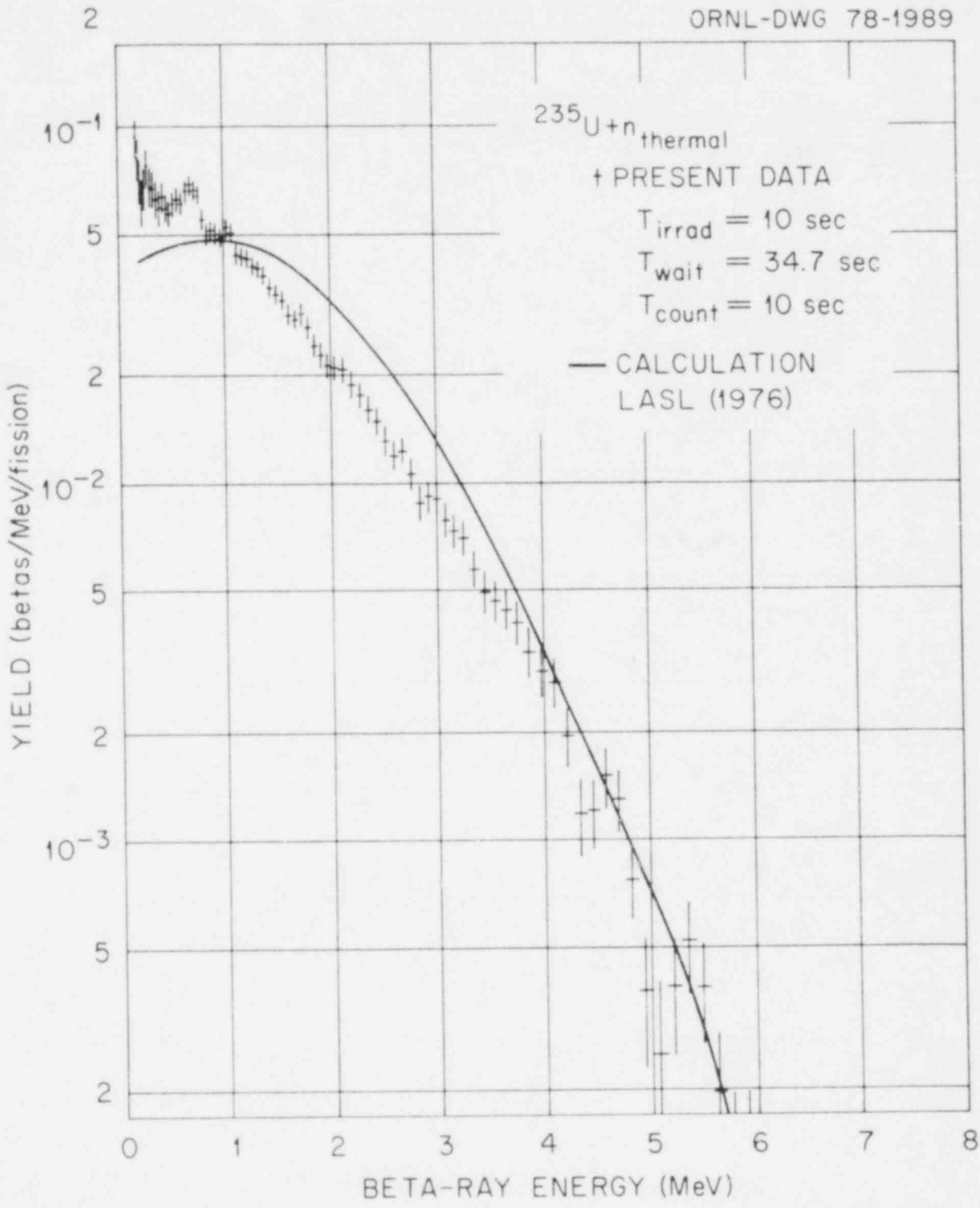

Fig. 24. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

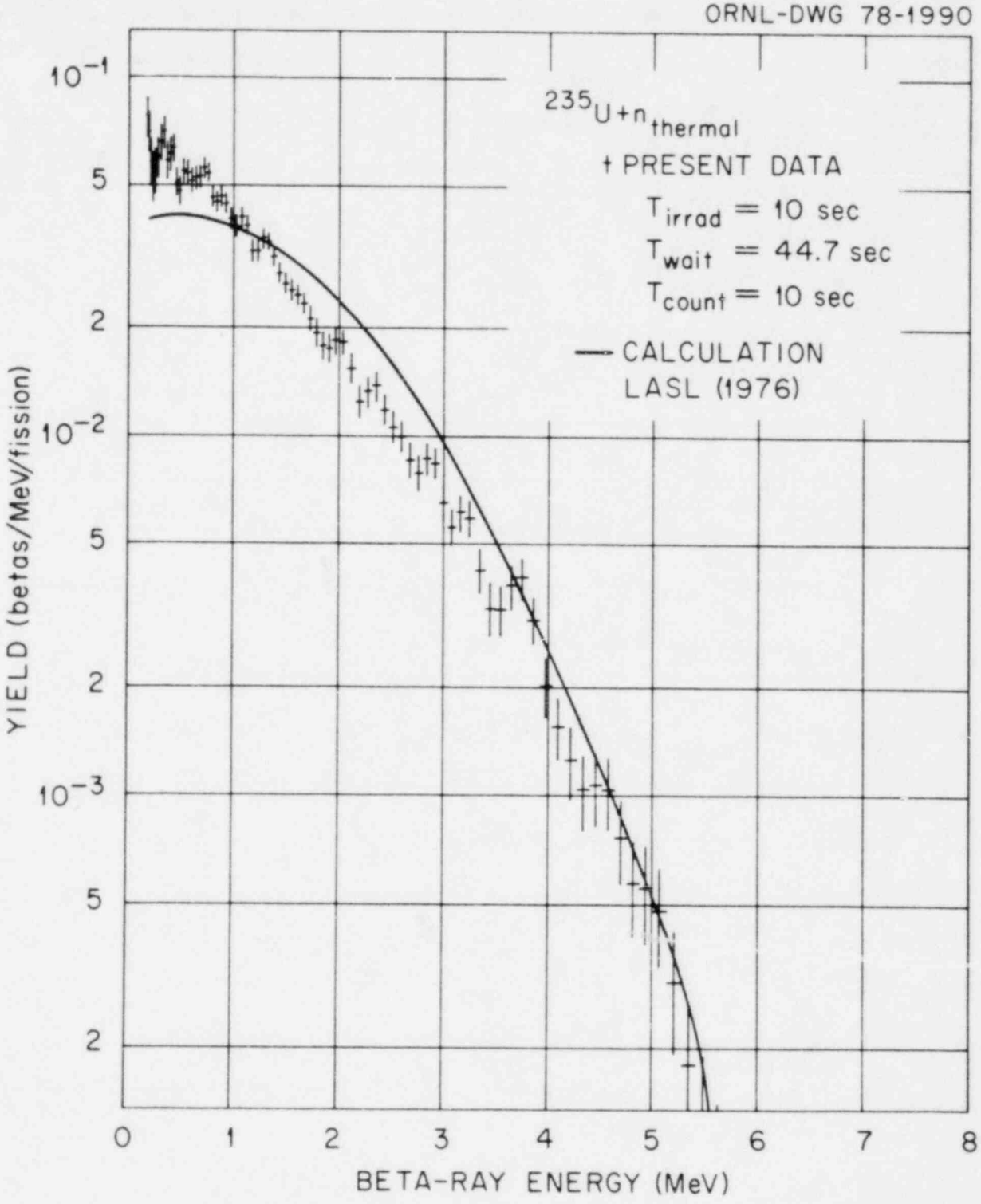

Fig. 25. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

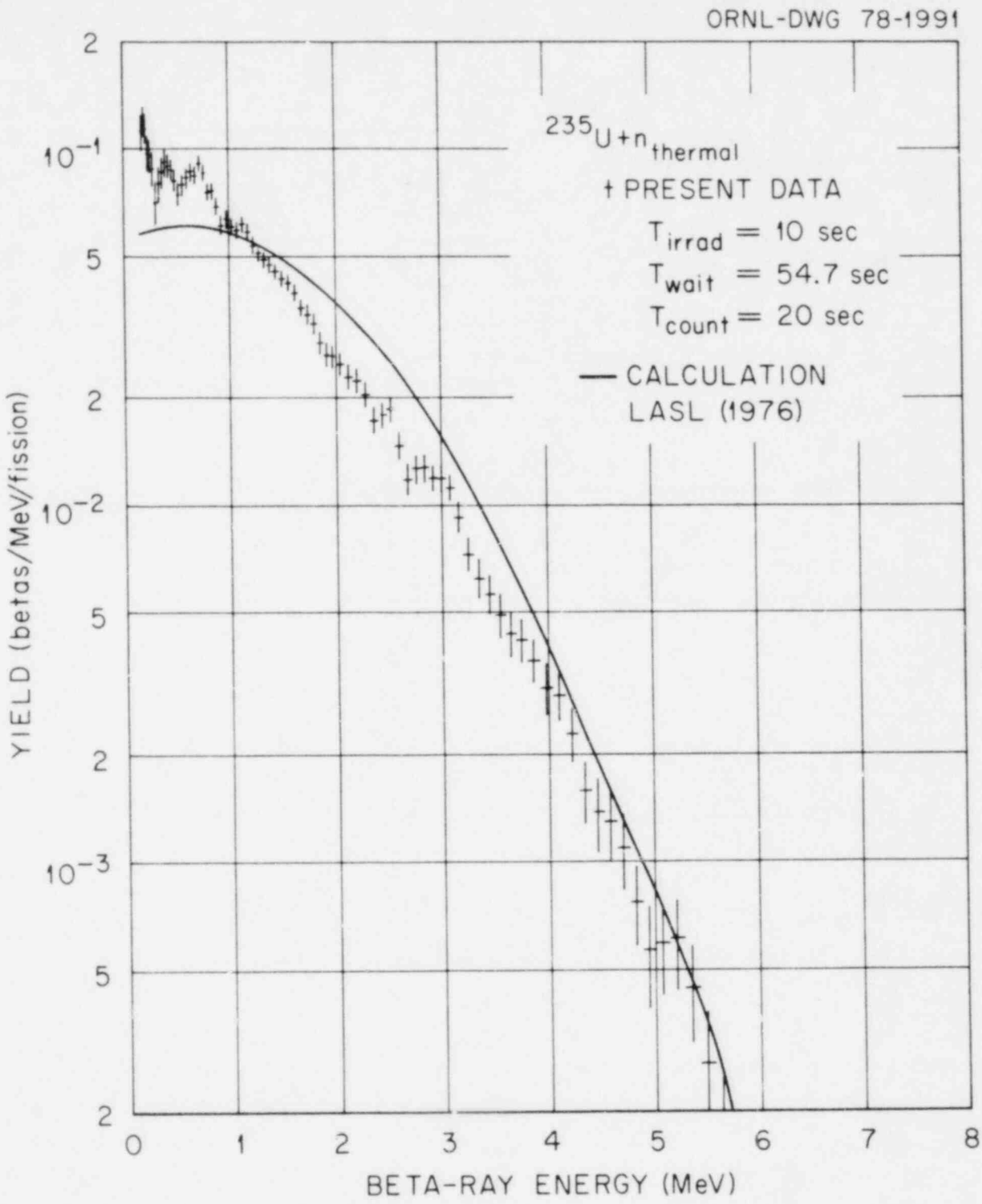

Fig. 26. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

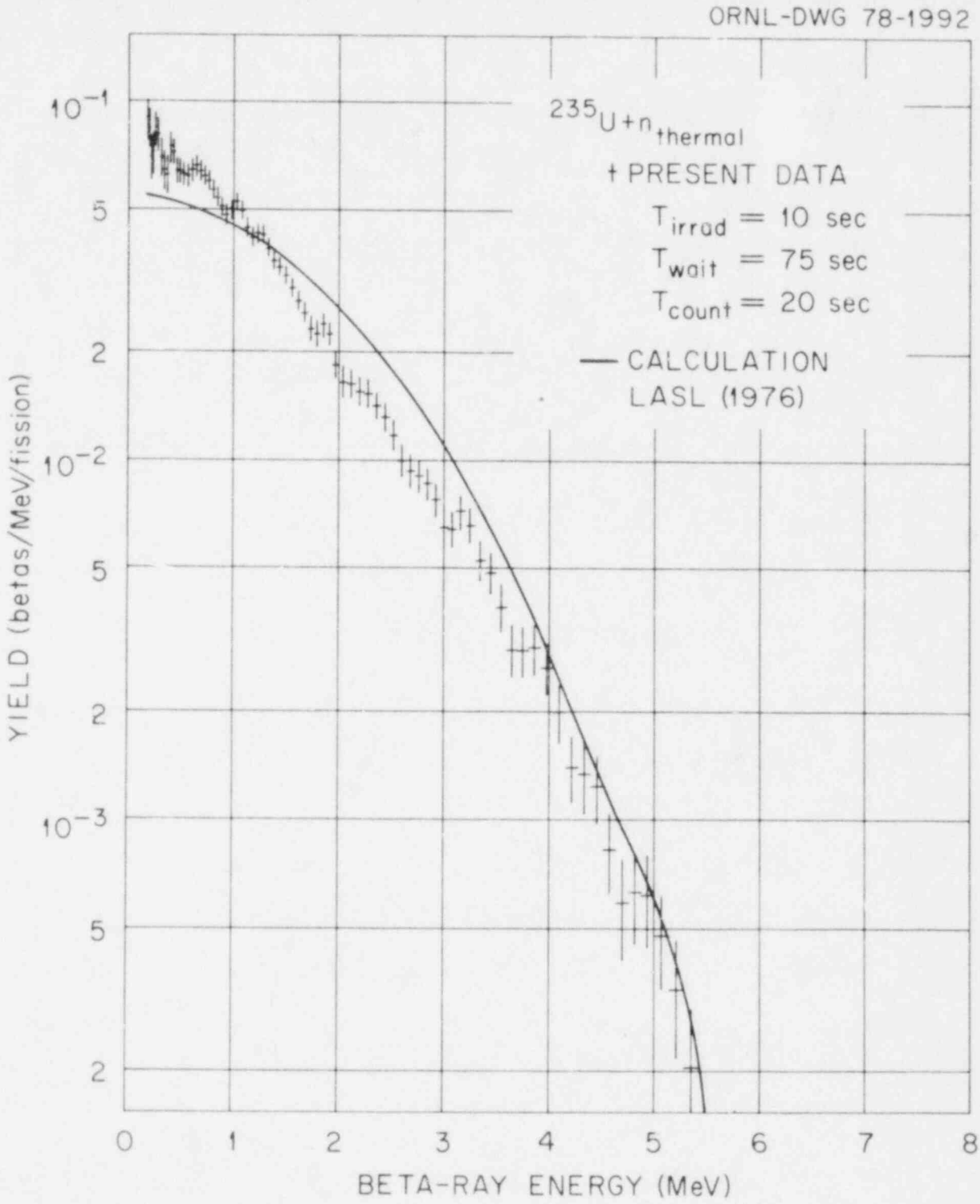

Fig. 27. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

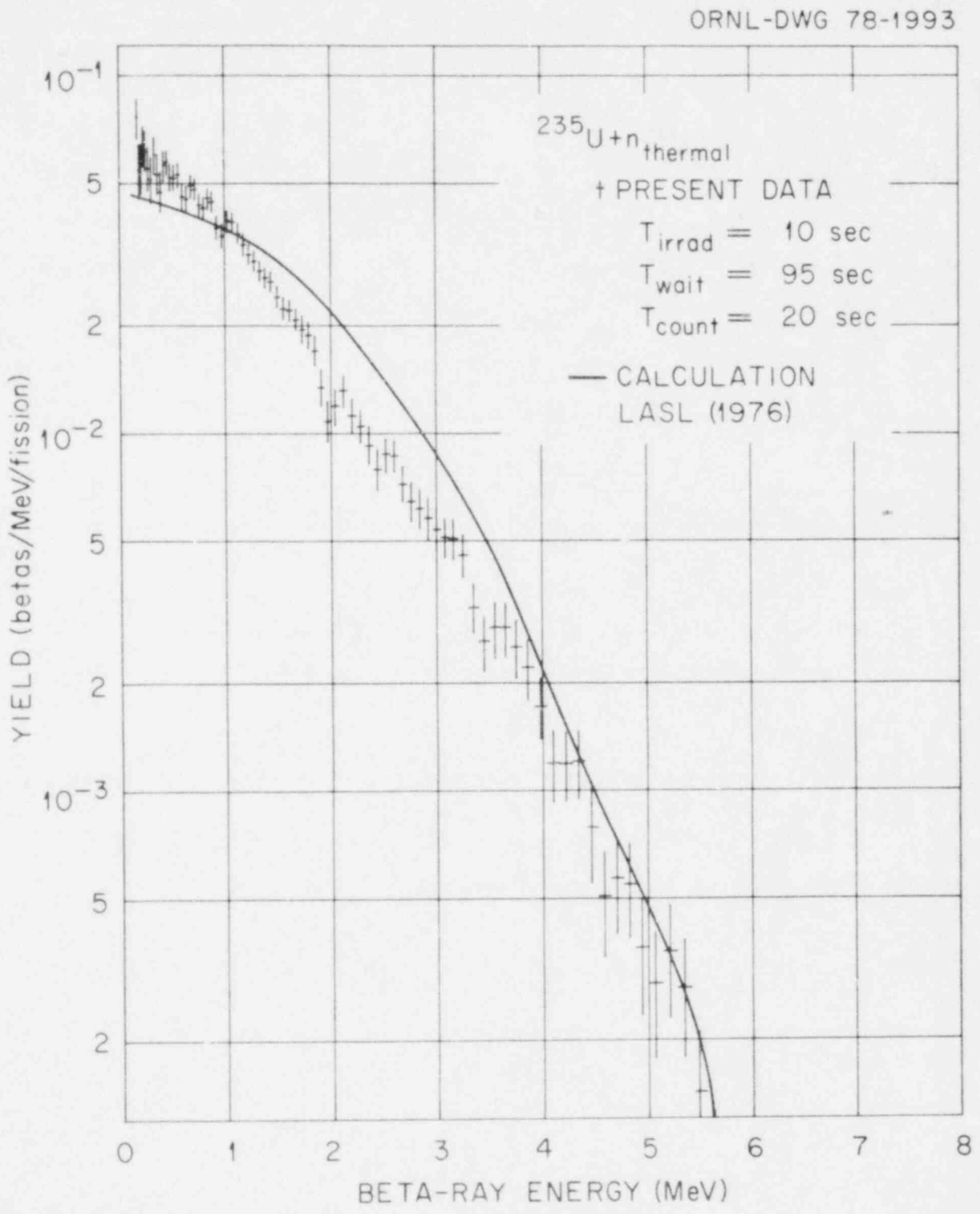

Fig. 28. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

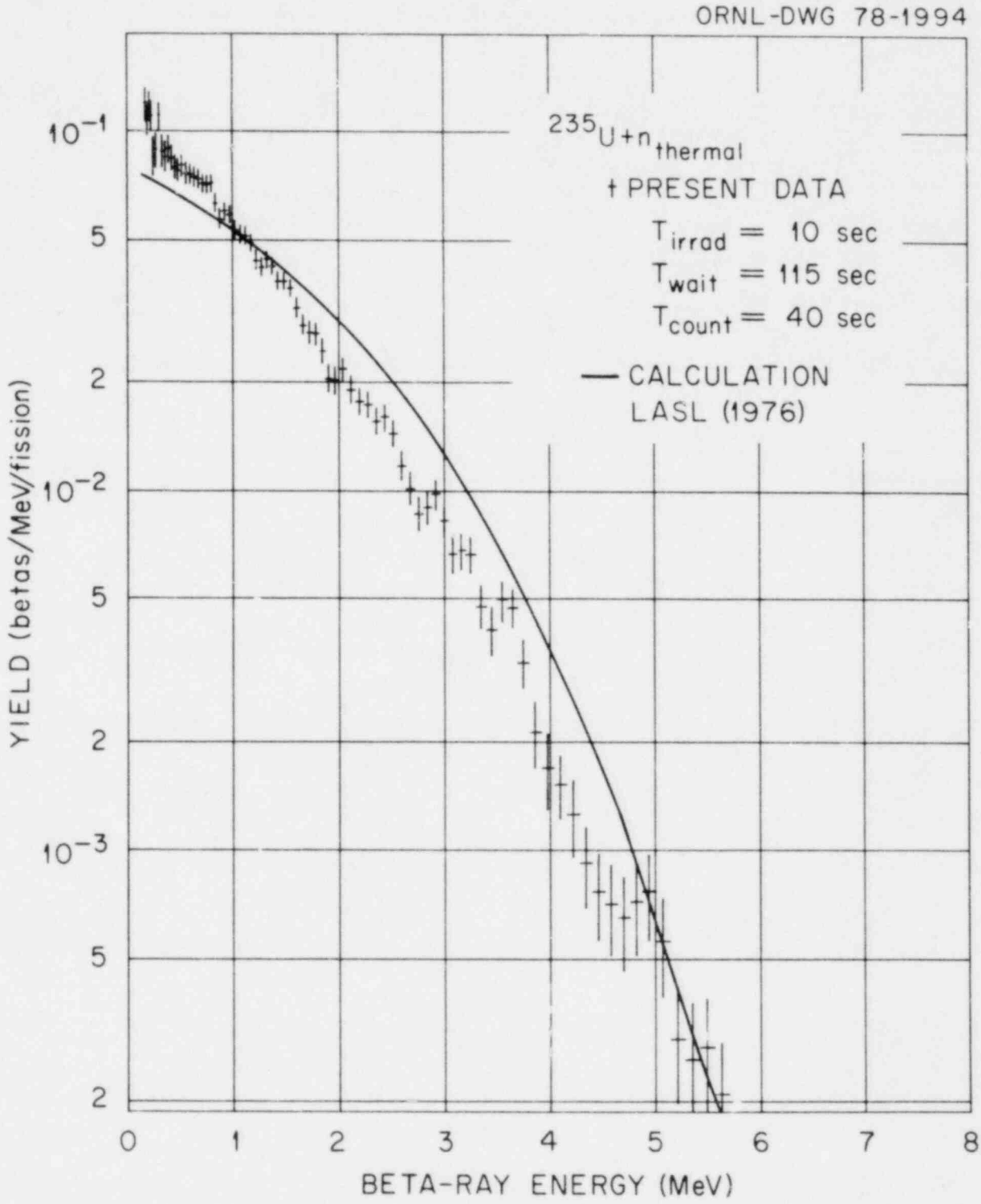

Fig. 29. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The solid points are the data of Tsoulfanidis et al. (Ref. 11) and the calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

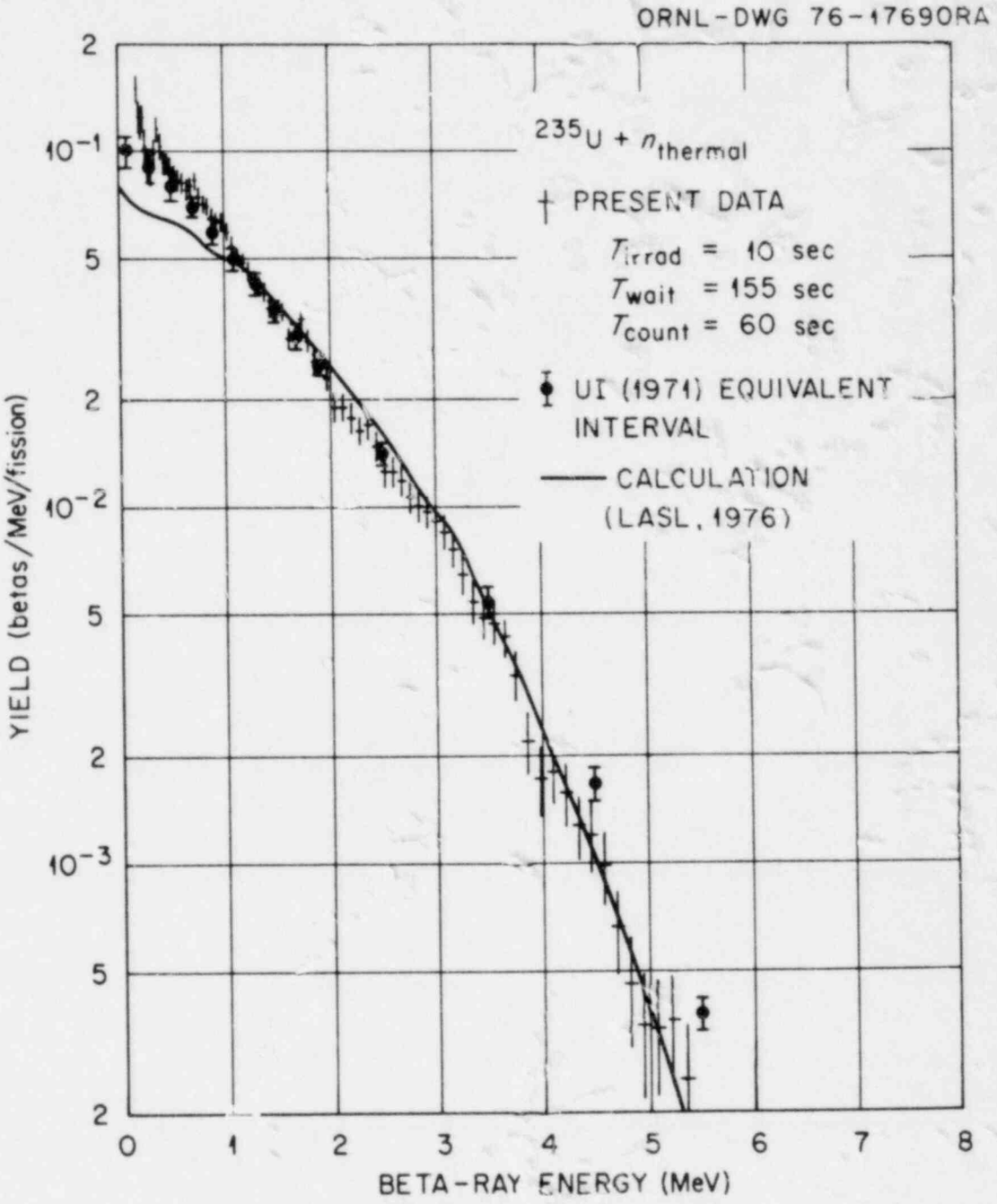

Fig. 30. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

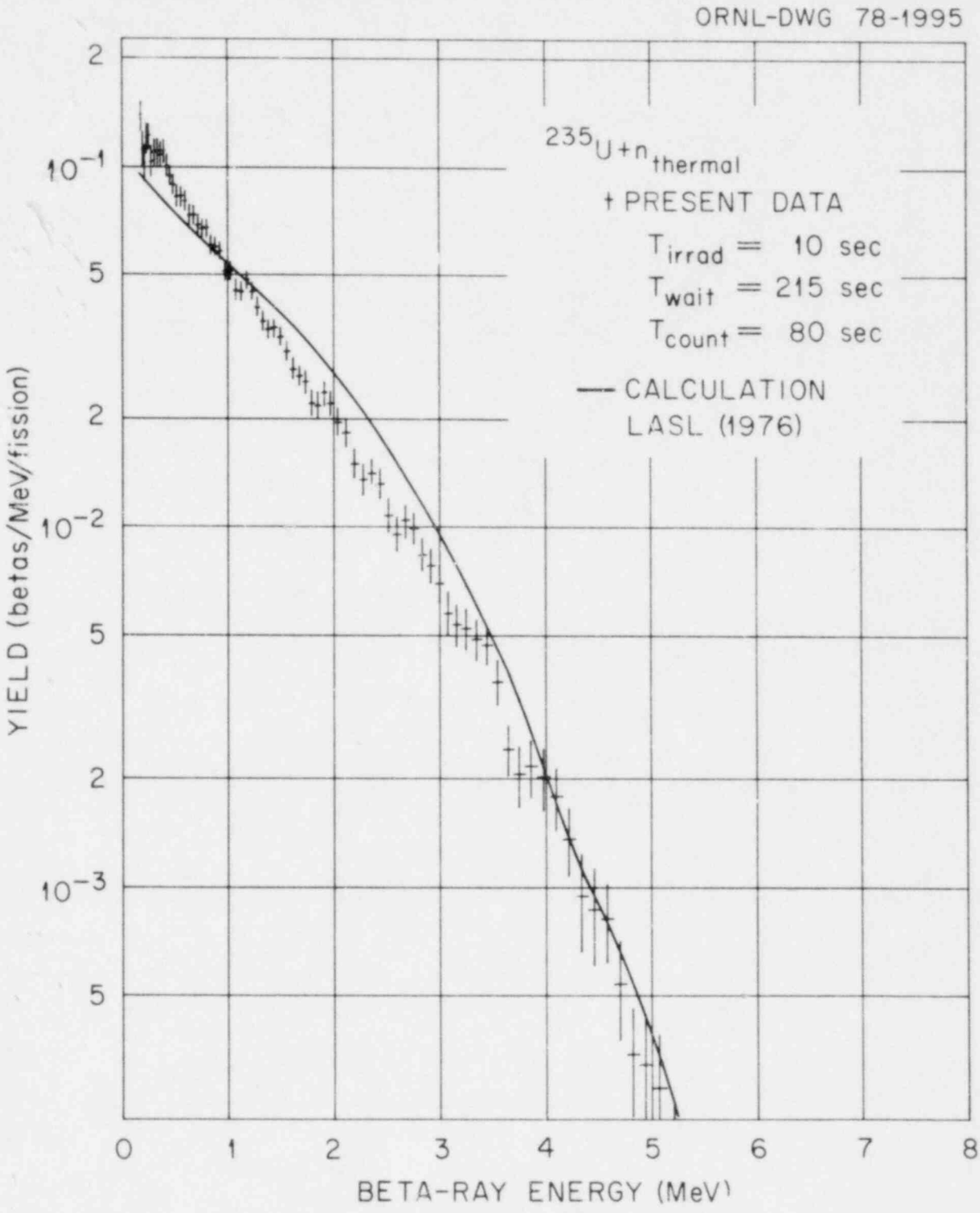

Fig. 31. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

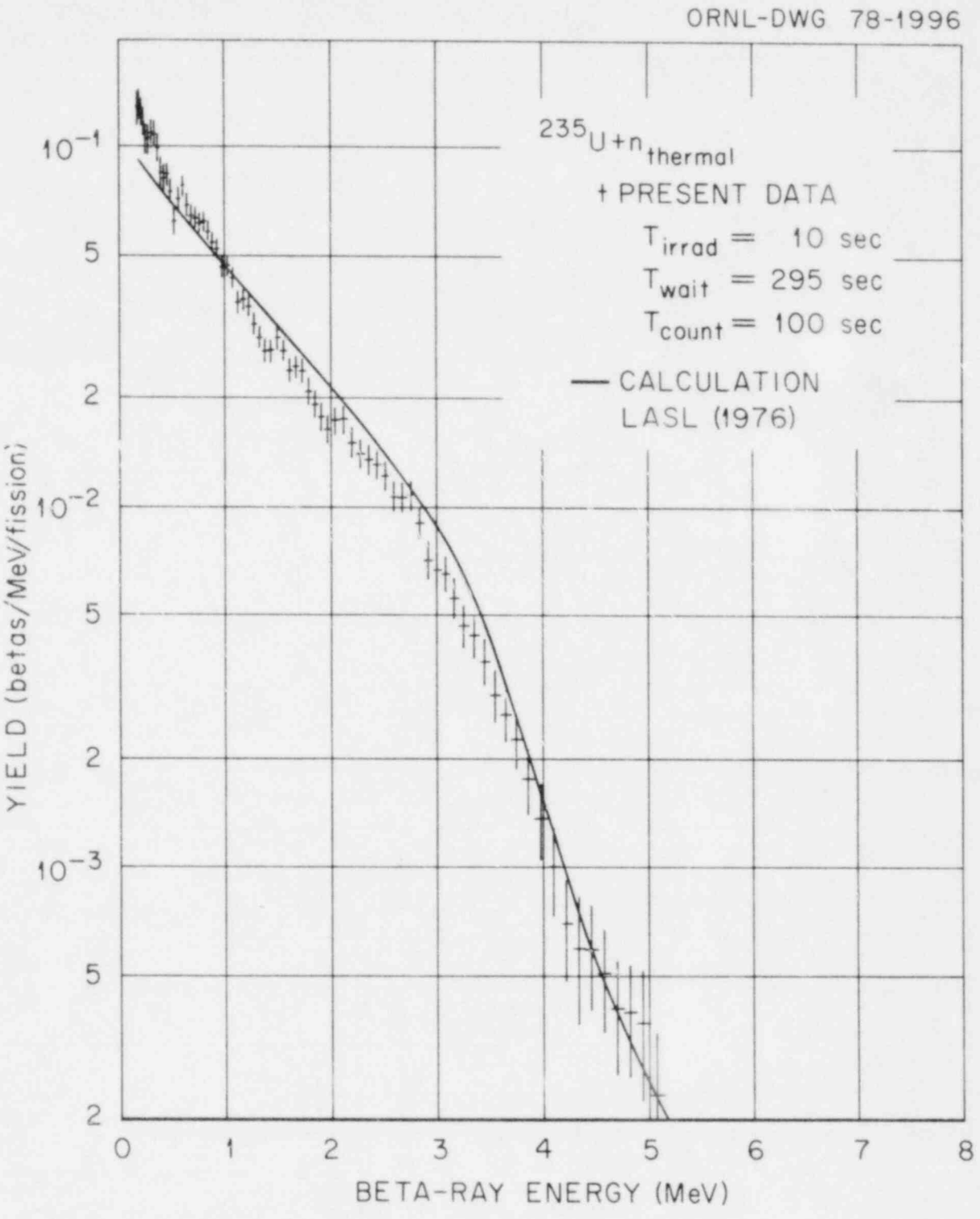

Fig. 32. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

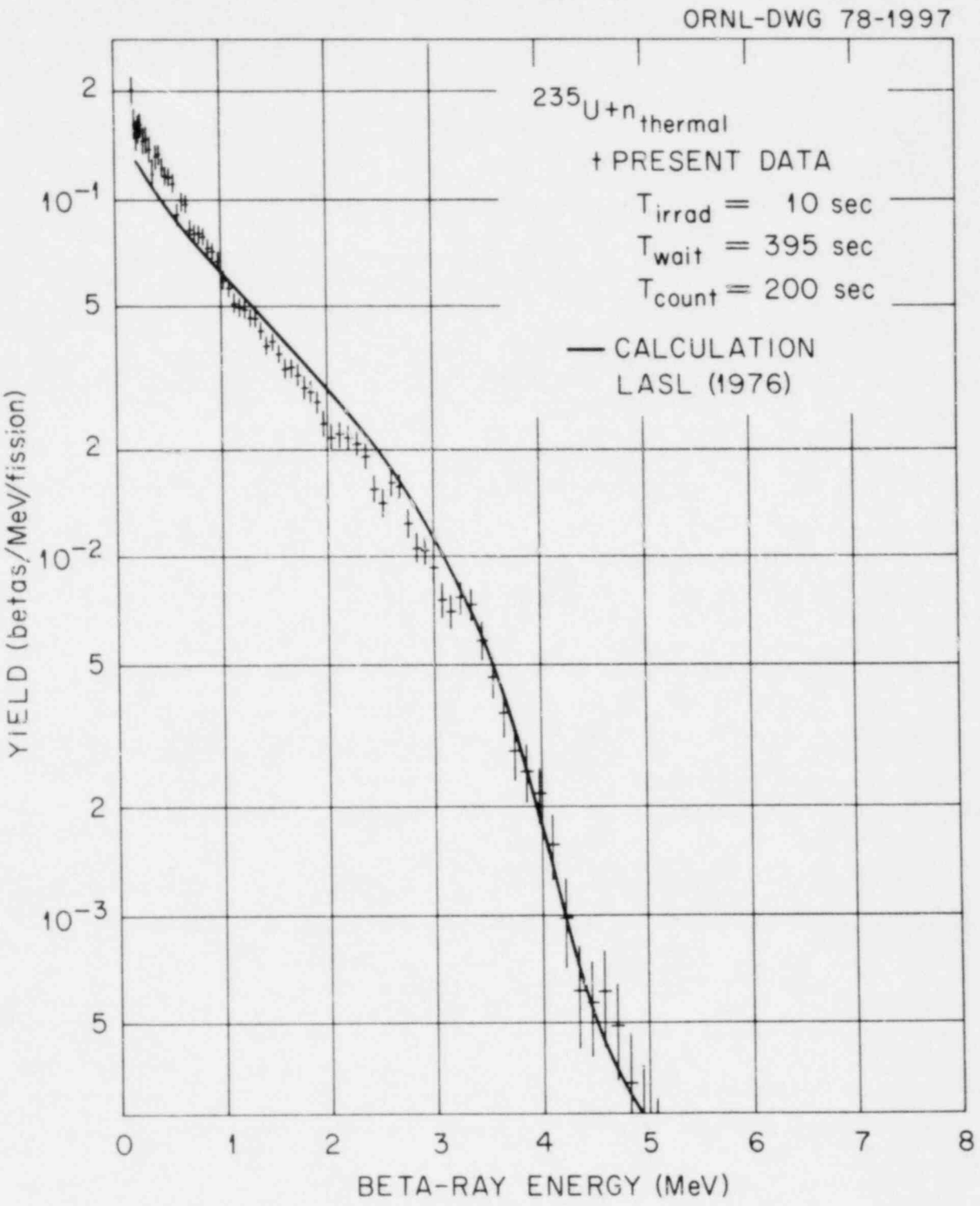

Fig. 33. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

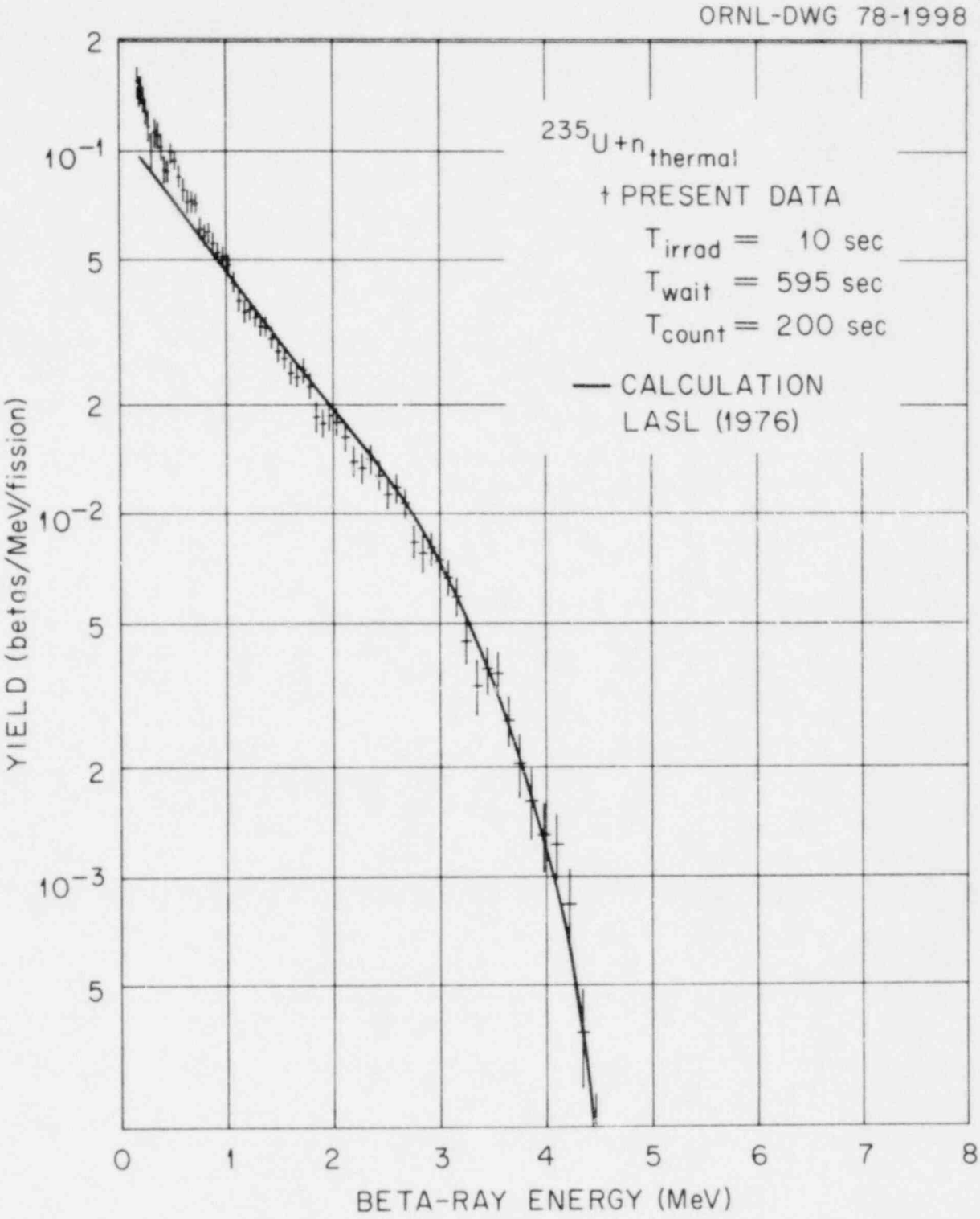

Fig. 34. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

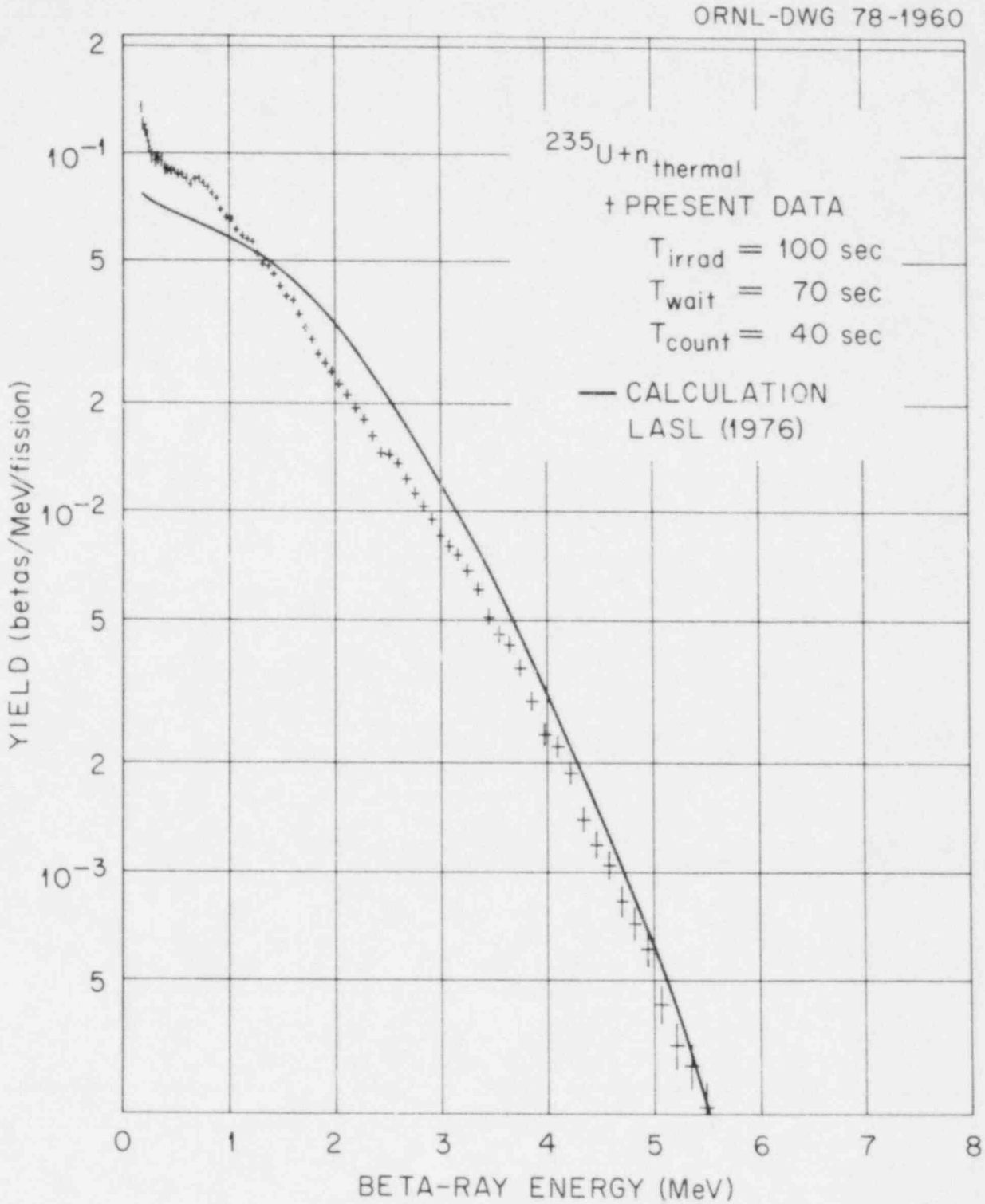

Fig. 35. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

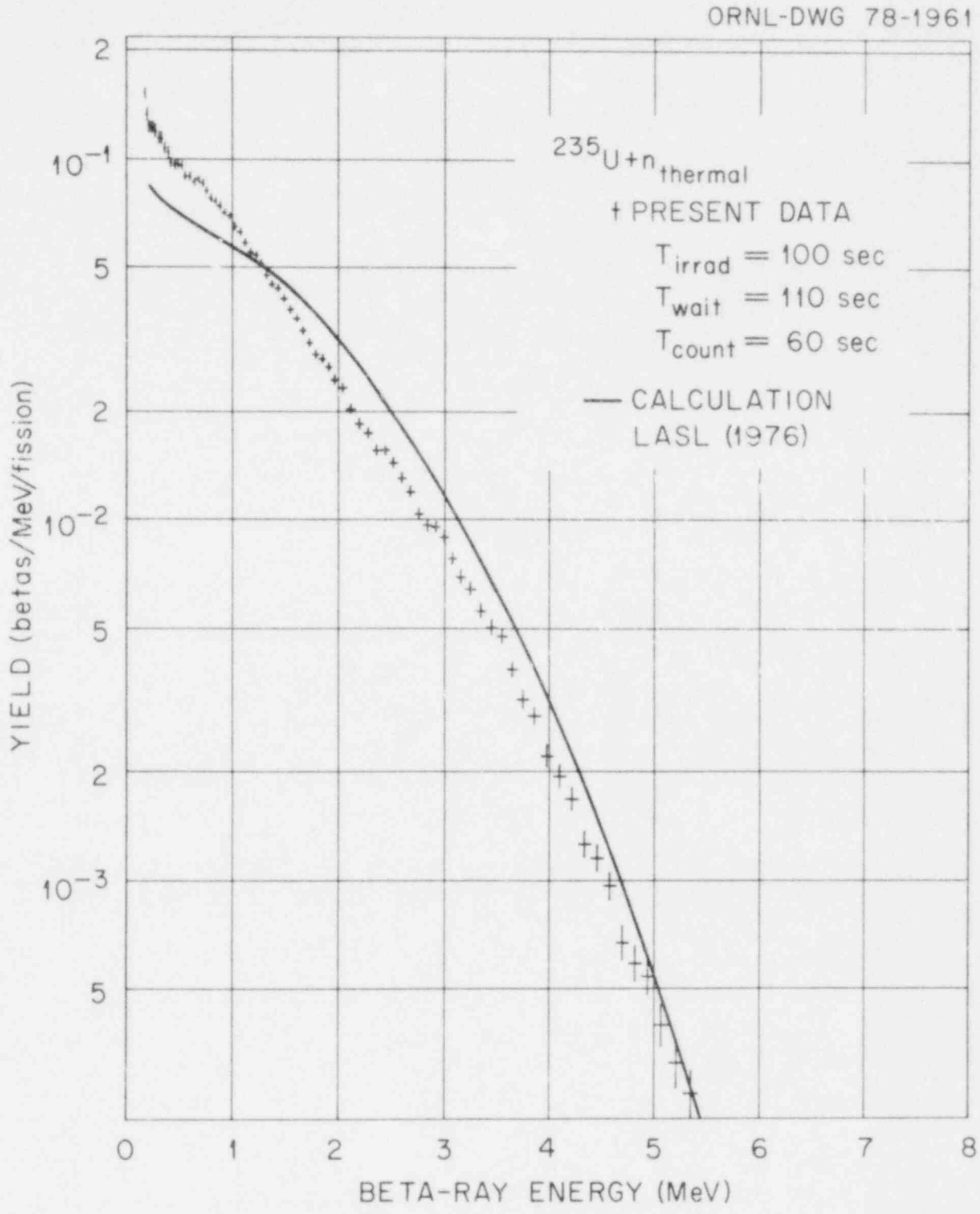

Fig. 36. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

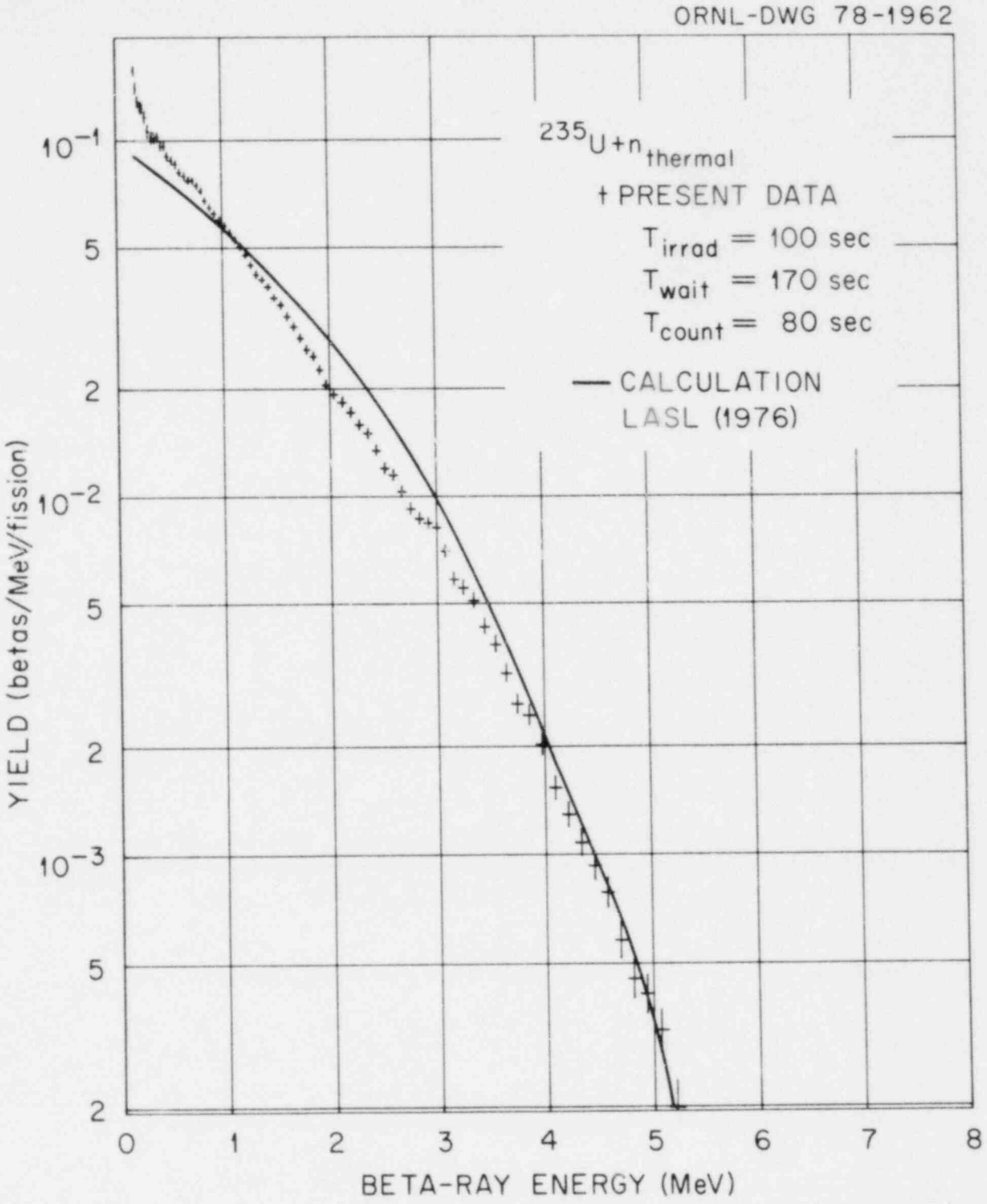

Fig. 37. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

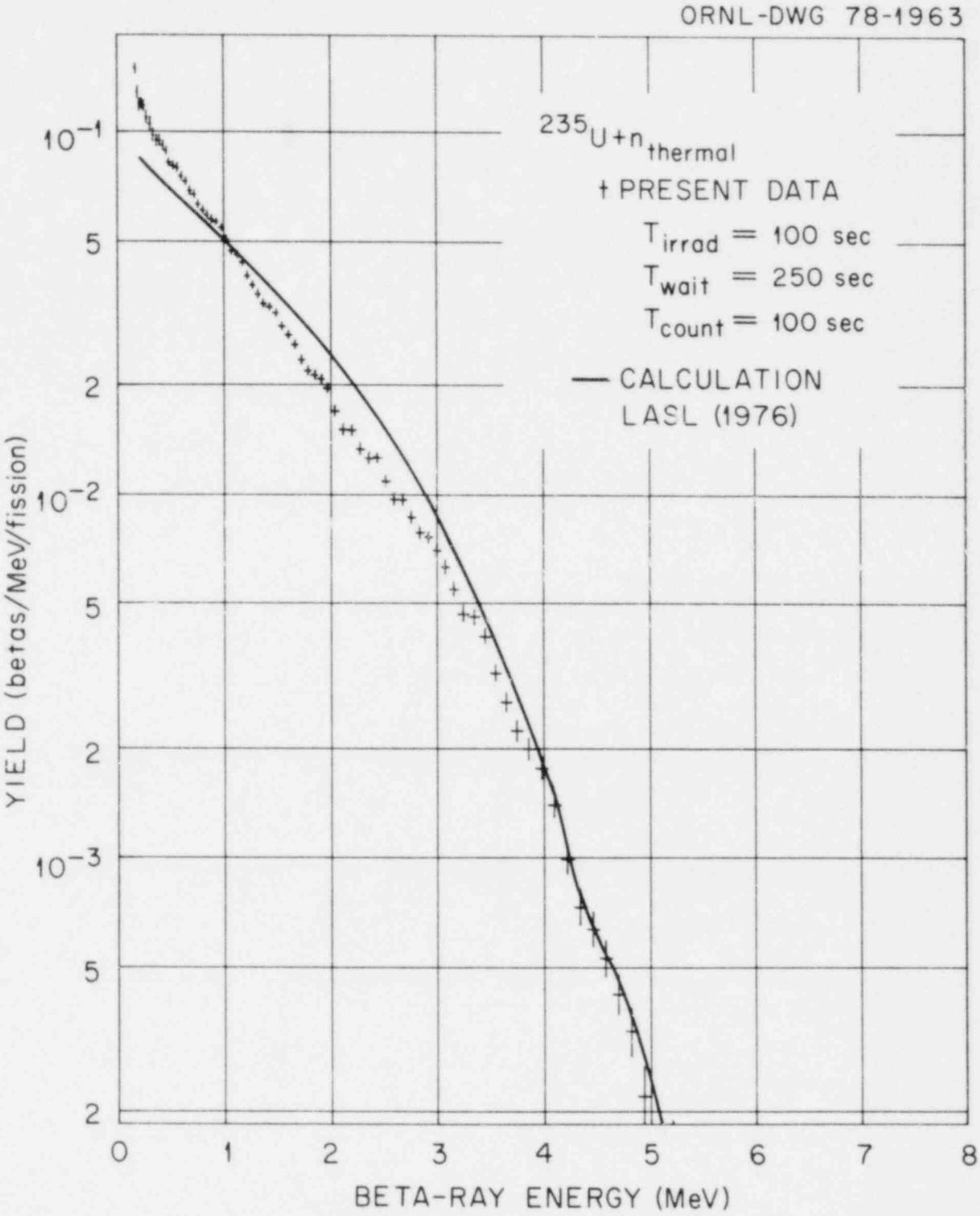

Fig. 38. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

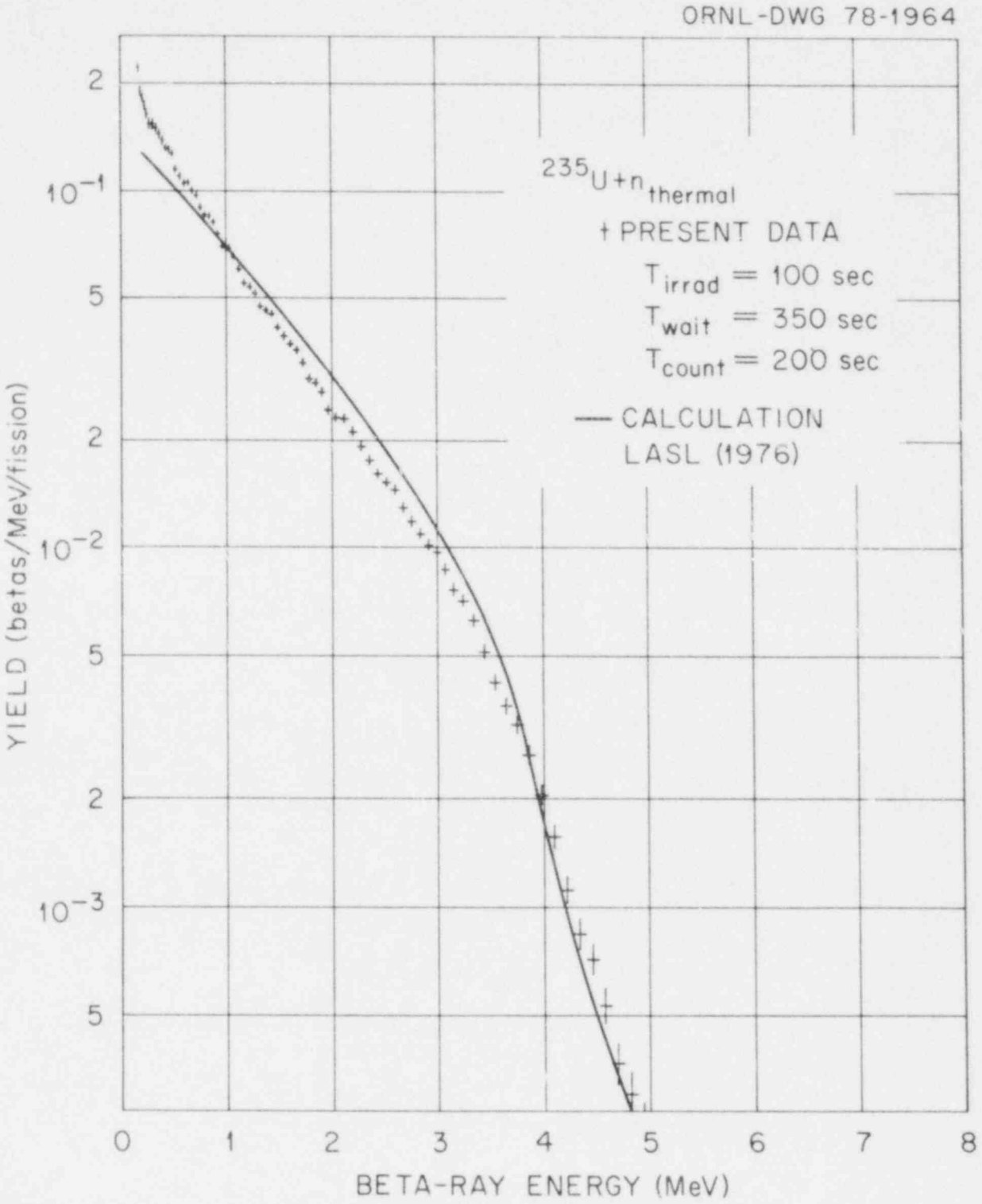

Fig. 39. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

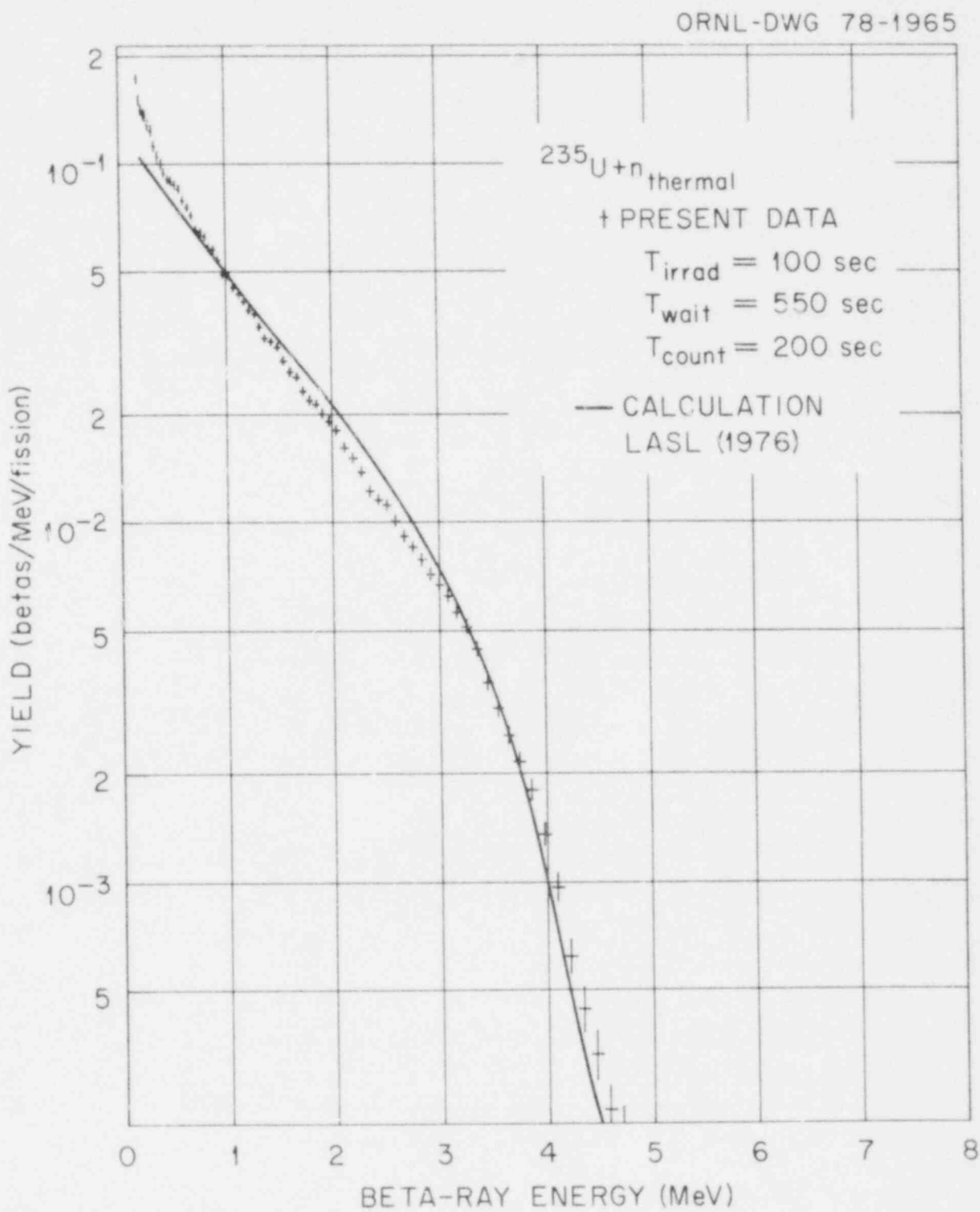

Fig. 40. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

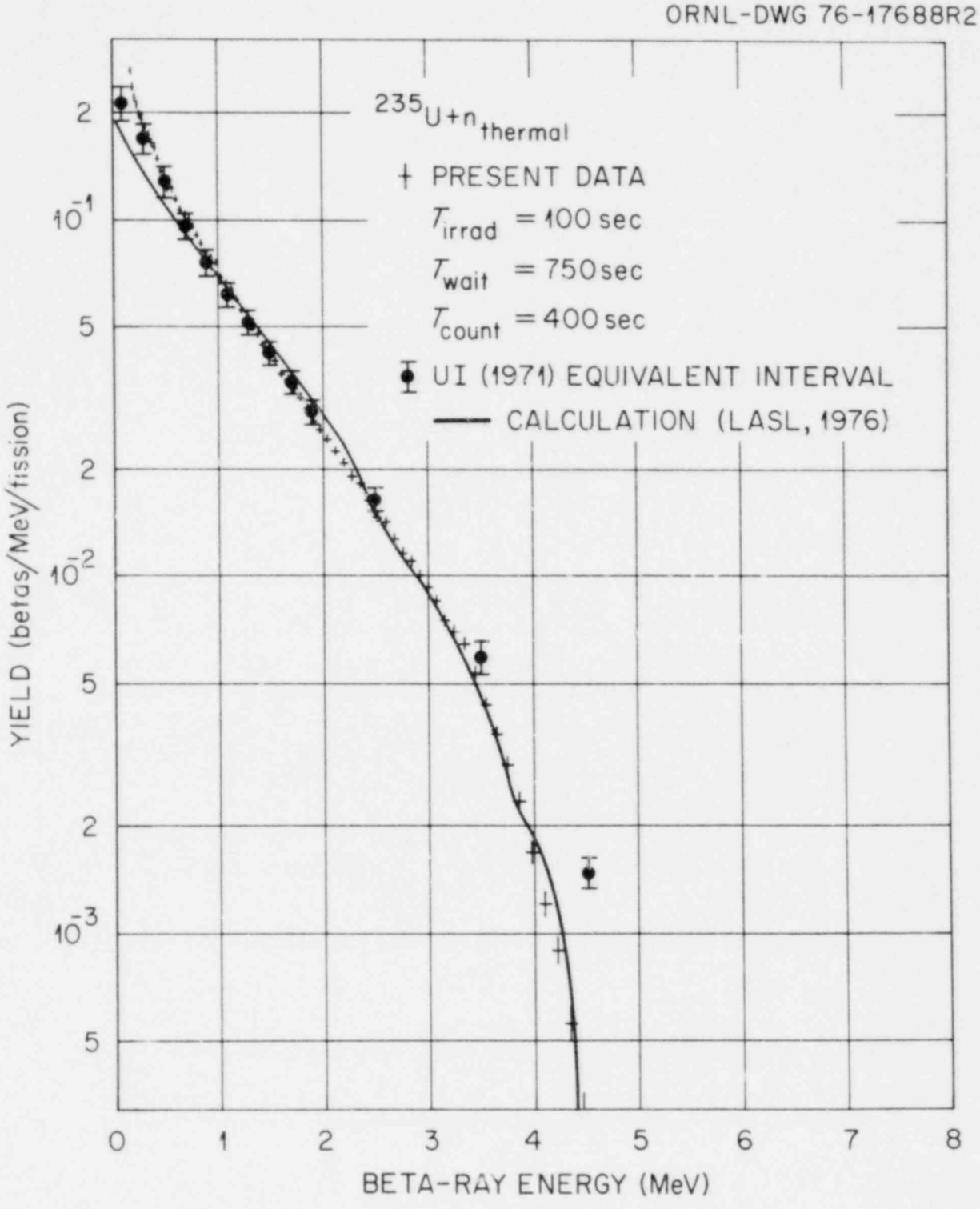

Fig. 41. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The solid points are the data of Tsoulfanidis et al. (Ref. 11) and the calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

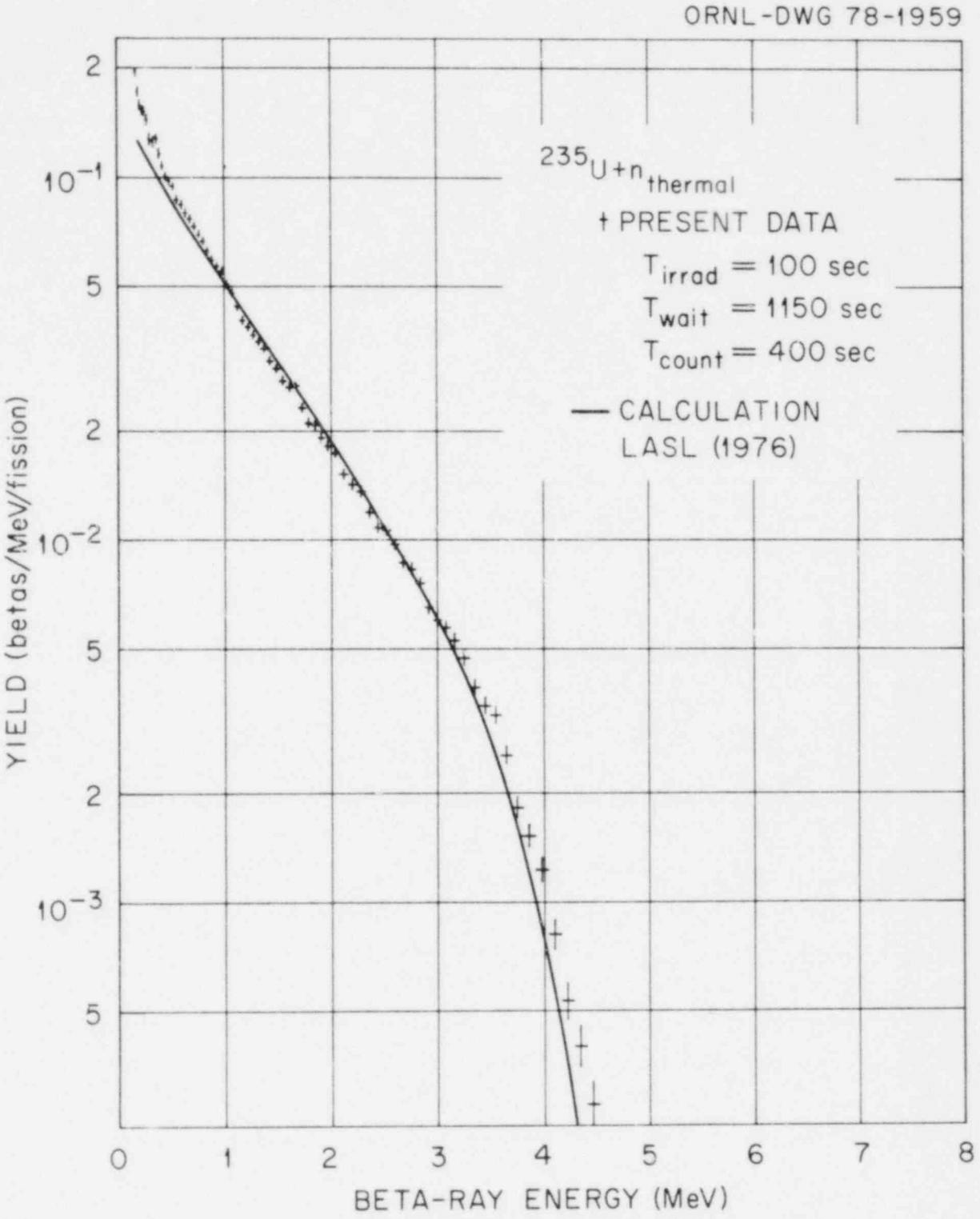

Fig. 42. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The solid points are the data of Tsoulfanidis et al. (Ref. 11) and the calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

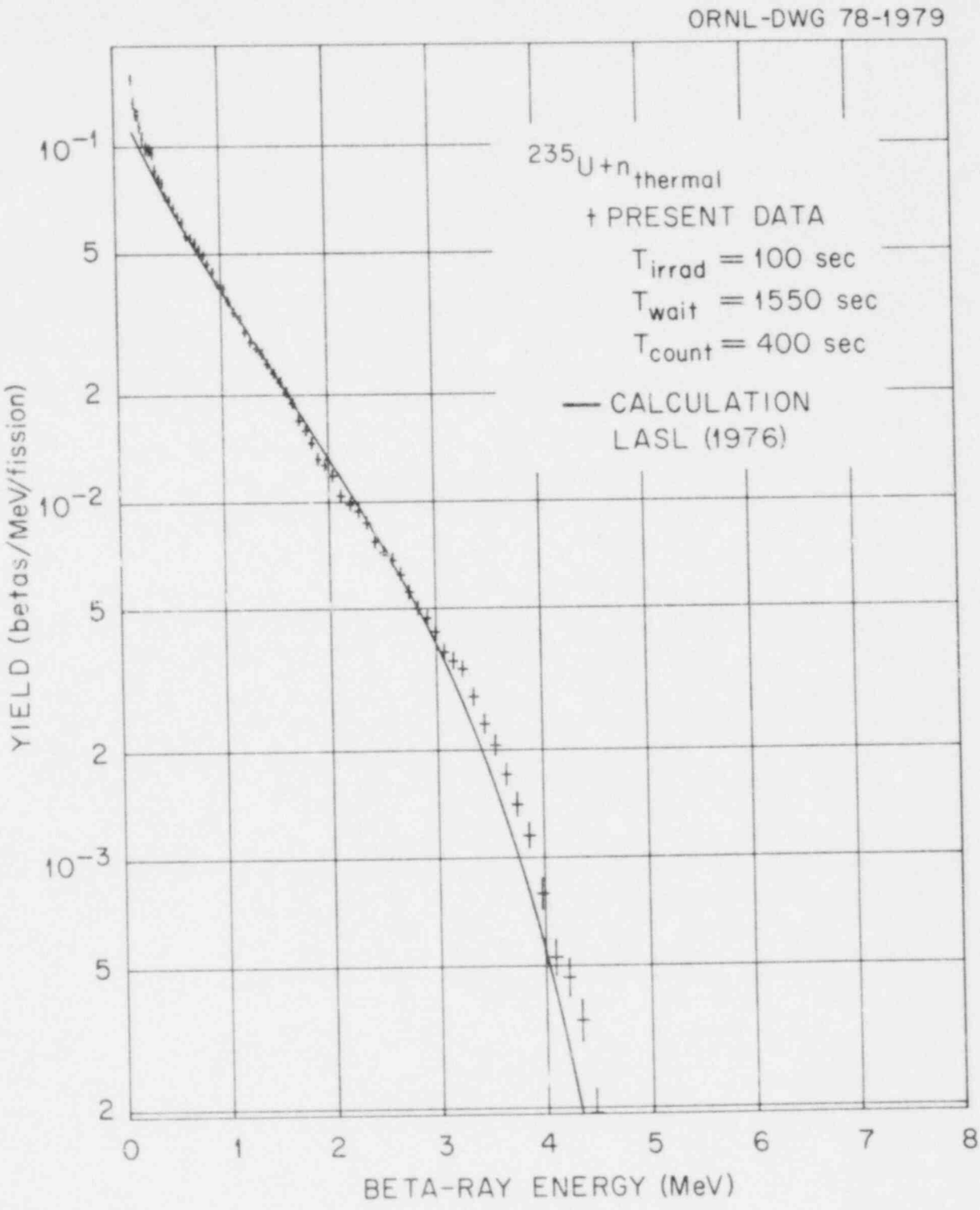

Fig. 43. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

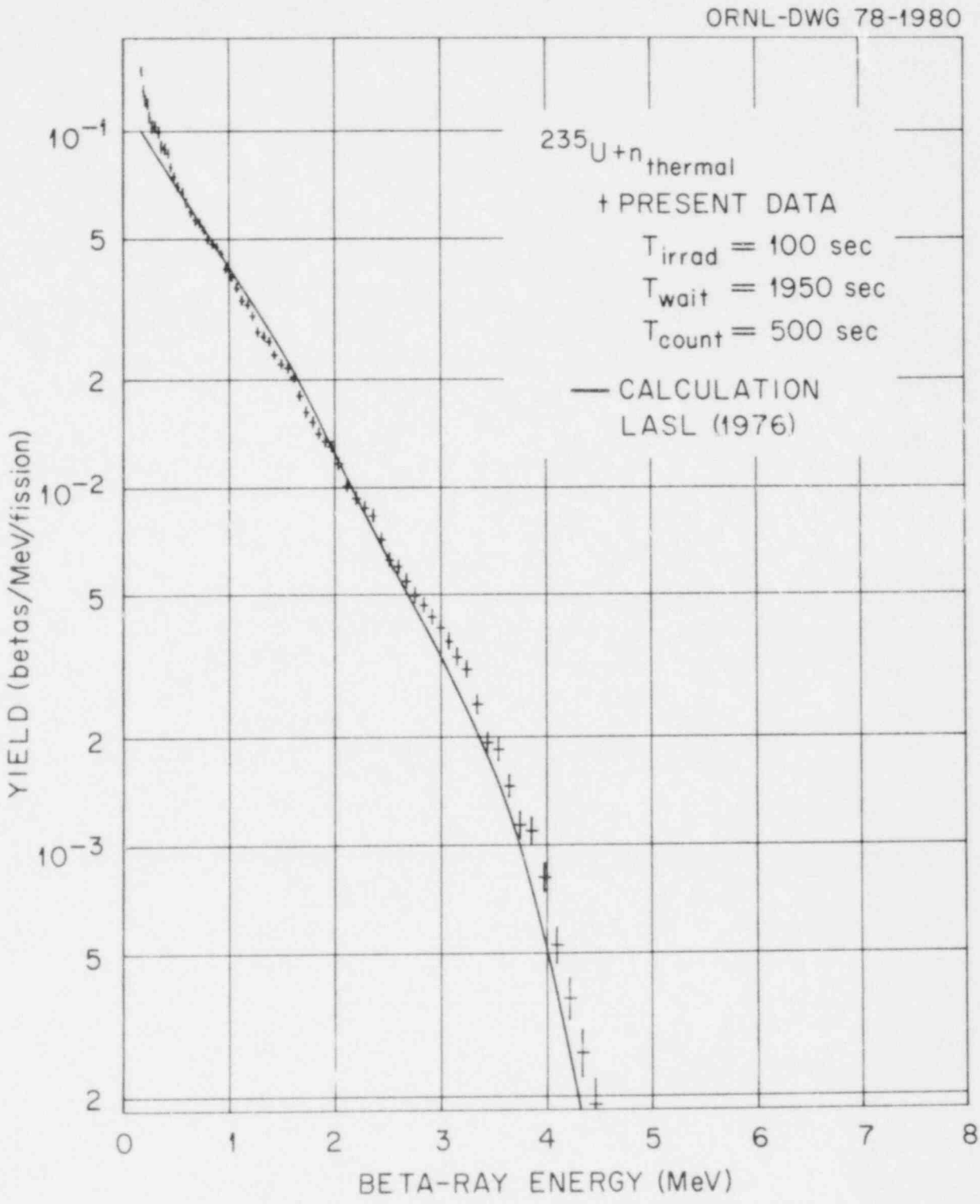

Fig. 44. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

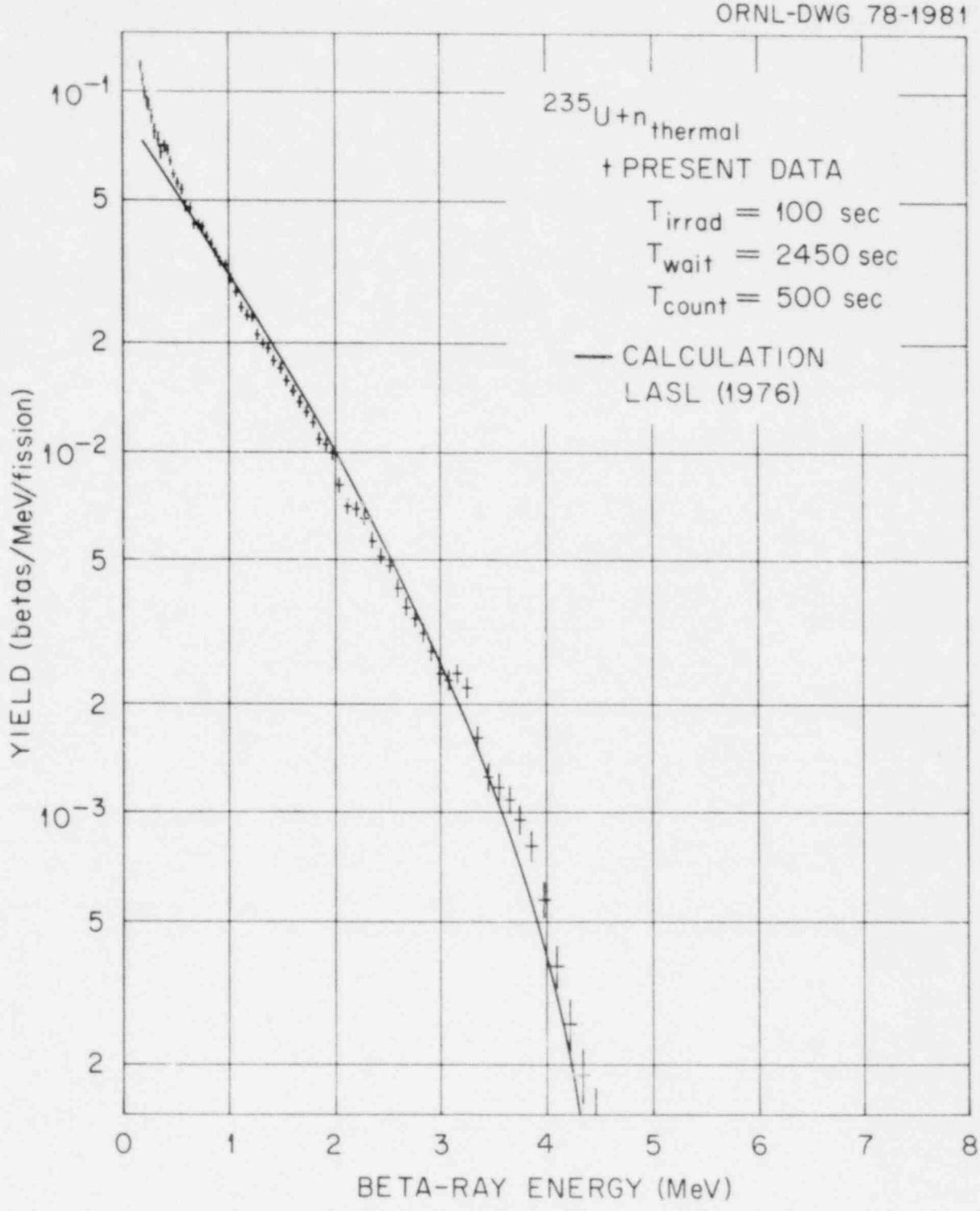

Fig. 45. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

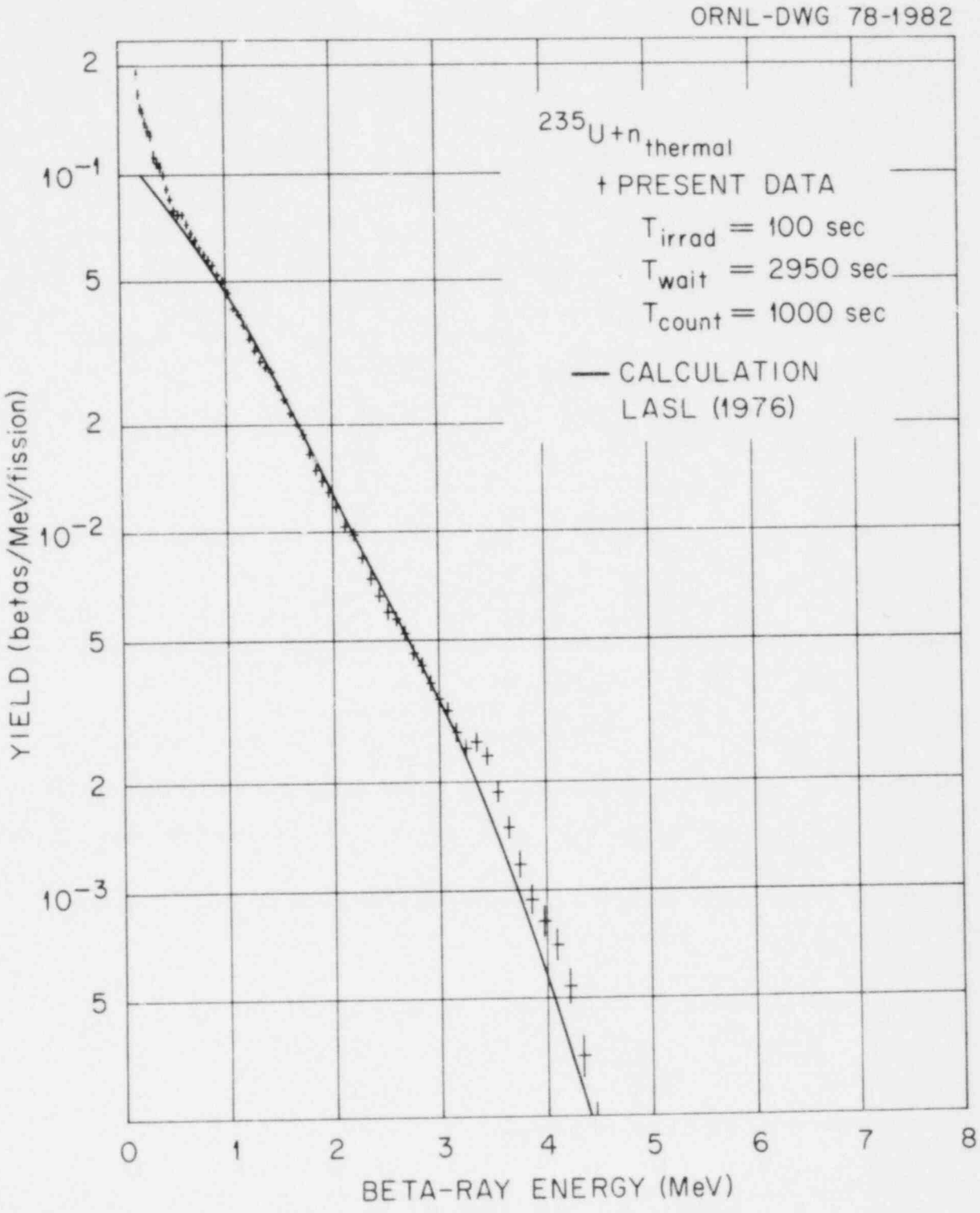

Fig. 46. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

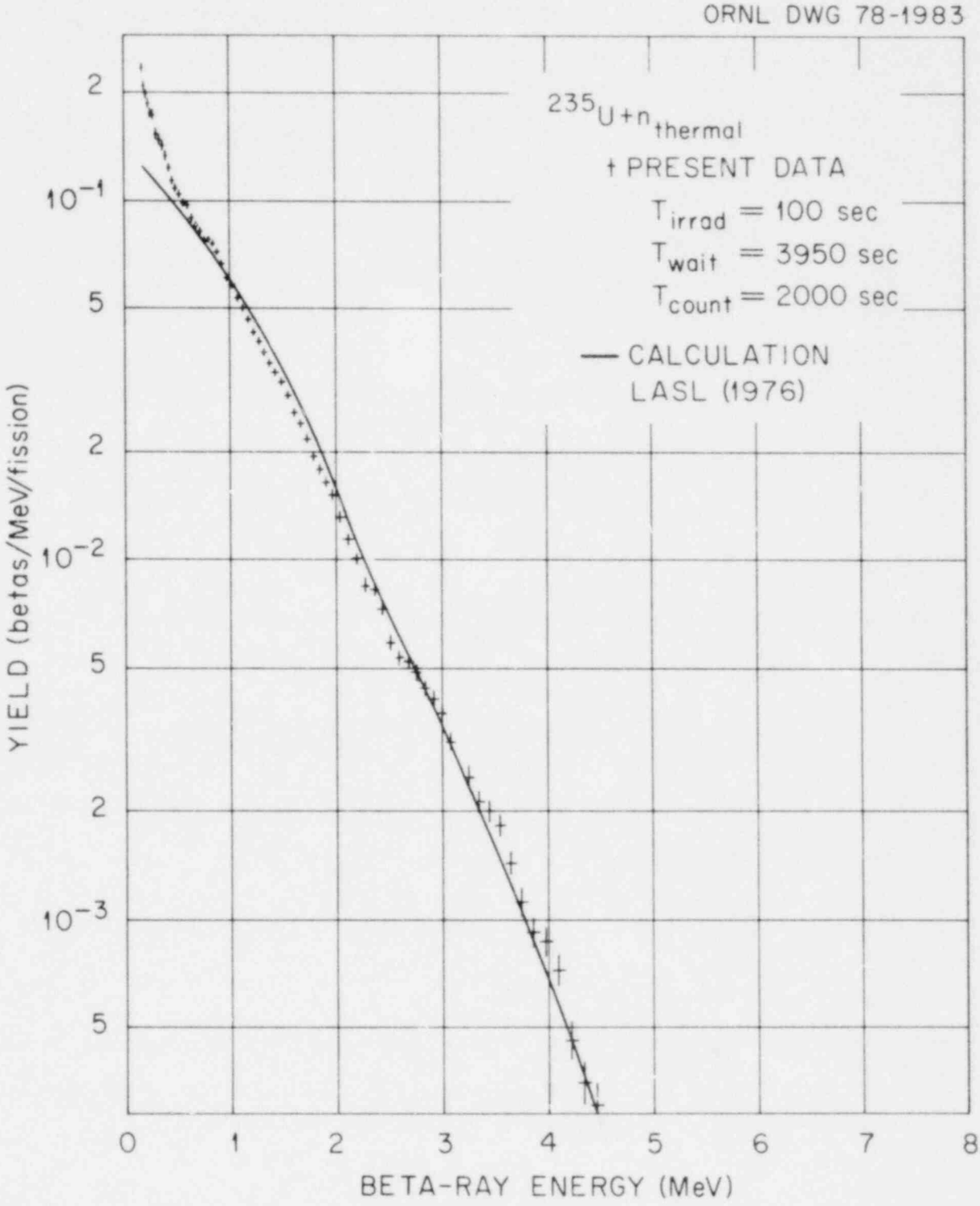

Fig. 47. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

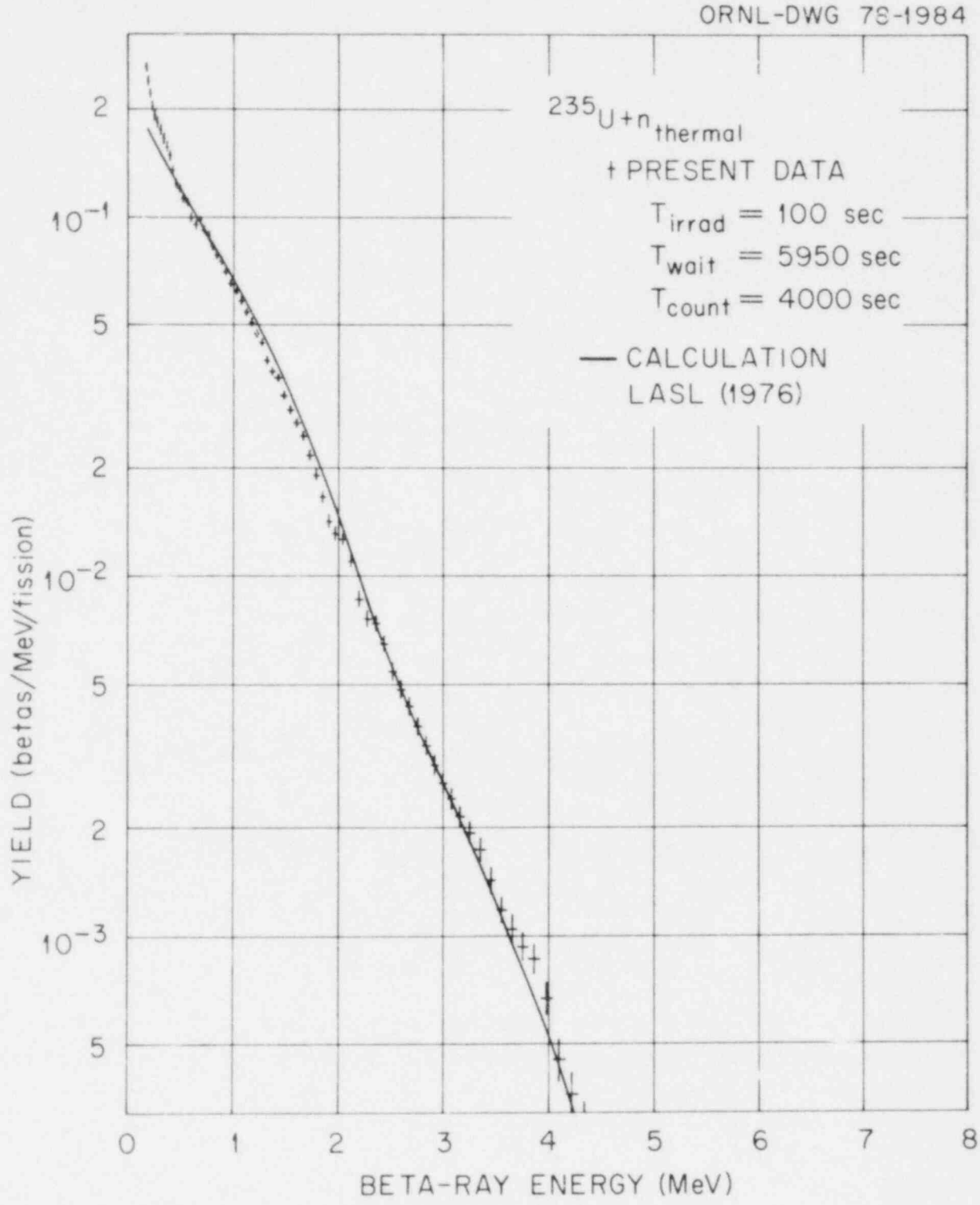

Fig. 48. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

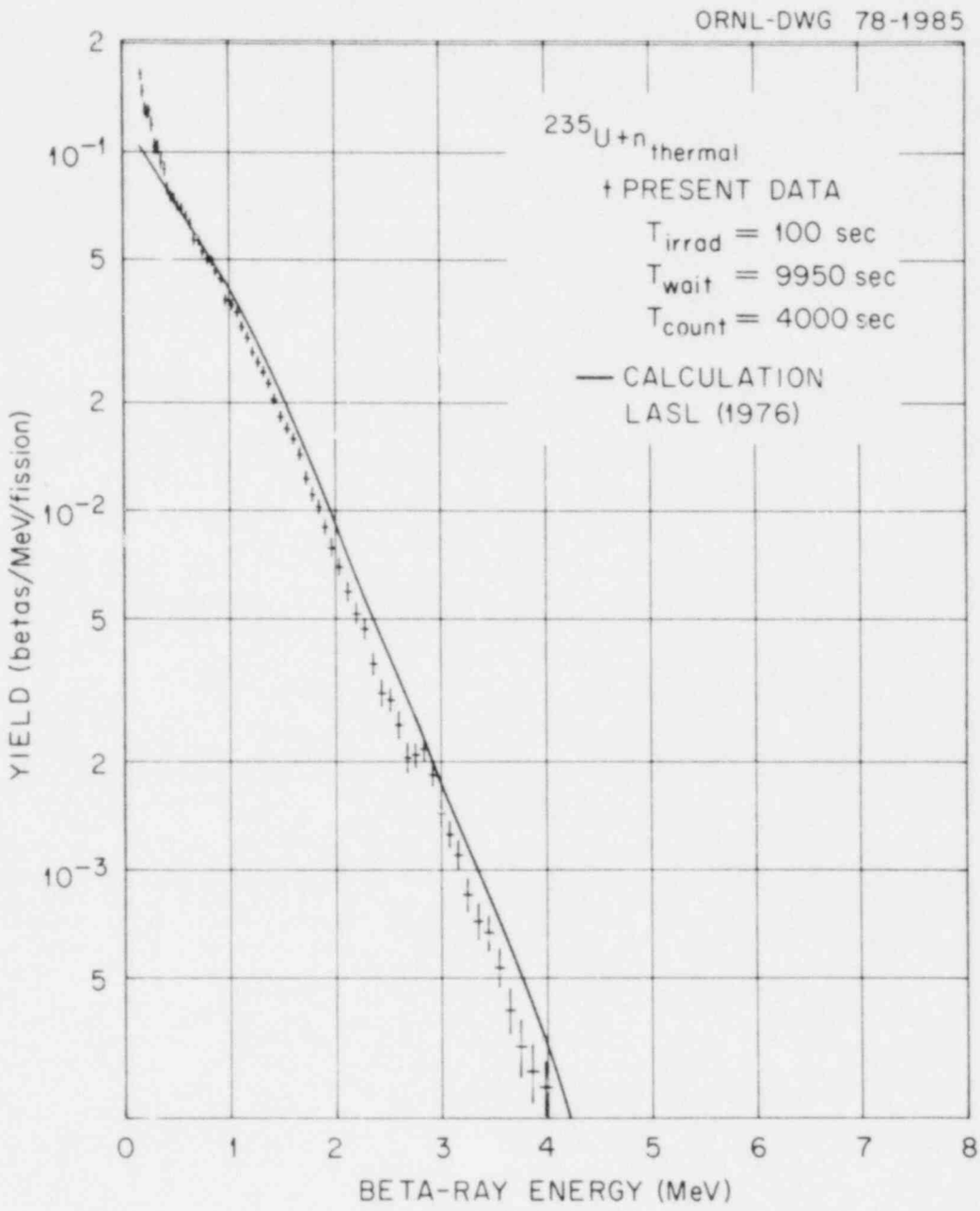

Fig. 49. Spectrum of Beta Rays Due to Thermal-neutron Fission of ^{235}U. The calculations are the work of England and Stamatelatos (Ref. 9). The irradiation time, waiting time, and counting time intervals are given in the legend.

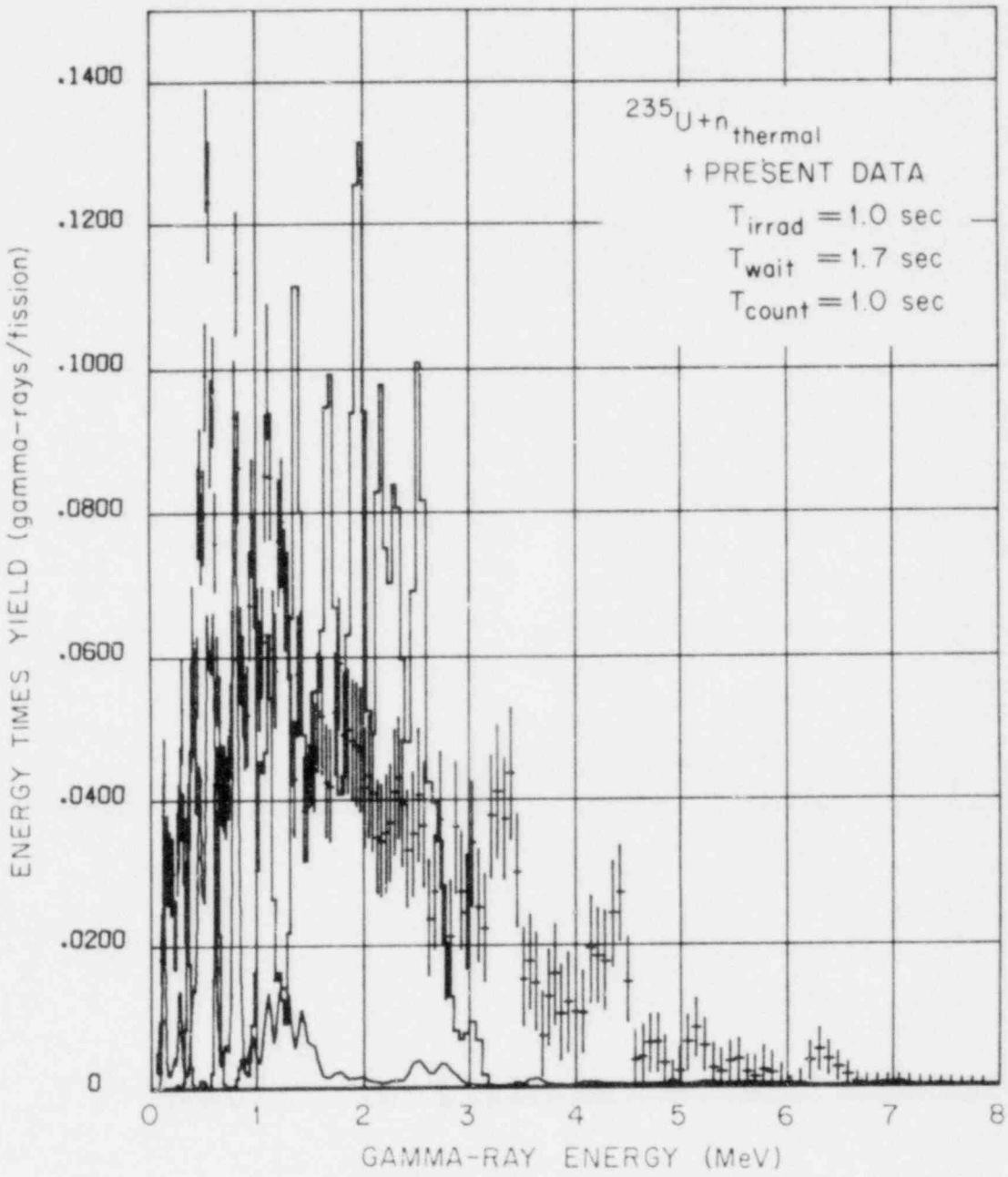

Fig. 50. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

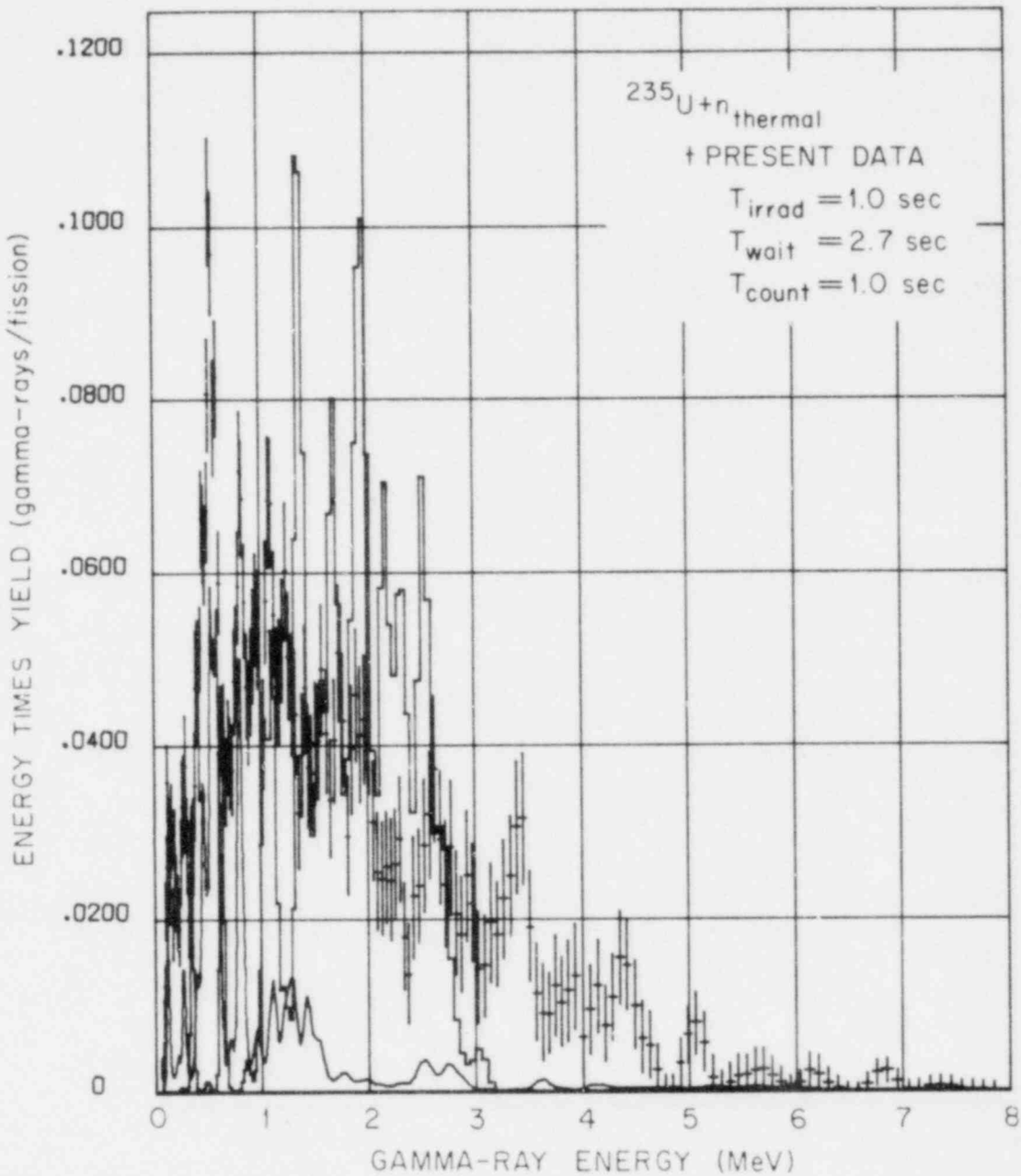

Fig. 51. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 3600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

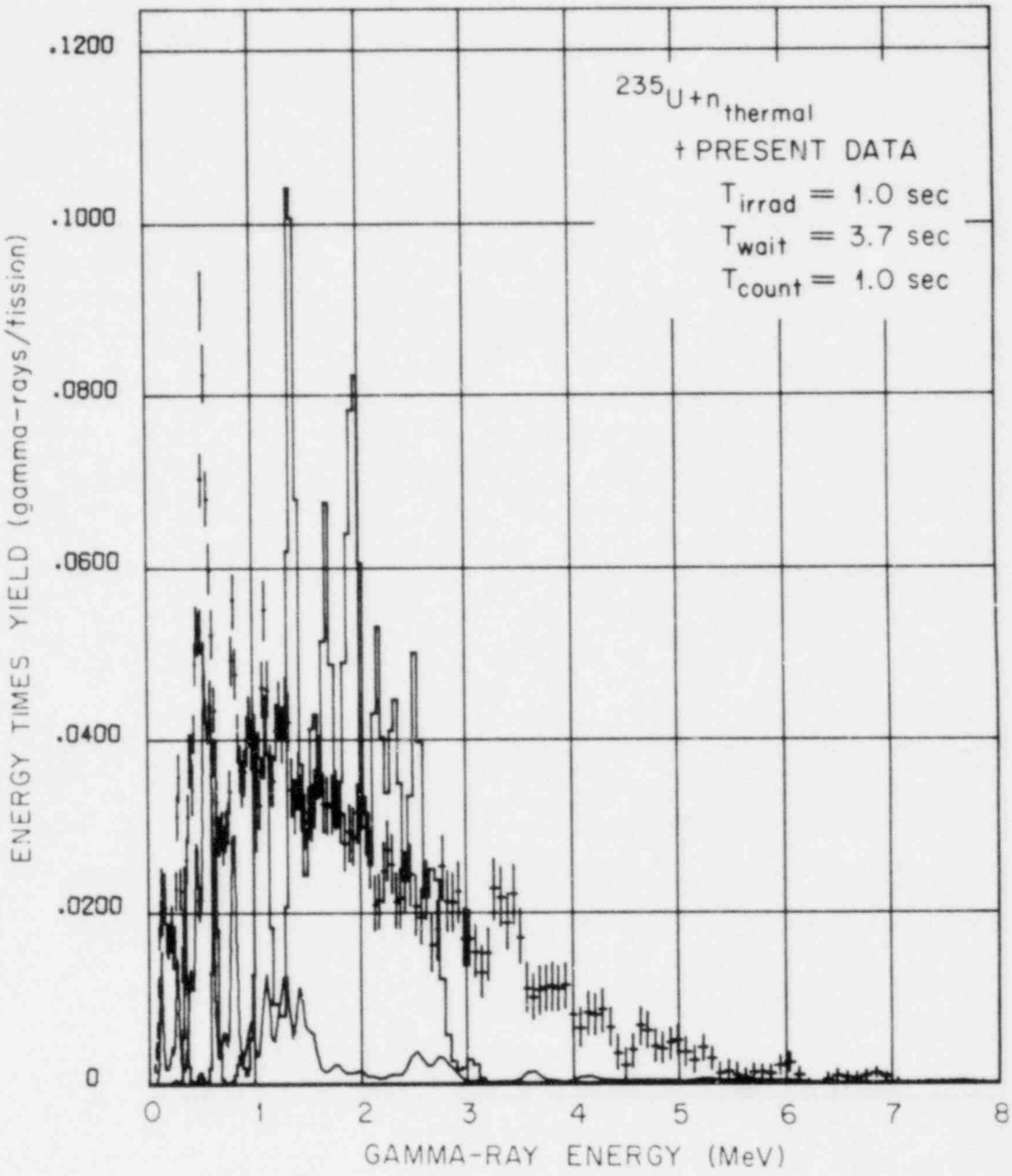

Fig. 52. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

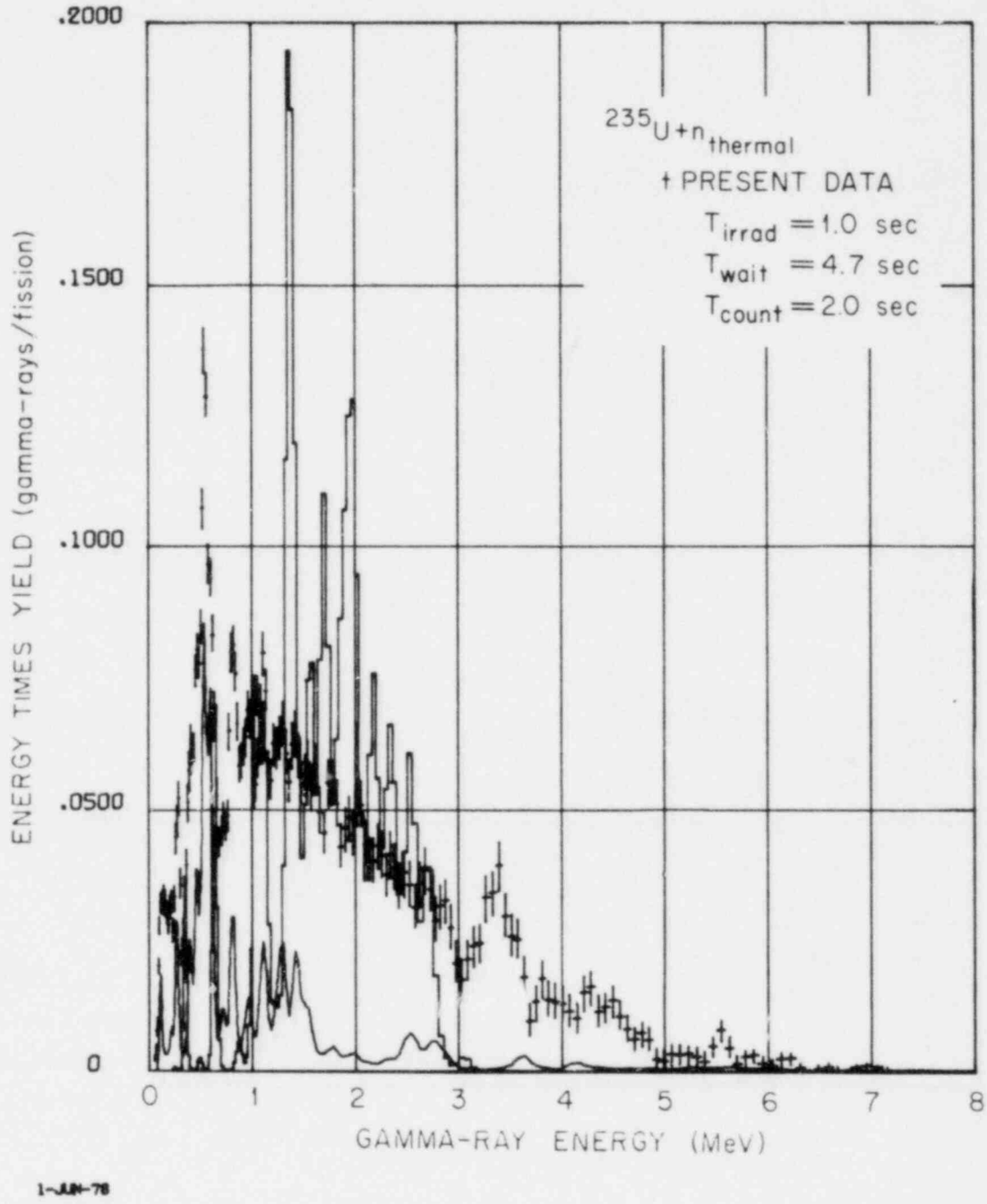

Fig. 53. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ~600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

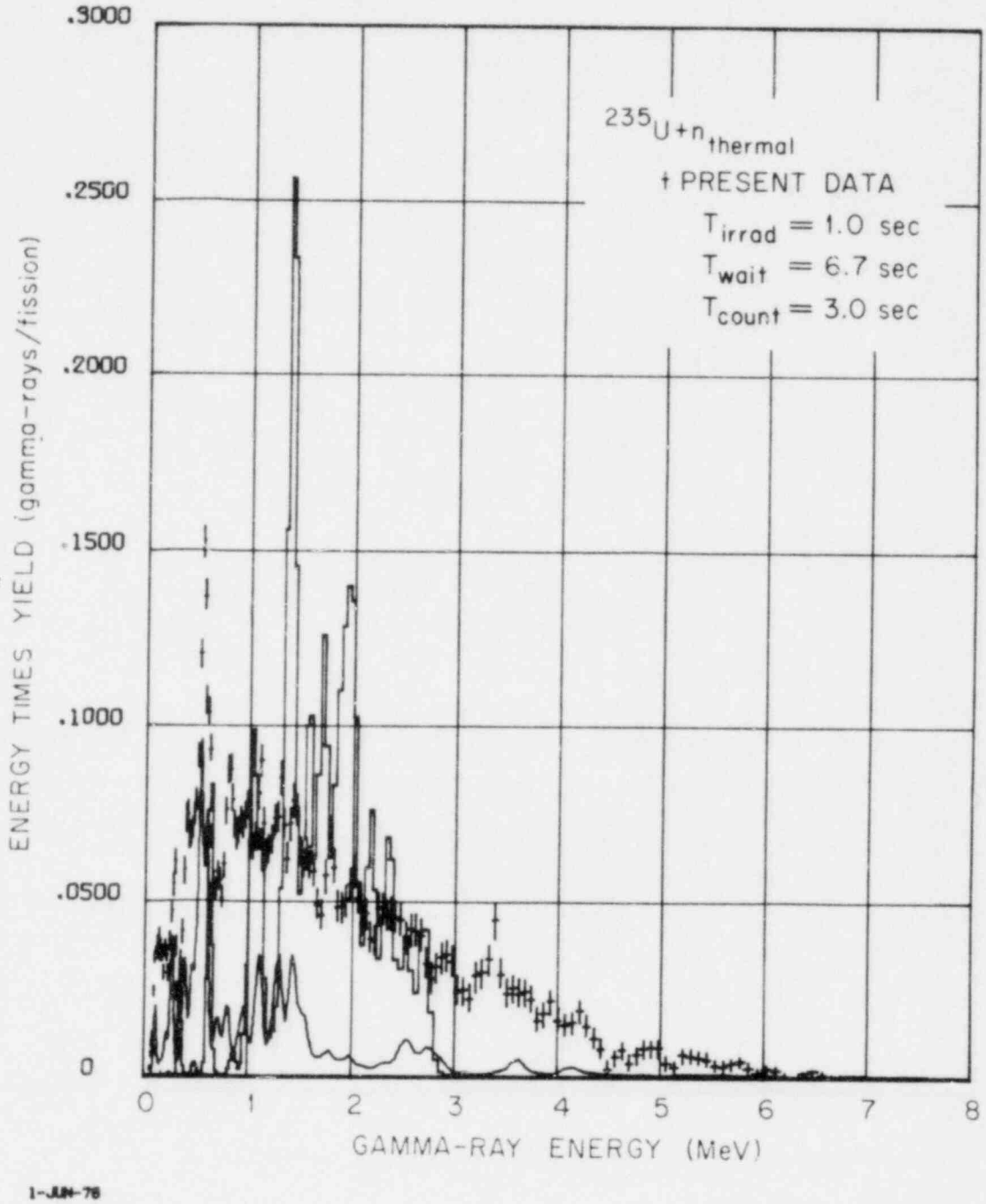

Fig. 54. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ~600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

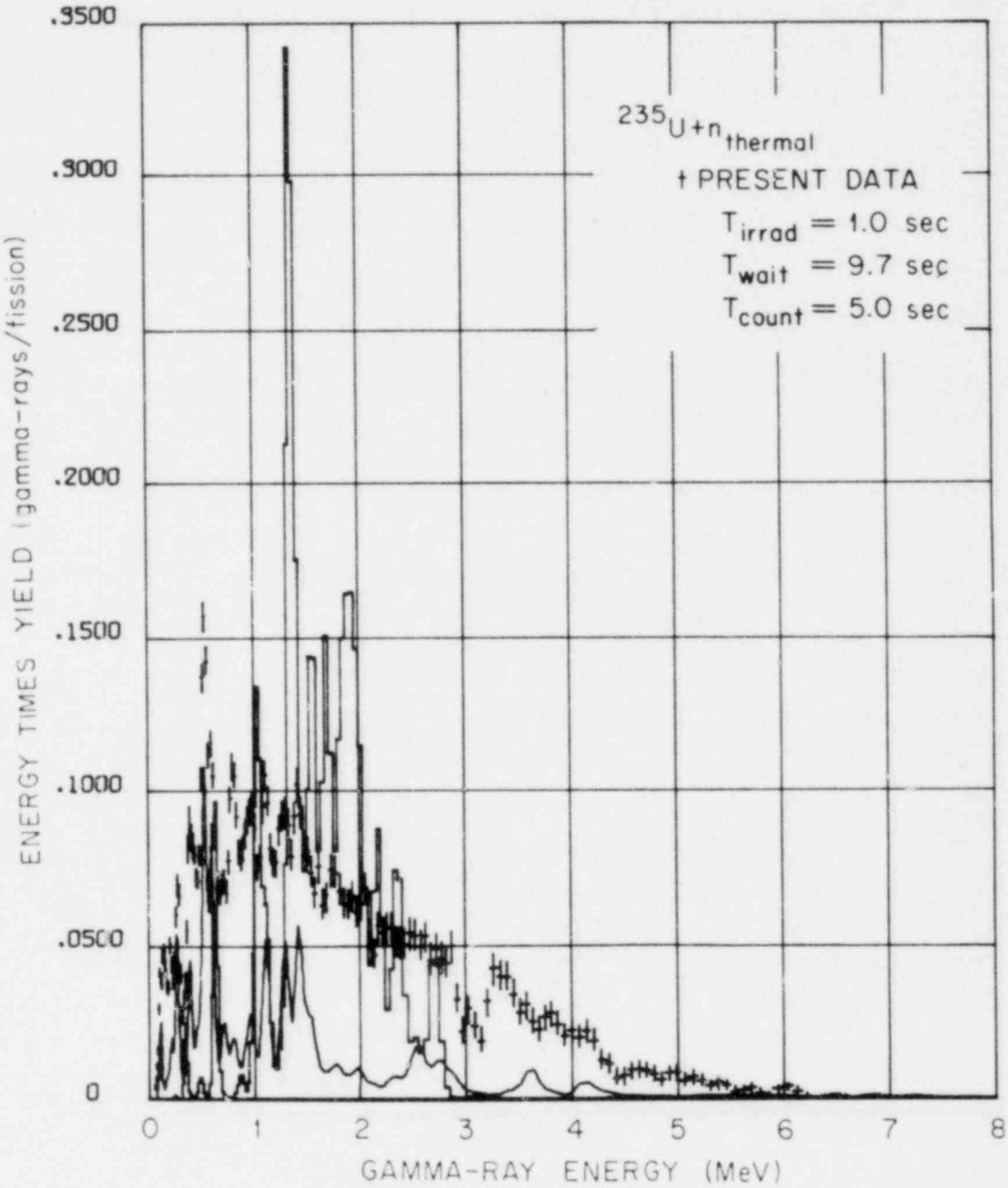

Fig. 55. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 3600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

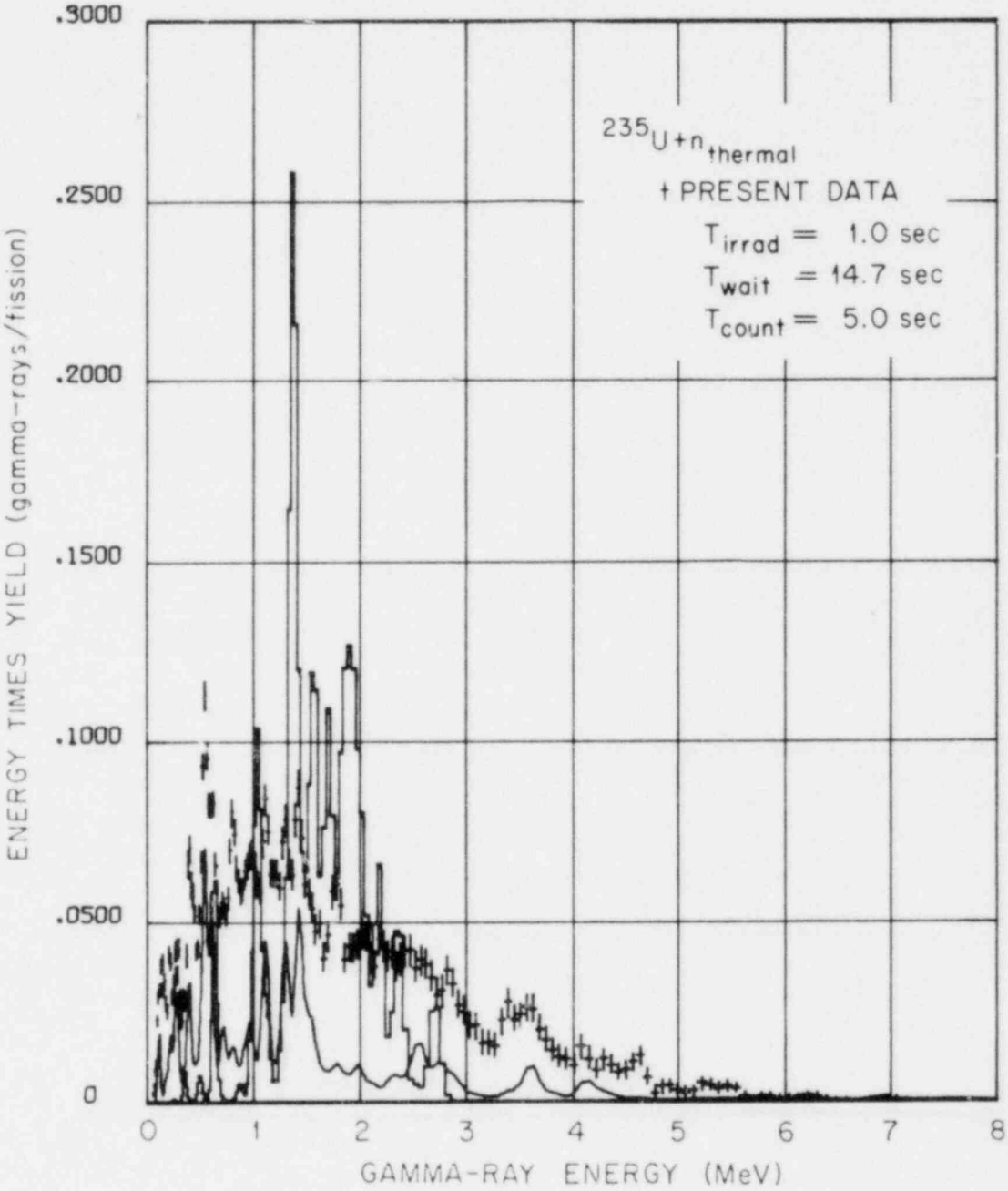

Fig. 56. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ~600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

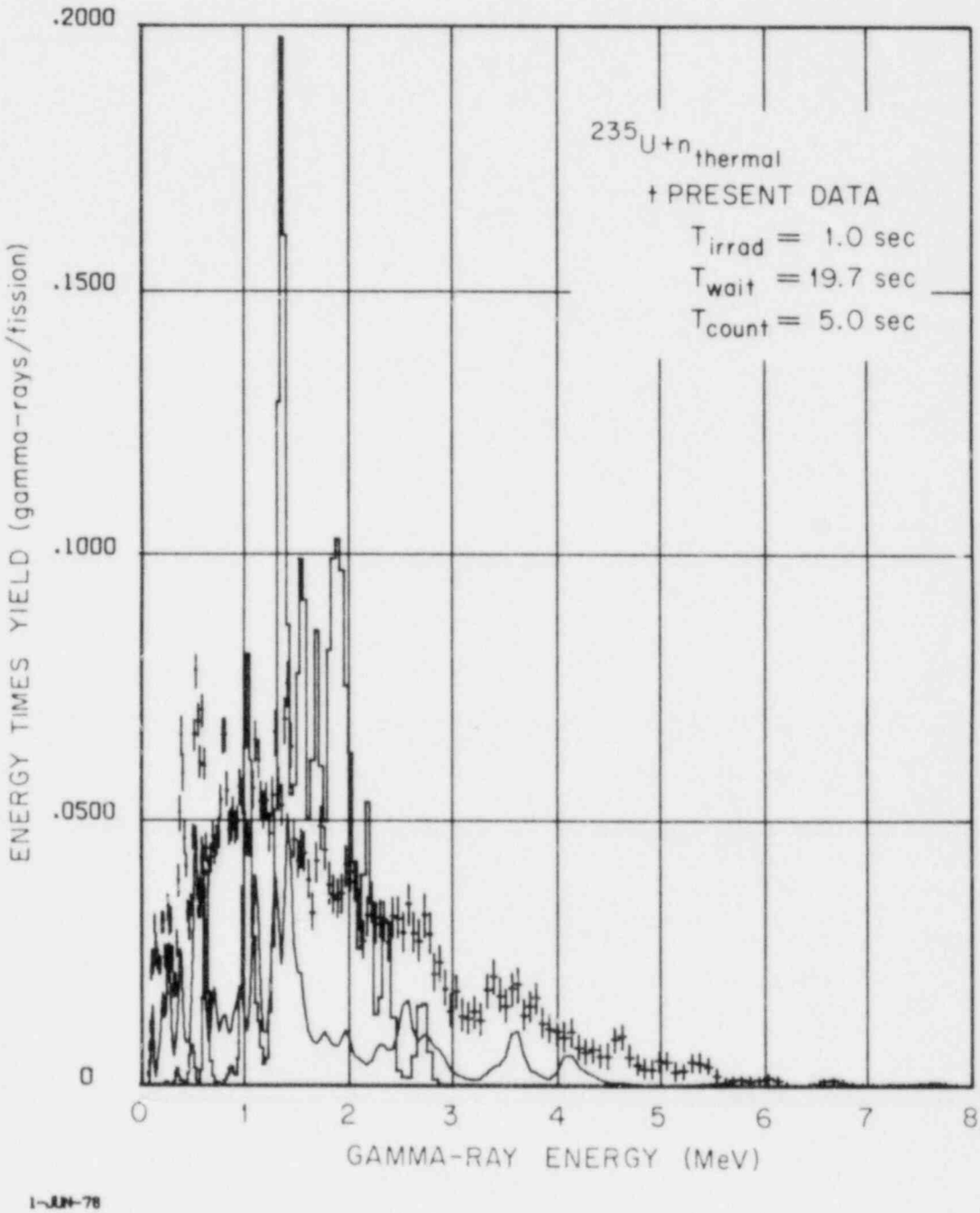

Fig. 57. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

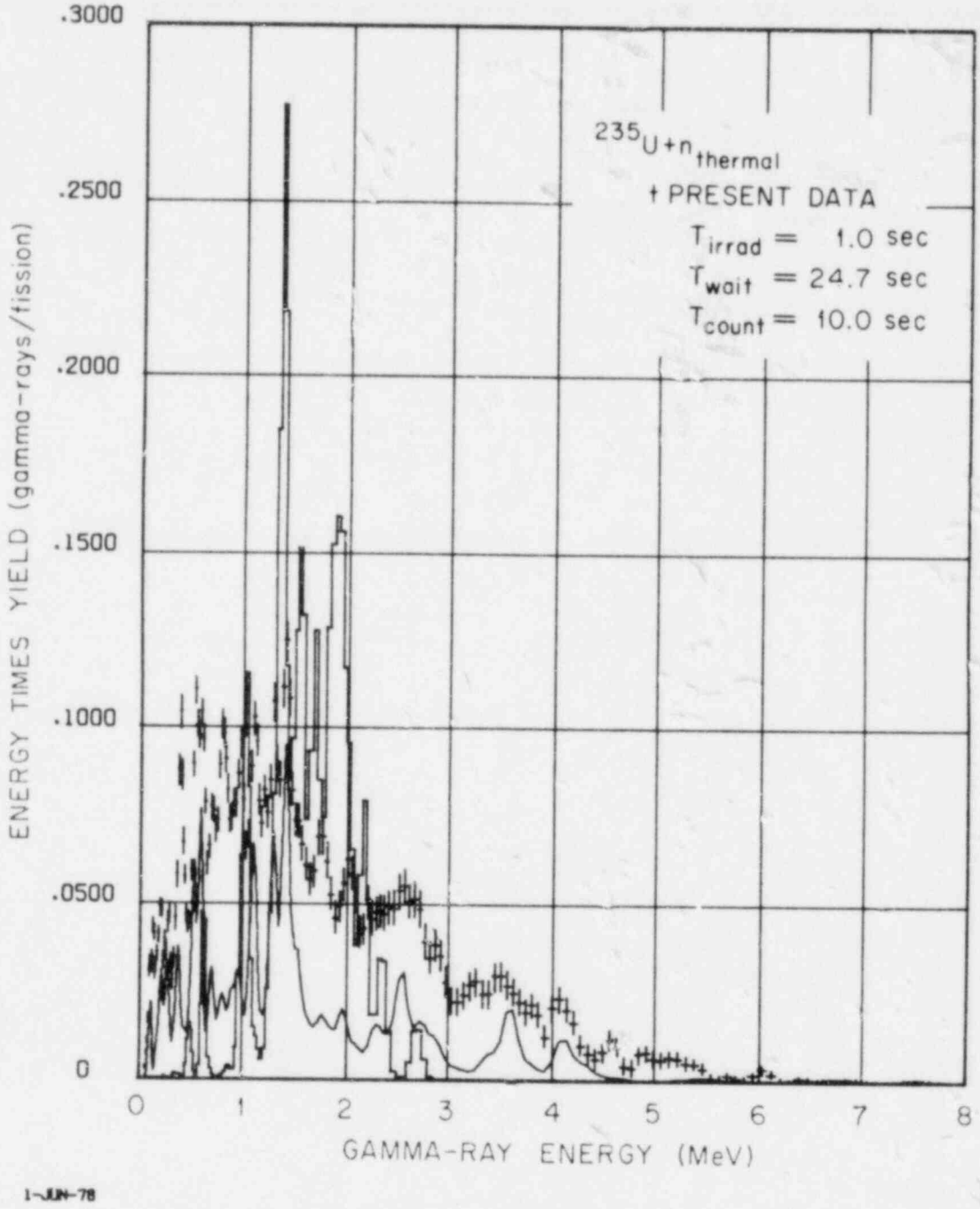

Fig. 58. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

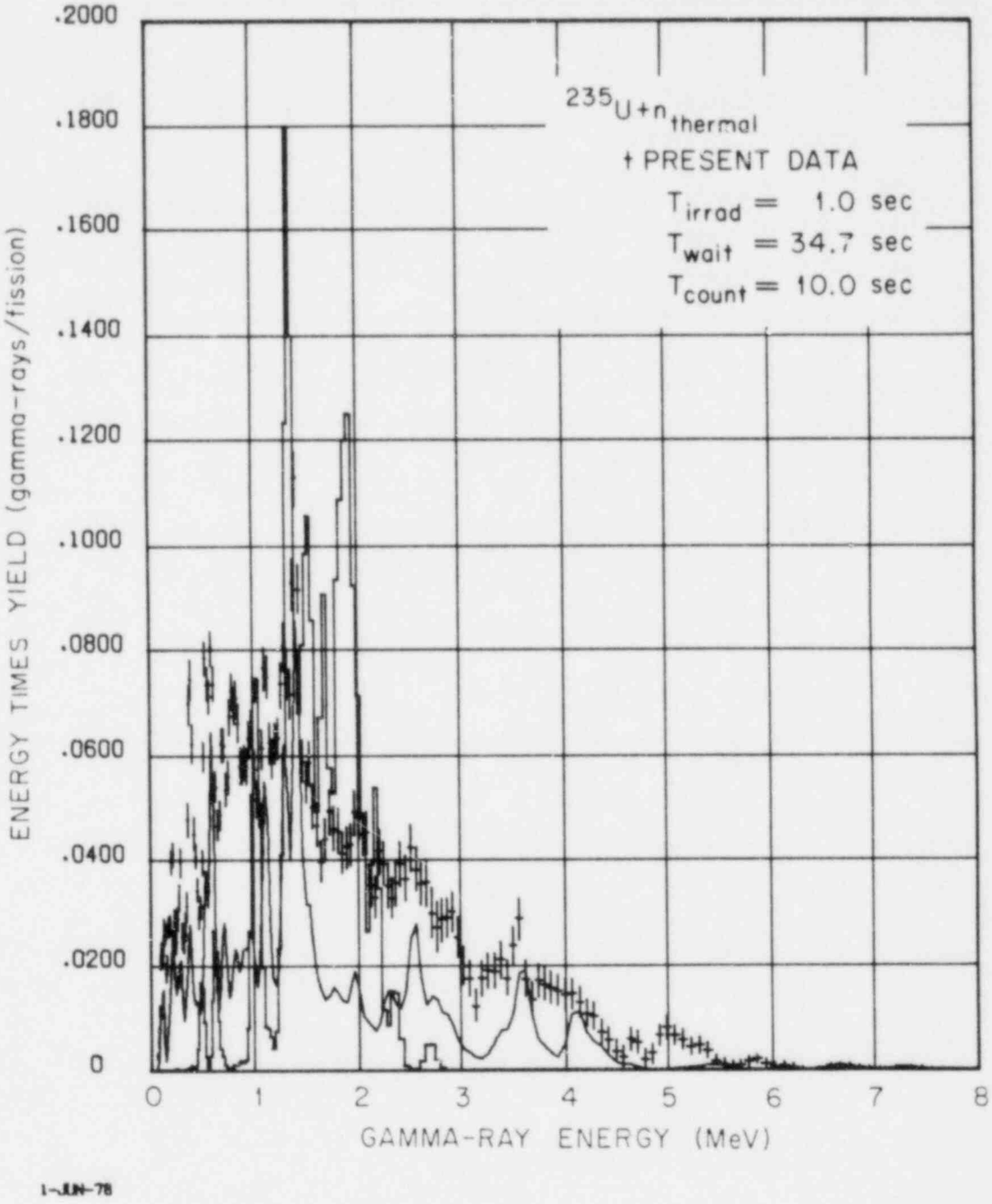

Fig. 59. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ~600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

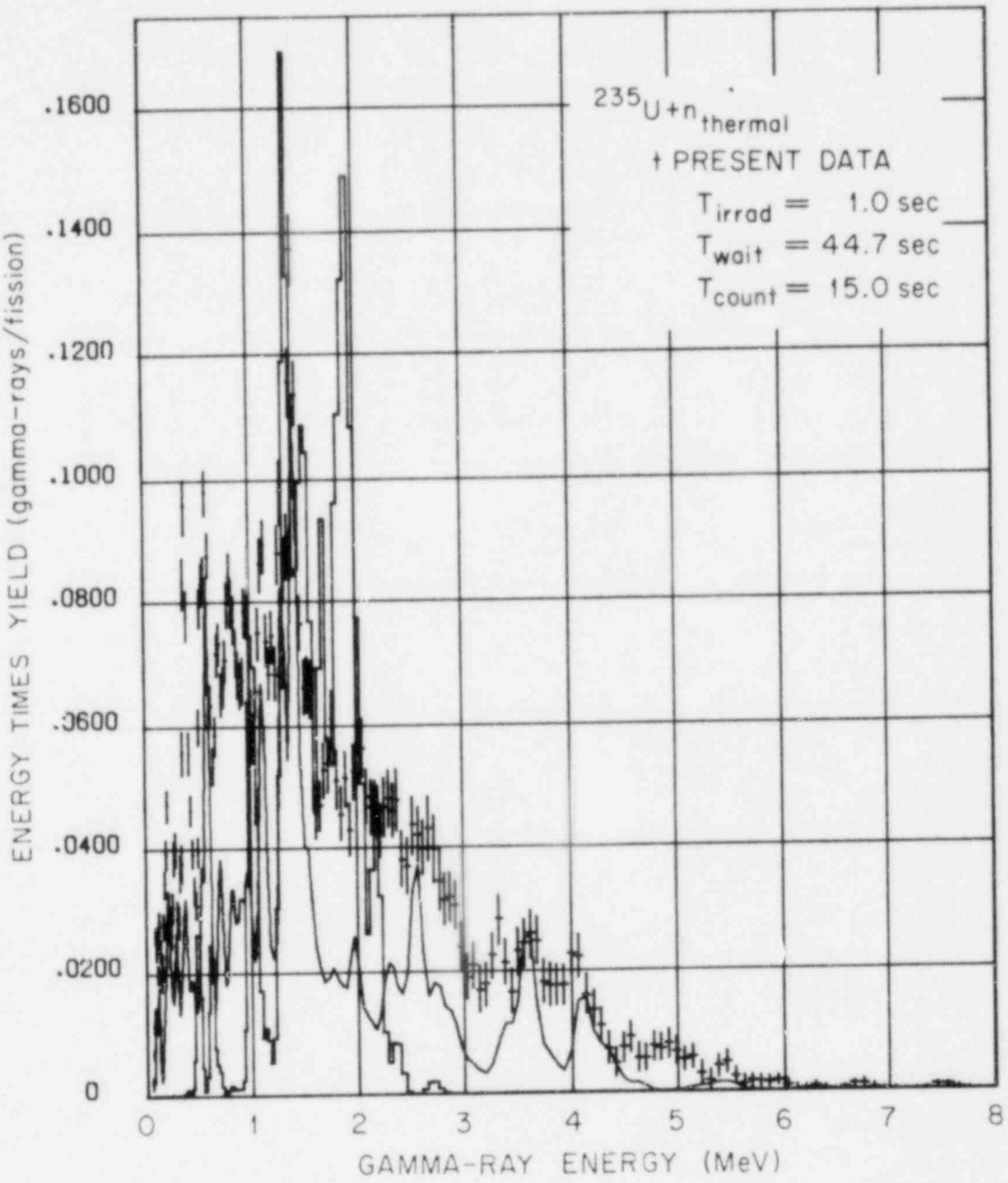

Fig. 60. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

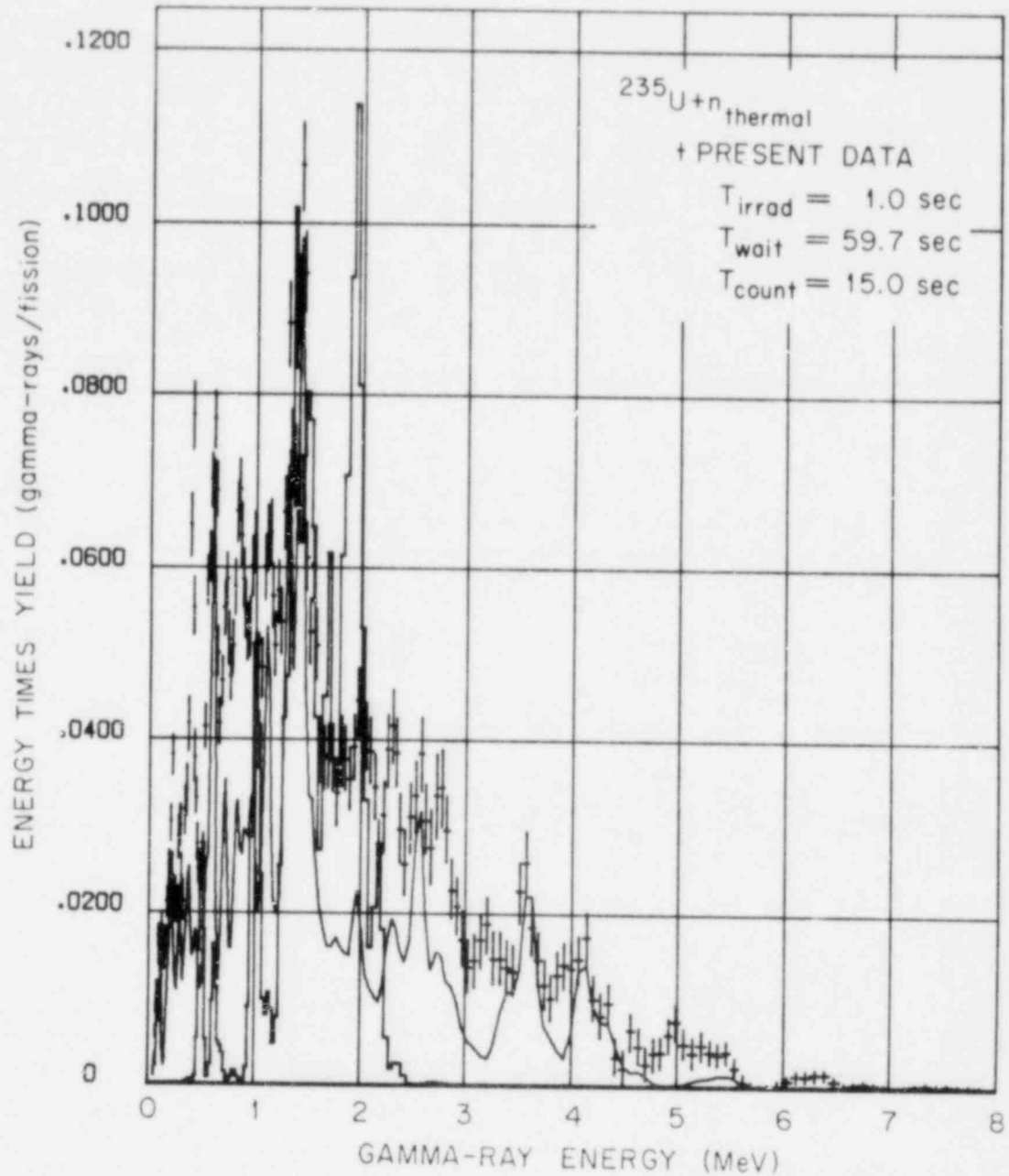

Fig. 61. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

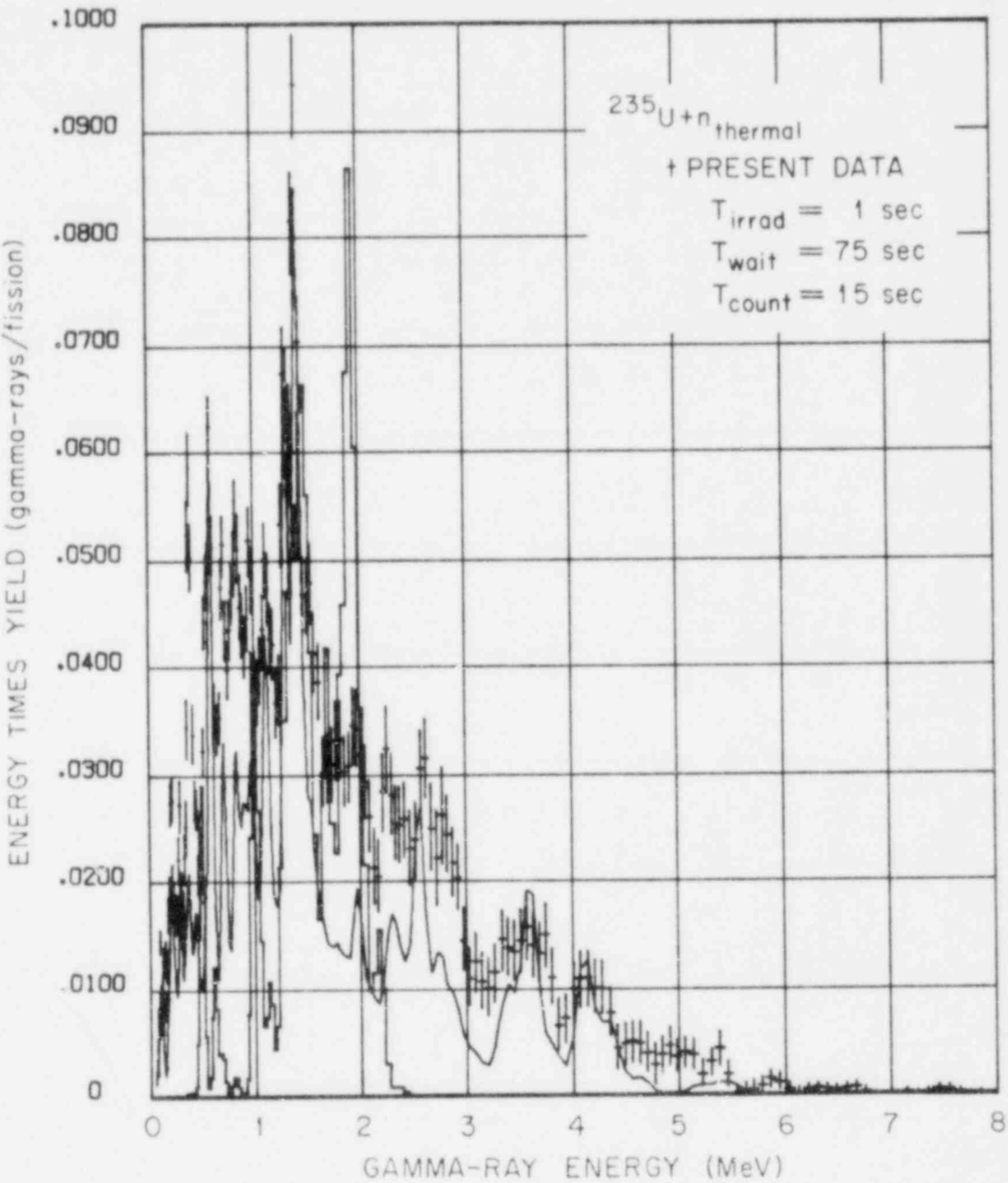

Fig. 62. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

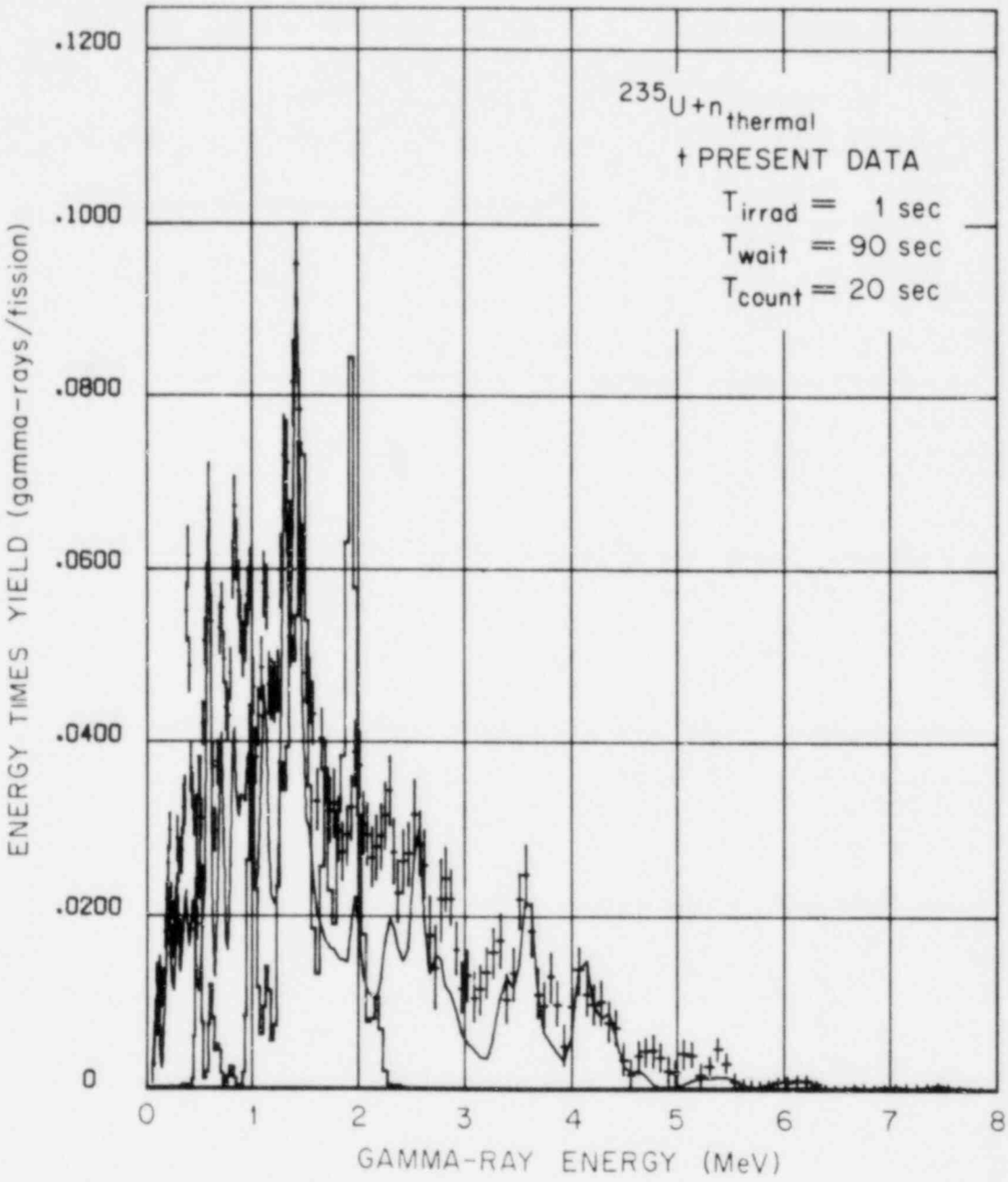

Fig. 63. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

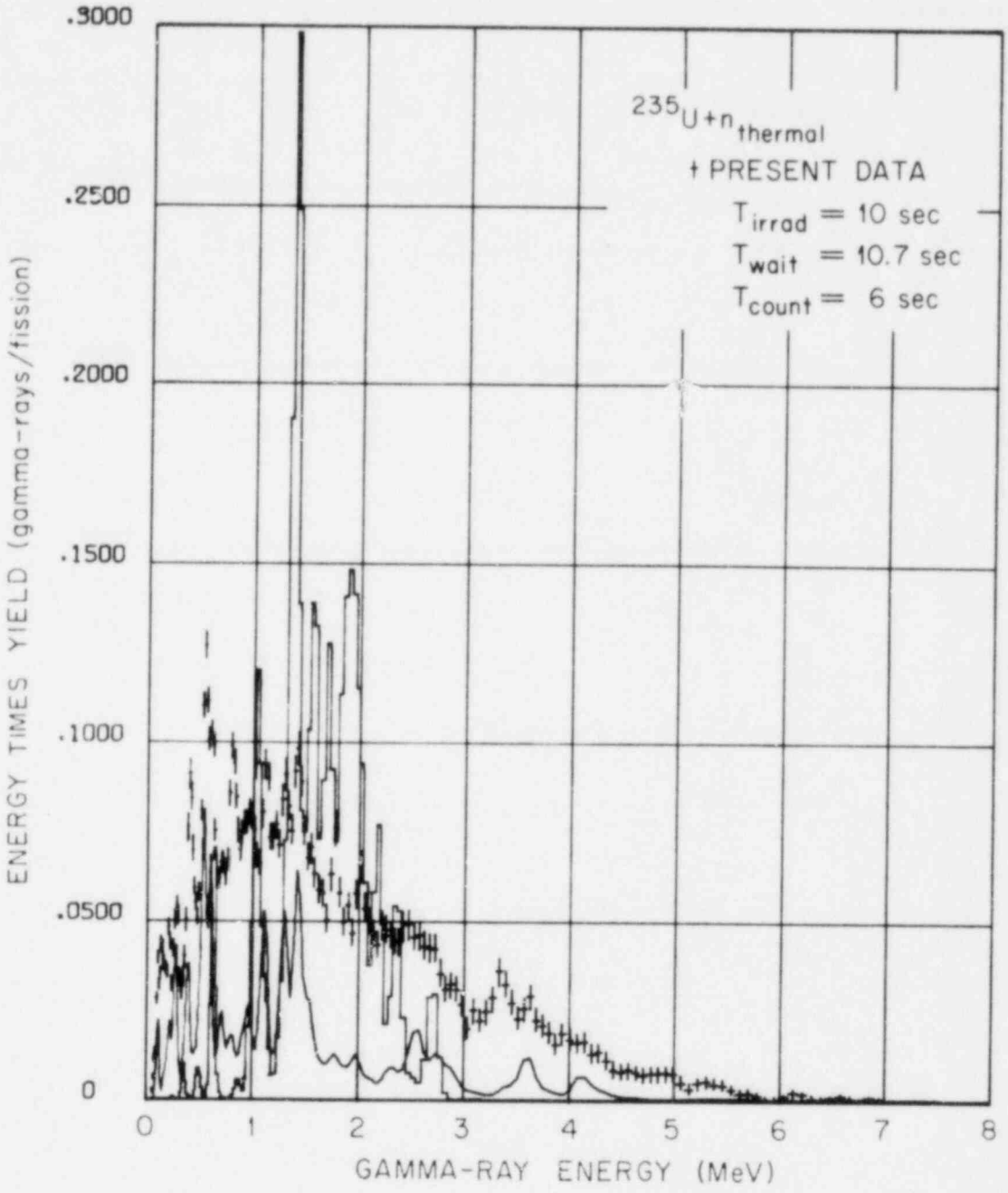

Fig. 64. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 3600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

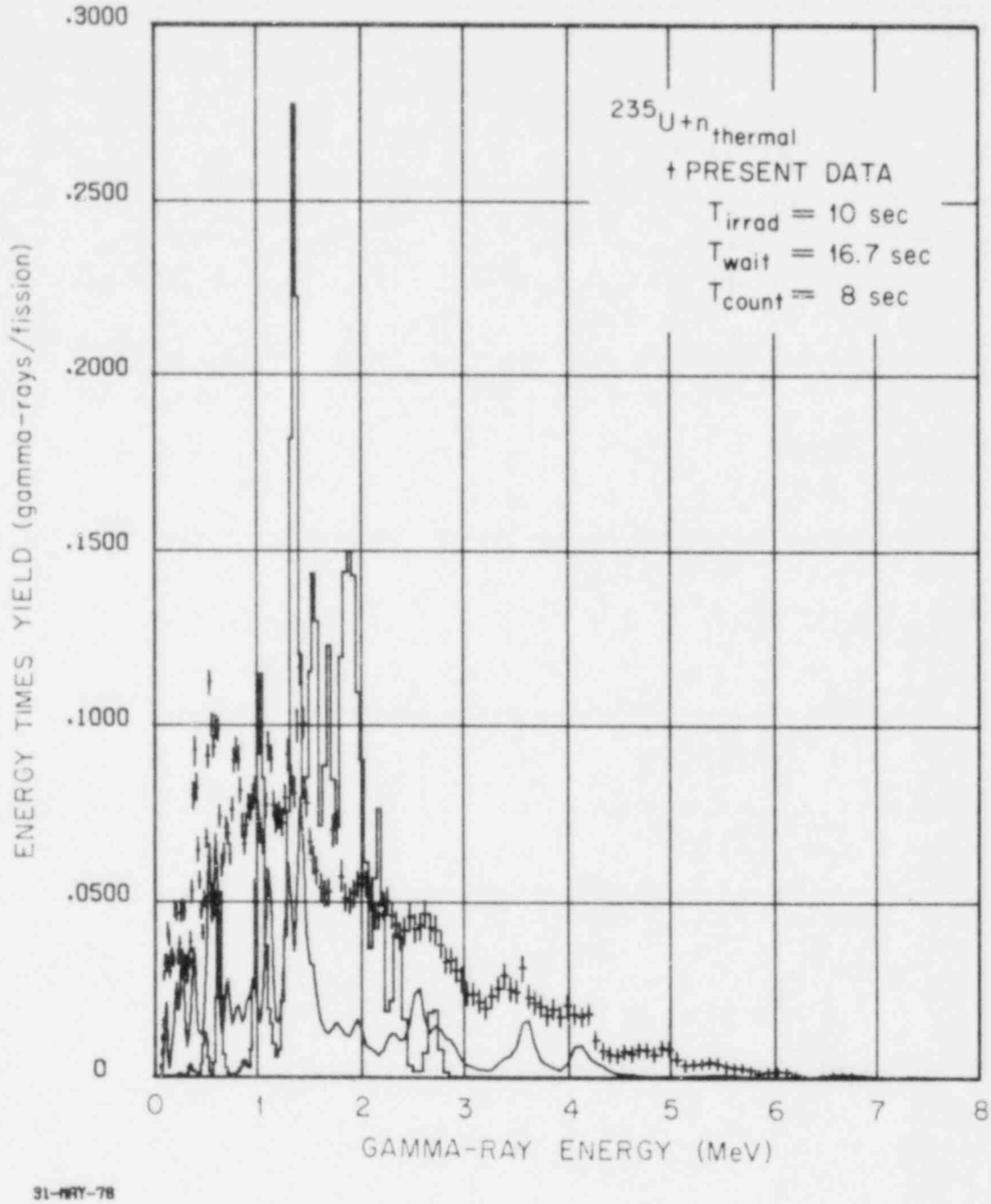

Fig. 65. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 3600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

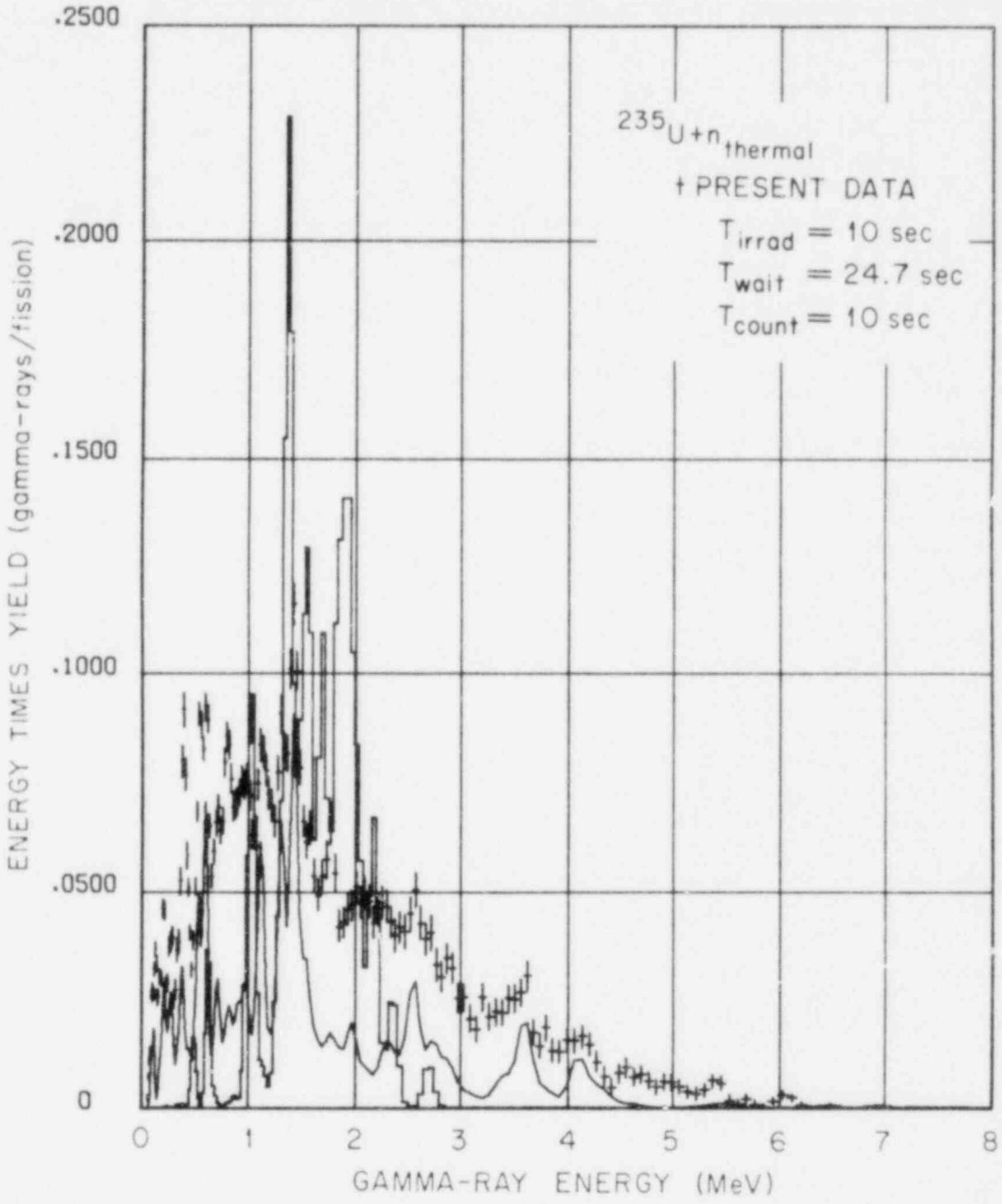

Fig. 66. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ~600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

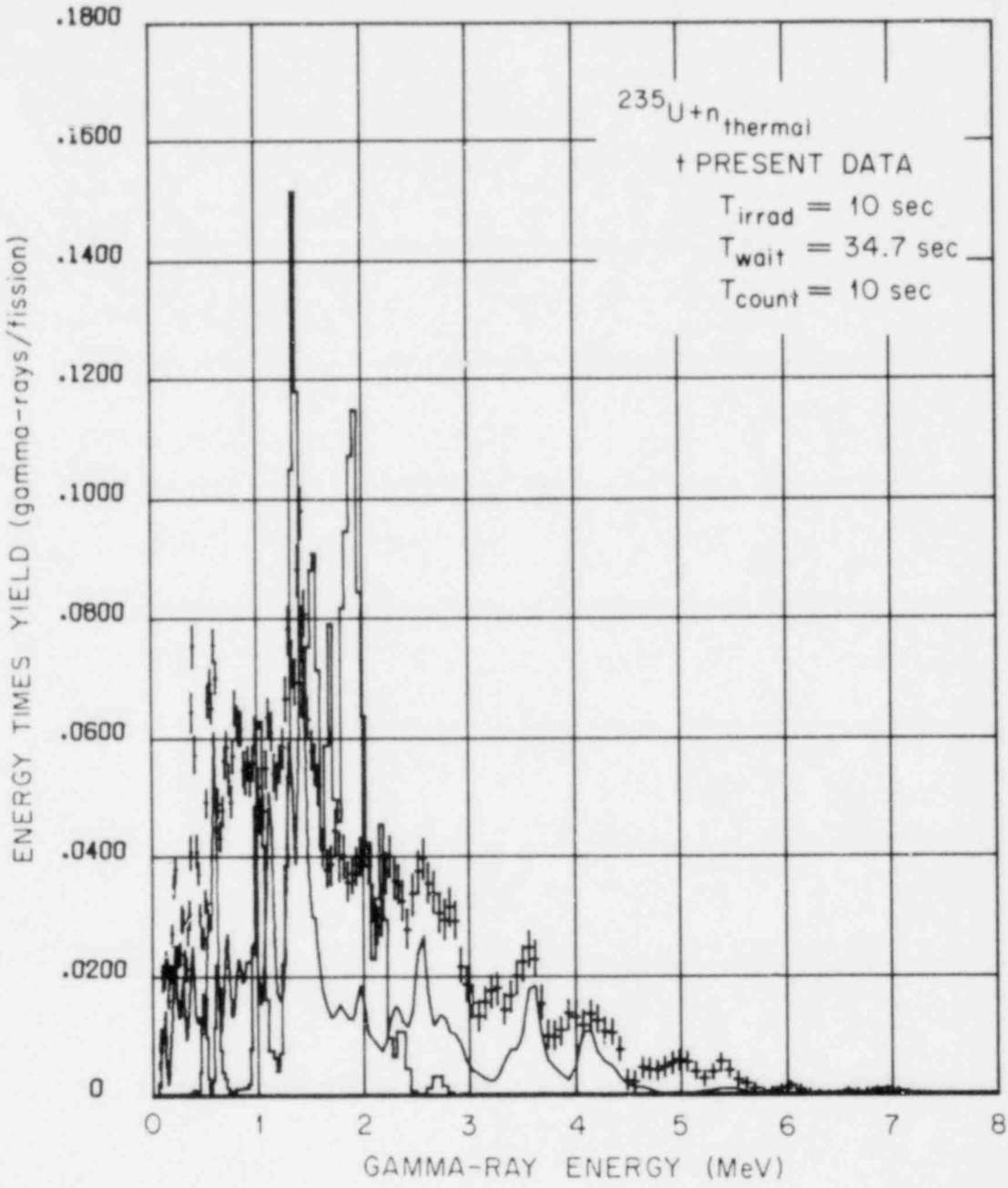

Fig. 67. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≳600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

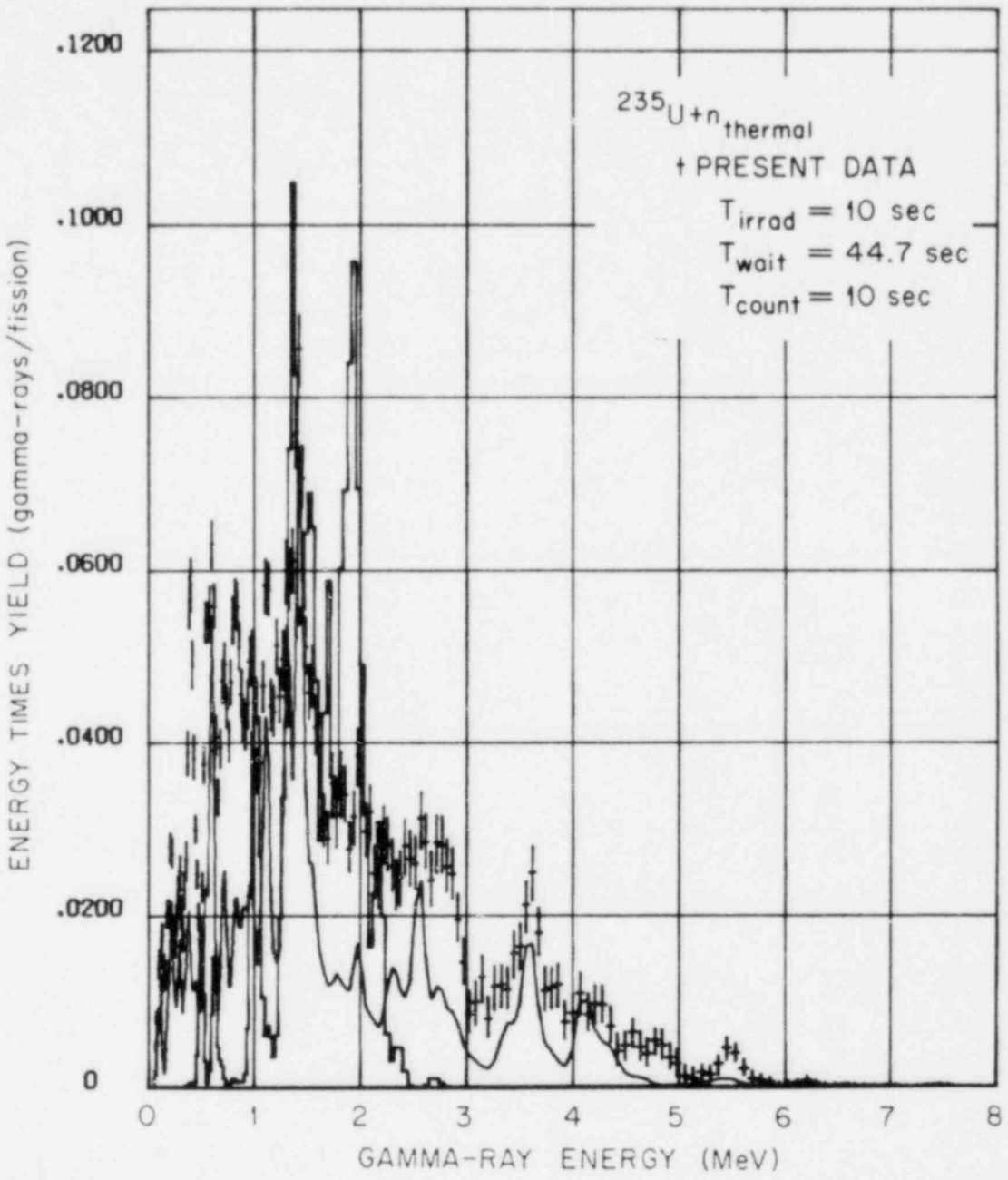

Fig. 68. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

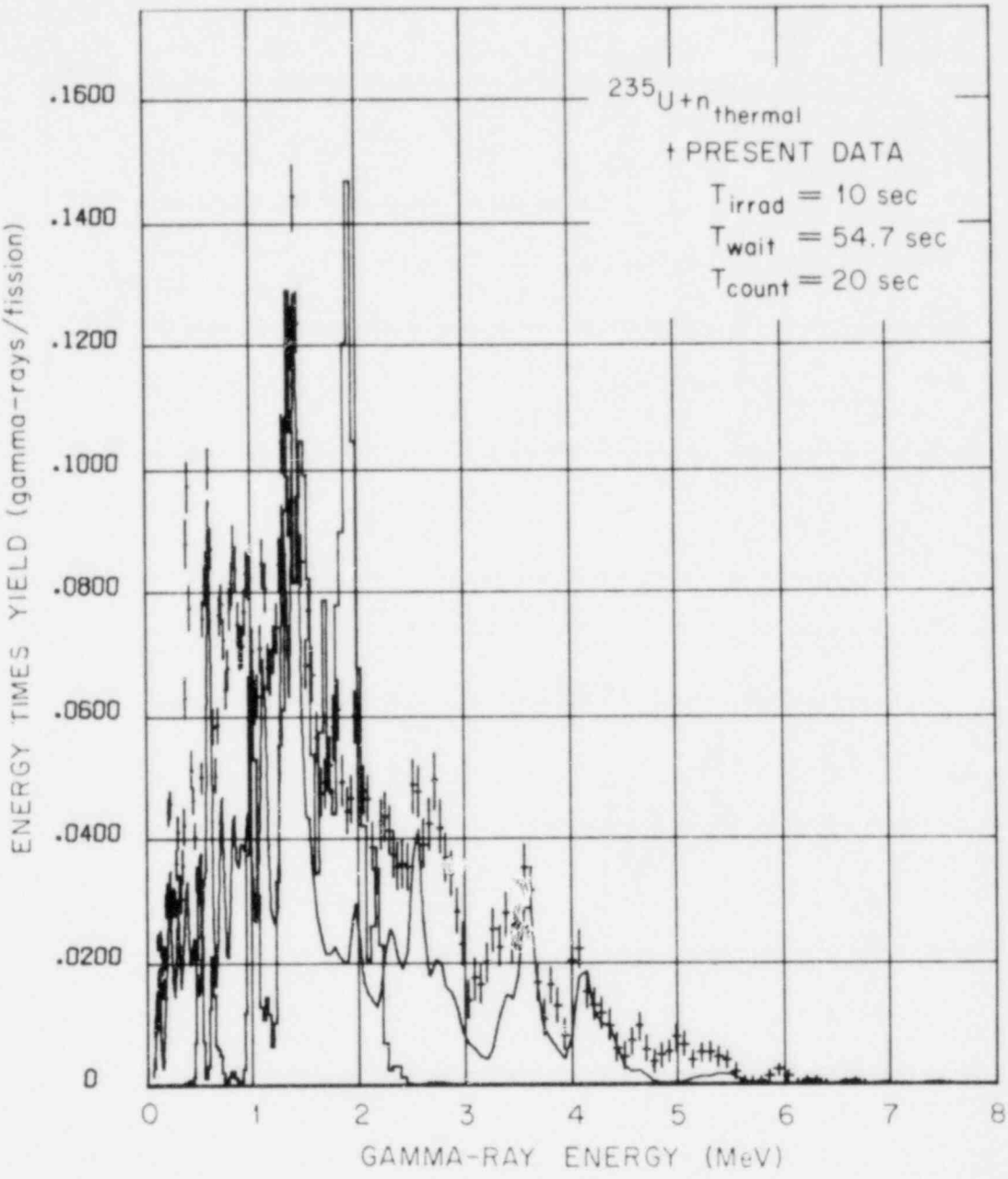

Fig. 69. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the ≈600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

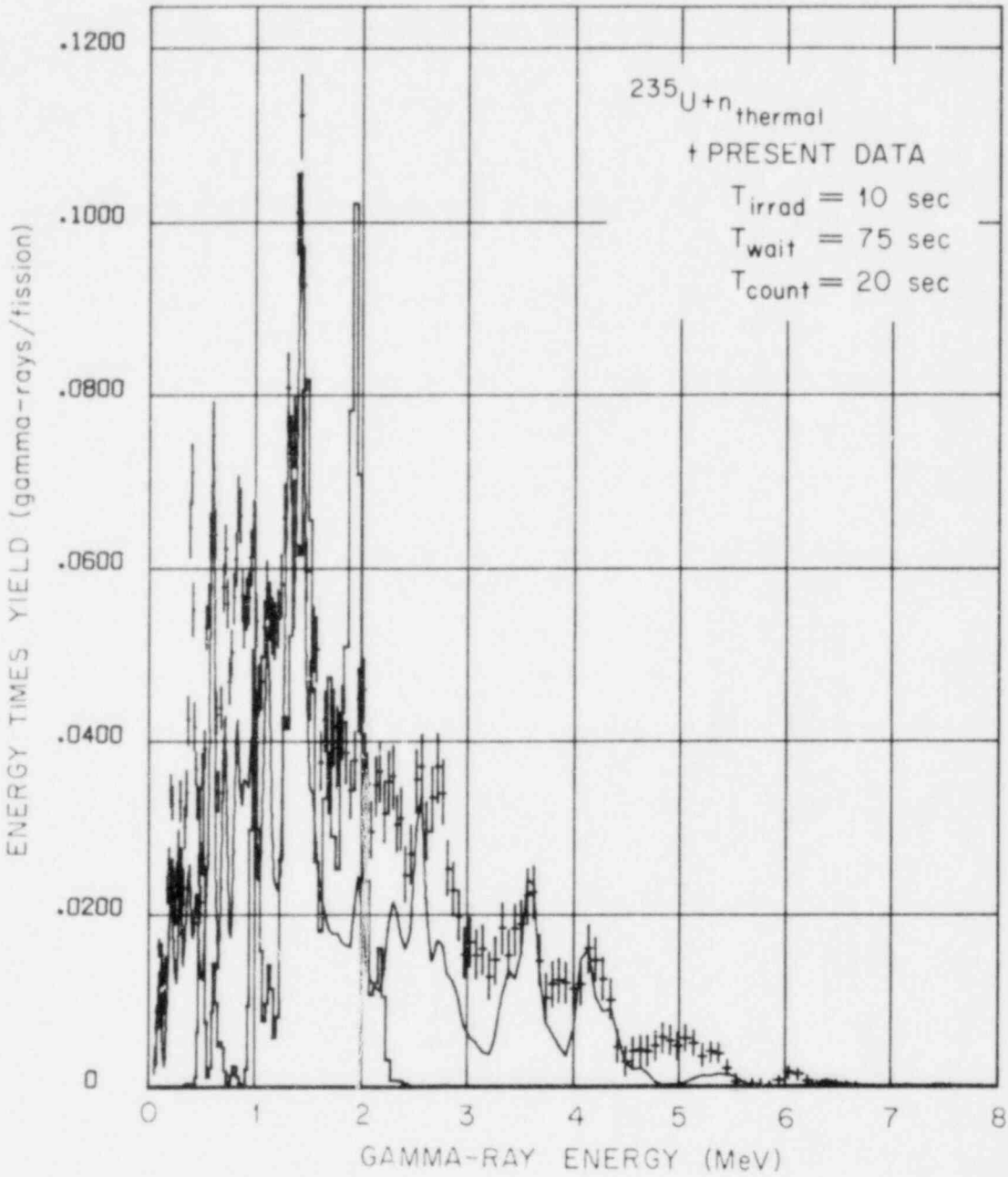

Fig. 70. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 3600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

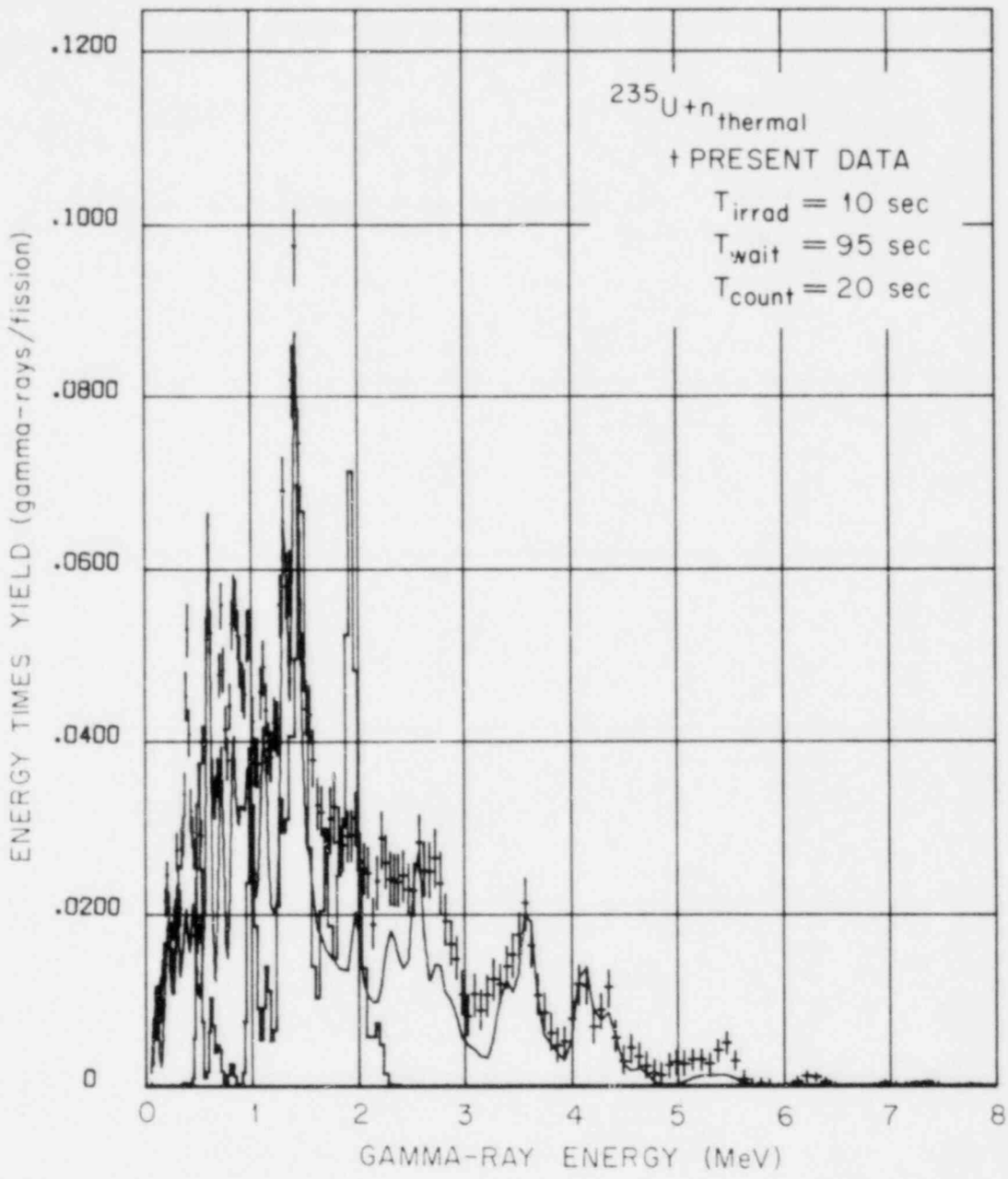

Fig. 71. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as Energy Times Yield, that is $E_\gamma \times N(E_\gamma)$, vs Gamma-Ray Energy. Summation calculation using the ENDF/B-IV data file have been split; the histogram indicates "average" contributions from the 2600 nuclides in the file having only an "average" gamma-ray energy, while the non-histogram curve indicates contributions from the 180 nuclides having complete decay data in the files. The irradiation, waiting and counting time intervals are given in the legend.

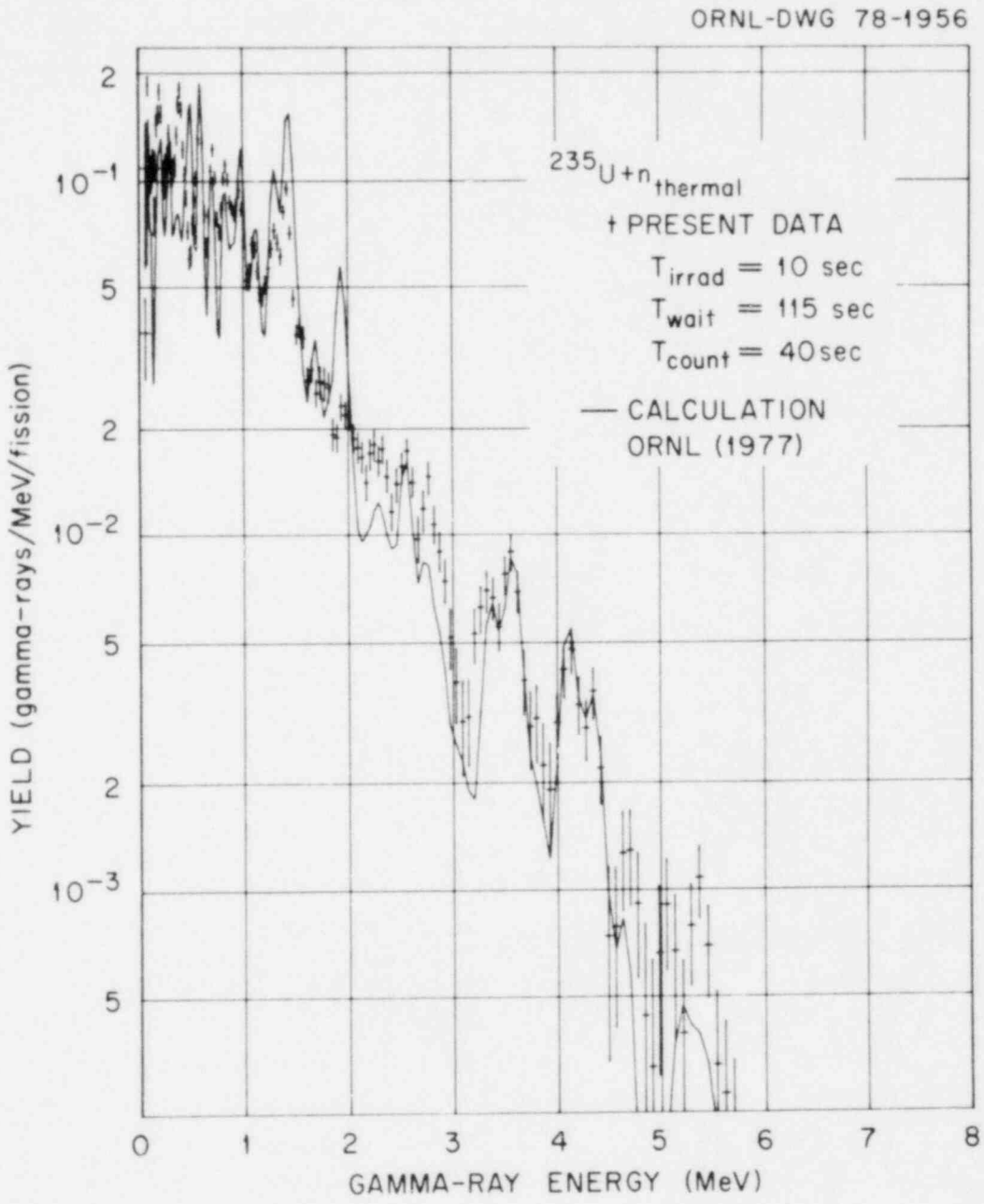

Fig. 72. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.9 MeV and portions of those at 1.3 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

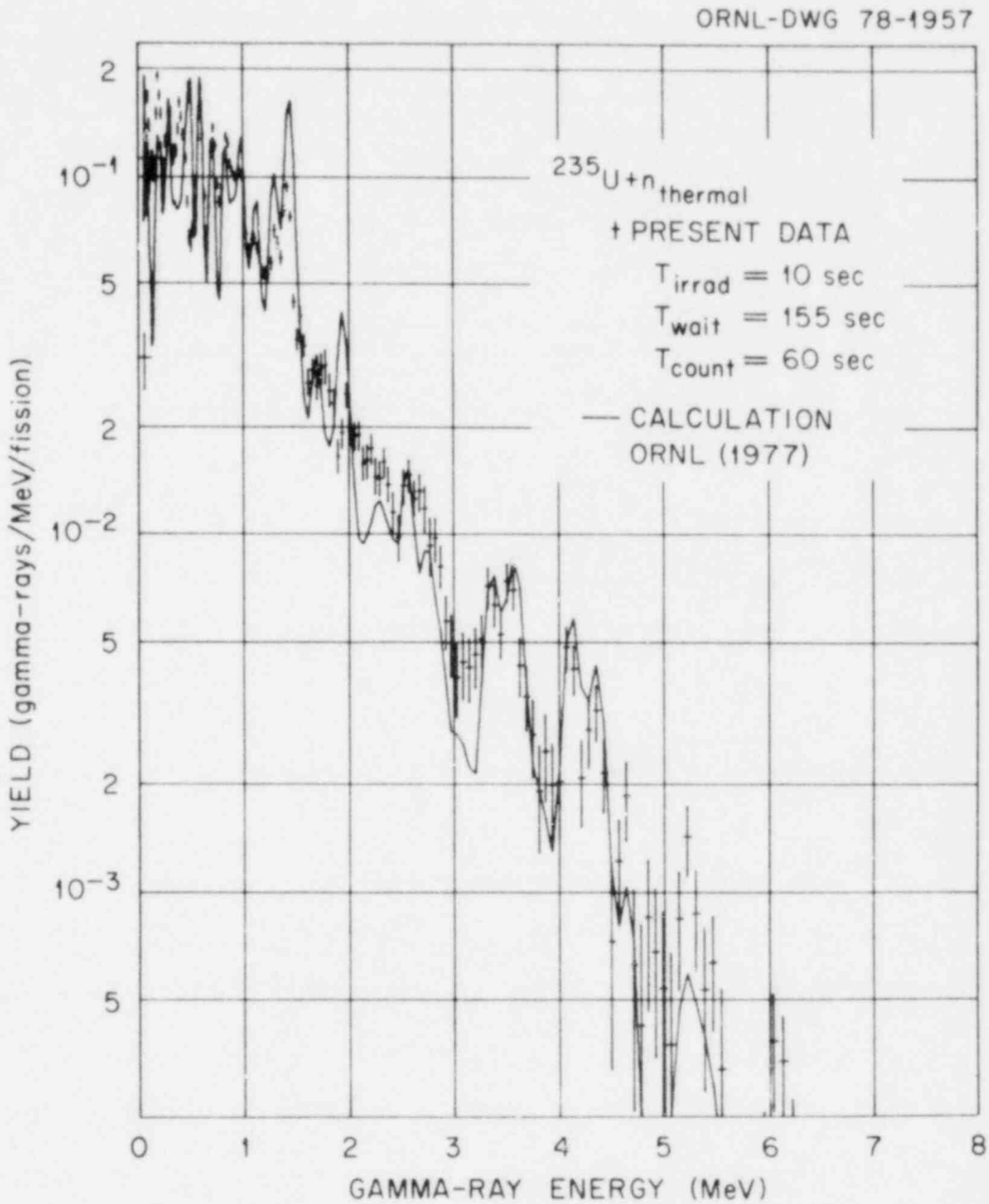

Fig. 73. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.9 MeV and portions of those at 1.3 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

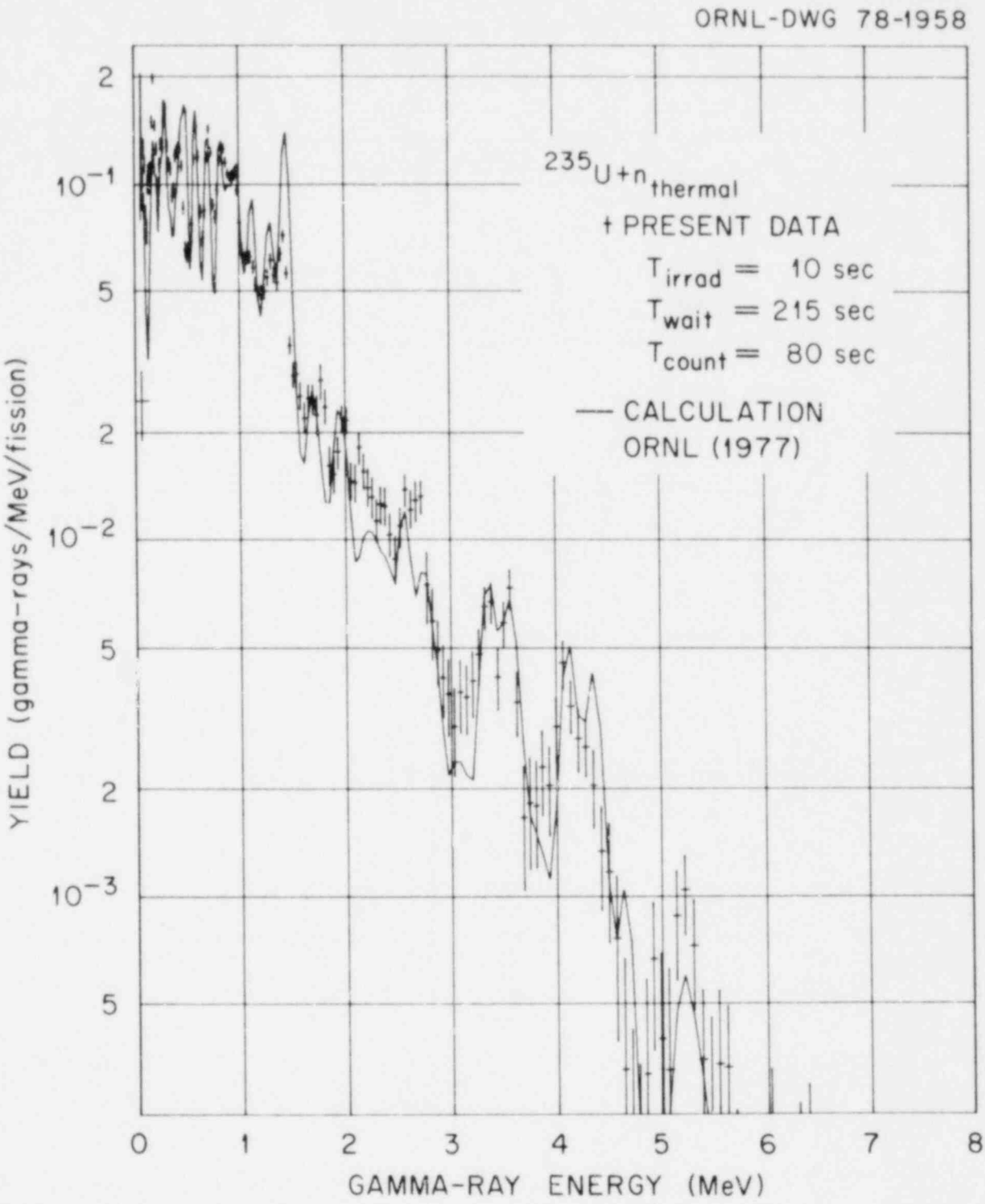

Fig. 74. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.9 MeV and a portion of that at 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

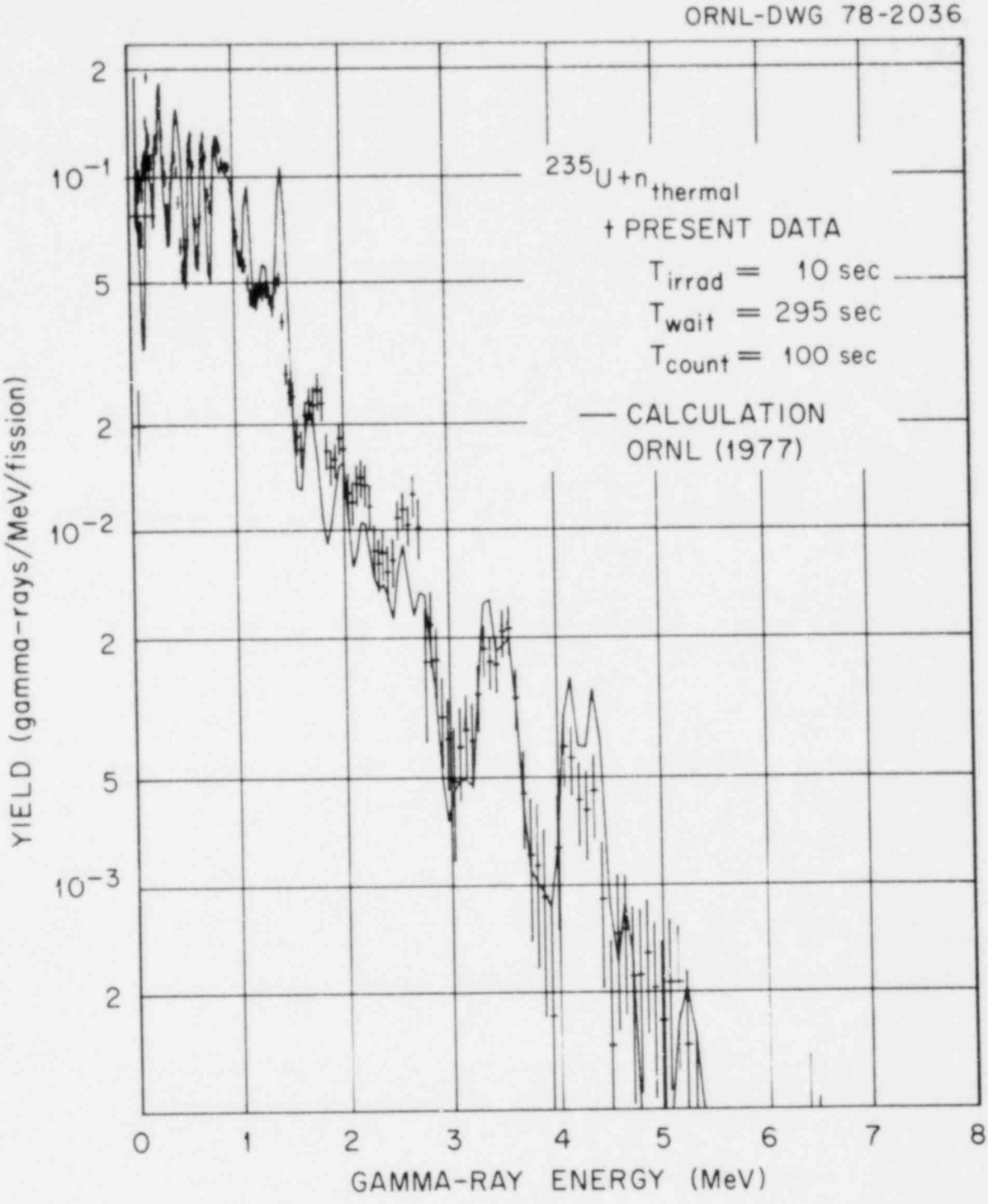

Fig. 75. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peaks" at 1.1 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

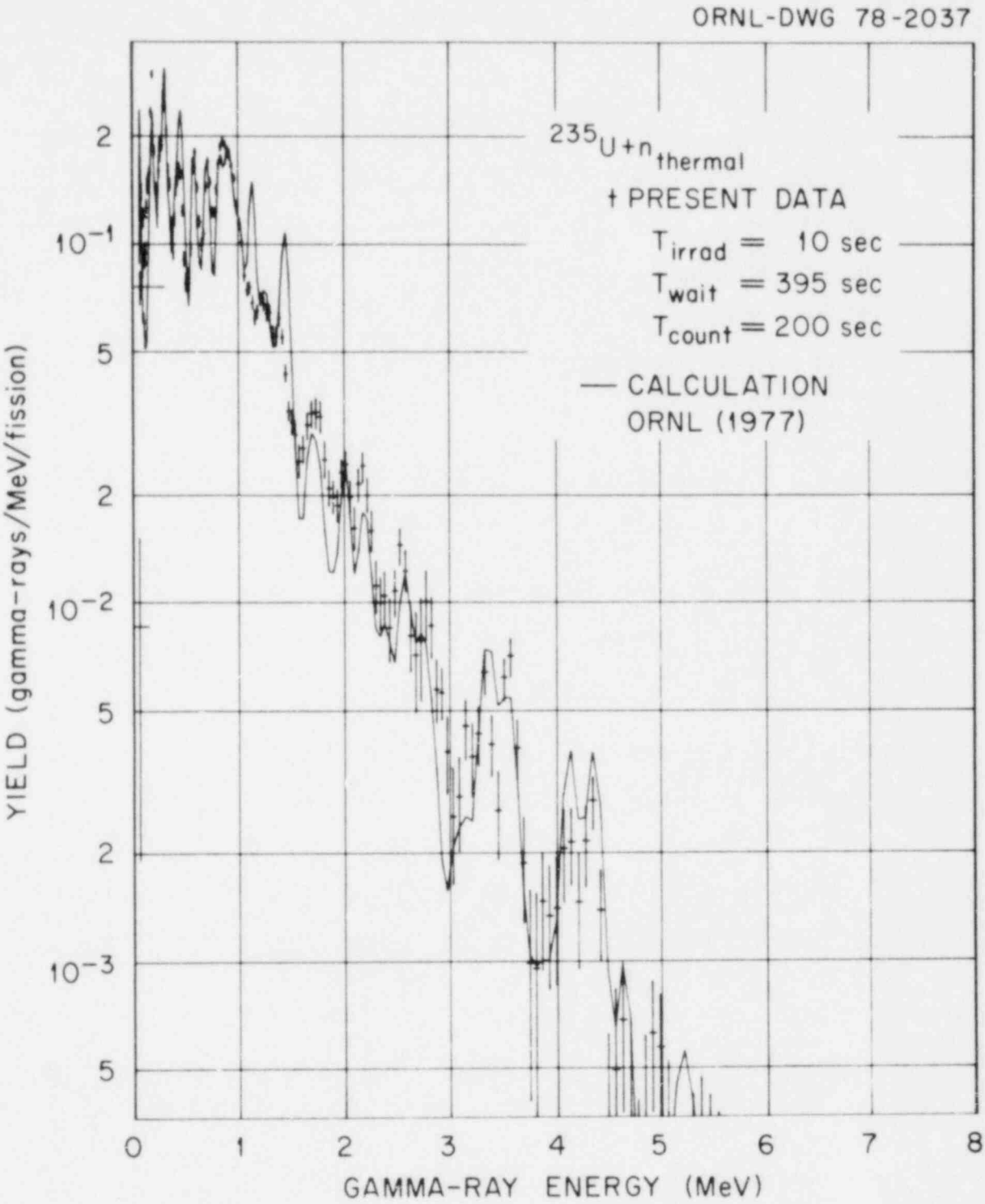

Fig. 76. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peaks" at 1.1 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

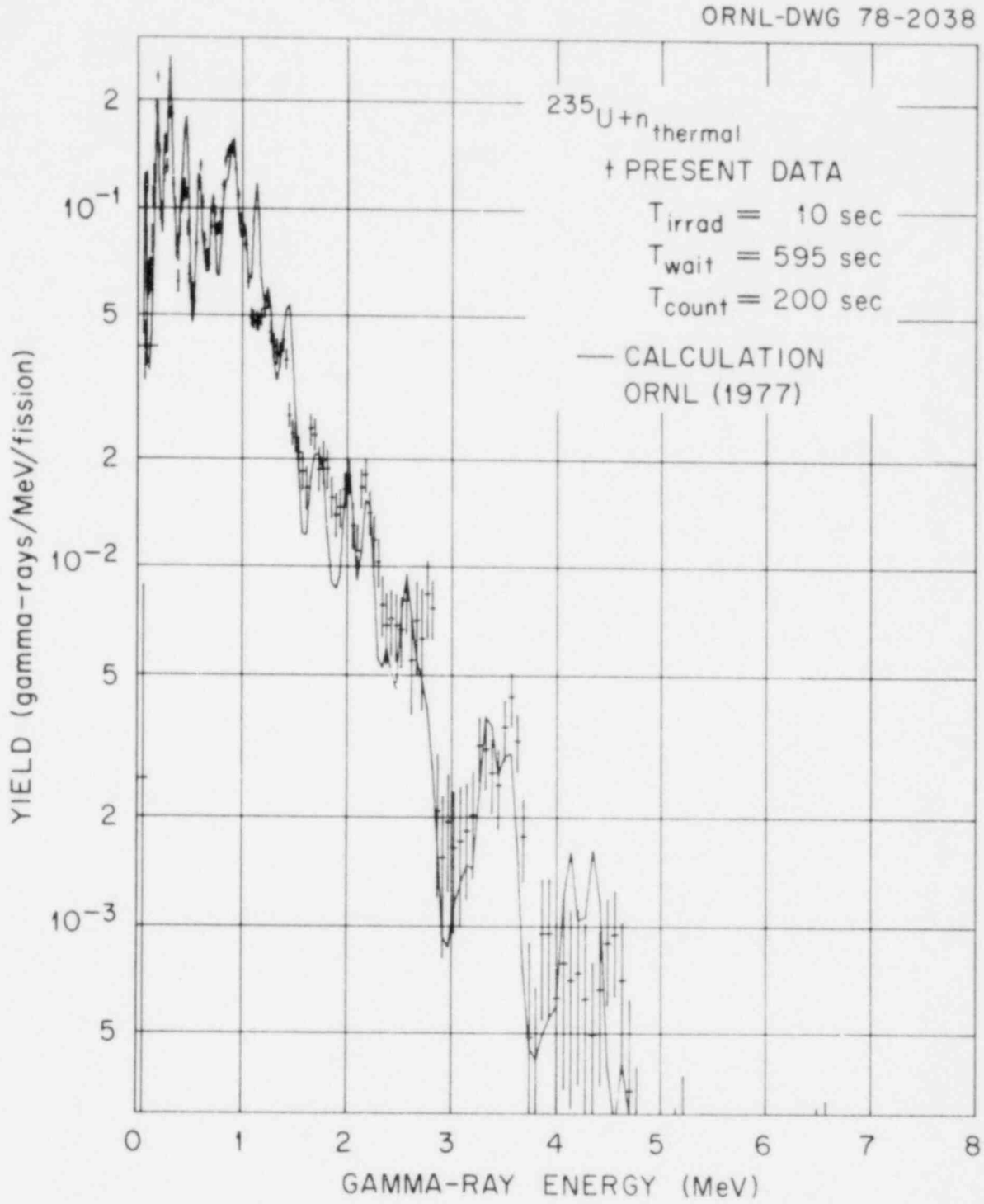

Fig. 77. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peaks" at 1.1 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

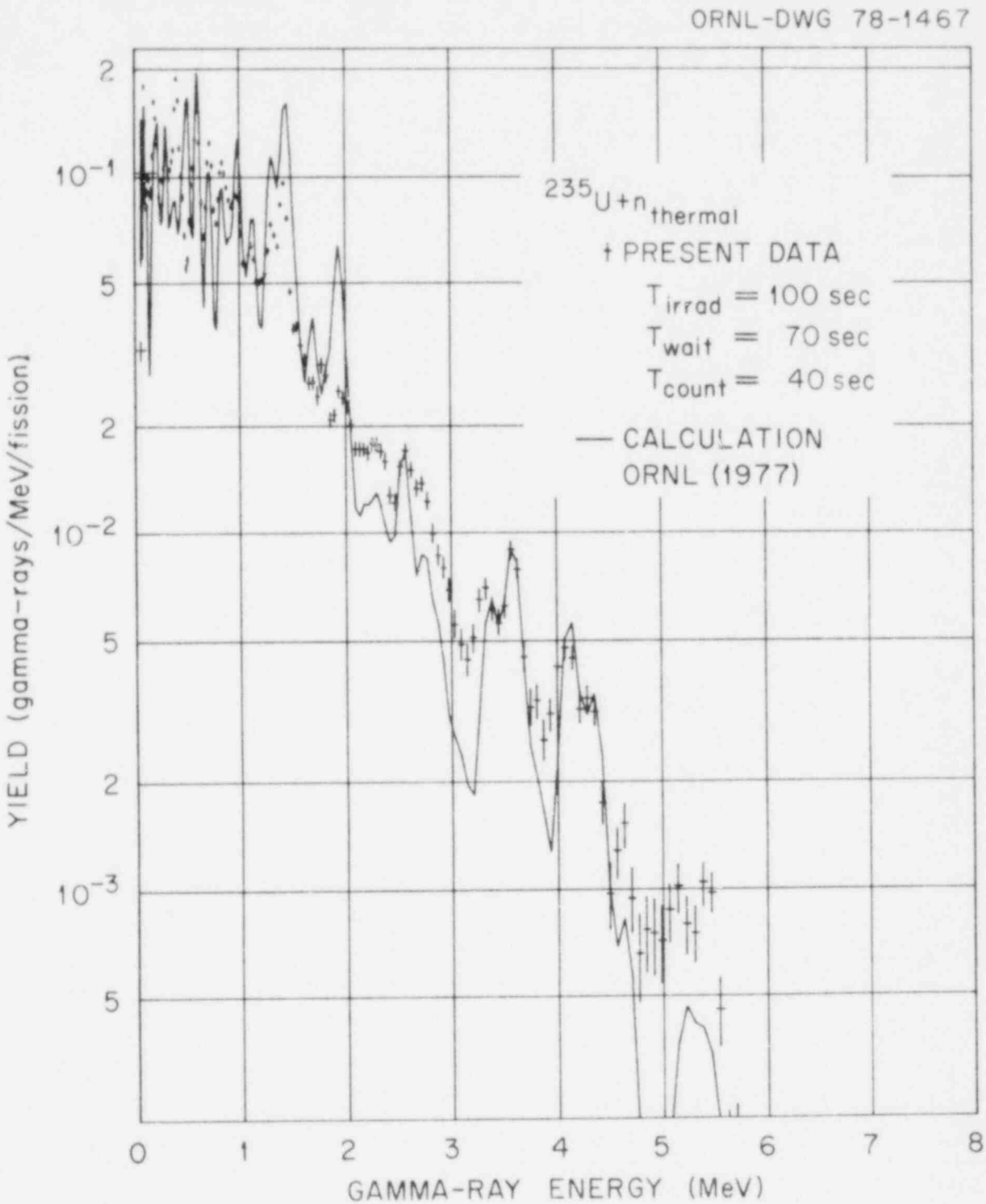

Fig. 78. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ~600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.9 MeV and portions of those at 1.3 and 1.4 MeV are due to contributions from the ~600 "average" nuclides.

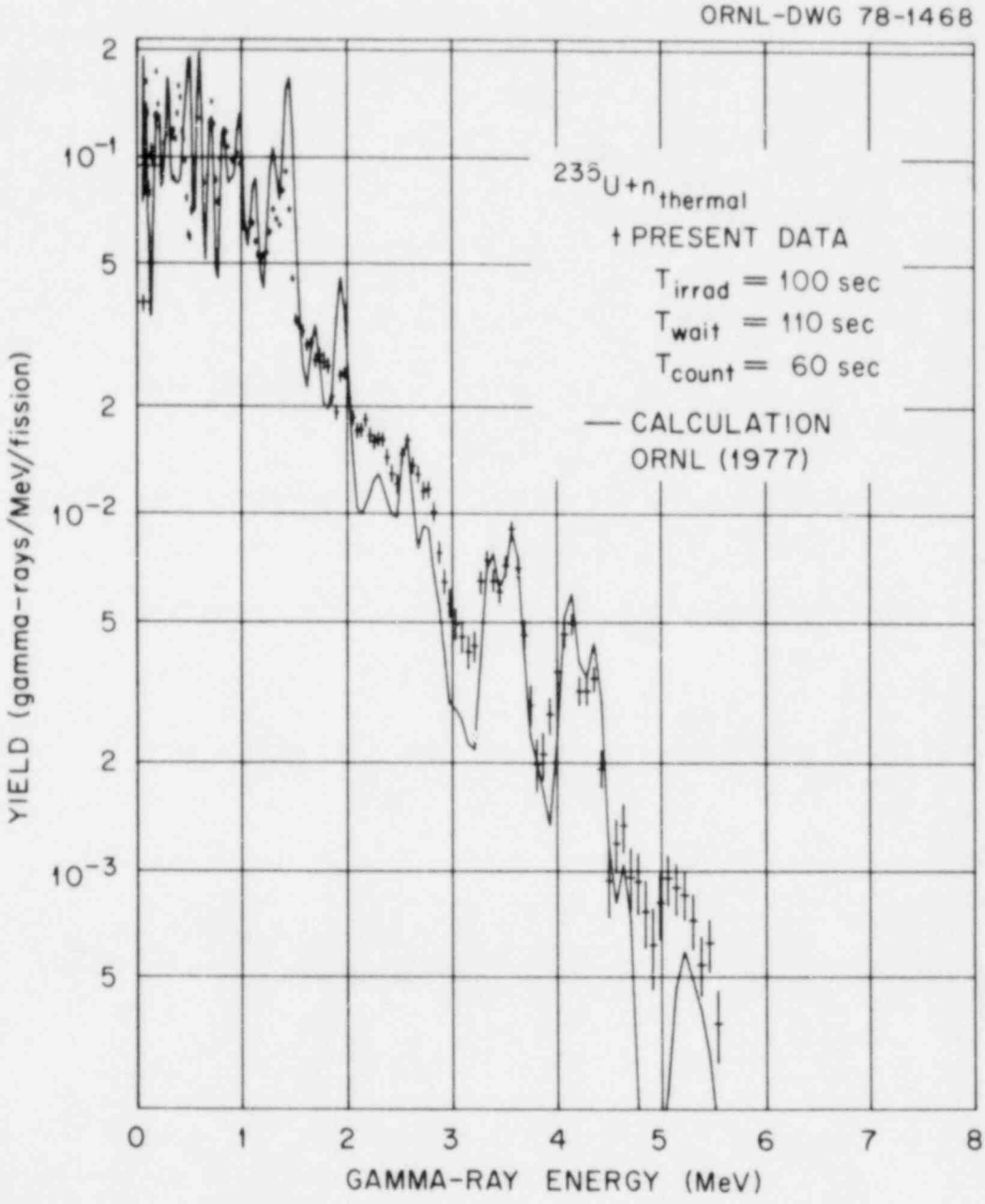

Fig. 79. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.9 MeV and portions of those at 1.3 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

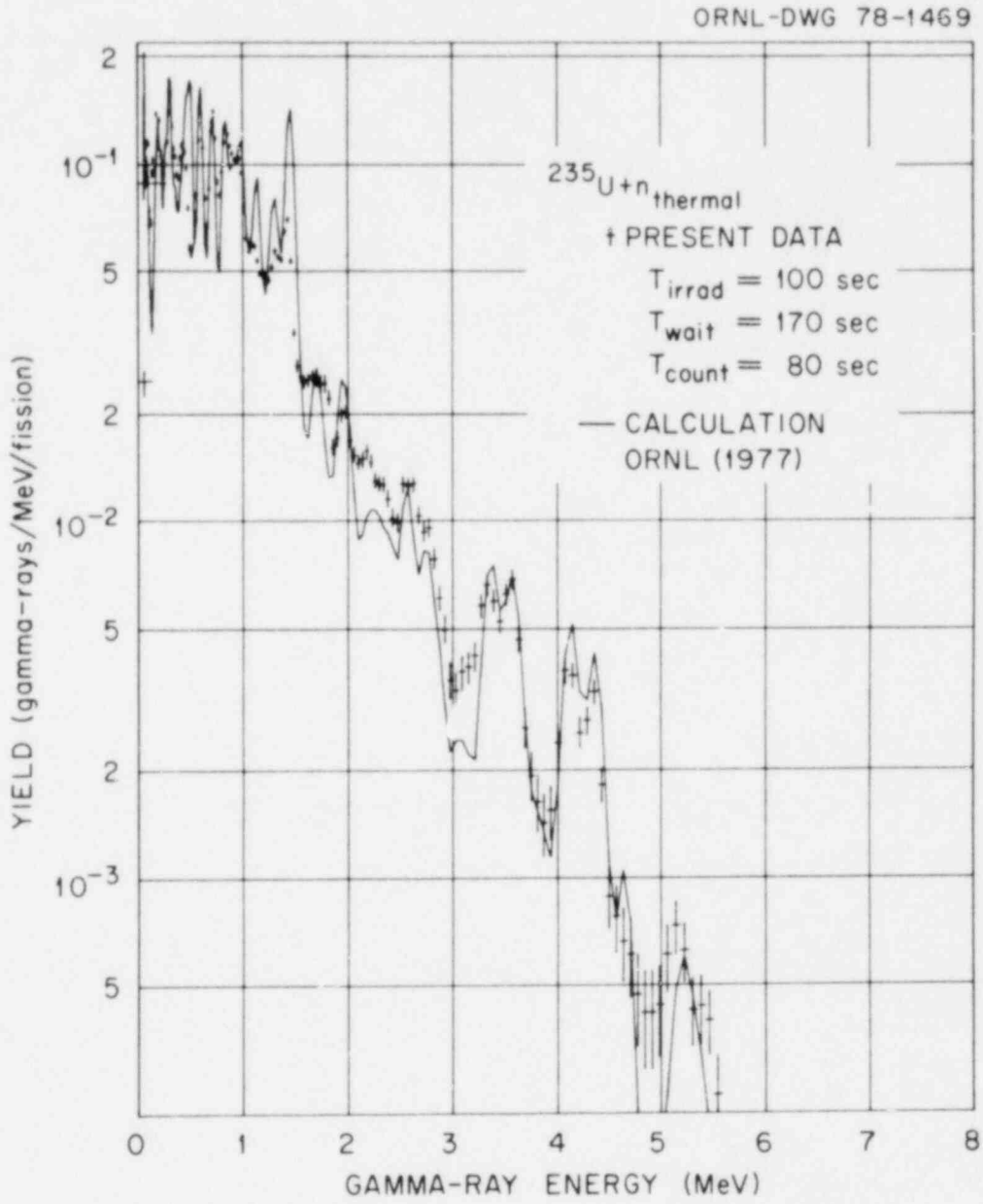

Fig. 80. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.9 MeV and a portion of that at 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

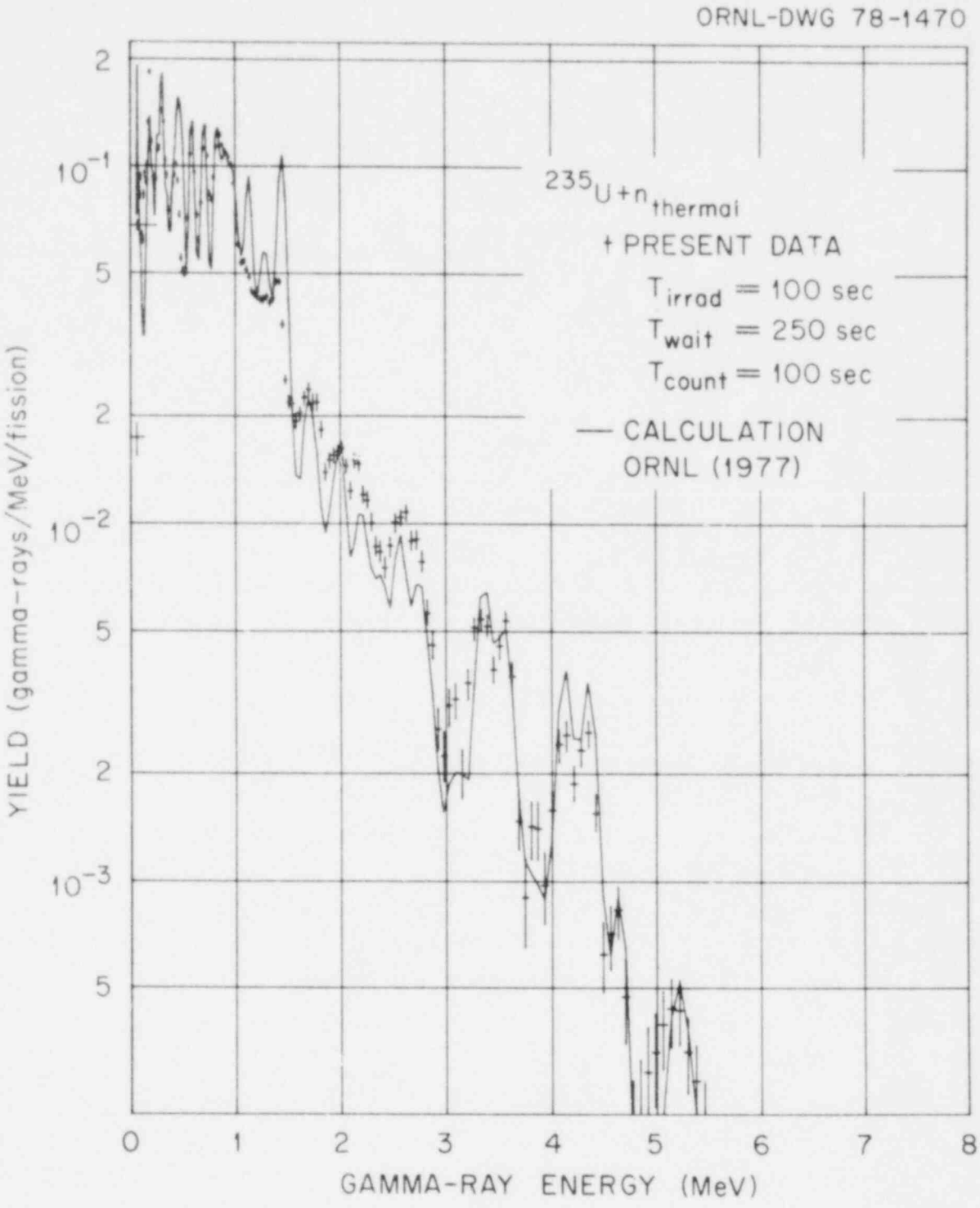

Fig. 81. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peaks" at 1.1 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

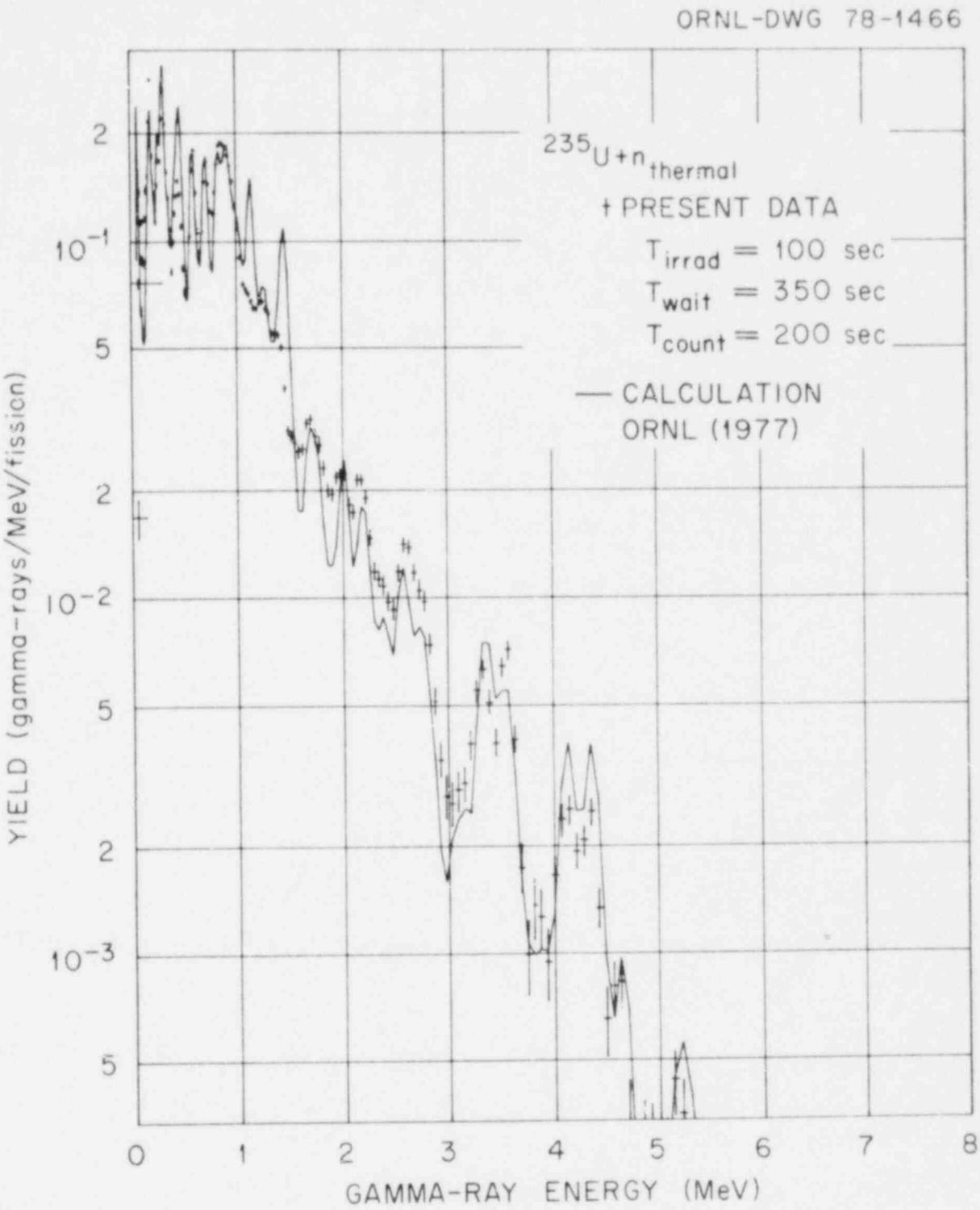

Fig. 82. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peaks" at 1.1 and 1.4 MeV are due to contributions from the ≈600 "average" nuclides.

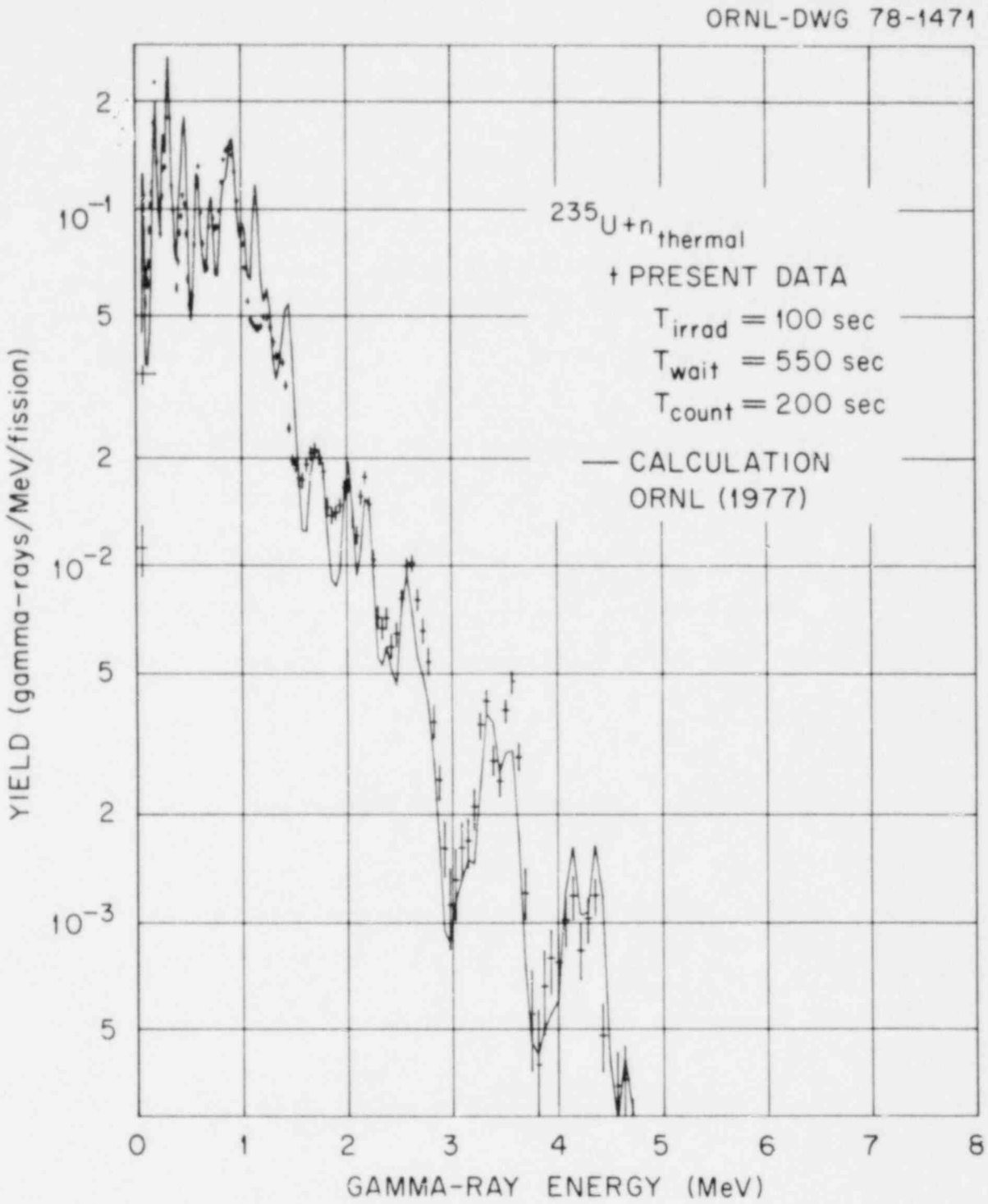

Fig. 83. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ~600 nuclides having only an "average" gamma-ray energy in the file. The "peaks" at 1.1 and 1.4 MeV are due to contributions from the ~600 "average" nuclides.

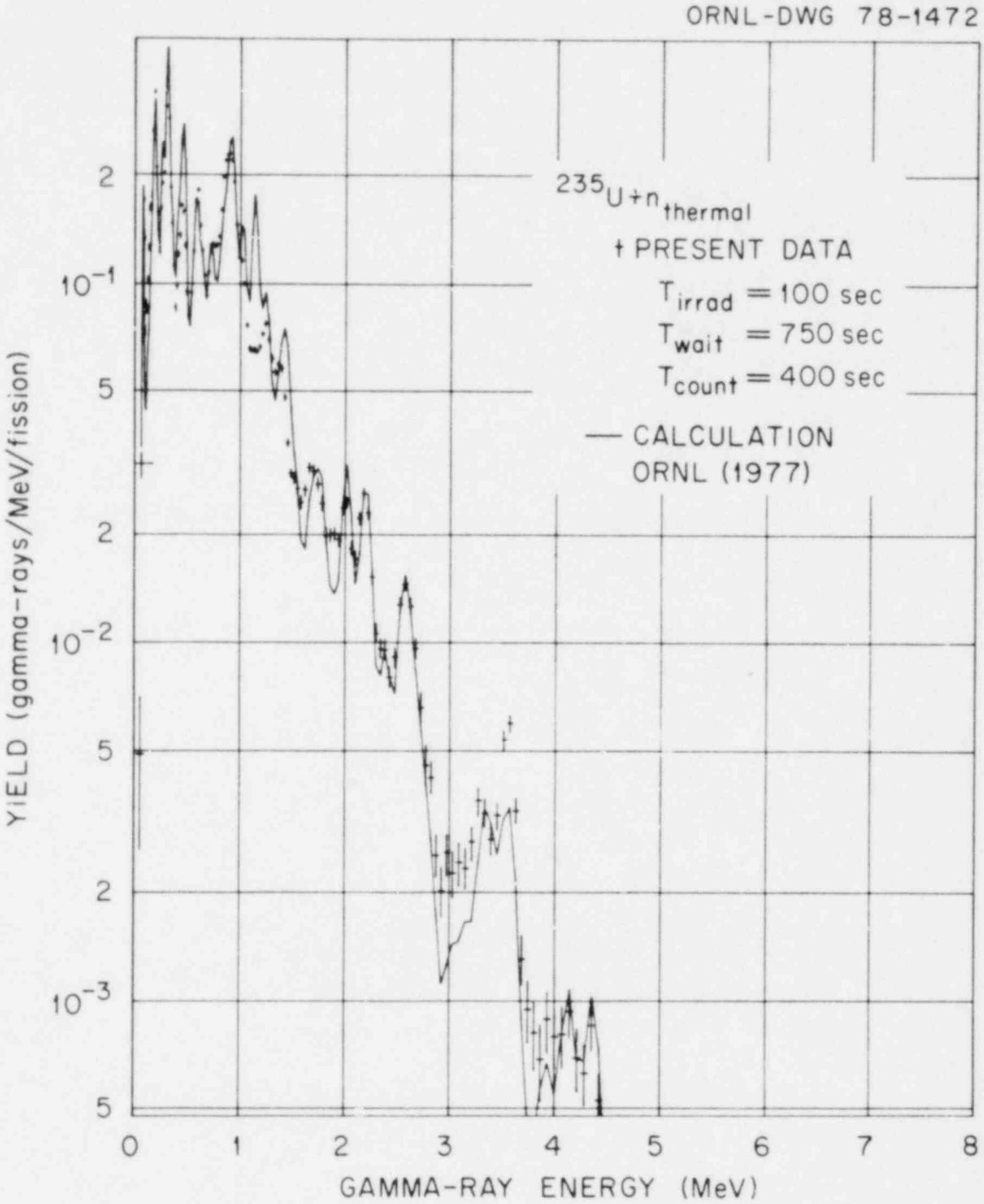

Fig. 84. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.1 MeV is due to a contribution from the ≈600 "average" nuclides.

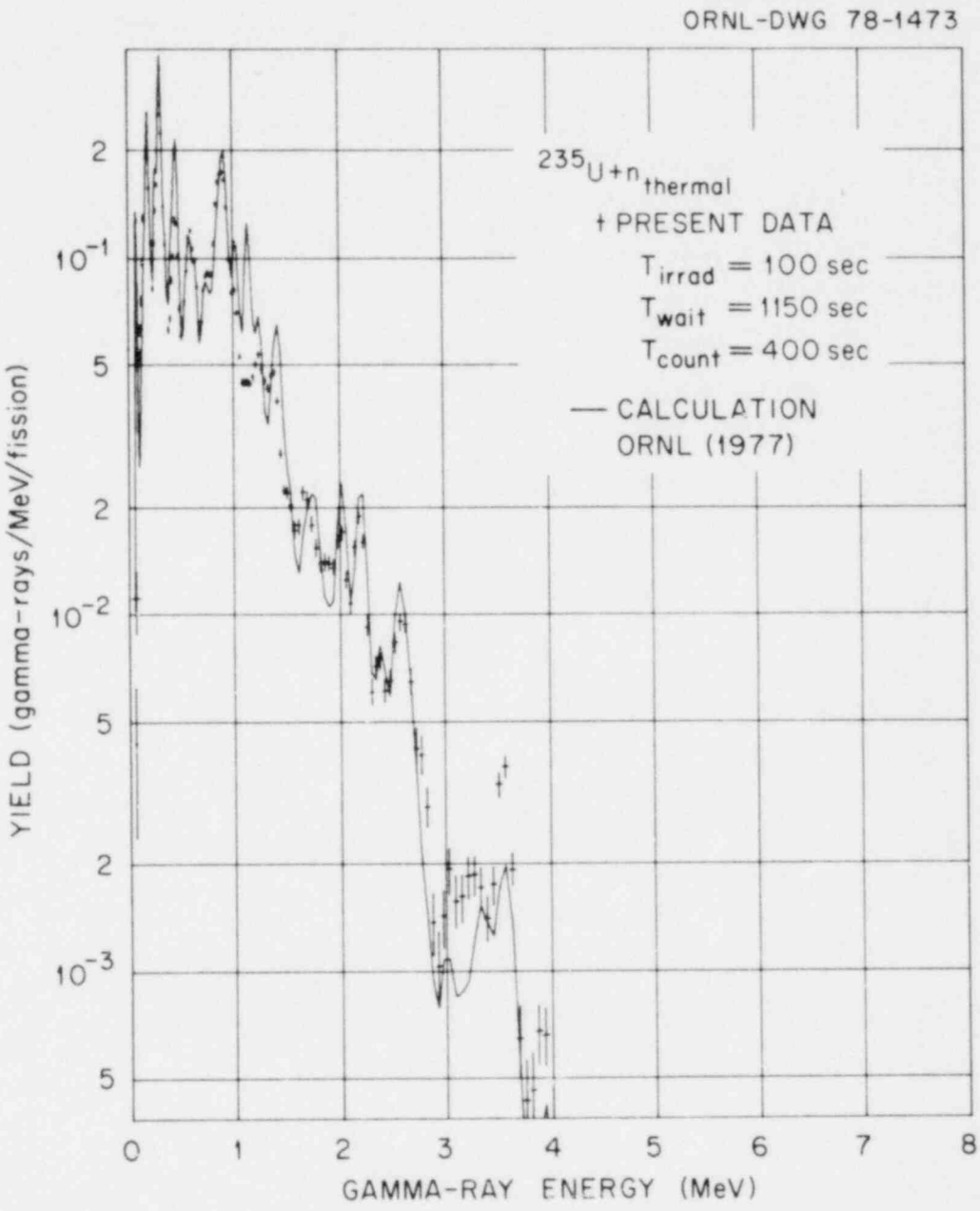

Fig. 85. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ~600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.1 MeV is due to a contribution from the ~600 "average" nuclides.

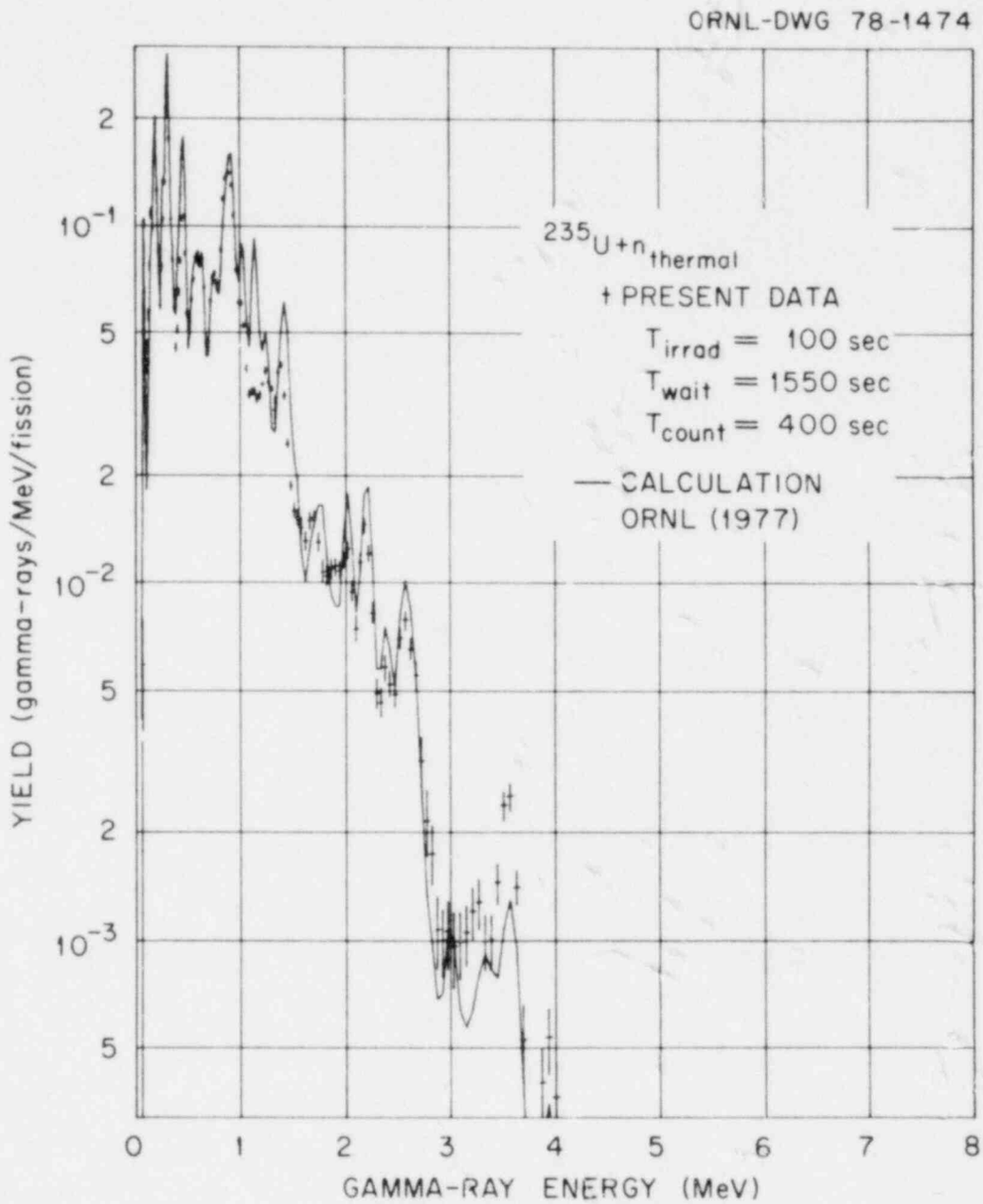

Fig. 86 Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ~600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.1 MeV is due to a contribution from the ~600 "average" nuclides.

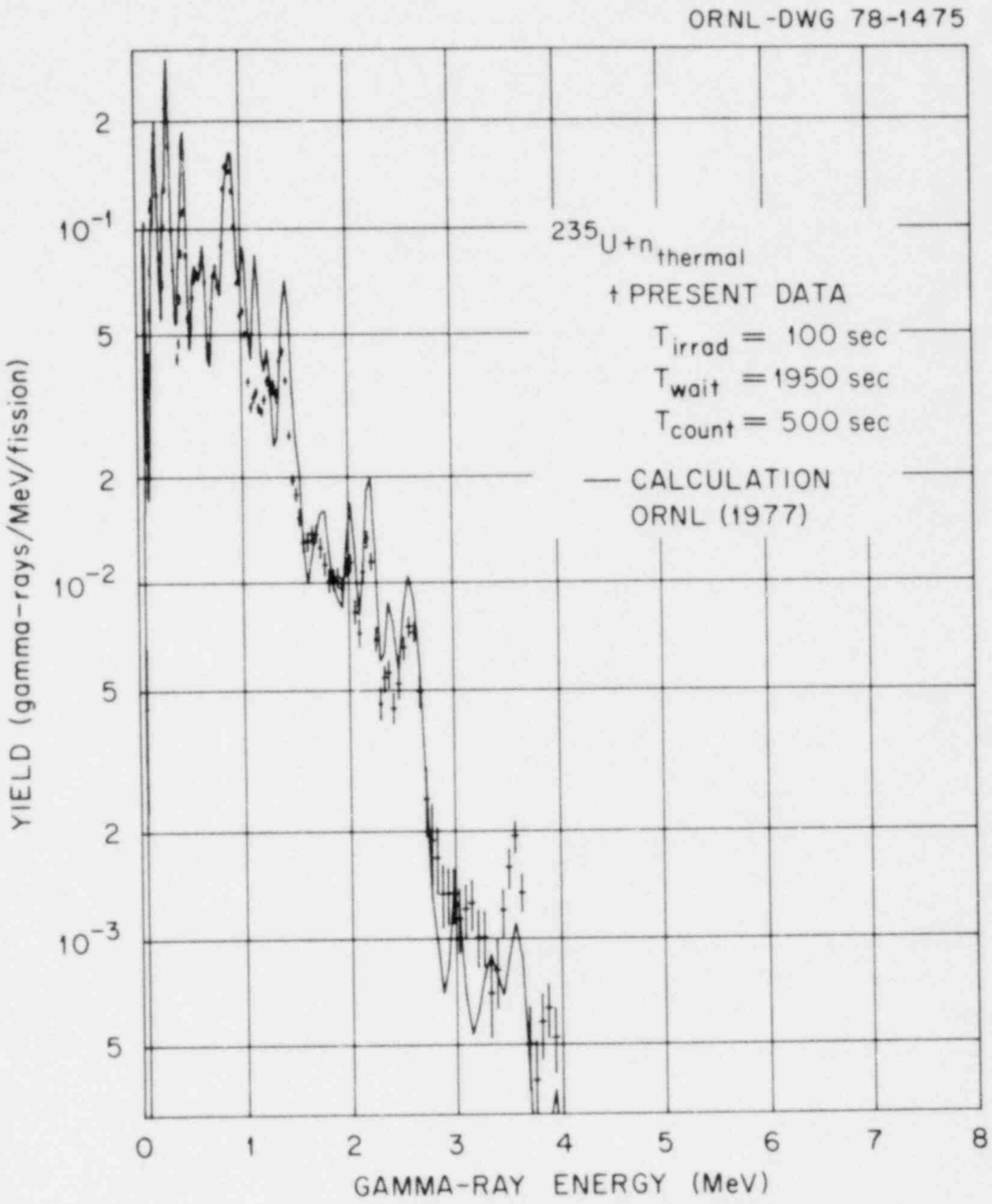

Fig. 87. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ~600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.1 MeV is due to a contribution from the ~600 "average" nuclides.

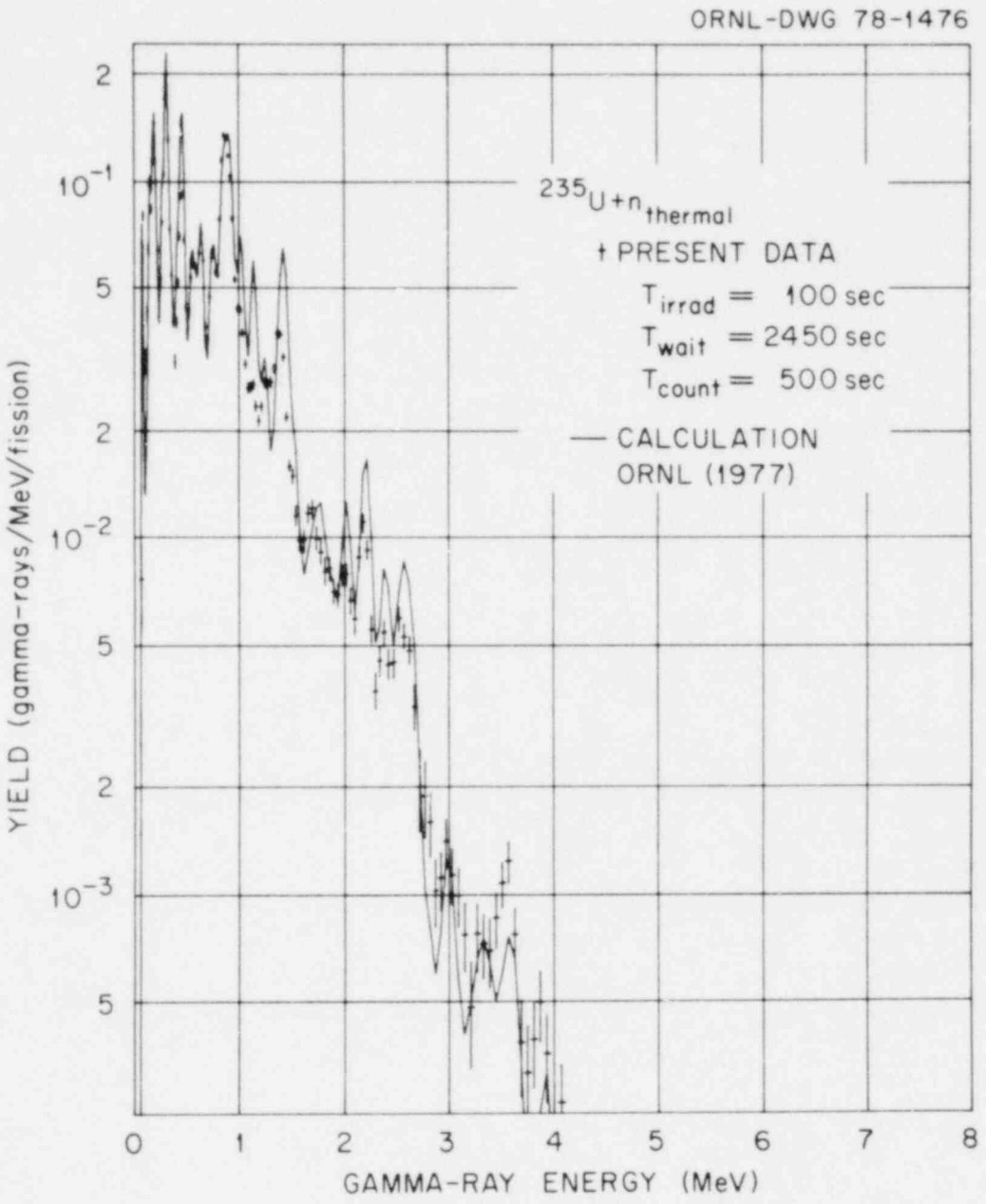

Fig. 88. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ∿600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.1 MeV is due to a contribution from the ∿600 "average" nuclides.

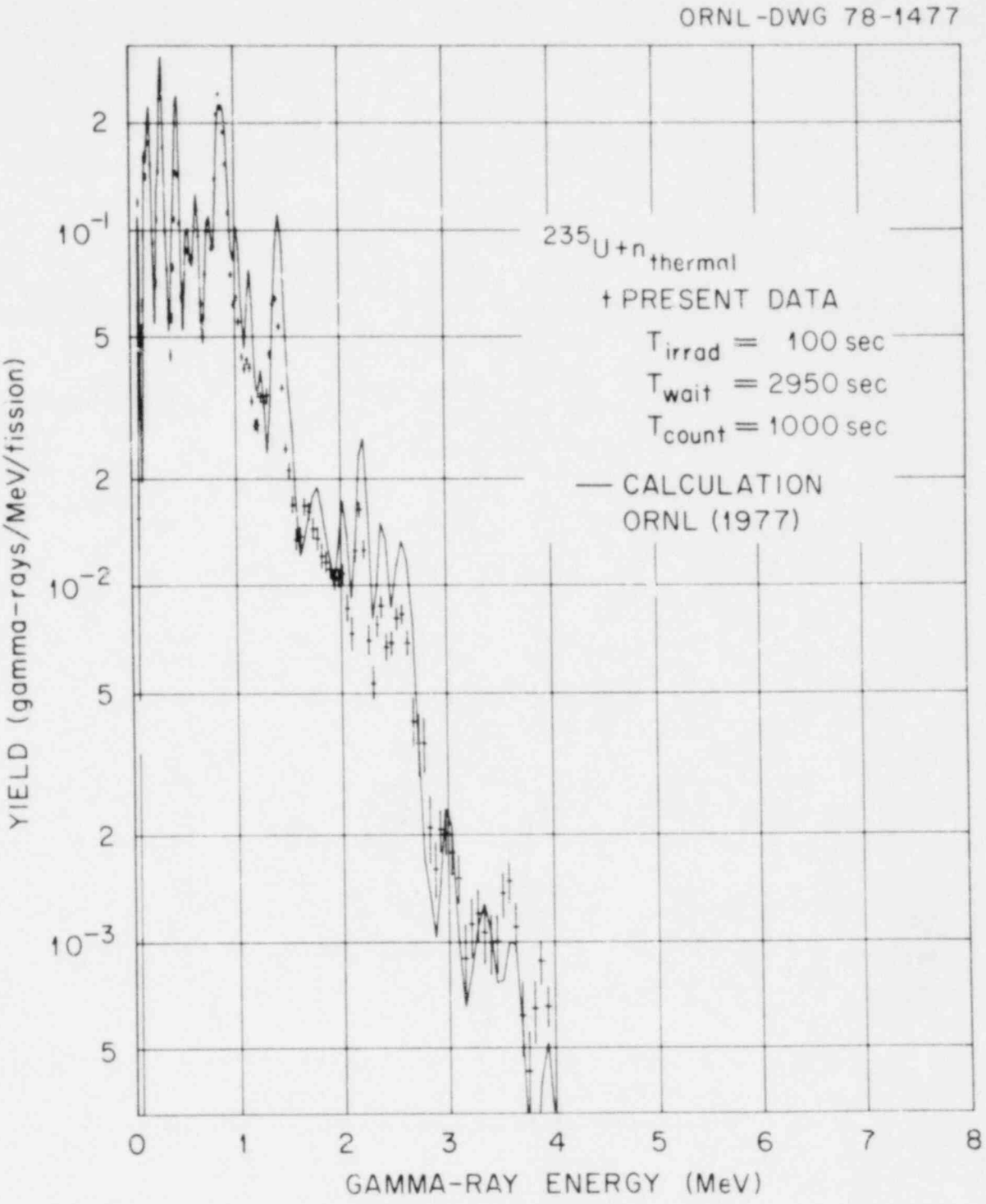

Fig. 89. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the ≈600 nuclides having only an "average" gamma-ray energy in the file. The "peak" at 1.1 MeV is due to a contribution from the ≈600 "average" nuclides. There is an estimated loss of 1.6% of $N(E\gamma)$ due to escape of fission gases not included in these data (see Ref. 1).

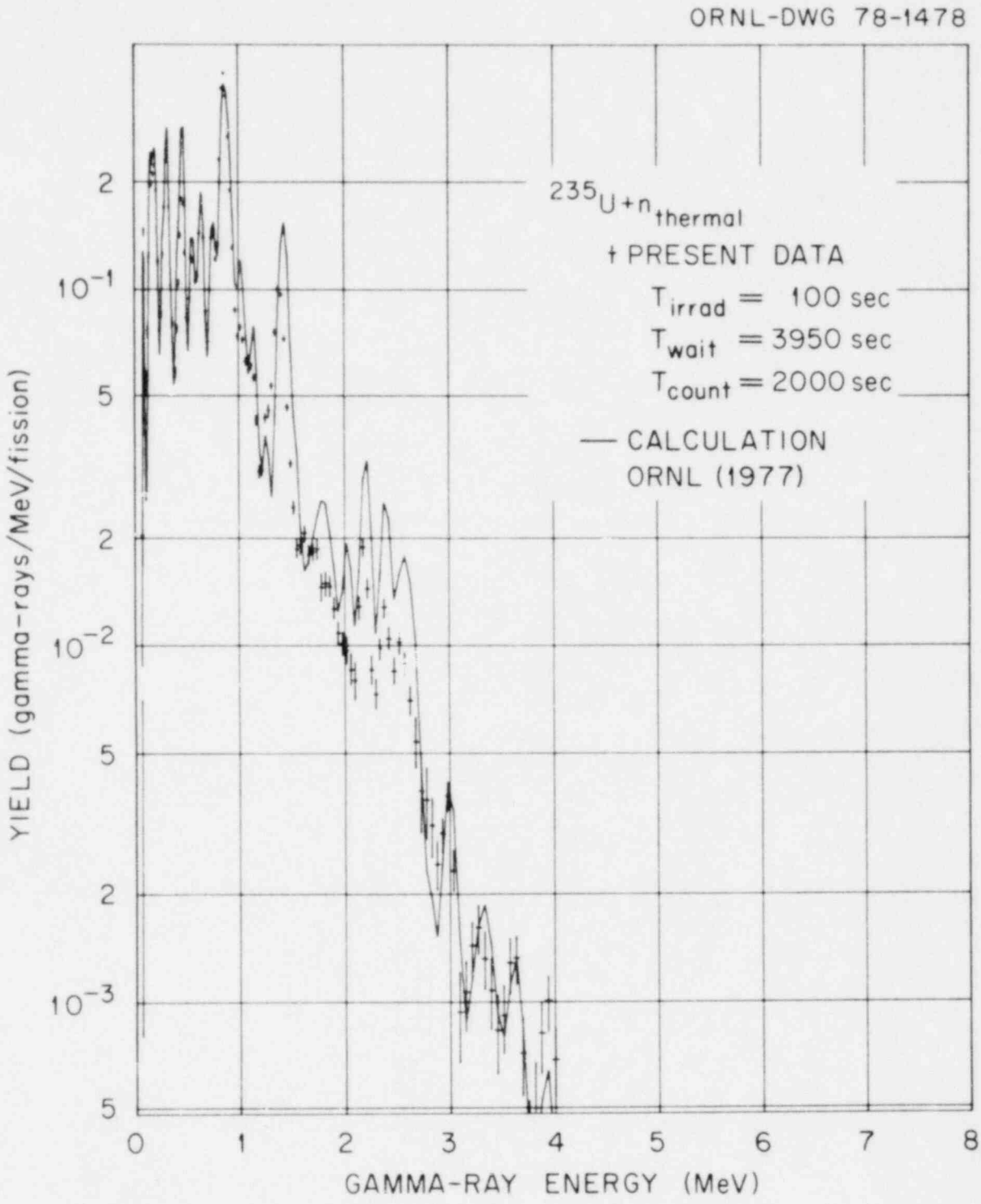

Fig. 90. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the 8600 nuclides having only an "average" gamma-ray energy in the file. There is an estimated loss of 4.4% of $N(E_\gamma)$ due to escape of fission gases (see Ref. 1).

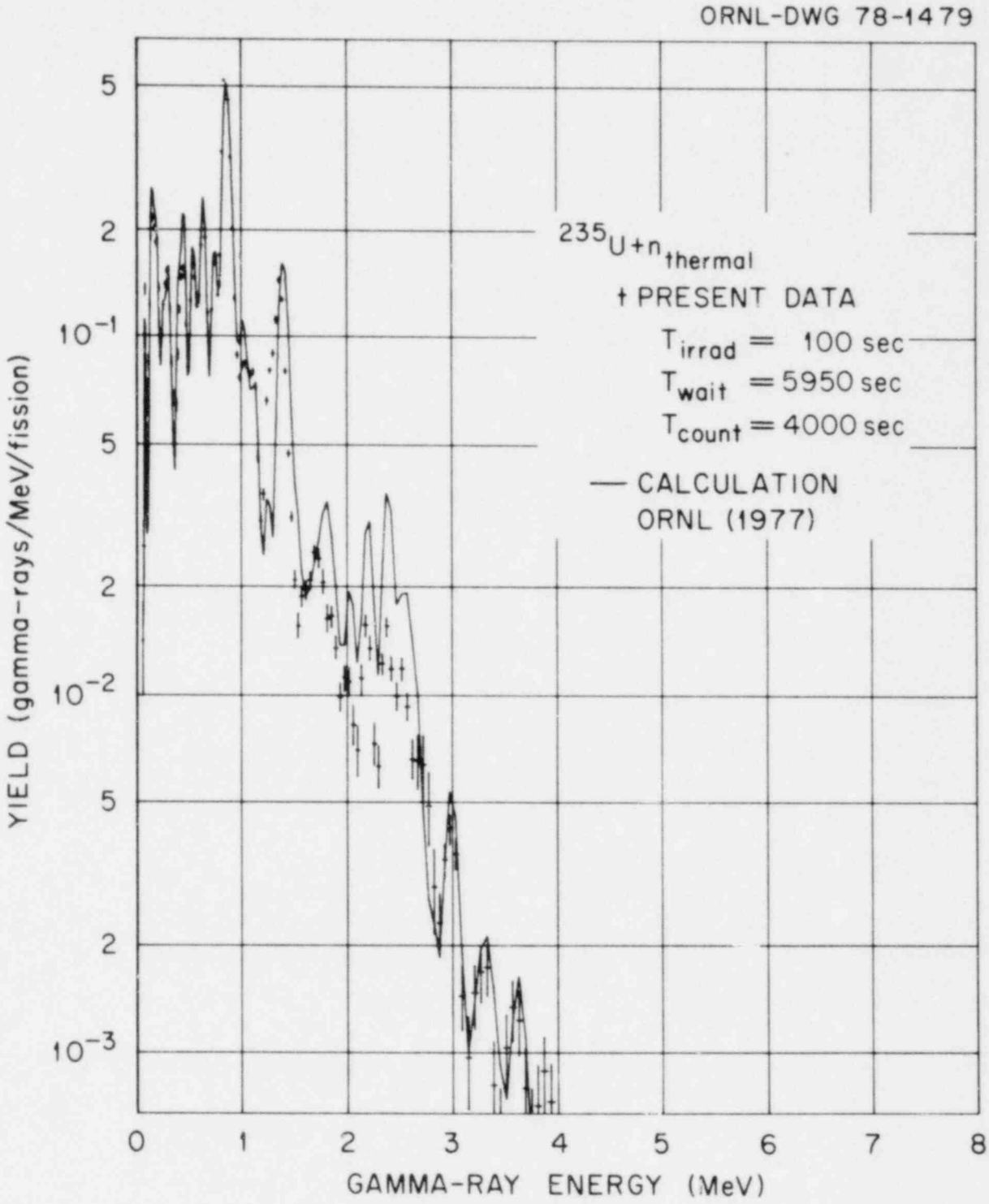

Fig. 91. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the 600 nuclides having only an "average" gamma-ray energy in the file. There is an estimated loss of 7.2% of $N(E_\gamma)$ due to escape of fission gases (Ref. 1).

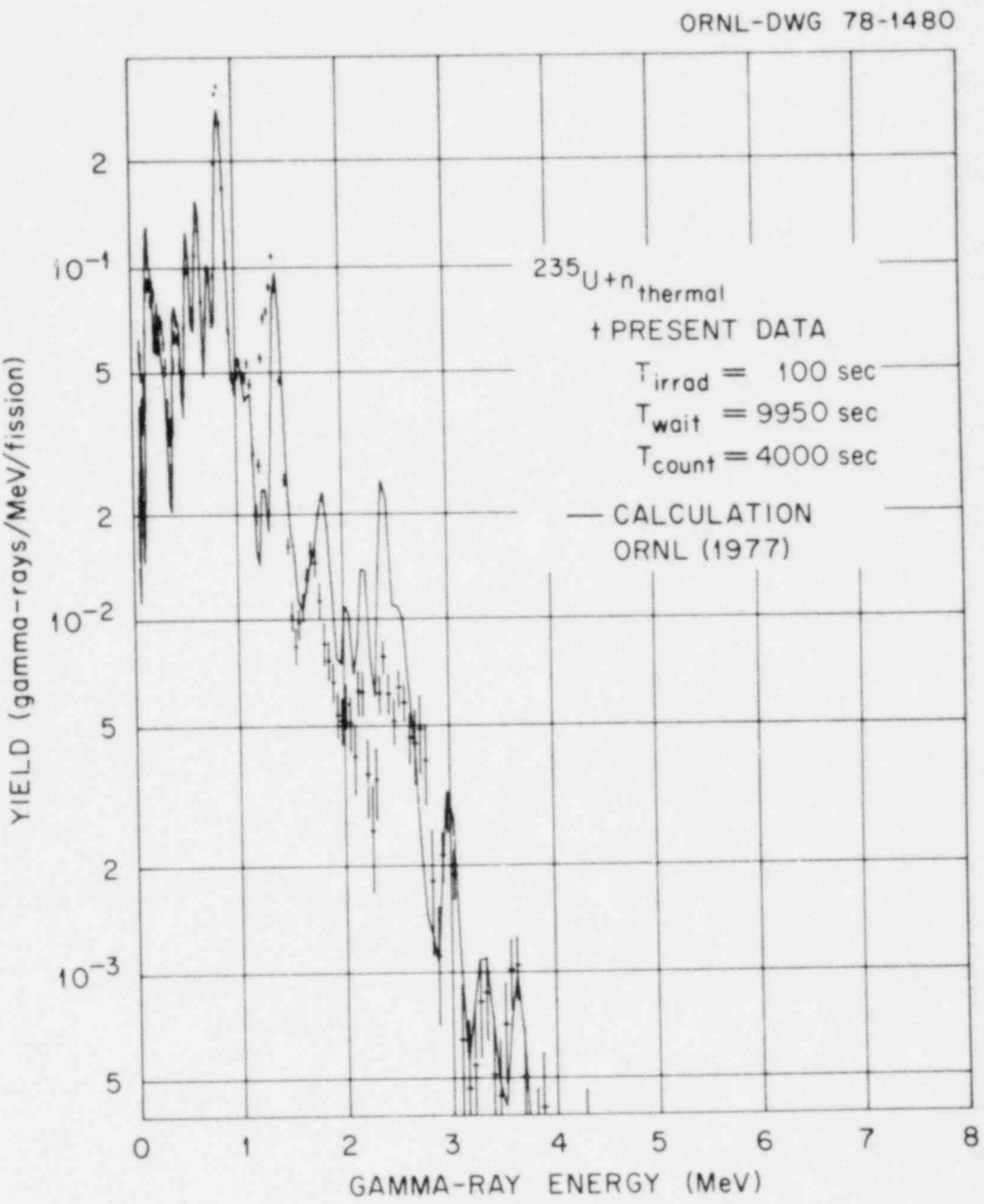

Fig. 92. Gamma Rays Due to Thermal-neutron Fission of ^{235}U. The present data are shown as $N(E_\gamma)$ vs E_γ. The calculation represents the sum of contributions from the 180 "complete" nuclides with contributions from the 600 nuclides having only an "average" gamma-ray energy in the file. There is an estimated loss of 7.0% of $N(E_\gamma)$ due to escape of fission gases (see Ref. 1).

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1.7 SEC AFTER END OF IRRADIATION
COUNT FOR 1 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	6.165E-02	1.725E-02	2.360	2.876E-02	2.376E-03
0.190	6.137E-02	1.621E-02	2.440	2.979E-02	2.225E-03
0.210	6.526E-02	1.503E-02	2.520	2.762E-02	2.236E-03
0.230	6.306E-02	1.416E-02	2.600	2.541E-02	1.992E-03
0.250	5.038E-02	1.428E-02	2.680	2.455E-02	2.028E-03
0.275	3.483E-02	1.351E-02	2.760	2.152E-02	1.790E-03
0.305	4.465E-02	1.261E-02	2.840	2.110E-02	1.829E-03
0.335	5.053E-02	1.200E-02	2.920	2.286E-02	1.798E-03
0.365	3.868E-02	1.181E-02	3.000	2.161E-02	1.723E-03
0.395	4.900E-02	1.040E-02	3.080	1.979E-02	1.605E-03
0.425	5.634E-02	8.080E-03	3.160	1.912E-02	1.641E-03
0.455	4.487E-02	6.473E-03	3.250	1.731E-02	1.453E-03
0.485	4.124E-02	6.370E-03	3.350	1.511E-02	1.459E-03
0.520	4.157E-02	6.210E-03	3.450	1.423E-02	1.427E-03
0.560	3.806E-02	6.607E-03	3.550	1.381E-02	1.325E-03
0.600	4.655E-02	5.720E-03	3.650	1.194E-02	1.211E-03
0.640	4.650E-02	5.494E-03	3.750	1.124E-02	1.192E-03
0.680	4.170E-02	5.041E-03	3.860	1.096E-02	1.118E-03
0.720	4.542E-02	5.219E-03	3.980	8.477E-03	9.773E-04
0.760	4.723E-02	5.027E-03	4.100	7.076E-03	8.971E-04
0.800	4.563E-02	4.847E-03	4.220	8.038E-03	9.409E-04
0.840	4.492E-02	4.314E-03	4.340	7.549E-03	8.592E-04
0.880	4.421E-02	4.224E-03	4.460	6.097E-03	7.555E-04
0.925	4.146E-02	4.096E-03	4.580	5.240E-03	7.304E-04
0.975	3.910E-02	4.052E-03	4.700	4.450E-03	6.177E-04
1.025	3.927E-02	3.813E-03	4.820	3.616E-03	6.158E-04
1.075	3.916E-02	3.792E-03	4.940	3.298E-03	5.290E-04
1.125	4.191E-02	3.772E-03	5.070	3.512E-03	5.573E-04
1.175	4.227E-02	3.596E-03	5.210	3.117E-03	5.103E-04
1.225	3.825E-02	3.450E-03	5.350	2.046E-03	3.981E-04
1.275	4.006E-02	3.443E-03	5.490	1.186E-03	3.331E-04
1.325	4.434E-02	3.359E-03	5.630	8.613E-04	2.784E-04
1.375	4.259E-02	3.204E-03	5.770	1.206E-03	3.043E-04
1.430	3.922E-02	3.117E-03	5.910	1.661E-03	3.705E-04
1.490	4.012E-02	3.046E-03	6.050	1.563E-03	3.282E-04
1.550	4.006E-02	3.078E-03	6.190	1.179E-03	2.962E-04
1.610	3.899E-02	2.929E-03	6.330	8.528E-04	2.444E-04
1.670	4.172E-02	2.833E-03	6.480	5.584E-04	1.968E-04
1.730	4.210E-02	3.054E-03	6.640	3.596E-04	1.556E-04
1.790	3.591E-02	2.930E-03	6.800	3.175E-04	1.506E-04
1.850	3.206E-02	2.776E-03	6.960	3.000E-04	1.360E-04
1.910	3.434E-02	2.729E-03	7.120	2.205E-04	1.096E-04
1.970	3.638E-02	2.796E-03	7.280	1.163E-04	8.782E-05
2.040	3.519E-02	2.687E-03	7.440	4.344E-05	7.642E-05
2.120	3.387E-02	2.592E-03	7.600	1.247E-05	7.216E-05
2.200	3.238E-02	2.461E-03	7.760	6.546E-06	7.580E-05
2.280	2.949E-02	2.442E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 2.7 SEC AFTER END OF IRRADIATION
COUNT FOR 1 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	3.599E-02	1.473E-02	2.360	2.298E-02	2.055E-03
0.190	3.015E-02	1.435E-02	2.440	2.230E-02	1.946E-03
0.210	4.510E-02	1.330E-02	2.520	2.390E-02	2.003E-03
0.230	5.778E-02	1.316E-02	2.600	2.336E-02	1.912E-03
0.250	4.886E-02	1.213E-02	2.680	2.085E-02	1.770E-03
0.275	2.870E-02	1.149E-02	2.760	2.013E-02	1.740E-03
0.305	3.711E-02	1.081E-02	2.840	1.847E-02	1.574E-03
0.335	4.792E-02	1.053E-02	2.920	1.453E-02	1.515E-03
0.365	3.545E-02	1.049E-02	3.000	1.315E-02	1.363E-03
0.395	3.831E-02	9.900E-03	3.080	1.428E-02	1.356E-03
0.425	4.288E-02	7.998E-03	3.160	1.355E-02	1.321E-03
0.455	3.638E-02	6.033E-03	3.250	1.193E-02	1.256E-03
0.485	3.303E-02	5.713E-03	3.350	1.158E-02	1.257E-03
0.520	3.337E-02	5.474E-03	3.450	9.622E-03	1.115E-03
0.560	3.339E-02	5.422E-03	3.550	9.205E-03	1.053E-03
0.600	3.584E-02	5.026E-03	3.650	1.068E-02	1.093E-03
0.640	3.705E-02	4.768E-03	3.750	9.143E-03	1.060E-03
0.680	3.819E-02	4.720E-03	3.860	7.301E-03	8.715E-04
0.720	3.878E-02	4.529E-03	3.980	8.111E-03	9.940E-04
0.760	3.838E-02	4.333E-03	4.100	7.225E-03	8.499E-04
0.800	3.775E-02	4.132E-03	4.220	6.207E-03	8.123E-04
0.840	3.409E-02	3.914E-03	4.340	5.929E-03	7.920E-04
0.880	3.451E-02	4.005E-03	4.460	5.156E-03	7.089E-04
0.925	3.583E-02	3.881E-03	4.580	4.714E-03	6.501E-04
0.975	3.459E-02	3.716E-03	4.700	4.261E-03	6.189E-04
1.025	3.934E-02	3.578E-03	4.820	3.190E-03	5.183E-04
1.075	3.843E-02	3.403E-03	4.940	2.431E-03	4.765E-04
1.125	3.574E-02	3.449E-03	5.070	2.518E-03	4.857E-04
1.175	3.546E-02	3.177E-03	5.210	2.611E-03	4.658E-04
1.225	3.510E-02	3.163E-03	5.350	1.928E-03	4.209E-04
1.275	3.557E-02	3.058E-03	5.490	1.377E-03	3.577E-04
1.325	3.405E-02	2.926E-03	5.630	1.423E-03	3.523E-04
1.375	2.981E-02	2.948E-03	5.770	1.346E-03	3.192E-04
1.430	3.062E-02	2.628E-03	5.910	1.021E-03	2.748E-04
1.490	3.645E-02	2.803E-03	6.050	9.139E-04	2.699E-04
1.550	3.524E-02	2.619E-03	6.190	9.447E-04	2.587E-04
1.610	3.245E-02	2.620E-03	6.330	8.028E-04	2.257E-04
1.670	3.127E-02	2.558E-03	6.480	5.002E-04	1.733E-04
1.730	2.808E-02	2.539E-03	6.640	2.938E-04	1.327E-04
1.790	2.695E-02	2.574E-03	6.800	1.464E-04	8.907E-05
1.850	2.732E-02	2.512E-03	6.960	1.710E-04	9.850E-05
1.910	2.573E-02	2.327E-03	7.120	1.750E-04	9.660E-05
1.970	2.622E-02	2.313E-03	7.280	1.175E-04	8.561E-05
2.040	2.936E-02	2.376E-03	7.440	5.095E-05	7.695E-05
2.120	2.818E-02	2.447E-03	7.600	1.360E-05	7.139E-05
2.200	2.602E-02	2.254E-03	7.760	5.020E-06	7.555E-05
2.280	2.501E-02	2.191E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 3.7 SEC AFTER END OF IRRADIATION
COUNT FOR 1 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	6.650E-02	1.364E-02	2.360	1.988E-02	1.868E-03
0.190	6.919E-02	1.236E-02	2.440	1.554E-02	1.555E-03
0.210	6.712E-02	1.150E-02	2.520	1.433E-02	1.556E-03
0.230	4.339E-02	1.003E-02	2.600	1.633E-02	1.576E-03
0.250	1.898E-02	9.053E-03	2.680	1.698E-02	1.547E-03
0.275	1.243E-02	8.696E-03	2.760	1.491E-02	1.498E-03
0.305	2.035E-02	9.570E-03	2.840	1.286E-02	1.371E-03
0.335	2.241E-02	9.864E-03	2.920	1.336E-02	1.337E-03
0.365	3.503E-02	9.133E-03	3.000	1.326E-02	1.307E-03
0.395	4.581E-02	8.117E-03	3.080	1.068E-02	1.241E-03
0.425	3.670E-02	6.589E-03	3.160	9.133E-03	1.153E-03
0.455	2.648E-02	5.302E-03	3.250	9.892E-03	1.119E-03
0.485	3.040E-02	5.023E-03	3.350	9.818E-03	1.103E-03
0.520	3.793E-02	4.765E-03	3.450	8.787E-03	1.021E-03
0.560	3.363E-02	4.819E-03	3.550	8.037E-03	1.005E-03
0.600	3.184E-02	4.616E-03	3.650	7.302E-03	9.013E-04
0.640	3.829E-02	4.393E-03	3.750	6.978E-03	9.257E-04
0.680	3.311E-02	4.145E-03	3.860	6.674E-03	8.334E-04
0.720	3.585E-02	4.006E-03	3.980	6.174E-03	8.392E-04
0.760	3.814E-02	3.931E-03	4.100	5.227E-03	7.300E-04
0.800	3.291E-02	3.704E-03	4.220	3.806E-03	6.209E-04
0.840	2.873E-02	3.423E-03	4.340	3.342E-03	5.680E-04
0.880	3.151E-02	3.579E-03	4.460	3.714E-03	6.175E-04
0.925	3.559E-02	3.601E-03	4.580	3.760E-03	5.618E-04
0.975	3.531E-02	3.426E-03	4.700	3.306E-03	5.554E-04
1.025	3.334E-02	3.266E-03	4.820	2.590E-03	4.608E-04
1.075	3.246E-02	3.058E-03	4.940	2.084E-03	4.298E-04
1.125	3.037E-02	2.975E-03	5.070	1.943E-03	4.124E-04
1.175	2.997E-02	2.901E-03	5.210	1.861E-03	3.779E-04
1.225	2.878E-02	2.788E-03	5.350	1.496E-03	3.697E-04
1.275	2.875E-02	2.635E-03	5.490	1.246E-03	3.002E-04
1.325	3.152E-02	2.645E-03	5.630	1.405E-03	3.305E-04
1.375	3.291E-02	2.523E-03	5.770	1.379E-03	3.188E-04
1.430	3.339E-02	2.534E-03	5.910	1.054E-03	2.630E-04
1.490	3.239E-02	2.522E-03	6.050	8.101E-04	2.290E-04
1.550	2.821E-02	2.343E-03	6.190	5.792E-04	2.094E-04
1.610	2.537E-02	2.362E-03	6.330	3.503E-04	1.372E-04
1.670	2.647E-02	2.273E-03	6.480	2.777E-04	1.231E-04
1.730	2.863E-02	2.416E-03	6.640	2.823E-04	1.194E-04
1.790	2.731E-02	2.372E-03	6.800	1.854E-04	1.013E-04
1.850	2.653E-02	2.287E-03	6.960	6.867E-05	6.969E-05
1.910	2.729E-02	2.264E-03	7.120	3.603E-05	6.413E-05
1.970	2.373E-02	2.170E-03	7.280	4.654E-05	6.708E-05
2.040	2.098E-02	1.964E-03	7.440	4.188E-05	6.796E-05
2.120	2.540E-02	2.227E-03	7.600	2.181E-05	6.973E-05
2.200	2.358E-02	2.048E-03	7.760	7.181E-06	7.490E-05
2.280	2.124E-02	2.010E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 4.7 SEC AFTER END OF IRRADIATION
COUNT FOR 2 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	7.491E-02	1.801E-02	2.360	3.024E-02	2.242E-03
0.190	8.641E-02	1.612E-02	2.440	2.948E-02	2.212E-03
0.210	9.279E-02	1.584E-02	2.520	2.650E-02	2.021E-03
0.230	8.101E-02	1.433E-02	2.600	2.441E-02	1.993E-03
0.250	7.167E-02	1.449E-02	2.680	2.152E-02	1.904E-03
0.275	6.948E-02	1.310E-02	2.760	2.117E-02	1.835E-03
0.305	5.411E-02	1.350E-02	2.840	2.334E-02	1.824E-03
0.335	4.769E-02	1.311E-02	2.920	2.218E-02	1.705E-03
0.365	6.956E-02	1.234E-02	3.000	1.822E-02	1.647E-03
0.395	7.187E-02	1.111E-02	3.080	1.633E-02	1.454E-03
0.425	5.655E-02	8.671E-03	3.160	1.666E-02	1.512E-03
0.455	6.065E-02	7.037E-03	3.250	1.610E-02	1.403E-03
0.485	6.093E-02	6.805E-03	3.350	1.461E-02	1.415E-03
0.520	5.121E-02	6.276E-03	3.450	1.414E-02	1.302E-03
0.560	5.302E-02	6.386E-03	3.550	1.349E-02	1.293E-03
0.600	4.526E-02	6.065E-03	3.650	1.189E-02	1.161E-03
0.640	4.686E-02	5.661E-03	3.750	1.033E-02	1.127E-03
0.680	5.128E-02	5.567E-03	3.860	9.374E-03	9.880E-04
0.720	5.233E-02	5.348E-03	3.980	8.645E-03	1.009E-03
0.760	5.530E-02	4.966E-03	4.100	7.717E-03	8.922E-04
0.800	4.998E-02	4.910E-03	4.220	7.235E-03	8.698E-04
0.840	4.718E-02	4.893E-03	4.340	6.311E-03	7.922E-04
0.880	4.874E-02	4.561E-03	4.460	5.157E-03	7.005E-04
0.925	4.934E-02	4.478E-03	4.580	4.951E-03	7.105E-04
0.975	5.250E-02	4.490E-03	4.700	5.065E-03	6.972E-04
1.025	5.357E-02	4.310E-03	4.820	4.542E-03	6.590E-04
1.075	5.324E-02	4.038E-03	4.940	3.606E-03	5.955E-04
1.125	5.101E-02	3.763E-03	5.070	3.109E-03	4.992E-04
1.175	5.191E-02	3.795E-03	5.210	2.976E-03	5.383E-04
1.225	5.527E-02	3.718E-03	5.350	2.486E-03	4.400E-04
1.275	5.059E-02	3.517E-03	5.490	2.403E-03	4.705E-04
1.325	4.359E-02	3.347E-03	5.630	2.452E-03	4.284E-04
1.375	4.427E-02	3.311E-03	5.770	1.936E-03	3.986E-04
1.430	4.865E-02	3.319E-03	5.910	1.454E-03	3.227E-04
1.490	4.929E-02	3.248E-03	6.050	1.441E-03	3.276E-04
1.550	5.034E-02	3.219E-03	6.190	1.367E-03	2.966E-04
1.610	4.957E-02	3.095E-03	6.330	9.508E-04	2.343E-04
1.670	4.835E-02	2.954E-03	6.480	4.512E-04	1.704E-04
1.730	4.256E-02	3.020E-03	6.640	2.008E-04	9.081E-05
1.790	4.209E-02	3.084E-03	6.800	1.818E-04	1.048E-04
1.850	3.910E-02	2.777E-03	6.960	1.505E-04	9.063E-05
1.910	4.021E-02	2.859E-03	7.120	1.037E-04	7.184E-05
1.970	4.312E-02	2.783E-03	7.280	8.570E-05	6.726E-05
2.040	3.757E-02	2.748E-03	7.440	6.968E-05	7.128E-05
2.120	3.036E-02	2.560E-03	7.600	4.061E-05	7.086E-05
2.200	3.100E-02	2.589E-03	7.760	1.407E-05	7.414E-05
2.280	2.963E-02	2.304E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 6.7 SEC AFTER END OF IRRADIATION
COUNT FOR 3 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.417E-01	1.705E-02	2.360	3.403E-02	2.434E-03
0.190	1.007E-01	1.635E-02	2.440	3.295E-02	2.354E-03
0.210	6.023E-02	1.645E-02	2.520	2.954E-02	2.152E-03
0.230	6.990E-02	1.588E-02	2.600	3.018E-02	2.184E-03
0.250	7.980E-02	1.495E-02	2.680	3.057E-02	2.122E-03
0.275	6.223E-02	1.467E-02	2.760	2.529E-02	1.853E-03
0.305	7.682E-02	1.451E-02	2.840	2.134E-02	1.778E-03
0.335	9.696E-02	1.347E-02	2.920	2.124E-02	1.717E-03
0.365	7.091E-02	1.267E-02	3.000	2.015E-02	1.663E-03
0.395	5.726E-02	1.197E-02	3.080	1.712E-02	1.556E-03
0.425	5.793E-02	9.718E-03	3.160	1.506E-02	1.450E-03
0.455	5.695E-02	7.432E-03	3.250	1.487E-02	1.420E-03
0.485	5.830E-02	7.190E-03	3.350	1.459E-02	1.382E-03
0.520	6.315E-02	6.912E-03	3.450	1.406E-02	1.307E-03
0.560	6.465E-02	6.938E-03	3.550	1.465E-02	1.316E-03
0.600	5.953E-02	6.544E-03	3.650	1.356E-02	1.289E-03
0.640	6.008E-02	6.191E-03	3.750	1.142E-02	1.145E-03
0.680	6.112E-02	6.014E-03	3.860	1.114E-02	1.202E-03
0.720	6.911E-02	5.716E-03	3.980	1.061E-02	1.075E-03
0.760	7.381E-02	5.522E-03	4.100	8.467E-03	9.394E-04
0.800	6.575E-02	5.236E-03	4.220	6.420E-03	8.220E-04
0.840	6.525E-02	4.892E-03	4.340	5.950E-03	7.487E-04
0.880	6.811E-02	4.802E-03	4.460	6.373E-03	7.985E-04
0.925	6.399E-02	4.789E-03	4.580	5.786E-03	7.237E-04
0.975	6.542E-02	4.833E-03	4.700	4.861E-03	6.707E-04
1.025	6.552E-02	4.335E-03	4.820	4.469E-03	6.363E-04
1.075	6.132E-02	4.269E-03	4.940	4.160E-03	6.335E-04
1.125	6.026E-02	4.095E-03	5.070	3.690E-03	5.799E-04
1.175	6.156E-02	4.031E-03	5.210	3.216E-03	5.225E-04
1.225	6.420E-02	3.997E-03	5.350	2.783E-03	4.980E-04
1.275	6.244E-02	3.766E-03	5.490	2.394E-03	4.318E-04
1.325	5.785E-02	3.654E-03	5.630	2.104E-03	4.353E-04
1.375	5.472E-02	3.672E-03	5.770	1.850E-03	3.706E-04
1.430	5.819E-02	3.450E-03	5.910	1.596E-03	3.549E-04
1.490	5.508E-02	3.437E-03	6.050	1.238E-03	3.160E-04
1.550	4.897E-02	3.230E-03	6.190	9.982E-04	2.580E-04
1.610	4.882E-02	3.236E-03	6.330	9.523E-04	2.689E-04
1.670	4.808E-02	3.189E-03	6.480	8.115E-04	2.255E-04
1.730	4.384E-02	3.201E-03	6.640	9.237E-04	1.737E-04
1.790	4.453E-02	3.071E-03	6.800	2.240E-04	1.195E-04
1.850	4.650E-02	3.103E-03	6.960	1.562E-04	9.058E-05
1.910	4.337E-02	2.963E-03	7.120	2.234E-04	1.151E-04
1.970	3.937E-02	2.907E-03	7.280	2.340E-04	1.116E-04
2.040	3.757E-02	2.722E-03	7.440	1.680E-04	8.955E-05
2.120	3.800E-02	2.790E-03	7.600	8.390E-05	7.721E-05
2.200	3.873E-02	2.688E-03	7.760	3.388E-05	7.857E-05
2.280	3.561E-02	2.630E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 9.7 SEC AFTER END OF IRRADIATION
COUNT FOR 5 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.040E-01	2.053E-02	2.360	3.933E-02	2.646E-03
0.190	1.059E-01	1.889E-02	2.440	3.367E-02	2.290E-03
0.210	9.192E-02	1.821E-02	2.520	2.975E-02	2.251E-03
0.230	7.536E-02	1.705E-02	2.600	2.892E-02	2.127E-03
0.250	7.803E-02	1.650E-02	2.680	2.577E-02	1.970E-03
0.275	7.906E-02	1.584E-02	2.760	2.490E-02	1.978E-03
0.305	7.090E-02	1.488E-02	2.840	2.540E-02	1.916E-03
0.335	9.109E-02	1.473E-02	2.920	2.412E-02	1.864E-03
0.365	9.350E-02	1.395E-02	3.000	2.211E-02	1.743E-03
0.395	7.604E-02	1.294E-02	3.080	1.971E-02	1.681E-03
0.425	7.910E-02	1.049E-02	3.160	1.799E-02	1.571E-03
0.455	8.993E-02	8.477E-03	3.250	1.762E-02	1.583E-03
0.485	9.423E-02	8.015E-03	3.350	1.686E-02	1.471E-03
0.520	9.085E-02	7.687E-03	3.450	1.536E-02	1.406E-03
0.560	8.165E-02	7.505E-03	3.550	1.386E-02	1.306E-03
0.600	8.212E-02	7.192E-03	3.650	1.240E-02	1.199E-03
0.640	8.668E-02	6.927E-03	3.750	1.181E-02	1.156E-03
0.680	8.475E-02	6.800E-03	3.860	1.120E-02	1.156E-03
0.720	8.095E-02	6.491E-03	3.980	9.683E-03	1.015E-03
0.760	8.692E-02	6.148E-03	4.100	8.783E-03	1.037E-03
0.800	9.273E-02	5.953E-03	4.220	8.007E-03	9.053E-04
0.840	8.452E-02	5.569E-03	4.340	6.948E-03	8.502E-04
0.880	7.607E-02	5.461E-03	4.460	6.285E-03	7.646E-04
0.925	7.121E-02	5.234E-03	4.580	5.790E-03	7.198E-04
0.975	7.058E-02	5.128E-03	4.700	4.675E-03	7.037E-04
1.025	7.902E-02	5.124E-03	4.820	3.766E-03	6.008E-04
1.075	7.899E-02	5.070E-03	4.940	3.884E-03	6.132E-04
1.125	7.394E-02	4.539E-03	5.070	3.874E-03	5.886E-04
1.175	7.027E-02	4.405E-03	5.210	2.976E-03	5.117E-04
1.225	7.008E-02	4.233E-03	5.350	2.344E-03	4.395E-04
1.275	6.987E-02	4.156E-03	5.490	2.181E-03	4.214E-04
1.325	6.830E-02	3.962E-03	5.630	1.846E-03	3.935E-04
1.375	6.791E-02	3.938E-03	5.770	1.670E-03	3.483E-04
1.430	6.470E-02	3.732E-03	5.910	1.684E-03	3.534E-04
1.490	5.894E-02	3.654E-03	6.050	1.309E-03	3.122E-04
1.550	5.756E-02	3.444E-03	6.190	8.048E-04	2.071E-04
1.610	6.041E-02	3.548E-03	6.330	5.349E-04	2.084E-04
1.670	5.841E-02	3.372E-03	6.480	4.470E-04	1.634E-04
1.730	5.221E-02	3.369E-03	6.640	3.890E-04	1.460E-04
1.790	4.948E-02	3.457E-03	6.800	2.646E-04	1.201E-04
1.850	4.871E-02	3.253E-03	6.960	1.249E-04	8.210E-05
1.910	4.865E-02	3.124E-03	7.120	3.496E-05	6.642E-05
1.970	4.703E-02	3.071E-03	7.280	5.044E-06	6.792E-05
2.040	4.209E-02	2.890E-03	7.440	6.077E-06	6.843E-05
2.120	3.874E-02	2.793E-03	7.600	1.160E-05	6.947E-05
2.200	3.510E-02	2.666E-03	7.760	1.074E-05	7.492E-05
2.280	3.611E-02	2.555E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 14.7 SEC AFTER END OF IRRADIATION
COUNT FOR 5 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	8.611E-02	1.772E-02	2.360	2.358E-02	1.983E-03
0.190	9.202E-02	1.678E-02	2.440	2.177E-02	1.875E-03
0.210	8.087E-02	1.653E-02	2.520	1.940E-02	1.752E-03
0.230	7.568E-02	1.526E-02	2.600	1.911E-02	1.789E-03
0.250	8.546E-02	1.409E-02	2.680	1.793E-02	1.658E-03
0.275	8.410E-02	1.385E-02	2.760	1.634E-02	1.562E-03
0.305	6.452E-02	1.292E-02	2.840	1.712E-02	1.588E-03
0.335	6.864E-02	1.230E-02	2.920	1.626E-02	1.500E-03
0.365	6.858E-02	1.187E-02	3.000	1.317E-02	1.263E-03
0.395	5.814E-02	1.137E-02	3.080	1.172E-02	1.228E-03
0.425	6.546E-02	9.178E-03	3.160	1.156E-02	1.226E-03
0.455	6.641E-02	7.234E-03	3.250	1.147E-02	1.231E-03
0.485	6.090E-02	6.856E-03	3.350	1.170E-02	1.191E-03
0.520	6.805E-02	6.865E-03	3.450	1.090E-02	1.165E-03
0.560	7.044E-02	6.938E-03	3.550	9.232E-03	1.006E-03
0.600	6.435E-02	6.328E-03	3.650	8.073E-03	9.941E-04
0.640	6.137E-02	6.097E-03	3.750	7.227E-03	8.837E-04
0.680	5.965E-02	5.686E-03	3.860	6.236E-03	8.053E-04
0.720	6.602E-02	5.502E-03	3.980	4.903E-03	7.600E-04
0.760	6.893E-02	5.324E-03	4.100	4.614E-03	6.900E-04
0.800	6.115E-02	5.014E-03	4.220	5.109E-03	7.311E-04
0.840	5.748E-02	5.162E-03	4.340	4.424E-03	6.575E-04
0.880	6.066E-02	4.929E-03	4.460	3.311E-03	5.780E-04
0.925	6.061E-02	4.755E-03	4.580	2.689E-03	4.533E-04
0.975	5.808E-02	4.671E-03	4.700	2.167E-03	4.651E-04
1.025	6.055E-02	4.564E-03	4.820	1.834E-03	3.599E-04
1.075	5.912E-02	4.169E-03	4.940	1.823E-03	4.074E-04
1.125	5.406E-02	3.811E-03	5.070	1.678E-03	3.710E-04
1.175	5.415E-02	3.788E-03	5.210	1.597E-03	3.470E-04
1.225	5.176E-02	3.791E-03	5.350	1.737E-03	3.737E-04
1.275	4.780E-02	3.376E-03	5.490	1.418E-03	3.138E-04
1.325	4.998E-02	3.367E-03	5.630	9.593E-04	2.764E-04
1.375	5.228E-02	3.314E-03	5.770	8.687E-04	2.361E-04
1.430	4.821E-02	3.266E-03	5.910	8.269E-04	2.358E-04
1.490	4.442E-02	3.153E-03	6.050	5.538E-04	1.869E-04
1.550	4.507E-02	3.064E-03	6.190	3.471E-04	1.283E-04
1.610	4.209E-02	3.040E-03	6.330	2.861E-04	1.301E-04
1.670	3.777E-02	2.858E-03	6.480	2.005E-04	1.149E-04
1.730	3.671E-02	2.963E-03	6.640	9.569E-05	7.477E-05
1.790	3.586E-02	2.905E-03	6.800	1.073E-04	6.542E-05
1.850	3.380E-02	2.876E-03	6.960	1.516E-04	7.366E-05
1.910	3.298E-02	2.692E-03	7.120	1.138E-04	6.778E-05
1.970	3.316E-02	2.559E-03	7.280	4.281E-05	6.830E-05
2.040	2.999E-02	2.536E-03	7.440	7.567E-06	6.745E-05
2.120	2.493E-02	2.411E-03	7.600	4.875E-06	6.944E-05
2.200	2.529E-02	2.229E-03	7.760	9.805E-06	7.509E-05
2.280	2.509E-02	2.253E-03			

C0021I STOP 0

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 19.7 SEC AFTER END OF IRRADIATION
COUNT FOR 5 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	9.146E-02	1.569E-02	2.360	1.561E-02	1.735E-03
0.190	6.116E-02	1.408E-02	2.440	1.338E-02	1.587E-03
0.210	5.244E-02	1.451E-02	2.520	1.316E-02	1.516E-03
0.230	6.212E-02	1.322E-02	2.600	1.599E-02	1.586E-03
0.250	7.173E-02	1.201E-02	2.680	1.573E-02	1.505E-03
0.275	7.673E-02	1.156E-02	2.760	1.148E-02	1.221E-03
0.305	7.779E-02	1.221E-02	2.840	9.550E-03	1.275E-03
0.335	8.488E-02	1.147E-02	2.920	9.944E-03	1.294E-03
0.365	7.545E-02	1.066E-02	3.000	9.395E-03	1.152E-03
0.395	5.433E-02	1.033E-02	3.080	9.156E-03	1.073E-03
0.425	5.234E-02	8.629E-03	3.160	9.610E-03	1.084E-03
0.455	5.299E-02	6.683E-03	3.250	8.184E-03	9.983E-04
0.485	5.093E-02	6.277E-03	3.350	5.883E-03	9.275E-04
0.520	5.483E-02	5.922E-03	3.450	6.245E-03	8.839E-04
0.560	5.804E-02	5.889E-03	3.550	6.656E-03	8.473E-04
0.600	5.730E-02	5.805E-03	3.650	5.619E-03	7.700E-04
0.640	5.679E-02	5.472E-03	3.750	5.233E-03	7.380E-04
0.680	5.825E-02	5.278E-03	3.860	5.154E-03	7.643E-04
0.720	5.895E-02	5.097E-03	3.980	4.045E-03	6.221E-04
0.760	5.777E-02	4.872E-03	4.100	3.407E-03	5.814E-04
0.800	5.513E-02	4.692E-03	4.220	3.147E-03	5.264E-04
0.840	5.357E-02	4.568E-03	4.340	2.394E-03	4.613E-04
0.880	5.052E-02	4.376E-03	4.460	1.780E-03	4.054E-04
0.925	4.850E-02	4.178E-03	4.580	1.960E-03	4.044E-04
0.975	5.317E-02	4.229E-03	4.700	2.171E-03	4.101E-04
1.025	5.138E-02	3.962E-03	4.820	1.675E-03	3.707E-04
1.075	4.494E-02	3.698E-03	4.940	1.166E-03	3.851E-04
1.125	4.539E-02	3.613E-03	5.070	1.161E-03	2.930E-04
1.175	4.447E-02	3.365E-03	5.210	1.133E-03	2.824E-04
1.225	3.919E-02	3.248E-03	5.350	7.925E-04	2.203E-04
1.275	3.636E-02	3.142E-03	5.490	5.165E-04	1.863E-04
1.325	3.823E-02	3.004E-03	5.630	3.635E-04	1.412E-04
1.375	4.005E-02	2.986E-03	5.770	2.289E-04	1.168E-04
1.430	3.792E-02	2.978E-03	5.910	1.392E-04	8.482E-05
1.490	3.527E-02	2.833E-03	6.050	1.322E-04	7.760E-05
1.550	3.524E-02	2.722E-03	6.190	1.225E-04	7.515E-05
1.610	3.485E-02	2.503E-03	6.330	7.360E-05	6.504E-05
1.670	3.285E-02	2.514E-03	6.480	4.106E-05	5.916E-05
1.730	3.011E-02	2.487E-03	6.640	5.795E-05	5.830E-05
1.790	2.849E-02	2.373E-03	6.800	8.142E-05	6.154E-05
1.850	2.622E-02	2.284E-03	6.960	6.328E-05	6.092E-05
1.910	2.157E-02	2.153E-03	7.120	2.335E-05	6.330E-05
1.970	1.888E-02	1.961E-03	7.280	4.189E-06	6.490E-05
2.040	2.123E-02	2.089E-03	7.440	5.392E-06	6.751E-05
2.120	2.304E-02	2.096E-03	7.600	1.140E-05	6.886E-05
2.200	2.021E-02	1.919E-03	7.760	1.109E-05	7.475E-05
2.280	1.740E-02	1.722E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 24.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.229E-01	1.961E-02	2.360	2.476E-02	2.127E-03
0.190	1.385E-01	1.800E-02	2.440	2.198E-02	1.951E-03
0.210	1.324E-01	1.720E-02	2.520	1.956E-02	1.832E-03
0.230	1.096E-01	1.750E-02	2.600	2.015E-02	1.743E-03
0.250	1.012E-01	1.577E-02	2.680	1.932E-02	1.738E-03
0.275	1.117E-01	1.510E-02	2.760	1.675E-02	1.653E-03
0.305	1.050E-01	1.542E-02	2.840	1.547E-02	1.502E-03
0.335	1.042E-01	1.467E-02	2.920	1.472E-02	1.377E-03
0.365	1.074E-01	1.378E-02	3.000	1.272E-02	1.274E-03
0.395	9.257E-02	1.254E-02	3.080	1.114E-02	1.239E-03
0.425	8.973E-02	1.032E-02	3.160	1.117E-02	1.231E-03
0.455	9.446E-02	8.313E-03	3.250	1.081E-02	1.198E-03
0.485	8.873E-02	7.886E-03	3.350	9.175E-03	1.017E-03
0.520	7.570E-02	7.767E-03	3.450	7.942E-03	1.017E-03
0.560	8.403E-02	7.510E-03	3.550	8.665E-03	9.365E-04
0.600	8.794E-02	7.328E-03	3.650	6.489E-03	8.853E-04
0.640	8.219E-02	6.773E-03	3.750	6.636E-03	8.571E-04
0.680	9.220E-02	6.504E-03	3.860	5.289E-03	7.544E-04
0.720	1.033E-01	6.648E-03	3.980	4.418E-03	7.611E-04
0.760	1.058E-01	6.292E-03	4.100	4.508E-03	6.994E-04
0.800	9.281E-02	5.872E-03	4.220	3.782E-03	6.380E-04
0.840	8.198E-02	5.554E-03	4.340	3.081E-03	5.666E-04
0.880	8.259E-02	5.480E-03	4.460	2.796E-03	5.204E-04
0.925	7.977E-02	5.260E-03	4.580	2.626E-03	4.951E-04
0.975	7.338E-02	4.951E-03	4.700	2.462E-03	4.626E-04
1.025	6.960E-02	4.830E-03	4.820	1.973E-03	3.862E-04
1.075	7.293E-02	4.628E-03	4.940	1.324E-03	3.077E-04
1.125	7.642E-02	4.513E-03	5.070	8.281E-04	2.510E-04
1.175	7.136E-02	4.496E-03	5.210	5.907E-04	2.017E-04
1.225	6.518E-02	4.206E-03	5.350	5.658E-04	2.106E-04
1.275	6.288E-02	3.967E-03	5.490	6.020E-04	1.917E-04
1.325	6.078E-02	3.684E-03	5.630	5.506E-04	1.868E-04
1.375	5.899E-02	3.809E-03	5.770	3.833E-04	1.505E-04
1.430	5.570E-02	3.576E-03	5.910	2.527E-04	1.082E-04
1.490	5.232E-02	3.426E-03	6.050	2.000E-04	1.150E-04
1.550	5.360E-02	3.386E-03	6.190	1.723E-04	1.203E-04
1.610	5.009E-02	3.135E-03	6.330	1.377E-04	9.442E-05
1.670	4.335E-02	2.921E-03	6.480	9.620E-05	7.094E-05
1.730	4.206E-02	3.210E-03	6.640	5.165E-05	6.470E-05
1.790	4.350E-02	2.996E-03	6.800	2.336E-05	6.199E-05
1.850	4.209E-02	2.952E-03	6.960	1.166E-05	6.054E-05
1.910	3.893E-02	2.696E-03	7.120	7.733E-06	6.313E-05
1.970	3.711E-02	2.646E-03	7.280	9.382E-06	6.505E-05
2.040	3.321E-02	2.624E-03	7.440	9.892E-06	6.759E-05
2.120	2.904E-02	2.317E-03	7.600	1.103E-05	6.893E-05
2.200	2.849E-02	2.559E-03	7.760	9.798E-06	7.476E-05
2.280	2.569E-02	2.021E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 34.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.062E-01	1.715E-02	2.360	1.783E-02	1.831E-03
0.190	9.286E-02	1.613E-02	2.440	1.593E-02	1.637E-03
0.210	9.926E-02	1.507E-02	2.520	1.425E-02	1.491E-03
0.230	1.048E-01	1.462E-02	2.600	1.349E-02	1.519E-03
0.250	8.741E-02	1.310E-02	2.680	1.228E-02	1.490E-03
0.275	6.511E-02	1.330E-02	2.760	1.206E-02	1.319E-03
0.305	8.141E-02	1.312E-02	2.840	1.267E-02	1.391E-03
0.335	9.041E-02	1.213E-02	2.920	1.126E-02	1.185E-03
0.365	8.039E-02	1.171E-02	3.000	8.502E-03	1.072E-03
0.395	7.624E-02	1.133E-02	3.080	8.911E-03	1.020E-03
0.425	7.578E-02	9.022E-03	3.160	7.292E-03	9.531E-04
0.455	6.769E-02	7.205E-03	3.250	8.061E-03	9.821E-04
0.485	5.867E-02	6.953E-03	3.350	7.195E-03	9.724E-04
0.520	6.404E-02	6.585E-03	3.450	5.476E-03	8.627E-04
0.560	6.882E-02	6.704E-03	3.550	4.324E-03	7.697E-04
0.600	6.777E-02	6.328E-03	3.650	4.578E-03	7.331E-04
0.640	6.693E-02	6.169E-03	3.750	5.096E-03	7.338E-04
0.680	5.959E-02	5.680E-03	3.860	4.093E-03	7.252E-04
0.720	6.520E-02	5.958E-03	3.980	2.678E-03	5.869E-04
0.760	7.323E-02	5.223E-03	4.100	2.651E-03	5.269E-04
0.800	6.388E-02	4.989E-03	4.220	2.813E-03	4.819E-04
0.840	5.593E-02	4.721E-03	4.340	2.361E-03	4.552E-04
0.880	5.701E-02	4.625E-03	4.460	1.709E-03	3.539E-04
0.925	5.682E-02	4.681E-03	4.580	1.198E-03	3.233E-04
0.975	5.602E-02	4.500E-03	4.700	9.021E-04	2.939E-04
1.025	5.388E-02	4.229E-03	4.820	8.211E-04	2.643E-04
1.075	5.279E-02	4.304E-03	4.940	7.777E-04	2.313E-04
1.125	5.373E-02	3.940E-03	5.070	6.195E-04	1.935E-04
1.175	5.095E-02	3.915E-03	5.210	3.723E-04	1.587E-04
1.225	4.765E-02	3.761E-03	5.350	2.452E-04	1.065E-04
1.275	4.531E-02	3.474E-03	5.490	2.137E-04	1.095E-04
1.325	4.286E-02	3.294E-03	5.630	1.377E-04	8.803E-05
1.375	4.236E-02	3.043E-03	5.770	7.765E-05	6.793E-05
1.430	4.274E-02	3.075E-03	5.910	9.421E-05	6.498E-05
1.490	4.056E-02	2.928E-03	6.050	1.365E-04	7.443E-05
1.550	3.716E-02	2.759E-03	6.190	1.315E-04	7.104E-05
1.610	3.471E-02	2.697E-03	6.330	1.027E-04	6.883E-05
1.670	3.289E-02	2.611E-03	6.480	6.610E-05	6.291E-05
1.730	3.388E-02	2.604E-03	6.640	3.400E-05	6.053E-05
1.790	2.998E-02	2.588E-03	6.800	1.396E-05	5.865E-05
1.850	2.911E-02	2.556E-03	6.960	6.774E-06	6.084E-05
1.910	2.566E-02	2.288E-03	7.120	8.548E-06	6.280E-05
1.970	2.360E-02	2.156E-03	7.280	1.014E-05	6.477E-05
2.040	2.451E-02	2.228E-03	7.440	9.898E-06	6.756E-05
2.120	2.331E-02	2.070E-03	7.600	1.085E-05	6.885E-05
2.200	2.106E-02	2.019E-03	7.760	9.738E-06	7.476E-05
2.280	1.931E-02	1.931E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 44.7 SEC AFTER END OF IRRADIATION
COUNT FOR 15 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.416E-01	1.959E-02	2.360	2.043E-02	1.911E-03
0.190	1.388E-01	1.718E-02	2.440	1.631E-02	1.616E-03
0.210	1.284E-01	1.660E-02	2.520	1.340E-02	1.655E-03
0.230	1.241E-01	1.546E-02	2.600	1.533E-02	1.542E-03
0.250	1.167E-01	1.457E-02	2.680	1.526E-02	1.468E-03
0.275	1.011E-01	1.448E-02	2.760	1.155E-02	1.363E-03
0.305	8.840E-02	1.462E-02	2.840	1.023E-02	1.152E-03
0.335	9.441E-02	1.370E-02	2.920	1.062E-02	1.196E-03
0.365	9.290E-02	1.282E-02	3.000	9.647E-03	1.213E-03
0.395	7.520E-02	1.241E-02	3.080	8.546E-03	1.139E-03
0.425	8.058E-02	9.963E-03	3.160	8.926E-03	1.131E-03
0.455	9.258E-02	7.666E-03	3.250	8.924E-03	1.048E-03
0.485	8.848E-02	7.512E-03	3.350	7.419E-03	9.930E-04
0.520	7.759E-02	7.350E-03	3.450	5.782E-03	8.327E-04
0.560	7.334E-02	7.333E-03	3.550	5.131E-03	7.528E-04
0.600	8.084E-02	6.890E-03	3.650	4.999E-03	7.359E-04
0.640	8.389E-02	6.757E-03	3.750	4.957E-03	7.109E-04
0.680	7.894E-02	6.236E-03	3.860	3.823E-03	6.107E-04
0.720	8.433E-02	6.033E-03	3.980	3.861E-03	6.522E-04
0.760	8.714E-02	5.700E-03	4.100	3.539E-03	5.827E-04
0.800	7.824E-02	5.321E-03	4.220	2.982E-03	4.563E-04
0.840	6.681E-02	5.317E-03	4.340	2.105E-03	4.455E-04
0.880	6.495E-02	5.146E-03	4.460	1.569E-03	3.842E-04
0.925	6.626E-02	5.065E-03	4.580	9.998E-04	2.970E-04
0.975	6.000E-02	4.863E-03	4.700	9.016E-04	2.580E-04
1.025	6.001E-02	4.617E-03	4.820	9.919E-04	2.308E-04
1.075	6.003E-02	4.343E-03	4.940	9.091E-04	2.476E-04
1.125	5.949E-02	4.132E-03	5.070	8.034E-04	2.350E-04
1.175	6.298E-02	4.122E-03	5.210	6.937E-04	2.307E-04
1.225	5.669E-02	3.959E-03	5.350	4.021E-04	1.604E-04
1.275	4.904E-02	3.607E-03	5.490	1.520E-04	8.419E-05
1.325	5.050E-02	3.454E-03	5.630	1.095E-04	7.567E-05
1.375	4.956E-02	3.434E-03	5.770	1.366E-04	8.336E-05
1.430	4.280E-02	3.236E-03	5.910	1.065E-04	6.957E-05
1.490	4.132E-02	3.057E-03	6.050	8.489E-05	6.274E-05
1.550	4.192E-02	2.893E-03	6.190	8.882E-05	6.017E-05
1.610	3.786E-02	2.810E-03	6.330	8.374E-05	6.027E-05
1.670	3.529E-02	2.629E-03	6.480	6.053E-05	6.141E-05
1.730	3.566E-02	2.809E-03	6.640	3.068E-05	5.810E-05
1.790	3.471E-02	2.743E-03	6.800	1.168E-05	5.953E-05
1.850	2.981E-02	2.636E-03	6.960	8.160E-06	6.049E-05
1.910	2.466E-02	2.564E-03	7.120	9.287E-06	6.257E-05
1.970	2.366E-02	2.168E-03	7.280	6.634E-06	6.473E-05
2.040	2.404E-02	2.355E-03	7.440	4.966E-06	6.698E-05
2.120	2.155E-02	2.098E-03	7.600	2.144E-05	6.755E-05
2.200	2.062E-02	2.048E-03	7.760	4.847E-05	7.562E-05
2.280	2.114E-02	1.900E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 59.7 SEC AFTER END OF IRRADIATION
COUNT FOR 15 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	8.665E-02	1.671E-02	2.360	1.436E-02	1.565E-03
0.190	8.475E-02	1.579E-02	2.440	1.277E-02	1.436E-03
0.210	8.834E-02	1.550E-02	2.520	1.011E-02	1.462E-03
0.230	9.191E-02	1.534E-02	2.600	9.159E-03	1.258E-03
0.250	8.976E-02	1.312E-02	2.680	9.615E-03	1.254E-03
0.275	7.799E-02	1.308E-02	2.760	8.975E-03	1.122E-03
0.305	6.893E-02	1.249E-02	2.840	7.653E-03	1.060E-03
0.335	7.224E-02	1.174E-02	2.920	7.301E-03	1.052E-03
0.365	7.658E-02	1.116E-02	3.000	7.898E-03	1.030E-03
0.395	6.994E-02	1.051E-02	3.080	7.884E-03	1.046E-03
0.425	6.512E-02	8.943E-03	3.160	6.378E-03	8.840E-04
0.455	6.403E-02	7.265E-03	3.250	4.779E-03	7.855E-04
0.485	6.283E-02	6.982E-03	3.350	4.884E-03	8.080E-04
0.520	6.239E-02	6.546E-03	3.450	5.111E-03	7.465E-04
0.560	6.418E-02	6.528E-03	3.550	4.392E-03	7.199E-04
0.600	7.014E-02	6.126E-03	3.650	4.030E-03	6.462E-04
0.640	7.502E-02	6.011E-03	3.750	3.947E-03	7.059E-04
0.690	7.181E-02	5.700E-03	3.860	3.248E-03	5.793E-04
0.740	6.161E-02	5.259E-03	3.980	2.354E-03	5.120E-04
0.760	6.133E-02	5.112E-03	4.100	1.860E-03	4.668E-04
0.800	6.304E-02	4.818E-03	4.220	1.805E-03	4.144E-04
0.840	6.225E-02	4.740E-03	4.340	1.617E-03	4.073E-04
0.880	5.810E-02	4.641E-03	4.460	1.281E-03	3.171E-04
0.925	4.672E-02	4.383E-03	4.580	1.218E-03	3.104E-04
0.975	4.273E-02	4.164E-03	4.700	1.116E-03	2.754E-04
1.025	4.606E-02	3.996E-03	4.820	7.774E-04	2.108E-04
1.075	4.699E-02	3.807E-03	4.940	4.011E-04	1.694E-04
1.125	4.395E-02	3.747E-03	5.070	1.918E-04	9.305E-05
1.175	4.065E-02	3.515E-03	5.210	1.832E-04	9.841E-05
1.225	4.324E-02	3.232E-03	5.350	1.429E-04	8.886E-05
1.275	4.668E-02	3.265E-03	5.490	5.836E-05	6.610E-05
1.325	4.306E-02	3.029E-03	5.630	2.285E-05	5.681E-05
1.375	3.770E-02	3.015E-03	5.770	5.021E-05	5.779E-05
1.430	3.566E-02	2.730E-03	5.910	7.859E-05	6.021E-05
1.490	3.302E-02	2.743E-03	6.050	6.829E-05	5.649E-05
1.550	2.867E-02	2.567E-03	6.190	4.695E-05	5.992E-05
1.610	2.725E-02	2.447E-03	6.330	3.802E-05	6.087E-05
1.670	2.734E-02	2.223E-03	6.480	2.927E-05	5.748E-05
1.730	2.485E-02	2.388E-03	6.640	1.552E-05	5.799E-05
1.790	2.168E-02	2.257E-03	6.800	8.088E-06	5.887E-05
1.850	2.203E-02	2.138E-03	6.960	8.272E-06	6.035E-05
1.910	2.150E-02	2.189E-03	7.120	9.647E-06	6.256E-05
1.970	1.748E-02	1.859E-03	7.280	1.000E-05	6.471E-05
2.040	1.618E-02	1.919E-03	7.440	9.598E-06	6.750E-05
2.120	1.786E-02	1.934E-03	7.600	1.082E-05	6.884E-05
2.200	1.518E-02	1.710E-03	7.760	9.826E-06	7.474E-05
2.280	1.390E-02	1.607E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 75 SEC AFTER END OF IRRADIATION
COUNT FOR 15 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	8.465E-02	1.675E-02	2.360	8.659E-03	1.448E-03
0.190	8.677E-02	1.425E-02	2.440	9.012E-03	1.215E-03
0.210	9.317E-02	1.398E-02	2.520	1.032E-02	1.214E-03
0.230	8.760E-02	1.269E-02	2.600	9.150E-03	1.198E-03
0.250	7.302E-02	1.185E-02	2.680	6.408E-03	1.099E-03
0.275	6.188E-02	1.217E-02	2.760	7.071E-03	9.738E-04
0.305	4.951E-02	1.181E-02	2.840	8.378E-03	1.041E-03
0.335	4.759E-02	1.069E-02	2.920	6.848E-03	9.027E-04
0.365	6.520E-02	1.058E-02	3.000	5.751E-03	9.392E-04
0.395	6.397E-02	1.004E-02	3.080	6.310E-03	8.606E-04
0.425	5.783E-02	8.176E-03	3.160	6.076E-03	8.809E-04
0.455	5.627E-02	6.294E-03	3.250	4.495E-03	7.115E-04
0.485	5.451E-02	5.928E-03	3.350	3.538E-03	7.318E-04
0.520	5.329E-02	5.779E-03	3.450	3.135E-03	6.509E-04
0.560	5.180E-02	5.863E-03	3.550	2.497E-03	5.859E-04
0.600	4.777E-02	5.547E-03	3.650	2.165E-03	5.390E-04
0.640	4.933E-02	5.151E-03	3.750	2.355E-03	4.899E-04
0.680	5.410E-02	4.954E-03	3.860	2.541E-03	4.773E-04
0.720	5.182E-02	4.833E-03	3.980	2.030E-03	4.378E-04
0.760	4.547E-02	4.705E-03	4.100	1.350E-03	3.373E-04
0.800	4.625E-02	4.476E-03	4.220	1.237E-03	3.128E-04
0.840	4.821E-02	4.336E-03	4.340	1.094E-03	3.037E-04
0.880	4.261E-02	4.054E-03	4.460	7.375E-04	2.189E-04
0.925	3.788E-02	3.814E-03	4.580	6.369E-04	2.254E-04
0.975	3.780E-02	3.814E-03	4.700	6.882E-04	2.245E-04
1.025	3.711E-02	3.640E-03	4.820	6.116E-04	1.995E-04
1.075	3.932E-02	3.514E-03	4.940	4.983E-04	1.749E-04
1.125	3.682E-02	3.227E-03	5.070	3.966E-04	1.583E-04
1.175	3.223E-02	3.251E-03	5.210	2.628E-04	1.139E-04
1.225	3.425E-02	2.907E-03	5.350	1.711E-04	1.079E-04
1.275	3.402E-02	2.827E-03	5.490	1.476E-04	1.137E-04
1.325	2.888E-02	2.650E-03	5.630	1.138E-04	8.951E-05
1.375	2.765E-02	2.602E-03	5.770	5.211E-05	6.590E-05
1.430	2.842E-02	2.509E-03	5.910	9.506E-06	5.850E-05
1.490	2.496E-02	2.313E-03	6.050	5.424E-06	5.808E-05
1.550	2.208E-02	2.203E-03	6.190	3.447E-05	6.034E-05
1.610	2.282E-02	2.172E-03	6.330	7.109E-05	6.116E-05
1.670	2.374E-02	2.184E-03	6.480	7.516E-05	6.071E-05
1.730	2.105E-02	2.227E-03	6.640	4.187E-05	5.876E-05
1.790	1.877E-02	2.029E-03	6.800	1.348E-05	5.800E-05
1.850	1.938E-02	2.004E-03	6.960	5.033E-06	6.050E-05
1.910	1.772E-02	1.874E-03	7.120	8.365E-06	6.256E-05
1.970	1.418E-02	1.783E-03	7.280	1.055E-05	6.472E-05
2.040	1.496E-02	1.717E-03	7.440	1.003E-05	6.750E-05
2.120	1.677E-02	1.844E-03	7.600	1.076E-05	6.884E-05
2.200	1.136E-02	1.356E-03	7.760	9.691E-06	7.475E-05
2.280	8.380E-03	1.309E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 90 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	9.370E-02	1.667E-02	2.360	1.024E-02	1.300E-03
0.190	9.193E-02	1.497E-02	2.440	1.041E-02	1.301E-03
0.210	7.963E-02	1.447E-02	2.520	9.467E-03	1.136E-03
0.230	7.208E-02	1.381E-02	2.600	9.295E-03	1.195E-03
0.250	7.639E-02	1.301E-02	2.680	8.456E-03	1.104E-03
0.275	8.175E-02	1.242E-02	2.760	6.970E-03	1.056E-03
0.305	7.860E-02	1.230E-02	2.840	6.474E-03	9.269E-04
0.335	7.447E-02	1.145E-02	2.920	6.683E-03	9.316E-04
0.365	6.459E-02	1.081E-02	3.000	6.627E-03	8.817E-04
0.395	5.723E-02	9.809E-03	3.080	6.193E-03	8.850E-04
0.425	6.605E-02	7.926E-03	3.160	5.322E-03	7.993E-04
0.455	6.694E-02	6.591E-03	3.250	4.290E-03	7.211E-04
0.485	6.167E-02	6.298E-03	3.350	3.939E-03	6.292E-04
0.520	6.330E-02	6.276E-03	3.450	3.775E-03	7.107E-04
0.560	5.851E-02	6.244E-03	3.550	3.163E-03	5.582E-04
0.600	4.453E-02	5.838E-03	3.650	2.988E-03	5.623E-04
0.640	4.944E-02	5.420E-03	3.750	2.883E-03	5.659E-04
0.680	5.919E-02	5.264E-03	3.860	2.311E-03	4.371E-04
0.720	5.813E-02	5.035E-03	3.980	1.638E-03	5.109E-04
0.760	5.597E-02	4.799E-03	4.100	1.146E-03	4.630E-04
0.800	5.526E-02	4.599E-03	4.220	1.241E-03	3.297E-04
0.840	5.349E-02	4.392E-03	4.340	1.368E-03	3.255E-04
0.880	4.395E-02	4.265E-03	4.460	9.474E-04	2.537E-04
0.925	3.522E-02	4.098E-03	4.580	5.867E-04	1.914E-04
0.975	4.157E-02	4.079E-03	4.700	4.792E-04	1.879E-04
1.025	4.184E-02	3.858E-03	4.820	3.503E-04	1.441E-04
1.075	3.452E-02	3.701E-03	4.940	2.191E-04	1.097E-04
1.125	3.771E-02	3.543E-03	5.070	1.619E-04	1.074E-04
1.175	4.030E-02	3.498E-03	5.210	1.513E-04	8.380E-05
1.225	3.533E-02	3.162E-03	5.350	1.363E-04	8.487E-05
1.275	3.354E-02	3.033E-03	5.490	8.771E-05	6.913E-05
1.325	3.286E-02	2.850E-03	5.630	6.134E-05	6.223E-05
1.375	2.992E-02	2.732E-03	5.770	7.094E-05	5.964E-05
1.430	2.960E-02	2.485E-03	5.910	7.057E-05	6.175E-05
1.490	3.049E-02	2.581E-03	6.050	4.295E-05	5.985E-05
1.550	2.850E-02	2.401E-03	6.190	1.587E-05	5.831E-05
1.610	2.594E-02	2.336E-03	6.330	9.808E-06	5.928E-05
1.670	2.290E-02	2.095E-03	6.480	7.395E-06	5.722E-05
1.730	2.046E-02	2.119E-03	6.640	1.033E-05	5.755E-05
1.790	1.943E-02	2.093E-03	6.800	1.015E-05	5.882E-05
1.850	1.759E-02	2.099E-03	6.960	9.634E-06	6.036E-05
1.910	1.509E-02	1.849E-03	7.120	9.362E-06	6.256E-05
1.970	1.473E-02	1.668E-03	7.280	9.608E-06	6.471E-05
2.040	1.683E-02	1.720E-03	7.440	9.595E-06	6.750E-05
2.120	1.650E-02	1.728E-03	7.600	1.092E-05	6.885E-05
2.200	1.142E-02	1.475E-03	7.760	9.844E-06	7.475E-05
2.280	8.887E-03	1.436E-03			

HC002I STOP 0

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 10.7 SEC AFTER END OF IRRADIATION
COUNT FOR 6 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.017E-01	1.257E-02	2.360	2.801E-02	1.851E-03
0.190	9.861E-02	1.173E-02	2.440	2.615E-02	1.770E-03
0.210	9.473E-02	1.140E-02	2.520	2.493E-02	1.777E-03
0.230	8.609E-02	1.078E-02	2.600	2.285E-02	1.599E-03
0.250	8.653E-02	1.033E-02	2.680	2.164E-02	1.560E-03
0.275	8.912E-02	1.050E-02	2.760	1.975E-02	1.435E-03
0.305	8.497E-02	1.006E-02	2.840	1.653E-02	1.294E-03
0.335	8.428E-02	9.633E-03	2.920	1.536E-02	1.226E-03
0.365	7.977E-02	9.008E-03	3.000	1.548E-02	1.281E-03
0.395	8.049E-02	8.383E-03	3.080	1.393E-02	1.168E-03
0.425	7.600E-02	6.853E-03	3.160	1.284E-02	1.122E-03
0.455	7.574E-02	5.845E-03	3.250	1.376E-02	1.137E-03
0.485	7.544E-02	5.675E-03	3.350	1.259E-02	1.036E-03
0.520	7.081E-02	5.387E-03	3.450	1.005E-02	9.582E-04
0.560	8.077E-02	5.224E-03	3.550	1.035E-02	9.353E-04
0.600	8.065E-02	5.097E-03	3.650	9.611E-03	9.268E-04
0.640	8.401E-02	4.879E-03	3.750	7.294E-03	7.856E-04
0.680	8.017E-02	4.876E-03	3.860	6.743E-03	7.995E-04
0.720	7.404E-02	4.480E-03	3.980	6.652E-03	7.122E-04
0.760	7.585E-02	4.287E-03	4.100	5.375E-03	6.717E-04
0.800	7.744E-02	4.126E-03	4.220	4.441E-03	5.591E-04
0.840	7.560E-02	3.981E-03	4.340	4.451E-03	5.894E-04
0.880	6.923E-02	3.876E-03	4.460	4.207E-03	5.330E-04
0.925	6.717E-02	3.601E-03	4.580	3.847E-03	4.909E-04
0.975	7.234E-02	3.728E-03	4.700	3.939E-03	4.845E-04
1.025	7.133E-02	3.642E-03	4.820	3.358E-03	4.471E-04
1.075	6.667E-02	3.433E-03	4.940	2.256E-03	3.559E-04
1.125	6.042E-02	3.220E-03	5.070	1.888E-03	3.508E-04
1.175	5.880E-02	3.028E-03	5.210	2.228E-03	3.563E-04
1.225	6.357E-02	3.040E-03	5.350	2.289E-03	3.547E-04
1.275	6.156E-02	2.892E-03	5.490	1.828E-03	3.092E-04
1.325	5.866E-02	2.849E-03	5.630	1.118E-03	2.373E-04
1.375	5.861E-02	2.760E-03	5.770	6.396E-04	1.761E-04
1.430	5.863E-02	2.679E-03	5.910	5.199E-04	1.688E-04
1.490	5.771E-02	2.548E-03	6.050	5.602E-04	1.679E-04
1.550	5.156E-02	2.408E-03	6.190	5.949E-04	1.632E-04
1.610	4.973E-02	2.352E-03	6.330	5.420E-04	1.563E-04
1.670	5.027E-02	2.386E-03	6.480	3.912E-04	1.249E-04
1.730	4.961E-02	2.543E-03	6.640	2.244E-04	8.805E-05
1.790	4.343E-02	2.688E-03	6.800	1.049E-04	7.342E-05
1.850	4.561E-02	2.530E-03	6.960	4.363E-05	4.968E-05
1.910	4.144E-02	2.508E-03	7.120	4.525E-05	4.570E-05
1.970	3.550E-02	2.409E-03	7.280	5.953E-05	4.679E-05
2.040	3.497E-02	2.232E-03	7.440	4.702E-05	4.742E-05
2.120	3.619E-02	2.216E-03	7.600	2.000E-05	4.848E-05
2.200	3.350E-02	2.110E-03	7.760	3.397E-06	5.211E-05
2.280	3.100E-02	2.022E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10 SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 16.7 SEC AFTER END OF IRRADIATION
COUNT FOR 8 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.219E-01	1.267E-02	2.360	2.427E-02	1.788E-03
0.190	1.044E-01	1.218E-02	2.440	2.163E-02	1.576E-03
0.210	7.365E-02	1.155E-02	2.520	2.203E-02	1.602E-03
0.230	7.438E-02	1.112E-02	2.600	2.079E-02	1.582E-03
0.250	8.137E-02	1.047E-02	2.680	1.663E-02	1.287E-03
0.275	7.429E-02	1.035E-02	2.760	1.554E-02	1.318E-03
0.305	6.313E-02	1.004E-02	2.840	1.569E-02	1.251E-03
0.335	7.163E-02	9.208E-03	2.920	1.499E-02	1.210E-03
0.365	8.881E-02	8.628E-03	3.000	1.456E-02	1.174E-03
0.395	8.021E-02	8.290E-03	3.080	1.324E-02	1.108E-03
0.425	8.089E-02	6.949E-03	3.160	1.128E-02	1.034E-03
0.455	8.416E-02	5.809E-03	3.250	1.111E-02	9.727E-04
0.485	8.724E-02	5.593E-03	3.350	1.171E-02	1.058E-03
0.520	8.713E-02	5.481E-03	3.450	9.792E-03	8.945E-04
0.560	8.029E-02	5.526E-03	3.550	8.363E-03	8.215E-04
0.600	7.979E-02	5.289E-03	3.650	8.574E-03	8.385E-04
0.640	7.601E-02	5.036E-03	3.750	7.826E-03	7.948E-04
0.680	8.213E-02	4.755E-03	3.860	6.045E-03	7.134E-04
0.720	9.152E-02	4.636E-03	3.980	5.433E-03	6.982E-04
0.760	8.510E-02	4.397E-03	4.100	5.360E-03	6.262E-04
0.800	7.544E-02	4.032E-03	4.220	4.317E-03	5.812E-04
0.840	7.359E-02	3.940E-03	4.340	3.416E-03	4.697E-04
0.880	7.246E-02	3.891E-03	4.460	3.041E-03	4.611E-04
0.925	7.087E-02	3.792E-03	4.580	2.543E-03	3.951E-04
0.975	7.027E-02	3.719E-03	4.700	1.975E-03	3.711E-04
1.025	6.771E-02	3.543E-03	4.820	1.631E-03	3.079E-04
1.075	6.354E-02	3.342E-03	4.940	1.644E-03	3.112E-04
1.125	6.325E-02	3.239E-03	5.070	1.583E-03	3.064E-04
1.175	6.125E-02	3.133E-03	5.210	1.213E-03	2.500E-04
1.225	5.719E-02	2.925E-03	5.350	1.030E-03	2.487E-04
1.275	5.679E-02	2.863E-03	5.490	9.786E-04	2.183E-04
1.325	5.652E-02	2.784E-03	5.630	7.891E-04	2.048E-04
1.375	5.405E-02	2.691E-03	5.770	5.809E-04	1.625E-04
1.430	5.421E-02	2.559E-03	5.910	4.788E-04	1.519E-04
1.490	5.478E-02	2.539E-03	6.050	4.085E-04	1.338E-04
1.550	4.968E-02	2.431E-03	6.190	3.240E-04	1.176E-04
1.610	4.669E-02	2.231E-03	6.330	2.461E-04	9.161E-05
1.670	4.630E-02	2.200E-03	6.480	1.662E-04	7.664E-05
1.730	4.118E-02	2.429E-03	6.640	7.296E-05	5.461E-05
1.790	3.658E-02	2.404E-03	6.800	2.625E-05	4.304E-05
1.850	3.507E-02	2.354E-03	6.960	4.247E-05	4.469E-05
1.910	3.380E-02	2.214E-03	7.120	7.909E-05	4.521E-05
1.970	3.323E-02	2.250E-03	7.280	7.779E-05	4.734E-05
2.040	3.143E-02	2.088E-03	7.440	3.914E-05	4.789E-05
2.120	2.712E-02	2.067E-03	7.600	9.748E-06	4.736E-05
2.200	2.523E-02	1.902E-03	7.760	8.589E-07	5.208E-05
2.280	2.530E-02	1.857E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 24.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.188E-01	1.194E-02	2.360	2.248E-02	1.621E-03
0.190	8.809E-02	1.173E-02	2.440	1.909E-02	1.597E-03
0.210	6.862E-02	1.111E-02	2.520	1.794E-02	1.382E-03
0.230	7.586E-02	1.089E-02	2.600	1.667E-02	1.421E-03
0.250	9.029E-02	9.774E-03	2.680	1.492E-02	1.235E-03
0.275	8.879E-02	9.966E-03	2.760	1.526E-02	1.266E-03
0.305	8.300E-02	9.811E-03	2.840	1.434E-02	1.178E-03
0.335	8.344E-02	9.017E-03	2.920	1.168E-02	1.098E-03
0.365	7.655E-02	8.500E-03	3.000	1.098E-02	1.014E-03
0.395	8.706E-02	7.972E-03	3.080	1.155E-02	1.016E-03
0.425	8.495E-02	6.920E-03	3.160	1.063E-02	9.956E-04
0.455	7.538E-02	5.858E-03	3.250	8.740E-03	8.835E-04
0.485	8.179E-02	5.562E-03	3.350	8.210E-03	9.001E-04
0.520	9.049E-02	5.452E-03	3.450	8.184E-03	8.605E-04
0.560	8.158E-02	5.421E-03	3.550	7.237E-03	7.341E-04
0.600	7.297E-02	5.166E-03	3.650	6.006E-03	7.349E-04
0.640	7.795E-02	4.865E-03	3.750	5.178E-03	6.756E-04
0.680	8.505E-02	4.618E-03	3.860	5.033E-03	6.215E-04
0.720	8.327E-02	4.423E-03	3.980	4.716E-03	5.893E-04
0.760	7.957E-02	4.287E-03	4.100	3.562E-03	5.211E-04
0.800	7.348E-02	4.096E-03	4.220	2.665E-03	4.548E-04
0.840	7.168E-02	3.893E-03	4.340	2.432E-03	4.381E-04
0.880	7.011E-02	3.869E-03	4.460	2.390E-03	3.923E-04
0.925	6.636E-02	3.595E-03	4.580	2.020E-03	3.448E-04
0.975	6.376E-02	3.522E-03	4.700	1.493E-03	2.992E-04
1.025	6.082E-02	3.467E-03	4.820	1.226E-03	2.506E-04
1.075	5.891E-02	3.271E-03	4.940	1.052E-03	2.565E-04
1.125	5.635E-02	3.158E-03	5.070	7.688E-04	2.140E-04
1.175	5.756E-02	2.998E-03	5.210	5.491E-04	1.709E-04
1.225	5.768E-02	2.948E-03	5.350	4.163E-04	1.442E-04
1.275	5.350E-02	2.795E-03	5.490	3.453E-04	1.200E-04
1.325	5.205E-02	2.741E-03	5.630	3.394E-04	1.285E-04
1.375	5.203E-02	2.568E-03	5.770	3.060E-04	1.125E-04
1.430	4.988E-02	2.512E-03	5.910	2.321E-04	8.987E-05
1.490	4.782E-02	2.436E-03	6.050	1.489E-04	7.633E-05
1.550	4.711E-02	2.350E-03	6.190	7.942E-05	5.528E-05
1.610	4.297E-02	2.228E-03	6.330	6.437E-05	4.514E-05
1.670	3.904E-02	2.064E-03	6.480	4.525E-05	4.381E-05
1.730	3.872E-02	2.408E-03	6.640	5.252E-05	4.223E-05
1.790	3.636E-02	2.425E-03	6.800	4.328E-05	4.237E-05
1.850	3.192E-02	2.274E-03	6.960	3.207E-05	4.295E-05
1.910	3.089E-02	2.147E-03	7.120	2.802E-05	4.293E-05
1.970	3.045E-02	2.192E-03	7.280	3.258E-05	4.513E-05
2.040	2.718E-02	2.080E-03	7.440	3.497E-05	4.842E-05
2.120	2.572E-02	1.998E-03	7.600	2.521E-05	4.672E-05
2.200	2.552E-02	1.886E-03	7.760	1.061E-05	5.243E-05
2.280	2.461E-02	1.798E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 34.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	9.399E-02	1.123E-02	2.360	1.606E-02	1.407E-03
0.190	8.132E-02	1.040E-02	2.440	1.495E-02	1.348E-03
0.210	7.058E-02	9.766E-03	2.520	1.313E-02	1.241E-03
0.230	6.186E-02	9.362E-03	2.600	1.195E-02	1.097E-03
0.250	6.764E-02	8.897E-03	2.680	1.223E-02	1.116E-03
0.275	7.770E-02	9.026E-03	2.760	1.060E-02	1.070E-03
0.305	6.812E-02	8.755E-03	2.840	8.802E-03	9.722E-04
0.335	6.743E-02	8.140E-03	2.920	9.135E-03	9.460E-04
0.365	6.299E-02	7.551E-03	3.000	9.062E-03	9.198E-04
0.395	5.950E-02	6.958E-03	3.080	7.887E-03	8.740E-04
0.425	6.374E-02	6.099E-03	3.160	7.358E-03	8.185E-04
0.455	5.930E-02	5.174E-03	3.250	7.022E-03	7.881E-04
0.485	5.719E-02	4.767E-03	3.350	5.744E-03	6.818E-04
0.520	6.090E-02	4.746E-03	3.450	4.945E-03	6.658E-04
0.560	6.264E-02	4.813E-03	3.550	4.673E-03	6.291E-04
0.600	6.110E-02	4.532E-03	3.650	4.419E-03	5.663E-04
0.640	6.617E-02	4.402E-03	3.750	4.055E-03	5.718E-04
0.680	6.884E-02	4.293E-03	3.860	3.361E-03	5.321E-04
0.720	6.652E-02	4.004E-03	3.980	2.964E-03	4.730E-04
0.760	6.447E-02	3.766E-03	4.100	2.762E-03	4.346E-04
0.800	5.504E-02	3.575E-03	4.220	1.949E-03	3.603E-04
0.840	4.989E-02	3.446E-03	4.340	1.182E-03	2.878E-04
0.880	5.119E-02	3.382E-03	4.460	1.204E-03	2.613E-04
0.925	5.023E-02	3.319E-03	4.580	1.508E-03	2.817E-04
0.975	4.906E-02	3.135E-03	4.700	1.300E-03	2.553E-04
1.025	5.220E-02	3.049E-03	4.820	7.739E-04	1.778E-04
1.075	5.050E-02	2.960E-03	4.940	3.796E-04	1.497E-04
1.125	4.372E-02	2.756E-03	5.070	2.517E-04	1.503E-04
1.175	4.310E-02	2.549E-03	5.210	3.881E-04	1.366E-04
1.225	4.269E-02	2.547E-03	5.350	5.198E-04	1.473E-04
1.275	4.070E-02	2.456E-03	5.490	3.877E-04	1.198E-04
1.325	3.997E-02	2.311E-03	5.630	1.974E-04	8.925E-05
1.375	3.821E-02	2.271E-03	5.770	1.295E-04	6.583E-05
1.430	3.544E-02	2.128E-03	5.910	1.170E-04	8.103E-05
1.490	3.383E-02	2.078E-03	6.050	6.577E-05	7.076E-05
1.550	3.258E-02	1.987E-03	6.190	1.757E-05	5.076E-05
1.610	2.962E-02	1.926E-03	6.330	-5.184E-07	4.328E-05
1.670	2.876E-02	1.741E-03	6.480	4.956E-06	3.975E-05
1.730	2.980E-02	2.109E-03	6.640	1.748E-05	4.140E-05
1.790	2.731E-02	2.001E-03	6.800	2.906E-05	4.265E-05
1.850	2.425E-02	1.817E-03	6.960	2.982E-05	4.582E-05
1.910	2.286E-02	1.889E-03	7.120	1.615E-05	4.434E-05
1.970	2.138E-02	1.801E-03	7.280	6.779E-06	4.571E-05
2.040	2.102E-02	1.678E-03	7.440	4.935E-06	4.702E-05
2.120	2.077E-02	1.702E-03	7.600	7.148E-06	4.787E-05
2.200	1.889E-02	1.607E-03	7.760	7.313E-06	5.207E-05
2.280	1.767E-02	1.502E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 44.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	7.817E-02	9.915E-03	2.360	1.381E-02	1.326E-03
0.190	7.066E-02	9.201E-03	2.440	1.184E-02	1.124E-03
0.210	5.837E-02	8.912E-03	2.520	1.060E-02	1.050E-03
0.230	5.329E-02	8.541E-03	2.600	9.991E-03	1.012E-03
0.250	5.546E-02	8.028E-03	2.680	8.580E-03	9.644E-04
0.275	6.004E-02	7.719E-03	2.760	7.894E-03	8.854E-04
0.305	6.612E-02	7.650E-03	2.840	8.651E-03	8.931E-04
0.335	7.047E-02	7.355E-03	2.920	8.413E-03	8.324E-04
0.365	5.884E-02	7.065E-03	3.000	6.549E-03	7.700E-04
0.395	6.131E-02	6.680E-03	3.080	5.581E-03	6.519E-04
0.425	6.358E-02	5.487E-03	3.160	6.144E-03	7.075E-04
0.455	5.101E-02	4.639E-03	3.250	5.920E-03	7.033E-04
0.485	4.818E-02	4.400E-03	3.350	4.242E-03	6.073E-04
0.520	5.490E-02	4.372E-03	3.450	3.321E-03	5.700E-04
0.560	5.433E-02	4.421E-03	3.550	3.292E-03	5.359E-04
0.600	5.155E-02	4.134E-03	3.650	3.831E-03	5.231E-04
0.640	5.260E-02	3.991E-03	3.750	4.040E-03	5.443E-04
0.680	5.297E-02	3.864E-03	3.860	3.083E-03	4.573E-04
0.720	5.606E-02	3.665E-03	3.980	2.026E-03	3.871E-04
0.760	5.428E-02	3.498E-03	4.100	1.557E-03	3.019E-04
0.800	4.647E-02	3.172E-03	4.220	1.256E-03	2.834E-04
0.840	4.496E-02	3.049E-03	4.340	1.045E-03	2.490E-04
0.880	4.660E-02	3.134E-03	4.460	1.068E-03	2.410E-04
0.925	4.464E-02	2.999E-03	4.580	1.040E-03	2.360E-04
0.975	4.028E-02	2.825E-03	4.700	7.668E-04	1.997E-04
1.025	3.845E-02	2.714E-03	4.820	5.747E-04	1.700E-04
1.075	4.073E-02	2.722E-03	4.940	5.565E-04	1.696E-04
1.125	3.863E-02	2.561E-03	5.070	4.820E-04	1.475E-04
1.175	3.274E-02	2.388E-03	5.210	3.058E-04	1.131E-04
1.225	3.275E-02	2.308E-03	5.350	1.815E-04	8.564E-05
1.275	3.561E-02	2.224E-03	5.490	1.207E-04	6.959E-05
1.325	3.490E-02	2.131E-03	5.630	8.679E-05	5.398E-05
1.375	3.161E-02	2.018E-03	5.770	6.136E-05	4.871E-05
1.430	2.842E-02	1.885E-03	5.910	3.243E-05	4.466E-05
1.490	2.654E-02	1.793E-03	6.050	1.200E-05	4.100E-05
1.550	2.545E-02	1.731E-03	6.190	4.131E-06	4.143E-05
1.610	2.471E-02	1.654E-03	6.330	5.041E-06	4.123E-05
1.670	2.338E-02	1.596E-03	6.480	6.700E-06	3.983E-05
1.730	2.123E-02	1.696E-03	6.640	6.999E-06	4.003E-05
1.790	1.939E-02	1.772E-03	6.800	6.682E-06	4.089E-05
1.850	1.781E-02	1.681E-03	6.960	6.690E-06	4.195E-05
1.910	1.756E-02	1.612E-03	7.120	6.592E-06	4.348E-05
1.970	1.838E-02	1.627E-03	7.280	6.691E-06	4.497E-05
2.040	1.826E-02	1.500E-03	7.440	6.647E-06	4.691E-05
2.120	1.542E-02	1.468E-03	7.600	7.578E-06	4.785E-05
2.200	1.244E-02	1.271E-03	7.760	6.843E-06	5.194E-05
2.280	1.329E-02	1.320E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 54.7 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.120E-01	1.262E-02	2.360	1.727E-02	1.490E-03
0.190	1.203E-01	1.162E-02	2.440	1.788E-02	1.465E-03
0.210	1.144E-01	1.113E-02	2.520	1.859E-02	1.468E-03
0.230	9.766E-02	1.112E-02	2.600	1.470E-02	1.320E-03
0.250	9.585E-02	1.045E-02	2.680	1.184E-02	1.206E-03
0.275	8.826E-02	1.011E-02	2.760	1.266E-02	1.149E-03
0.305	7.102E-02	9.745E-03	2.840	1.276E-02	1.122E-03
0.335	8.021E-02	8.915E-03	2.920	1.193E-02	1.024E-03
0.365	8.627E-02	8.525E-03	3.000	1.193E-02	1.064E-03
0.395	9.164E-02	8.031E-03	3.080	1.120E-02	9.515E-04
0.425	8.975E-02	6.932E-03	3.160	9.258E-03	9.203E-04
0.455	8.757E-02	5.862E-03	3.250	7.290E-03	7.765E-04
0.485	8.143E-02	5.590E-03	3.350	6.241E-03	7.897E-04
0.520	7.463E-02	5.505E-03	3.450	5.654E-03	7.180E-04
0.560	7.928E-02	5.354E-03	3.550	4.936E-03	7.117E-04
0.600	8.304E-02	5.280E-03	3.650	4.379E-03	6.460E-04
0.640	8.636E-02	4.967E-03	3.750	4.194E-03	5.953E-04
0.680	8.441E-02	4.748E-03	3.860	3.667E-03	5.096E-04
0.720	9.120E-02	4.572E-03	3.980	3.073E-03	5.002E-04
0.760	8.571E-02	4.449E-03	4.100	2.926E-03	4.545E-04
0.800	7.553E-02	4.075E-03	4.220	2.286E-03	3.906E-04
0.840	7.624E-02	3.972E-03	4.340	1.583E-03	3.096E-04
0.880	6.887E-02	3.852E-03	4.460	1.381E-03	3.228E-04
0.925	6.106E-02	3.721E-03	4.580	1.296E-03	2.912E-04
0.975	6.369E-02	3.644E-03	4.700	1.094E-03	2.636E-04
1.025	6.057E-02	3.470E-03	4.820	7.734E-04	1.938E-04
1.075	5.918E-02	3.313E-03	4.940	5.672E-04	1.783E-04
1.125	6.156E-02	3.152E-03	5.070	5.908E-04	1.674E-04
1.175	5.842E-02	3.139E-03	5.210	6.087E-04	1.720E-04
1.225	5.356E-02	2.975E-03	5.350	4.451E-04	1.344E-04
1.275	5.107E-02	2.771E-03	5.490	2.733E-04	1.040E-04
1.325	4.884E-02	2.654E-03	5.630	1.633E-04	8.464E-05
1.375	4.722E-02	2.534E-03	5.770	8.966E-05	5.511E-05
1.430	4.537E-02	2.417E-03	5.910	5.861E-05	4.816E-05
1.490	4.310E-02	2.309E-03	6.050	4.757E-05	4.560E-05
1.550	4.195E-02	2.234E-03	6.190	5.114E-05	4.294E-05
1.610	3.932E-02	2.116E-03	6.330	5.497E-05	4.339E-05
1.670	3.569E-02	1.960E-03	6.480	3.899E-05	4.026E-05
1.730	3.422E-02	2.241E-03	6.640	1.397E-05	4.050E-05
1.790	3.236E-02	2.291E-03	6.800	3.518E-06	4.107E-05
1.850	2.850E-02	2.069E-03	6.960	4.864E-06	4.197E-05
1.910	2.638E-02	2.029E-03	7.120	7.055E-06	4.352E-05
1.970	2.614E-02	1.950E-03	7.280	7.236E-06	4.497E-05
2.040	2.483E-02	1.829E-03	7.440	6.628E-06	4.692E-05
2.120	2.288E-02	1.831E-03	7.600	7.427E-06	4.785E-05
2.200	2.233E-02	1.801E-03	7.760	6.822E-06	5.194E-05
2.280	2.038E-02	1.618E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 95 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(T)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(T)
0.170	7.669E-02	9.869E-03	2.360	9.294E-03	1.140E-03
0.190	5.459E-02	9.389E-03	2.440	7.980E-03	1.055E-03
0.210	5.548E-02	8.959E-03	2.520	8.742E-03	9.497E-04
0.230	6.402E-02	8.374E-03	2.600	8.679E-03	9.238E-04
0.250	6.191E-02	8.310E-03	2.680	7.244E-03	8.337E-04
0.275	5.515E-02	7.883E-03	2.760	6.481E-03	8.389E-04
0.305	5.154E-02	7.739E-03	2.840	6.195E-03	8.235E-04
0.335	6.024E-02	7.112E-03	2.920	5.829E-03	8.130E-04
0.365	5.311E-02	6.743E-03	3.000	5.422E-03	7.504E-04
0.395	4.745E-02	6.131E-03	3.080	5.144E-03	6.599E-04
0.425	5.613E-02	5.503E-03	3.160	5.105E-03	6.848E-04
0.455	5.736E-02	4.765E-03	3.250	4.580E-03	6.532E-04
0.485	5.197E-02	4.515E-03	3.350	3.265E-03	5.150E-04
0.520	5.192E-02	4.377E-03	3.450	2.633E-03	4.864E-04
0.560	5.277E-02	4.289E-03	3.550	2.858E-03	5.157E-04
0.600	4.587E-02	4.130E-03	3.650	2.861E-03	4.965E-04
0.640	4.526E-02	3.903E-03	3.750	2.529E-03	4.783E-04
0.680	4.929E-02	3.701E-03	3.860	2.220E-03	4.340E-04
0.720	4.822E-02	3.545E-03	3.980	1.727E-03	3.389E-04
0.760	4.366E-02	3.369E-03	4.100	1.196E-03	2.730E-04
0.800	4.273E-02	3.168E-03	4.220	1.195E-03	2.640E-04
0.840	4.541E-02	3.059E-03	4.340	1.206E-03	2.642E-04
0.880	4.449E-02	3.043E-03	4.460	7.918E-04	2.429E-04
0.925	3.759E-02	2.834E-03	4.580	5.076E-04	1.676E-04
0.975	3.566E-02	2.802E-03	4.700	5.669E-04	1.701E-04
1.025	3.927E-02	2.791E-03	4.820	5.488E-04	1.643E-04
1.075	3.901E-02	2.519E-03	4.990	3.654E-04	1.317E-04
1.125	3.623E-02	2.459E-03	5.070	2.896E-04	1.131E-04
1.175	3.368E-02	2.440E-03	5.210	3.539E-04	1.246E-04
1.225	3.173E-02	2.189E-03	5.350	2.812E-04	1.033E-04
1.275	3.038E-02	2.012E-03	5.490	1.439E-04	6.716E-05
1.325	2.849E-02	1.987E-03	5.630	7.513E-05	5.123E-05
1.375	2.724E-02	1.915E-03	5.770	8.760E-05	4.548E-05
1.430	2.661E-02	1.785E-03	5.910	2.610E-05	4.285E-05
1.490	2.411E-02	1.728E-03	6.050	9.982E-06	4.101E-05
1.550	2.233E-02	1.677E-03	6.190	4.207E-06	4.151E-05
1.610	2.211E-02	1.645E-03	6.330	5.548E-06	4.129E-05
1.670	2.085E-02	1.509E-03	6.480	6.784E-06	3.983E-05
1.730	1.955E-02	1.774E-03	6.640	6.902E-06	4.004E-05
1.790	1.883E-02	1.695E-03	6.800	6.654E-06	4.089E-05
1.850	1.705E-02	1.686E-03	6.960	6.706E-06	4.195E-05
1.910	1.348E-02	1.608E-03	7.120	6.599E-06	4.348E-05
1.970	1.085E-02	1.424E-03	7.280	6.686E-06	4.497E-05
2.040	1.191E-02	1.388E-03	7.440	6.642E-06	4.691E-05
2.120	1.314E-02	1.378E-03	7.600	7.576E-06	4.785E-05
2.200	1.126E-02	1.234E-03	7.760	6.841E-06	5.194E-05
2.280	1.052E-02	1.184E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 75 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(T)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(T)
0.170	8.991E-02	1.143E-02	2.360	1.418E-02	1.261E-03
0.190	8.431E-02	1.042E-02	2.440	1.320E-02	1.265E-03
0.210	6.995E-02	9.995E-03	2.520	1.176E-02	1.147E-03
0.230	7.207E-02	9.686E-03	2.600	9.965E-03	1.034E-03
0.250	8.406E-02	9.148E-03	2.680	9.355E-03	1.037E-03
0.275	8.090E-02	8.819E-03	2.760	9.059E-03	9.669E-04
0.305	6.984E-02	8.559E-03	2.840	8.625E-03	9.340E-04
0.335	6.434E-02	8.200E-03	2.920	7.773E-03	8.725E-04
0.365	6.253E-02	7.759E-03	3.000	6.510E-03	8.497E-04
0.395	7.462E-02	7.155E-03	3.080	6.436E-03	7.562E-04
0.425	7.227E-02	6.026E-03	3.160	7.207E-03	7.751E-04
0.455	6.392E-02	5.224E-03	3.250	6.597E-03	7.398E-04
0.485	6.371E-02	5.057E-03	3.350	5.264E-03	6.706E-04
0.520	6.273E-02	4.792E-03	3.450	4.867E-03	6.436E-04
0.560	6.187E-02	4.748E-03	3.550	3.903E-03	5.891E-04
0.600	6.429E-02	4.687E-03	3.650	2.972E-03	4.789E-04
0.640	6.602E-02	4.378E-03	3.750	2.960E-03	4.780E-04
0.680	6.368E-02	4.093E-03	3.860	3.004E-03	4.763E-04
0.720	6.186E-02	3.916E-03	3.980	2.649E-03	4.322E-04
0.760	5.995E-02	3.785E-03	4.100	1.993E-03	3.640E-04
0.800	5.667E-02	3.622E-03	4.220	1.401E-03	2.884E-04
0.840	5.383E-02	3.403E-03	4.340	1.382E-03	3.051E-04
0.880	5.094E-02	3.349E-03	4.460	1.238E-03	2.659E-04
0.925	4.820E-02	3.199E-03	4.580	8.282E-04	2.060E-04
0.975	4.983E-02	3.101E-03	4.700	5.902E-04	1.837E-04
1.025	5.237E-02	3.028E-03	4.820	6.270E-04	1.755E-04
1.075	4.948E-02	2.835E-03	4.940	6.194E-04	1.802E-04
1.125	4.443E-02	2.772E-03	5.070	4.772E-04	1.401E-04
1.175	4.164E-02	2.636E-03	5.210	3.388E-04	1.221E-04
1.225	4.265E-02	2.584E-03	5.350	2.054E-04	9.157E-05
1.275	4.242E-02	2.448E-03	5.490	9.730E-05	6.187E-05
1.325	3.894E-02	2.279E-03	5.630	7.946E-05	5.280E-05
1.375	3.590E-02	2.165E-03	5.770	9.779E-05	5.893E-05
1.430	3.448E-02	2.096E-03	5.910	7.218E-05	4.859E-05
1.490	3.274E-02	1.973E-03	6.050	2.806E-05	4.301E-05
1.550	3.009E-02	1.911E-03	6.190	1.071E-05	4.122E-05
1.610	2.779E-02	1.762E-03	6.330	2.027E-05	4.166E-05
1.670	2.571E-02	1.637E-03	6.480	3.251E-05	4.229E-05
1.730	2.317E-02	1.864E-03	6.640	2.528E-05	4.029E-05
1.790	2.251E-02	1.962E-03	6.800	1.024E-05	4.137E-05
1.850	2.382E-02	1.844E-03	6.960	4.484E-06	4.203E-05
1.910	2.238E-02	1.727E-03	7.120	5.499E-06	4.350E-05
1.970	1.841E-02	1.608E-03	7.280	7.030E-06	4.499E-05
2.040	1.648E-02	1.642E-03	7.440	6.995E-06	4.691E-05
2.120	1.631E-02	1.498E-03	7.600	7.558E-06	4.785E-05
2.200	1.555E-02	1.444E-03	7.760	6.744E-06	5.194E-05
2.280	1.535E-02	1.448E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 115 SEC AFTER END OF IRRADIATION
COUNT FOR 40 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.203E-01	1.276E-02	2.360	1.563E-02	1.367E-03
0.190	1.093E-01	1.207E-02	2.440	1.601E-02	1.401E-03
0.210	1.186E-01	1.102E-02	2.520	1.442E-02	1.226E-03
0.230	1.112E-01	1.087E-02	2.600	1.174E-02	1.100E-03
0.250	8.549E-02	1.055E-02	2.680	1.015E-02	1.050E-03
0.275	8.926E-02	9.984E-03	2.760	8.632E-03	9.214E-04
0.305	1.111E-01	9.604E-03	2.840	8.971E-03	9.357E-04
0.335	8.839E-02	9.234E-03	2.920	9.777E-03	9.451E-04
0.365	8.552E-02	8.551E-03	3.000	8.288E-03	8.780E-04
0.395	8.972E-02	8.010E-03	3.080	6.685E-03	8.262E-04
0.425	8.463E-02	7.036E-03	3.160	6.802E-03	7.827E-04
0.455	7.976E-02	5.954E-03	3.250	6.660E-03	7.912E-04
0.485	7.818E-02	5.794E-03	3.350	4.769E-03	6.682E-04
0.520	8.056E-02	5.484E-03	3.450	4.110E-03	6.594E-04
0.560	7.660E-02	5.395E-03	3.550	4.961E-03	6.272E-04
0.600	7.637E-02	5.222E-03	3.650	4.727E-03	5.870E-04
0.640	7.488E-02	4.947E-03	3.750	3.322E-03	5.176E-04
0.680	7.393E-02	4.615E-03	3.860	2.131E-03	4.455E-04
0.720	7.188E-02	4.399E-03	3.980	1.694E-03	4.048E-04
0.760	7.140E-02	4.190E-03	4.100	1.524E-03	3.078E-04
0.800	7.197E-02	4.074E-03	4.220	1.259E-03	3.055E-04
0.840	6.326E-02	3.829E-03	4.340	9.235E-04	2.337E-04
0.880	5.716E-02	3.739E-03	4.460	7.697E-04	2.103E-04
0.925	6.005E-02	3.477E-03	4.580	7.108E-04	2.032E-04
0.975	5.881E-02	3.462E-03	4.700	6.526E-04	1.926E-04
1.025	5.320E-02	3.332E-03	4.820	7.173E-04	2.067E-04
1.075	5.173E-02	3.150E-03	4.940	7.663E-04	2.050E-04
1.125	5.126E-02	3.043E-03	5.070	5.596E-04	1.700E-04
1.175	4.900E-02	2.884E-03	5.210	2.996E-04	1.026E-04
1.225	4.379E-02	2.701E-03	5.350	2.630E-04	1.144E-04
1.275	4.197E-02	2.499E-03	5.490	2.824E-04	1.060E-04
1.325	4.410E-02	2.395E-03	5.630	2.105E-04	8.159E-05
1.375	4.222E-02	2.405E-03	5.770	1.042E-04	6.436E-05
1.430	3.834E-02	2.204E-03	5.910	3.507E-05	4.888E-05
1.490	3.833E-02	2.157E-03	6.050	1.406E-05	4.210E-05
1.550	3.669E-02	2.113E-03	6.190	1.990E-05	4.195E-05
1.610	3.232E-02	2.027E-03	6.330	2.705E-05	4.280E-05
1.670	2.894E-02	1.818E-03	6.480	2.667E-05	3.947E-05
1.730	2.759E-02	2.019E-03	6.640	2.662E-05	4.069E-05
1.790	2.731E-02	2.040E-03	6.800	2.998E-05	4.263E-05
1.850	2.450E-02	1.945E-03	6.960	2.406E-05	4.211E-05
1.910	2.053E-02	1.853E-03	7.120	1.139E-05	4.395E-05
1.970	2.031E-02	1.848E-03	7.280	5.090E-06	4.504E-05
2.040	2.176E-02	1.652E-03	7.440	5.212E-06	4.691E-05
2.120	1.908E-02	1.708E-03	7.600	7.674E-06	4.785E-05
2.200	1.778E-02	1.566E-03	7.760	7.255E-06	5.194E-05
2.280	1.741E-02	1.498E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 155 SEC AFTER END OF IRRADIATION
COUNT FOR 60 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.517E-01	1.314E-02	2.360	1.712E-02	1.545E-03
0.190	1.259E-01	1.265E-02	2.440	1.488E-02	1.357E-03
0.210	1.211E-01	1.216E-02	2.520	1.269E-02	1.254E-03
0.230	1.256E-01	1.107E-02	2.600	1.267E-02	1.194E-03
0.250	1.043E-01	1.066E-02	2.680	1.196E-02	1.148E-03
0.275	8.912E-02	1.019E-02	2.760	1.077E-02	1.099E-03
0.305	8.822E-02	9.950E-03	2.840	1.017E-02	1.038E-03
0.335	9.968E-02	9.447E-03	2.920	9.765E-03	9.816E-04
0.365	1.169E-01	9.111E-03	3.000	9.222E-03	9.163E-04
0.395	1.041E-01	8.509E-03	3.080	8.581E-03	9.275E-04
0.425	9.389E-02	7.193E-03	3.160	7.686E-03	8.586E-04
0.455	9.310E-02	6.267E-03	3.250	6.536E-03	8.350E-04
0.485	9.079E-02	5.910E-03	3.350	5.482E-03	7.411E-04
0.520	8.575E-02	5.863E-03	3.450	4.931E-03	6.541E-04
0.560	8.354E-02	5.722E-03	3.550	4.750E-03	6.167E-04
0.600	8.229E-02	5.493E-03	3.650	4.366E-03	5.590E-04
0.640	7.859E-02	5.253E-03	3.750	3.401E-03	5.226E-04
0.680	8.075E-02	4.850E-03	3.860	2.223E-03	4.298E-04
0.720	8.374E-02	4.659E-03	3.980	1.747E-03	3.794E-04
0.760	7.491E-02	4.345E-03	4.100	1.818E-03	3.323E-04
0.800	7.145E-02	4.112E-03	4.220	1.595E-03	3.184E-04
0.840	7.067E-02	3.923E-03	4.340	1.291E-03	2.600E-04
0.880	6.576E-02	3.810E-03	4.460	1.216E-03	2.669E-04
0.925	6.401E-02	3.607E-03	4.580	9.955E-04	2.265E-04
0.975	6.371E-02	3.622E-03	4.700	6.731E-04	1.798E-04
1.025	5.988E-02	3.341E-03	4.820	8.646E-04	1.584E-04
1.075	5.464E-02	3.168E-03	4.940	3.565E-04	1.358E-04
1.125	5.126E-02	3.054E-03	5.070	3.484E-04	1.239E-04
1.175	4.903E-02	2.849E-03	5.210	3.656E-04	1.240E-04
1.225	4.629E-02	2.783E-03	5.350	2.504E-04	1.020E-04
1.275	4.376E-02	2.568E-03	5.490	1.264E-04	5.870E-05
1.325	4.231E-02	2.461E-03	5.630	8.886E-05	6.088E-05
1.375	4.044E-02	2.372E-03	5.770	8.146E-05	5.811E-05
1.430	3.642E-02	2.176E-03	5.910	7.997E-05	4.807E-05
1.490	3.559E-02	2.191E-03	6.050	9.079E-05	4.456E-05
1.550	3.575E-02	2.095E-03	6.190	8.339E-05	4.876E-05
1.610	2.991E-02	1.931E-03	6.330	4.694E-05	4.515E-05
1.670	3.036E-02	1.854E-03	6.480	1.276E-05	3.992E-05
1.730	3.402E-02	2.230E-03	6.640	1.963E-06	4.015E-05
1.790	2.946E-02	2.077E-03	6.800	5.280E-06	4.091E-05
1.850	2.581E-02	1.980E-03	6.960	7.662E-06	4.195E-05
1.910	2.592E-02	1.899E-03	7.120	7.082E-06	4.349E-05
1.970	2.307E-02	1.757E-03	7.280	6.531E-06	4.497E-05
2.040	1.919E-02	1.761E-03	7.440	6.496E-06	4.691E-05
2.120	1.918E-02	1.672E-03	7.600	7.594E-06	4.785E-05
2.200	1.787E-02	1.712E-03	7.760	6.892E-06	5.194E-05
2.280	1.650E-02	1.458E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 295 SEC AFTER END OF IRRADIATION
COUNT FOR 100 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.295E-01	1.360E-02	2.360	1.355E-02	1.316E-03
0.190	1.324E-01	1.256E-02	2.440	1.313E-02	1.246E-03
0.210	1.255E-01	1.139E-02	2.520	1.222E-02	1.142E-03
0.230	1.179E-01	1.098E-02	2.600	1.069E-02	1.002E-03
0.250	1.053E-01	1.028E-02	2.680	1.065E-02	1.034E-03
0.275	1.054E-01	1.017E-02	2.760	1.079E-02	1.008E-03
0.305	1.094E-01	1.019E-02	2.840	9.041E-03	9.413E-04
0.335	1.090E-01	9.447E-03	2.920	7.136E-03	8.569E-04
0.365	9.938E-02	8.818E-03	3.000	6.705E-03	7.742E-04
0.395	8.495E-02	8.222E-03	3.080	6.551E-03	7.525E-04
0.425	8.187E-02	7.214E-03	3.160	5.604E-03	7.244E-04
0.455	8.396E-02	6.150E-03	3.250	4.690E-03	6.360E-04
0.485	7.533E-02	5.933E-03	3.350	4.408E-03	6.260E-04
0.520	6.229E-02	5.627E-03	3.450	3.731E-03	5.425E-04
0.560	7.166E-02	5.531E-03	3.550	3.006E-03	4.954E-04
0.600	7.803E-02	5.234E-03	3.650	2.657E-03	4.362E-04
0.640	6.891E-02	5.046E-03	3.750	2.266E-03	3.972E-04
0.680	6.429E-02	4.563E-03	3.860	1.758E-03	3.627E-04
0.720	6.348E-02	4.277E-03	3.980	1.370E-03	3.258E-04
0.760	6.163E-02	4.023E-03	4.100	1.008E-03	2.774E-04
0.800	6.207E-02	3.788E-03	4.220	7.014E-04	2.189E-04
0.840	5.832E-02	3.654E-03	4.340	5.977E-04	2.326E-04
0.880	5.429E-02	3.442E-03	4.460	5.923E-04	1.921E-04
0.925	5.212E-02	3.261E-03	4.580	5.104E-04	1.626E-04
0.975	4.631E-02	3.173E-03	4.700	4.074E-04	1.420E-04
1.025	4.676E-02	3.078E-03	4.820	3.979E-04	1.374E-04
1.075	4.323E-02	2.873E-03	4.940	3.709E-04	1.459E-04
1.125	3.686E-02	2.640E-03	5.070	2.339E-04	1.114E-04
1.175	3.753E-02	2.509E-03	5.210	9.958E-05	5.776E-05
1.225	3.608E-02	2.399E-03	5.350	7.933E-05	5.944E-05
1.275	3.212E-02	2.198E-03	5.490	8.647E-05	5.265E-05
1.325	2.953E-02	2.025E-03	5.630	7.436E-05	4.866E-05
1.375	2.709E-02	2.021E-03	5.770	4.832E-05	4.396E-05
1.430	2.710E-02	1.987E-03	5.910	3.142E-05	4.323E-05
1.490	2.941E-02	1.950E-03	6.050	2.917E-05	4.179E-05
1.550	2.716E-02	1.833E-03	6.190	2.698E-05	4.355E-05
1.610	2.389E-02	1.761E-03	6.330	1.880E-05	4.219E-05
1.670	2.449E-02	1.700E-03	6.480	9.828E-06	3.944E-05
1.730	2.390E-02	1.886E-03	6.640	5.286E-06	4.012E-05
1.790	2.097E-02	1.815E-03	6.800	6.074E-06	4.088E-05
1.850	1.928E-02	1.717E-03	6.960	6.994E-06	4.195E-05
1.910	1.780E-02	1.624E-03	7.120	6.784E-06	4.388E-05
1.970	1.644E-02	1.520E-03	7.280	6.647E-06	4.497E-05
2.040	1.737E-02	1.509E-03	7.440	6.591E-06	4.691E-05
2.120	1.754E-02	1.582E-03	7.600	7.578E-06	4.785E-05
2.200	1.511E-02	1.473E-03	7.760	6.860E-06	5.194E-05
2.280	1.406E-02	1.314E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 215 SEC AFTER END OF IRRADIATION
COUNT FOR 80 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.395E-01	1.311E-02	2.360	1.403E-02	1.386E-03
0.190	1.117E-01	1.266E-02	2.440	1.318E-02	1.261E-03
0.210	1.027E-01	1.217E-02	2.520	1.080E-02	1.165E-03
0.230	1.208E-01	1.122E-02	2.600	9.568E-03	1.051E-03
0.250	1.217E-01	9.914E-03	2.680	1.039E-02	1.114E-03
0.275	1.034E-01	1.021E-02	2.760	9.921E-03	1.021E-03
0.305	1.093E-01	1.002E-02	2.840	8.382E-03	8.799E-04
0.335	1.102E-01	9.459E-03	2.920	7.809E-03	8.851E-04
0.365	1.081E-01	9.115E-03	3.000	6.970E-03	8.668E-04
0.395	1.105E-01	8.509E-03	3.080	5.766E-03	8.061E-04
0.425	1.005E-01	7.044E-03	3.160	5.372E-03	7.505E-04
0.455	9.443E-02	6.037E-03	3.250	5.231E-03	7.016E-04
0.485	8.916E-02	5.961E-03	3.350	4.884E-03	6.554E-04
0.520	8.269E-02	5.690E-03	3.450	4.706E-03	5.739E-04
0.560	8.270E-02	5.607E-03	3.550	3.721E-03	5.357E-04
0.600	8.045E-02	5.399E-03	3.650	2.414E-03	3.915E-04
0.640	7.324E-02	4.940E-03	3.750	2.058E-03	3.988E-04
0.680	7.332E-02	4.549E-03	3.860	2.162E-03	3.908E-04
0.720	6.894E-02	4.364E-03	3.980	2.023E-03	3.987E-04
0.760	6.740E-02	4.093E-03	4.100	1.788E-03	3.558E-04
0.800	6.751E-02	3.912E-03	4.220	1.362E-03	2.911E-04
0.840	6.058E-02	3.745E-03	4.340	9.466E-04	2.869E-04
0.880	5.997E-02	3.581E-03	4.460	8.681E-04	2.624E-04
0.925	5.843E-02	3.482E-03	4.580	8.189E-04	2.034E-04
0.975	5.129E-02	3.274E-03	4.700	5.420E-04	1.669E-04
1.025	5.127E-02	3.221E-03	4.820	3.455E-04	1.144E-04
1.075	4.536E-02	3.031E-03	4.940	3.238E-04	1.218E-04
1.125	4.499E-02	2.847E-03	5.070	2.787E-04	1.131E-04
1.175	4.865E-02	2.759E-03	5.210	1.783E-04	7.175E-05
1.225	4.510E-02	2.621E-03	5.350	1.337E-04	7.430E-05
1.275	4.078E-02	2.437E-03	5.490	9.212E-05	5.997E-05
1.325	3.719E-02	2.328E-03	5.630	5.229E-05	4.620E-05
1.375	3.535E-02	2.195E-03	5.770	3.928E-05	4.279E-05
1.430	3.555E-02	2.059E-03	5.910	4.740E-05	4.628E-05
1.490	3.369E-02	2.056E-03	6.050	5.076E-05	4.506E-05
1.550	3.068E-02	1.913E-03	6.190	4.049E-05	4.438E-05
1.610	2.744E-02	1.873E-03	6.330	2.464E-05	4.236E-05
1.670	2.624E-02	1.674E-03	6.480	1.088E-05	3.979E-05
1.730	2.528E-02	1.961E-03	6.640	5.070E-06	4.020E-05
1.790	2.203E-02	1.785E-03	6.800	5.942E-06	4.099E-05
1.850	2.176E-02	1.916E-03	6.960	7.006E-06	4.198E-05
1.910	2.351E-02	1.795E-03	7.120	6.816E-06	4.351E-05
1.970	2.204E-02	1.815E-03	7.280	6.653E-06	4.497E-05
2.040	1.954E-02	1.712E-03	7.440	6.582E-06	4.692E-05
2.120	1.827E-02	1.650E-03	7.600	7.574E-06	4.785E-05
2.200	1.497E-02	1.382E-03	7.760	6.863E-06	5.194E-05
2.280	1.356E-02	1.401E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 395 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	2.028E-01	1.729E-02	2.360	1.913E-02	1.602E-03
0.190	1.627E-01	1.560E-02	2.440	1.550E-02	1.322E-03
0.210	1.509E-01	1.452E-02	2.520	1.416E-02	1.332E-03
0.230	1.581E-01	1.384E-02	2.600	1.614E-02	1.316E-03
0.250	1.616E-01	1.270E-02	2.680	1.572E-02	1.244E-03
0.275	1.463E-01	1.263E-02	2.760	1.240E-02	1.139E-03
0.305	1.473E-01	1.247E-02	2.840	1.060E-02	9.825E-04
0.335	1.383E-01	1.165E-02	2.920	1.044E-02	9.599E-04
0.365	1.181E-01	1.096E-02	3.000	9.356E-03	9.556E-04
0.395	1.319E-01	1.004E-02	3.080	7.584E-03	8.545E-04
0.425	1.337E-01	8.563E-03	3.160	7.043E-03	7.828E-04
0.455	1.226E-01	7.388E-03	3.250	7.687E-03	7.947E-04
0.485	1.164E-01	7.141E-03	3.350	7.356E-03	7.550E-04
0.520	1.155E-01	6.903E-03	3.450	5.836E-03	7.101E-04
0.560	1.108E-01	6.797E-03	3.550	4.620E-03	6.041E-04
0.600	9.057E-02	6.455E-03	3.650	3.660E-03	5.423E-04
0.640	9.881E-02	5.923E-03	3.750	2.869E-03	5.028E-04
0.680	9.717E-02	5.689E-03	3.860	2.512E-03	4.632E-04
0.720	8.227E-02	5.187E-03	3.980	2.185E-03	3.883E-04
0.760	8.014E-02	4.736E-03	4.100	1.570E-03	3.235E-04
0.800	8.015E-02	4.574E-03	4.220	9.837E-04	2.723E-04
0.840	7.864E-02	4.401E-03	4.340	6.176E-04	1.969E-04
0.880	7.320E-02	4.272E-03	4.460	5.709E-04	1.688E-04
0.925	7.146E-02	3.882E-03	4.580	6.109E-04	1.837E-04
0.975	6.729E-02	3.882E-03	4.700	4.920E-04	1.477E-04
1.025	5.932E-02	3.574E-03	4.820	3.405E-04	1.278E-04
1.075	5.645E-02	3.319E-03	4.940	2.672E-04	1.128E-04
1.125	5.120E-02	3.111E-03	5.070	2.206E-04	8.696E-05
1.175	4.983E-02	3.012E-03	5.210	1.375E-04	7.476E-05
1.225	4.912E-02	2.893E-03	5.350	4.910E-05	5.189E-05
1.275	4.654E-02	2.675E-03	5.490	3.474E-05	4.023E-05
1.325	4.639E-02	2.597E-03	5.630	9.212E-05	4.522E-05
1.375	4.287E-02	2.472E-03	5.770	1.339E-04	5.418E-05
1.430	3.888E-02	2.307E-03	5.910	9.553E-05	5.858E-05
1.490	3.993E-02	2.260E-03	6.050	3.705E-05	4.678E-05
1.550	3.692E-02	2.182E-03	6.190	2.049E-05	4.197E-05
1.610	3.351E-02	2.059E-03	6.330	2.712E-05	4.257E-05
1.670	3.373E-02	1.958E-03	6.480	2.376E-05	4.002E-05
1.730	3.218E-02	2.277E-03	6.640	1.091E-05	4.031E-05
1.790	2.963E-02	2.174E-03	6.800	4.824E-06	4.092E-05
1.850	2.887E-02	2.066E-03	6.960	5.623E-06	4.194E-05
1.910	2.704E-02	1.926E-03	7.120	6.862E-06	4.349E-05
1.970	2.355E-02	1.923E-03	7.280	7.001E-06	4.496E-05
2.040	2.158E-02	1.813E-03	7.440	6.632E-06	4.691E-05
2.120	2.205E-02	1.749E-03	7.600	7.485E-06	4.785E-05
2.200	2.153E-02	1.707E-03	7.760	6.823E-06	5.194E-05
2.280	2.072E-02	1.643E-03			

*C002I STOP 0

SPECTRUM OF BETA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 595 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.557E-01	1.501E-02	2.360	1.415E-02	1.325E-03
0.190	1.456E-01	1.392E-02	2.440	1.275E-02	1.178E-03
0.210	1.476E-01	1.297E-02	2.520	1.133E-02	1.070E-03
0.230	1.387E-01	1.218E-02	2.600	1.177E-02	1.073E-03
0.250	1.288E-01	1.102E-02	2.680	1.072E-02	1.032E-03
0.275	1.169E-01	1.146E-02	2.760	8.373E-03	8.759E-04
0.305	1.003E-01	1.115E-02	2.840	7.798E-03	9.558E-04
0.335	1.126E-01	1.020E-02	2.920	8.045E-03	8.559E-04
0.365	1.108E-01	9.595E-03	3.000	7.470E-03	8.298E-04
0.395	1.026E-01	8.886E-03	3.080	6.673E-03	7.737E-04
0.425	8.890E-02	7.678E-03	3.160	5.912E-03	7.370E-04
0.455	8.795E-02	6.453E-03	3.250	4.463E-03	6.293E-04
0.485	9.907E-02	6.178E-03	3.350	3.356E-03	5.872E-04
0.520	9.430E-02	5.914E-03	3.450	3.721E-03	5.520E-04
0.560	8.495E-02	5.887E-03	3.550	3.643E-03	5.227E-04
0.600	7.804E-02	5.661E-03	3.650	2.697E-03	4.311E-04
0.640	7.246E-02	5.281E-03	3.750	2.048E-03	4.062E-04
0.680	7.264E-02	4.984E-03	3.860	1.618E-03	3.585E-04
0.720	7.170E-02	4.532E-03	3.980	1.305E-03	2.851E-04
0.760	6.097E-02	4.217E-03	4.100	1.226E-03	2.553E-04
0.800	5.842E-02	4.054E-03	4.220	8.368E-04	2.066E-04
0.840	5.971E-02	3.829E-03	4.340	3.726E-04	1.134E-04
0.880	5.563E-02	3.609E-03	4.460	1.563E-04	9.478E-05
0.925	5.260E-02	3.446E-03	4.580	1.120E-04	6.611E-05
0.975	5.127E-02	3.333E-03	4.700	1.316E-04	6.795E-05
1.025	4.861E-02	3.115E-03	4.820	1.235E-04	6.761E-05
1.075	4.338E-02	2.916E-03	4.940	6.748E-05	4.982E-05
1.125	3.867E-02	2.757E-03	5.070	3.062E-05	4.345E-05
1.175	3.596E-02	2.605E-03	5.210	6.309E-05	4.240E-05
1.225	3.643E-02	2.346E-03	5.350	1.126E-04	4.924E-05
1.275	3.479E-02	2.223E-03	5.490	9.431E-05	4.949E-05
1.325	3.284E-02	2.155E-03	5.630	4.462E-05	4.519E-05
1.375	3.276E-02	2.126E-03	5.770	2.536E-05	4.026E-05
1.430	3.045E-02	2.062E-03	5.910	2.740E-05	4.198E-05
1.490	2.803E-02	1.953E-03	6.050	2.416E-05	4.080E-05
1.550	2.679E-02	1.868E-03	6.190	1.235E-05	4.141E-05
1.610	2.436E-02	1.769E-03	6.330	5.367E-06	4.117E-05
1.670	2.381E-02	1.587E-03	6.480	4.996E-06	3.976E-05
1.730	2.485E-02	1.965E-03	6.640	6.742E-06	4.000E-05
1.790	2.249E-02	1.871E-03	6.800	7.052E-06	4.088E-05
1.850	1.849E-02	1.777E-03	6.960	6.796E-06	4.194E-05
1.910	1.775E-02	1.586E-03	7.120	6.517E-06	4.348E-05
1.970	1.855E-02	1.612E-03	7.280	6.652E-06	4.496E-05
2.040	1.791E-02	1.560E-03	7.440	6.659E-06	4.691E-05
2.120	1.630E-02	1.500E-03	7.600	7.590E-06	4.785E-05
2.200	1.395E-02	1.337E-03	7.760	6.840E-06	5.194E-05
2.280	1.335E-02	1.283E-03			

SPECTRUM OF BETA RAYS FOLLOWING A
1(1)-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 69.7 SEC AFTER END OF IRRADIATION
COUNT FOR 40 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.362E-01	5.011E-03	2.360	1.636E-02	5.175E-04
0.190	1.220E-01	4.766E-03	2.440	1.464E-02	4.589E-04
0.210	1.168E-01	4.510E-03	2.520	1.451E-02	4.456E-04
0.230	1.126E-01	4.345E-03	2.600	1.370E-02	4.337E-04
0.250	1.051E-01	4.137E-03	2.680	1.237E-02	4.030E-04
0.275	9.902E-02	4.447E-03	2.760	1.130E-02	3.879E-04
0.305	9.645E-02	4.389E-03	2.840	1.041E-02	3.675E-04
0.335	9.792E-02	3.999E-03	2.920	9.555E-03	3.457E-04
0.365	9.618E-02	3.754E-03	3.000	8.597E-03	3.212E-04
0.395	9.142E-02	3.465E-03	3.080	8.059E-03	3.087E-04
0.425	9.082E-02	2.708E-03	3.160	7.628E-03	2.963E-04
0.455	8.998E-02	2.317E-03	3.250	6.885E-03	2.715E-04
0.485	9.046E-02	2.135E-03	3.350	6.111E-03	2.589E-04
0.520	8.802E-02	2.156E-03	3.450	5.126E-03	2.455E-04
0.560	8.817E-02	2.156E-03	3.550	4.577E-03	2.166E-04
0.600	8.579E-02	2.068E-03	3.650	4.295E-03	2.076E-04
0.640	8.251E-02	1.945E-03	3.750	3.703E-03	1.891E-04
0.680	8.581E-02	1.914E-03	3.860	2.989E-03	1.738E-04
0.720	8.566E-02	1.806E-03	3.980	2.420E-03	1.619E-04
0.760	8.322E-02	1.739E-03	4.100	2.242E-03	1.476E-04
0.800	8.103E-02	1.625E-03	4.220	1.896E-03	1.268E-04
0.840	7.785E-02	1.555E-03	4.340	1.409E-03	1.090E-04
0.880	7.536E-02	1.488E-03	4.460	1.202E-03	9.920E-05
0.925	7.012E-02	1.387E-03	4.580	1.051E-03	9.046E-05
0.975	6.672E-02	1.361E-03	4.700	8.376E-04	7.817E-05
1.025	6.596E-02	1.338E-03	4.820	7.275E-04	7.398E-05
1.075	6.148E-02	1.286E-03	4.940	6.213E-04	6.795E-05
1.125	5.933E-02	1.234E-03	5.070	4.367E-04	5.182E-05
1.175	5.795E-02	1.192E-03	5.210	3.367E-04	4.887E-05
1.225	5.701E-02	1.129E-03	5.350	2.957E-04	4.225E-05
1.275	5.277E-02	1.050E-03	5.490	2.260E-04	3.776E-05
1.325	4.941E-02	9.946E-04	5.630	1.765E-04	3.069E-05
1.375	4.860E-02	9.658E-04	5.770	1.418E-04	2.879E-05
1.430	4.617E-02	9.157E-04	5.910	9.936E-05	2.054E-05
1.490	4.263E-02	8.845E-04	6.050	5.971E-05	1.630E-05
1.550	4.018E-02	8.353E-04	6.190	3.055E-05	1.188E-05
1.610	3.906E-02	8.229E-04	6.330	1.350E-05	6.716E-06
1.670	3.573E-02	7.692E-04	6.480	8.485E-06	6.390E-06
1.730	3.266E-02	7.736E-04	6.640	7.090E-06	5.408E-06
1.790	3.029E-02	7.248E-04	6.800	5.012E-06	4.696E-06
1.850	2.767E-02	6.827E-04	6.960	2.466E-06	4.625E-06
1.910	2.605E-02	6.862E-04	7.120	9.674E-07	4.550E-06
1.970	2.467E-02	6.529E-04	7.280	4.833E-07	4.749E-06
2.040	2.281E-02	6.077E-04	7.440	6.318E-07	4.923E-06
2.120	2.128E-02	5.903E-04	7.600	8.296E-07	5.010E-06
2.200	1.955E-02	5.672E-04	7.760	7.412E-07	5.436E-06
2.280	1.816E-02	5.228E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 110 SEC AFTER END OF IRRADIATION
COUNT FOR 60 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.537E-01	5.113E-03	2.360	1.571E-02	4.957E-04
0.190	1.339E-01	4.838E-03	2.440	1.573E-02	4.625E-04
0.210	1.237E-01	4.651E-03	2.520	1.489E-02	4.323E-04
0.230	1.239E-01	4.421E-03	2.600	1.312E-02	4.168E-04
0.250	1.234E-01	4.274E-03	2.680	1.204E-02	3.959E-04
0.275	1.207E-01	4.660E-03	2.760	1.045E-02	3.591E-04
0.305	1.149E-01	4.580E-03	2.840	9.708E-03	3.515E-04
0.335	1.149E-01	4.110E-03	2.920	9.612E-03	3.391E-04
0.365	1.080E-01	3.834E-03	3.000	9.014E-03	3.197E-04
0.395	1.053E-01	3.578E-03	3.080	7.858E-03	2.947E-04
0.425	9.849E-02	2.779E-03	3.160	7.001E-03	2.820E-04
0.455	9.702E-02	2.401E-03	3.250	6.488E-03	2.616E-04
0.485	9.772E-02	2.202E-03	3.350	5.632E-03	2.498E-04
0.520	9.687E-02	2.197E-03	3.450	5.067E-03	2.308E-04
0.560	9.044E-02	2.213E-03	3.550	4.782E-03	2.123E-04
0.600	9.024E-02	2.106E-03	3.650	3.881E-03	1.852E-04
0.640	8.694E-02	2.005E-03	3.750	3.190E-03	1.828E-04
0.680	8.862E-02	1.924E-03	3.860	2.879E-03	1.592E-04
0.720	8.648E-02	1.833E-03	3.980	2.233E-03	1.524E-04
0.760	8.237E-02	1.720E-03	4.100	1.959E-03	1.425E-04
0.800	7.852E-02	1.619E-03	4.220	1.699E-03	1.213E-04
0.840	7.704E-02	1.544E-03	4.340	1.274E-03	1.041E-04
0.880	7.476E-02	1.494E-03	4.460	1.164E-03	9.627E-05
0.925	7.179E-02	1.393E-03	4.580	9.749E-04	8.379E-05
0.975	7.020E-02	1.366E-03	4.700	6.799E-04	7.272E-05
1.025	6.557E-02	1.335E-03	4.820	5.966E-04	6.588E-05
1.075	6.331E-02	1.282E-03	4.940	5.477E-04	6.211E-05
1.125	5.916E-02	1.229E-03	5.070	4.021E-04	5.328E-05
1.175	5.569E-02	1.162E-03	5.210	3.160E-04	4.932E-05
1.225	5.477E-02	1.105E-03	5.350	2.591E-04	4.006E-05
1.275	5.184E-02	1.023E-03	5.490	1.721E-04	3.230E-05
1.325	4.799E-02	9.822E-04	5.630	1.187E-04	2.439E-05
1.375	4.529E-02	9.409E-04	5.770	9.508E-05	2.322E-05
1.430	4.420E-02	8.984E-04	5.910	7.114E-05	1.754E-05
1.490	4.144E-02	8.682E-04	6.050	4.873E-05	1.429E-05
1.550	3.863E-02	8.255E-04	6.190	2.799E-05	1.070E-05
1.610	3.644E-02	7.878E-04	6.330	1.094E-05	6.239E-06
1.670	3.375E-02	7.345E-04	6.480	3.584E-06	4.763E-06
1.730	3.120E-02	7.638E-04	6.640	5.190E-06	4.350E-06
1.790	2.893E-02	7.163E-04	6.800	8.051E-06	5.043E-06
1.850	2.817E-02	6.725E-04	6.960	6.202E-06	4.668E-06
1.910	2.671E-02	6.799E-04	7.120	2.660E-06	4.764E-06
1.970	2.461E-02	6.537E-04	7.280	8.025E-07	4.661E-06
2.040	2.342E-02	6.170E-04	7.440	4.139E-07	4.946E-06
2.120	2.038E-02	5.637E-04	7.600	7.539E-07	5.020E-06
2.200	1.859E-02	5.592E-04	7.760	7.816E-07	5.429E-06
2.280	1.754E-02	5.264E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 170 SEC AFTER END OF IRRADIATION
COUNT FOR 80 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.593E-01	5.037E-03	2.360	1.509E-02	4.758E-04
0.190	1.416E-01	4.745E-03	2.440	1.354E-02	4.354E-04
0.210	1.298E-01	4.482E-03	2.520	1.205E-02	4.012E-04
0.230	1.259E-01	4.283E-03	2.600	1.154E-02	3.897E-04
0.250	1.245E-01	4.127E-03	2.680	1.037E-02	3.652E-04
0.275	1.177E-01	4.541E-03	2.760	9.293E-03	3.391E-04
0.305	1.074E-01	4.434E-03	2.840	8.733E-03	3.308E-04
0.335	1.027E-01	3.955E-03	2.920	8.482E-03	3.136E-04
0.365	1.030E-01	3.689E-03	3.000	8.228E-03	2.949E-04
0.395	1.033E-01	3.427E-03	3.080	7.079E-03	2.690E-04
0.425	9.729E-02	2.672E-03	3.160	5.902E-03	2.561E-04
0.455	9.718E-02	2.317E-03	3.250	5.603E-03	2.558E-04
0.485	9.094E-02	2.139E-03	3.350	5.154E-03	2.402E-04
0.520	8.855E-02	2.130E-03	3.450	4.360E-03	2.094E-04
0.560	8.631E-02	2.135E-03	3.550	3.890E-03	1.990E-04
0.600	8.214E-02	2.032E-03	3.650	3.230E-03	1.769E-04
0.640	8.017E-02	1.913E-03	3.750	2.647E-03	1.588E-04
0.680	7.775E-02	1.814E-03	3.860	2.466E-03	1.529E-04
0.720	7.802E-02	1.755E-03	3.980	2.046E-03	1.308E-04
0.760	7.529E-02	1.634E-03	4.100	1.550E-03	1.175E-04
0.800	7.265E-02	1.547E-03	4.220	1.303E-03	1.032E-04
0.840	6.834E-02	1.428E-03	4.340	1.092E-03	9.161E-05
0.880	6.531E-02	1.375E-03	4.460	9.388E-04	8.280E-05
0.925	6.292E-02	1.291E-03	4.580	7.929E-04	7.499E-05
0.975	6.041E-02	1.274E-03	4.700	5.863E-04	6.782E-05
1.025	5.800E-02	1.234E-03	4.820	4.583E-04	5.640E-05
1.075	5.541E-02	1.196E-03	4.940	4.166E-04	5.138E-05
1.125	5.298E-02	1.131E-03	5.070	3.296E-04	4.433E-05
1.175	5.071E-02	1.068E-03	5.210	2.015E-04	3.617E-05
1.225	4.800E-02	1.001E-03	5.350	1.596E-04	2.956E-05
1.275	4.500E-02	9.374E-04	5.490	1.547E-04	2.861E-05
1.325	4.234E-02	9.081E-04	5.630	9.435E-05	2.184E-05
1.375	4.082E-02	8.882E-04	5.770	4.775E-05	1.365E-05
1.430	3.906E-02	8.336E-04	5.910	5.264E-05	1.560E-05
1.490	3.633E-02	8.030E-04	6.050	5.606E-05	1.457E-05
1.550	3.469E-02	7.772E-04	6.190	3.486E-05	1.038E-05
1.610	3.219E-02	7.323E-04	6.330	1.144E-05	6.161E-06
1.670	3.015E-02	6.916E-04	6.480	1.325E-06	4.455E-06
1.730	2.793E-02	6.906E-04	6.640	2.250E-07	4.356E-06
1.790	2.594E-02	6.795E-04	6.800	7.578E-07	4.328E-06
1.850	2.488E-02	6.316E-04	6.960	8.179E-07	4.416E-06
1.910	2.277E-02	5.947E-04	7.120	6.870E-07	4.554E-06
1.970	2.062E-02	6.045E-04	7.280	6.707E-07	4.709E-06
2.040	1.946E-02	5.646E-04	7.440	6.896E-07	4.905E-06
2.120	1.847E-02	5.490E-04	7.600	7.976E-07	5.003E-06
2.200	1.730E-02	5.189E-04	7.760	7.171E-07	5.430E-06
2.280	1.594E-02	4.847E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 250 SEC AFTER END OF IRRADIATION
COUNT FOR 100 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.514E-01	4.829E-03	2.360	1.263E-02	4.296E-04
0.190	1.299E-01	4.563E-03	2.440	1.271E-02	4.101E-04
0.210	1.197E-01	4.359E-03	2.520	1.094E-02	3.808E-04
0.230	1.213E-01	4.092E-03	2.600	9.750E-03	3.667E-04
0.250	1.191E-01	3.959E-03	2.680	9.703E-03	3.453E-04
0.275	1.109E-01	4.385E-03	2.760	8.690E-03	3.178E-04
0.305	1.061E-01	4.261E-03	2.840	7.905E-03	2.938E-04
0.335	9.881E-02	3.777E-03	2.920	7.672E-03	2.902E-04
0.365	9.559E-02	3.511E-03	3.000	7.040E-03	2.668E-04
0.395	9.547E-02	3.294E-03	3.080	6.347E-03	2.685E-04
0.425	9.228E-02	2.575E-03	3.160	5.522E-03	2.446E-04
0.455	8.925E-02	2.229E-03	3.250	4.724E-03	2.306E-04
0.485	8.308E-02	2.050E-03	3.350	4.635E-03	2.226E-04
0.520	8.141E-02	2.035E-03	3.450	4.100E-03	1.958E-04
0.560	8.027E-02	2.044E-03	3.550	3.250E-03	1.668E-04
0.600	7.614E-02	1.957E-03	3.650	2.702E-03	1.611E-04
0.640	7.352E-02	1.859E-03	3.750	2.251E-03	1.447E-04
0.680	6.927E-02	1.730E-03	3.860	2.009E-03	1.354E-04
0.720	6.774E-02	1.619E-03	3.980	1.777E-03	1.217E-04
0.760	6.357E-02	1.539E-03	4.100	1.407E-03	1.071E-04
0.800	6.133E-02	1.428E-03	4.220	9.995E-04	9.086E-05
0.840	5.942E-02	1.348E-03	4.340	7.393E-04	8.228E-05
0.880	5.780E-02	1.296E-03	4.460	6.441E-04	7.066E-05
0.925	5.703E-02	1.207E-03	4.580	5.351E-04	6.034E-05
0.975	5.486E-02	1.188E-03	4.700	4.269E-04	5.266E-05
1.025	5.087E-02	1.133E-03	4.820	3.370E-04	5.006E-05
1.075	4.719E-02	1.087E-03	4.940	2.234E-04	4.546E-05
1.125	4.615E-02	1.030E-03	5.070	1.552E-04	2.838E-05
1.175	4.395E-02	9.849E-04	5.210	1.595E-04	3.167E-05
1.225	4.039E-02	9.232E-04	5.350	1.419E-04	2.655E-05
1.275	3.800E-02	8.678E-04	5.490	1.050E-04	2.377E-05
1.325	3.585E-02	8.286E-04	5.630	7.829E-05	1.980E-05
1.375	3.386E-02	8.104E-04	5.770	5.546E-05	1.565E-05
1.430	3.316E-02	7.558E-04	5.910	3.363E-05	1.201E-05
1.490	3.184E-02	7.456E-04	6.050	1.637E-05	7.893E-06
1.550	2.926E-02	7.124E-04	6.190	6.060E-06	5.602E-06
1.610	2.765E-02	6.777E-04	6.330	2.916E-06	4.751E-06
1.670	2.601E-02	6.454E-04	6.480	4.415E-06	4.412E-06
1.730	2.362E-02	6.405E-04	6.640	6.205E-06	4.336E-06
1.790	2.207E-02	6.192E-04	6.800	4.479E-06	4.340E-06
1.850	2.142E-02	5.850E-04	6.960	1.573E-06	4.444E-06
1.910	2.089E-02	5.721E-04	7.120	2.897E-07	4.563E-06
1.970	1.971E-02	5.610E-04	7.280	4.388E-07	4.707E-06
2.040	1.706E-02	5.158E-04	7.440	7.440E-07	4.905E-06
2.120	1.517E-02	5.007E-04	7.600	8.745E-07	5.003E-06
2.200	1.511E-02	4.887E-04	7.760	7.199E-07	5.430E-06
2.280	1.341E-02	4.519E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 350 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	2.242E-01	5.850E-03	2.360	1.766E-02	5.206E-04
0.190	1.918E-01	5.477E-03	2.440	1.618E-02	4.523E-04
0.210	1.835E-01	5.189E-03	2.520	1.532E-02	4.354E-04
0.230	1.752E-01	4.878E-03	2.600	1.462E-02	4.248E-04
0.250	1.658E-01	4.811E-03	2.680	1.303E-02	3.959E-04
0.275	1.558E-01	5.304E-03	2.760	1.189E-02	3.786E-04
0.305	1.549E-01	5.149E-03	2.840	1.101E-02	3.573E-04
0.335	1.511E-01	4.594E-03	2.920	1.025E-02	3.307E-04
0.365	1.452E-01	4.270E-03	3.000	9.792E-03	3.191E-04
0.395	1.391E-01	3.977E-03	3.080	8.758E-03	2.989E-04
0.425	1.329E-01	3.063E-03	3.160	7.684E-03	2.837E-04
0.455	1.316E-01	2.655E-03	3.250	7.154E-03	2.692E-04
0.485	1.281E-01	2.459E-03	3.350	6.320E-03	2.524E-04
0.520	1.155E-01	2.432E-03	3.450	5.168E-03	2.323E-04
0.560	1.105E-01	2.441E-03	3.550	4.244E-03	2.004E-04
0.600	1.054E-01	2.330E-03	3.650	3.653E-03	1.810E-04
0.640	1.063E-01	2.206E-03	3.750	3.241E-03	1.721E-04
0.680	1.008E-01	2.091E-03	3.860	2.668E-03	1.507E-04
0.720	9.759E-02	1.948E-03	3.980	2.068E-03	1.341E-04
0.760	9.044E-02	1.815E-03	4.100	1.584E-03	1.136E-04
0.800	8.613E-02	1.712E-03	4.220	1.126E-03	9.538E-05
0.840	8.556E-02	1.610E-03	4.340	8.497E-04	8.099E-05
0.880	8.198E-02	1.526E-03	4.460	7.238E-04	7.032E-05
0.925	7.638E-02	1.387E-03	4.580	5.399E-04	6.027E-05
0.975	7.038E-02	1.369E-03	4.700	3.740E-04	4.798E-05
1.025	6.969E-02	1.332E-03	4.820	3.076E-04	4.513E-05
1.075	6.571E-02	1.260E-03	4.940	2.507E-04	3.984E-05
1.125	6.061E-02	1.191E-03	5.070	1.926E-04	3.422E-05
1.175	5.548E-02	1.129E-03	5.210	1.723E-04	3.721E-05
1.225	5.408E-02	1.073E-03	5.350	1.365E-04	2.751E-05
1.275	5.192E-02	9.978E-04	5.490	8.272E-05	2.104E-05
1.325	4.755E-02	9.538E-04	5.630	5.202E-05	1.531E-05
1.375	4.646E-02	9.396E-04	5.770	4.247E-05	1.521E-05
1.430	4.539E-02	8.975E-04	5.910	3.731E-05	1.178E-05
1.490	4.157E-02	8.570E-04	6.050	3.021E-05	1.101E-05
1.550	3.928E-02	8.282E-04	6.190	1.914E-05	8.958E-06
1.610	3.737E-02	7.954E-04	6.330	8.477E-06	5.668E-06
1.670	3.592E-02	7.549E-04	6.480	2.931E-06	4.871E-06
1.730	3.312E-02	7.638E-04	6.640	1.125E-06	4.374E-06
1.790	2.981E-02	7.159E-04	6.800	8.040E-07	4.398E-06
1.850	2.908E-02	6.875E-04	6.960	7.024E-07	4.421E-06
1.910	2.741E-02	6.738E-04	7.120	6.646E-07	4.564E-06
1.970	2.439E-02	6.454E-04	7.280	7.046E-07	4.722E-06
2.040	2.328E-02	5.956E-04	7.440	6.998E-07	4.908E-06
2.120	2.310E-02	5.841E-04	7.600	7.917E-07	5.007E-06
2.200	2.122E-02	5.499E-04	7.760	7.142E-07	5.431E-06
2.280	1.933E-02	5.286E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 550 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.752E-01	5.131E-03	2.360	1.229E-02	4.221E-04
0.190	1.520E-01	4.762E-03	2.440	1.162E-02	3.817E-04
0.210	1.432E-01	4.482E-03	2.520	1.124E-02	3.689E-04
0.230	1.394E-01	4.198E-03	2.600	1.016E-02	3.485E-04
0.250	1.372E-01	4.190E-03	2.680	9.208E-03	3.413E-04
0.275	1.300E-01	4.563E-03	2.760	8.598E-03	3.053E-04
0.305	1.248E-01	4.465E-03	2.840	7.929E-03	3.025E-04
0.335	1.130E-01	3.923E-03	2.920	7.228E-03	2.814E-04
0.365	1.058E-01	3.657E-03	3.000	6.761E-03	2.702E-04
0.395	1.007E-01	3.373E-03	3.080	6.281E-03	2.561E-04
0.425	9.505E-02	2.610E-03	3.160	5.666E-03	2.393E-04
0.455	9.118E-02	2.263E-03	3.250	5.154E-03	2.165E-04
0.485	9.029E-02	2.086E-03	3.350	4.478E-03	2.075E-04
0.520	8.822E-02	2.085E-03	3.450	3.615E-03	1.801E-04
0.560	8.587E-02	2.083E-03	3.550	3.061E-03	1.712E-04
0.600	7.989E-02	2.011E-03	3.650	2.571E-03	1.446E-04
0.640	7.630E-02	1.908E-03	3.750	2.172E-03	1.343E-04
0.680	7.238E-02	1.776E-03	3.860	1.818E-03	1.222E-04
0.720	6.599E-02	1.640E-03	3.980	1.368E-03	9.853E-05
0.760	6.501E-02	1.552E-03	4.100	9.753E-04	8.659E-05
0.800	6.325E-02	1.414E-03	4.220	6.294E-04	6.818E-05
0.840	5.855E-02	1.354E-03	4.340	4.504E-04	6.499E-05
0.880	5.821E-02	1.277E-03	4.460	3.370E-04	5.272E-05
0.925	5.439E-02	1.192E-03	4.580	2.359E-04	3.767E-05
0.975	4.952E-02	1.147E-03	4.700	2.035E-04	3.686E-05
1.025	4.893E-02	1.108E-03	4.820	1.801E-04	3.136E-05
1.075	4.546E-02	1.053E-03	4.940	1.291E-04	2.668E-05
1.125	4.357E-02	1.000E-03	5.070	7.363E-05	1.958E-05
1.175	4.155E-02	9.450E-04	5.210	4.209E-05	1.546E-05
1.225	3.923E-02	8.784E-04	5.350	4.437E-05	1.431E-05
1.275	3.831E-02	8.513E-04	5.490	5.092E-05	1.518E-05
1.325	3.530E-02	7.993E-04	5.630	3.595E-05	1.273E-05
1.375	3.281E-02	7.870E-04	5.770	1.939E-05	7.667E-06
1.430	3.213E-02	7.523E-04	5.910	1.338E-05	7.468E-06
1.490	3.099E-02	7.297E-04	6.050	8.870E-06	6.214E-06
1.550	2.835E-02	6.968E-04	6.190	4.143E-06	4.894E-06
1.610	2.638E-02	6.582E-04	6.330	9.614E-07	4.530E-06
1.670	2.552E-02	6.435E-04	6.480	3.015E-07	4.204E-06
1.730	2.330E-02	6.268E-04	6.640	6.282E-07	4.212E-06
1.790	2.198E-02	6.032E-04	6.800	7.755E-07	4.281E-06
1.850	2.145E-02	5.801E-04	6.960	7.332E-07	4.391E-06
1.910	2.018E-02	5.672E-04	7.120	6.755E-07	4.547E-06
1.970	1.928E-02	5.535E-04	7.280	6.879E-07	4.702E-06
2.040	1.818E-02	5.189E-04	7.440	6.559E-07	4.904E-06
2.120	1.625E-02	4.953E-04	7.600	7.952E-07	5.002E-06
2.200	1.520E-02	4.709E-04	7.760	7.152E-07	5.430E-06
2.280	1.388E-02	4.341E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 750 SEC AFTER END OF IRRADIATION
COUNT FOR 400 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	2.652E-01	6.263E-03	2.360	1.837E-02	5.015E-04
0.190	2.379E-01	5.819E-03	2.440	1.642E-02	4.523E-04
0.210	2.218E-01	5.437E-03	2.520	1.472E-02	4.048E-04
0.230	2.121E-01	5.184E-03	2.600	1.331E-02	4.070E-04
0.250	1.995E-01	5.078E-03	2.680	1.282E-02	3.848E-04
0.275	1.934E-01	5.659E-03	2.760	1.167E-02	3.578E-04
0.305	1.828E-01	5.451E-03	2.840	1.110E-02	3.440E-04
0.335	1.749E-01	4.820E-03	2.920	1.012E-02	3.173E-04
0.365	1.661E-01	4.474E-03	3.000	9.397E-03	3.032E-04
0.395	1.610E-01	4.141E-03	3.080	8.565E-03	2.932E-04
0.425	1.498E-01	3.160E-03	3.160	7.566E-03	2.793E-04
0.455	1.384E-01	2.716E-03	3.250	7.050E-03	2.553E-04
0.485	1.317E-01	2.516E-03	3.350	6.509E-03	2.465E-04
0.520	1.233E-01	2.521E-03	3.450	5.402E-03	2.250E-04
0.560	1.239E-01	2.534E-03	3.550	4.401E-03	1.912E-04
0.600	1.150E-01	2.414E-03	3.650	3.638E-03	1.732E-04
0.640	1.075E-01	2.277E-03	3.750	2.994E-03	1.579E-04
0.680	1.041E-01	2.157E-03	3.860	2.367E-03	1.357E-04
0.720	1.014E-01	2.011E-03	3.980	1.700E-03	1.197E-04
0.760	9.670E-02	1.877E-03	4.100	1.222E-03	9.493E-05
0.800	9.148E-02	1.753E-03	4.220	9.059E-04	7.894E-05
0.840	8.649E-02	1.634E-03	4.340	5.663E-04	6.307E-05
0.880	8.178E-02	1.547E-03	4.460	3.192E-04	4.126E-05
0.925	7.934E-02	1.439E-03	4.580	2.167E-04	3.816E-05
0.975	7.641E-02	1.408E-03	4.700	1.551E-04	2.935E-05
1.025	6.954E-02	1.352E-03	4.820	1.198E-04	2.575E-05
1.075	6.693E-02	1.286E-03	4.940	1.068E-04	2.538E-05
1.125	6.438E-02	1.213E-03	5.070	8.952E-05	2.171E-05
1.175	6.030E-02	1.149E-03	5.210	6.781E-05	2.319E-05
1.225	5.604E-02	1.074E-03	5.350	4.049E-05	1.680E-05
1.275	5.274E-02	1.018E-03	5.490	1.730E-05	7.572E-06
1.325	4.977E-02	9.556E-04	5.630	1.424E-05	7.578E-06
1.375	4.799E-02	9.458E-04	5.770	1.724E-05	7.720E-06
1.430	4.482E-02	8.850E-04	5.910	1.401E-05	6.697E-06
1.490	4.280E-02	8.704E-04	6.050	8.551E-06	4.949E-06
1.550	4.041E-02	8.295E-04	6.190	6.005E-06	4.372E-06
1.610	3.713E-02	7.798E-04	6.330	4.231E-06	4.345E-06
1.670	3.577E-02	7.661E-04	6.480	1.831E-06	4.180E-06
1.730	3.389E-02	7.643E-04	6.640	4.724E-07	4.189E-06
1.790	3.188E-02	7.279E-04	6.800	4.612E-07	4.270E-06
1.850	2.973E-02	6.888E-04	6.960	7.219E-07	4.381E-06
1.910	2.740E-02	6.502E-04	7.120	7.505E-07	4.544E-06
1.970	2.592E-02	6.617E-04	7.280	7.043E-07	4.700E-06
2.040	2.431E-02	5.952E-04	7.440	6.796E-07	4.903E-06
2.120	2.255E-02	5.823E-04	7.600	7.878E-07	5.001E-06
2.200	2.093E-02	5.499E-04	7.760	7.187E-07	5.430E-06
2.280	1.916E-02	5.251E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1150 SEC AFTER END OF IRRADIATION
COUNT FOR 400 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.984E-01	5.437E-03	2.360	1.198E-02	4.039E-04
0.190	1.739E-01	5.046E-03	2.440	1.088E-02	3.639E-04
0.210	1.605E-01	4.740E-03	2.520	1.071E-02	3.396E-04
0.230	1.546E-01	4.474E-03	2.600	9.774E-03	3.333E-04
0.250	1.538E-01	4.332E-03	2.680	8.711E-03	3.174E-04
0.275	1.456E-01	4.851E-03	2.760	8.361E-03	2.977E-04
0.305	1.284E-01	4.647E-03	2.840	7.644E-03	2.806E-04
0.335	1.265E-01	4.146E-03	2.920	6.551E-03	2.587E-04
0.365	1.293E-01	3.862E-03	3.000	6.037E-03	2.473E-04
0.395	1.174E-01	3.542E-03	3.080	5.761E-03	2.366E-04
0.425	1.084E-01	2.720E-03	3.160	5.315E-03	2.254E-04
0.455	1.016E-01	2.321E-03	3.250	4.748E-03	2.130E-04
0.485	9.850E-02	2.139E-03	3.350	3.944E-03	1.897E-04
0.520	9.528E-02	2.148E-03	3.450	3.511E-03	1.840E-04
0.560	8.745E-02	2.134E-03	3.550	3.313E-03	1.694E-04
0.600	8.473E-02	2.063E-03	3.650	2.566E-03	1.436E-04
0.640	8.037E-02	1.957E-03	3.750	1.841E-03	1.200E-04
0.680	7.757E-02	1.935E-03	3.860	1.538E-03	1.092E-04
0.720	7.391E-02	1.707E-03	3.980	1.240E-03	9.472E-05
0.760	7.041E-02	1.600E-03	4.100	8.284E-04	7.651E-05
0.800	6.732E-02	1.482E-03	4.220	5.411E-04	6.236E-05
0.840	6.316E-02	1.397E-03	4.340	4.055E-04	5.294E-05
0.880	5.964E-02	1.312E-03	4.460	2.803E-04	4.118E-05
0.925	5.735E-02	1.230E-03	4.580	1.641E-04	2.766E-05
0.975	5.552E-02	1.203E-03	4.700	8.246E-05	1.978E-05
1.025	5.111E-02	1.151E-03	4.820	3.054E-05	1.095E-05
1.075	4.840E-02	1.095E-03	4.940	7.072E-06	5.921E-06
1.125	4.435E-02	1.016E-03	5.070	3.694E-06	4.506E-06
1.175	4.066E-02	9.742E-04	5.210	7.114E-06	4.582E-06
1.225	3.896E-02	9.103E-04	5.350	6.819E-06	5.786E-06
1.275	3.702E-02	8.682E-04	5.490	3.244E-06	4.763E-06
1.325	3.548E-02	8.109E-04	5.630	2.031E-06	4.181E-06
1.375	3.399E-02	7.989E-04	5.770	2.455E-06	4.343E-06
1.430	3.136E-02	7.484E-04	5.910	2.757E-06	4.169E-06
1.490	2.986E-02	7.117E-04	6.050	3.509E-06	4.363E-06
1.550	2.762E-02	7.004E-04	6.190	5.078E-06	4.429E-06
1.610	2.652E-02	6.569E-04	6.330	5.061E-06	4.445E-06
1.670	2.681E-02	6.493E-04	6.480	2.688E-06	4.239E-06
1.730	2.325E-02	6.285E-04	6.640	9.233E-07	4.122E-06
1.790	2.118E-02	6.049E-04	6.800	3.746E-07	4.280E-06
1.850	2.089E-02	5.819E-04	6.960	6.413E-07	4.384E-06
1.910	1.924E-02	5.575E-04	7.120	7.574E-07	4.545E-06
1.970	1.830E-02	5.477E-04	7.280	7.259E-07	4.701E-06
2.040	1.749E-02	4.993E-04	7.440	6.825E-07	4.903E-06
2.120	1.525E-02	4.692E-04	7.600	7.826E-07	5.001E-06
2.200	1.429E-02	4.603E-04	7.760	7.172E-07	5.430E-06
2.280	1.366E-02	4.447E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1550 SEC AFTER END OF IRRADIATION
COUNT FOR 400 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.573E-01	4.905E-03	2.360	8.804E-03	3.406E-04
0.190	1.341E-01	4.571E-03	2.440	7.804E-03	3.196E-04
0.210	1.253E-01	4.226E-03	2.520	7.255E-03	2.875E-04
0.230	1.239E-01	4.034E-03	2.600	6.924E-03	2.792E-04
0.250	1.146E-01	3.866E-03	2.680	6.316E-03	2.678E-04
0.275	1.062E-01	4.203E-03	2.760	5.623E-03	2.474E-04
0.305	1.001E-01	4.070E-03	2.840	5.065E-03	2.301E-04
0.335	9.898E-02	3.661E-03	2.920	4.749E-03	2.210E-04
0.365	9.656E-02	3.413E-03	3.000	4.337E-03	2.028E-04
0.395	8.679E-02	3.121E-03	3.080	3.813E-03	1.908E-04
0.425	8.213E-02	2.410E-03	3.160	3.598E-03	1.912E-04
0.455	7.980E-02	2.053E-03	3.250	3.416E-03	1.783E-04
0.485	7.341E-02	1.892E-03	3.350	2.849E-03	1.624E-04
0.520	7.125E-02	1.881E-03	3.450	2.389E-03	1.469E-04
0.560	6.779E-02	1.880E-03	3.550	2.080E-03	1.310E-04
0.600	6.425E-02	1.805E-03	3.650	1.725E-03	1.185E-04
0.640	6.195E-02	1.741E-03	3.750	1.421E-03	1.063E-04
0.680	5.655E-02	1.636E-03	3.860	1.165E-03	9.192E-05
0.720	5.582E-02	1.511E-03	3.980	8.025E-04	7.788E-05
0.760	5.418E-02	1.386E-03	4.100	5.332E-04	6.084E-05
0.800	5.215E-02	1.307E-03	4.220	4.690E-04	5.634E-05
0.840	5.014E-02	1.223E-03	4.340	3.549E-04	4.663E-05
0.880	4.729E-02	1.146E-03	4.460	1.953E-04	3.197E-05
0.925	4.481E-02	1.074E-03	4.580	1.089E-04	2.489E-05
0.975	4.132E-02	1.045E-03	4.700	6.870E-05	1.759E-05
1.025	4.025E-02	1.015E-03	4.820	3.843E-05	1.302E-05
1.075	3.701E-02	9.787E-04	4.940	1.712E-05	7.767E-06
1.125	3.439E-02	9.205E-04	5.070	6.228E-06	5.193E-06
1.175	3.294E-02	8.606E-04	5.210	2.113E-06	4.391E-06
1.225	3.030E-02	7.993E-04	5.350	3.337E-07	4.151E-06
1.275	2.830E-02	7.705E-04	5.490	5.633E-07	4.151E-06
1.325	2.722E-02	7.177E-04	5.630	6.692E-07	4.153E-06
1.375	2.624E-02	7.098E-04	5.770	6.975E-07	4.166E-06
1.430	2.462E-02	6.631E-04	5.910	6.783E-07	4.201E-06
1.490	2.315E-02	6.361E-04	6.050	6.677E-07	4.236E-06
1.550	2.185E-02	6.107E-04	6.190	6.646E-07	4.281E-06
1.610	2.048E-02	5.815E-04	6.330	6.692E-07	4.286E-06
1.670	1.909E-02	5.503E-04	6.480	6.755E-07	4.156E-06
1.730	1.707E-02	5.406E-04	6.640	7.356E-07	4.180E-06
1.790	1.599E-02	5.317E-04	6.800	7.298E-07	4.272E-06
1.850	1.475E-02	5.091E-04	6.960	5.360E-07	4.384E-06
1.910	1.329E-02	4.931E-04	7.120	4.342E-07	4.525E-06
1.970	1.283E-02	4.705E-04	7.280	1.293E-06	4.613E-06
2.040	1.194E-02	4.117E-04	7.440	2.806E-06	4.950E-06
2.120	1.049E-02	4.083E-04	7.600	3.824E-06	5.158E-06
2.200	9.930E-03	3.897E-04	7.760	2.972E-06	5.437E-06
2.280	9.500E-03	3.737E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1950 SEC AFTER END OF IRRADIATION
COUNT FOR 500 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.501E-01	4.842E-03	2.360	8.393E-03	3.374E-04
0.190	1.317E-01	4.545E-03	2.440	7.201E-03	2.939E-04
0.210	1.238E-01	4.235E-03	2.520	6.316E-03	2.753E-04
0.230	1.208E-01	4.021E-03	2.600	6.046E-03	2.683E-04
0.250	1.109E-01	3.843E-03	2.680	5.509E-03	2.586E-04
0.275	1.042E-01	4.226E-03	2.760	5.021E-03	2.360E-04
0.305	1.045E-01	4.123E-03	2.840	4.714E-03	2.143E-04
0.335	1.005E-01	3.680E-03	2.920	4.366E-03	2.057E-04
0.365	9.149E-02	3.407E-03	3.000	4.077E-03	1.915E-04
0.395	9.037E-02	3.142E-03	3.080	3.731E-03	1.896E-04
0.425	8.757E-02	2.449E-03	3.160	3.376E-03	1.862E-04
0.455	7.994E-02	2.088E-03	3.250	3.117E-03	1.629E-04
0.485	7.535E-02	1.882E-03	3.350	2.489E-03	1.536E-04
0.520	7.050E-02	1.883E-03	3.450	1.945E-03	1.268E-04
0.560	6.777E-02	1.879E-03	3.550	1.858E-03	1.304E-04
0.600	6.351E-02	1.815E-03	3.650	1.470E-03	1.072E-04
0.640	5.986E-02	1.722E-03	3.750	1.147E-03	9.725E-05
0.680	5.683E-02	1.641E-03	3.860	1.106E-03	9.352E-05
0.720	5.570E-02	1.505E-03	3.980	8.210E-04	7.450E-05
0.760	5.328E-02	1.375E-03	4.100	5.330E-04	6.140E-05
0.800	5.013E-02	1.293E-03	4.220	3.794E-04	5.012E-05
0.840	4.866E-02	1.210E-03	4.340	2.669E-04	3.991E-05
0.880	4.784E-02	1.167E-03	4.460	1.916E-04	3.354E-05
0.925	4.570E-02	1.077E-03	4.580	1.278E-04	2.431E-05
0.975	4.141E-02	1.052E-03	4.700	7.214E-05	1.844E-05
1.025	3.930E-02	1.002E-03	4.820	3.367E-05	1.114E-05
1.075	3.659E-02	9.667E-04	4.940	1.271E-05	7.368E-06
1.125	3.379E-02	9.148E-04	5.070	3.248E-06	4.885E-06
1.175	3.288E-02	8.802E-04	5.210	7.567E-07	4.141E-06
1.225	3.052E-02	7.852E-04	5.350	5.389E-07	4.146E-06
1.275	2.757E-02	7.536E-04	5.490	6.105E-07	4.118E-06
1.325	2.673E-02	7.141E-04	5.630	4.818E-07	4.145E-06
1.375	2.584E-02	6.937E-04	5.770	9.084E-07	4.096E-06
1.430	2.371E-02	6.440E-04	5.910	2.074E-06	4.236E-06
1.490	2.231E-02	6.289E-04	6.050	3.693E-06	4.315E-06
1.550	2.174E-02	6.032E-04	6.190	4.359E-06	4.321E-06
1.610	2.050E-02	5.703E-04	6.330	3.784E-06	4.482E-06
1.670	1.818E-02	5.388E-04	6.480	2.321E-06	4.185E-06
1.730	1.637E-02	5.451E-04	6.640	9.294E-07	4.213E-06
1.790	1.534E-02	5.171E-04	6.800	5.190E-07	4.277E-06
1.850	1.424E-02	4.811E-04	6.960	6.951E-07	4.381E-06
1.910	1.357E-02	4.678E-04	7.120	7.673E-07	4.541E-06
1.970	1.312E-02	4.621E-04	7.280	4.835E-07	4.688E-06
2.040	1.173E-02	4.105E-04	7.440	2.948E-07	4.849E-06
2.120	1.020E-02	3.955E-04	7.600	1.485E-06	4.888E-06
2.200	9.402E-03	3.804E-04	7.760	3.452E-06	5.441E-06
2.280	8.826E-03	3.583E-04			

SPECTRUM OF BETA RAYS FOLLOWING F
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 2450 SEC AFTER END OF IRRADIATION
COUNT FOR 500 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.187E-01	4.452E-03	2.360	5.705E-03	2.844E-04
0.190	1.089E-01	4.066E-03	2.440	5.172E-03	2.552E-04
0.210	1.002E-01	3.839E-03	2.520	4.879E-03	2.455E-04
0.230	9.418E-02	3.657E-03	2.600	4.207E-03	2.288E-04
0.250	9.243E-02	3.466E-03	2.680	3.734E-03	2.058E-04
0.275	8.599E-02	3.766E-03	2.760	3.466E-03	1.866E-04
0.305	7.806E-02	3.652E-03	2.840	3.161E-03	1.759E-04
0.335	7.418E-02	3.263E-03	2.920	2.813E-03	1.740E-04
0.365	6.846E-02	3.016E-03	3.000	2.440E-03	1.606E-04
0.395	7.135E-02	2.792E-03	3.080	2.341E-03	1.516E-04
0.425	6.928E-02	2.193E-03	3.160	2.449E-03	1.503E-04
0.455	6.453E-02	1.862E-03	3.250	2.232E-03	1.393E-04
0.485	5.940E-02	1.717E-03	3.350	1.622E-03	1.174E-04
0.520	5.617E-02	1.702E-03	3.450	1.265E-03	1.095E-04
0.560	5.384E-02	1.687E-03	3.550	1.187E-03	9.978E-05
0.600	4.827E-02	1.596E-03	3.650	1.096E-03	9.320E-05
0.640	4.787E-02	1.536E-03	3.750	9.651E-04	8.587E-05
0.680	4.320E-02	1.446E-03	3.860	8.163E-04	7.791E-05
0.720	4.305E-02	1.354E-03	3.980	5.809E-04	6.455E-05
0.760	4.247E-02	1.236E-03	4.100	3.802E-04	4.941E-05
0.800	3.981E-02	1.161E-03	4.220	2.630E-04	4.216E-05
0.840	3.802E-02	1.103E-03	4.340	1.899E-04	3.197E-05
0.880	3.596E-02	1.030E-03	4.460	1.448E-04	2.900E-05
0.925	3.389E-02	9.649E-04	4.580	9.915E-05	2.034E-05
0.975	3.319E-02	9.614E-04	4.700	5.111E-05	1.465E-05
1.025	3.042E-02	9.139E-04	4.820	1.770E-05	8.177E-06
1.075	2.788E-02	8.641E-04	4.940	3.928E-06	5.280E-06
1.125	2.530E-02	8.154E-04	5.070	6.782E-07	4.204E-06
1.175	2.406E-02	7.740E-04	5.210	5.830E-07	4.151E-06
1.225	2.376E-02	7.177E-04	5.350	6.397E-07	4.100E-06
1.275	2.120E-02	6.719E-04	5.490	4.429E-07	4.121E-06
1.325	2.004E-02	6.342E-04	5.630	1.005E-06	4.064E-06
1.375	1.943E-02	6.223E-04	5.770	2.346E-06	4.192E-06
1.430	1.795E-02	5.832E-04	5.910	3.262E-06	4.348E-06
1.490	1.712E-02	5.561E-04	6.050	2.588E-06	4.250E-06
1.550	1.581E-02	5.255E-04	6.190	1.199E-06	4.323E-06
1.610	1.479E-02	5.069E-04	6.330	5.508E-07	4.292E-06
1.670	1.379E-02	4.660E-04	6.480	5.853E-07	4.151E-06
1.730	1.296E-02	4.762E-04	6.640	5.958E-07	4.180E-06
1.790	1.211E-02	4.443E-04	6.800	4.032E-07	4.264E-06
1.850	1.091E-02	4.257E-04	6.960	1.035E-06	4.302E-06
1.910	1.055E-02	4.088E-04	7.120	2.475E-06	4.470E-06
1.970	9.975E-03	4.194E-04	7.280	3.587E-06	4.735E-06
2.040	8.144E-03	3.768E-04	7.440	3.001E-06	4.889E-06
2.120	7.118E-03	3.499E-04	7.600	1.497E-06	5.031E-06
2.200	6.988E-03	3.309E-04	7.760	5.156E-07	5.437E-06
2.280	6.611E-03	3.052E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 2950 SEC AFTER END OF IRRADIATION
COUNT FOR 1000 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.905E-01	5.535E-03	2.360	7.378E-03	3.407E-04
0.190	1.660E-01	5.117E-03	2.440	6.614E-03	2.981E-04
0.210	1.511E-01	4.753E-03	2.520	5.952E-03	2.839E-04
0.230	1.481E-01	4.514E-03	2.600	5.698E-03	2.669E-04
0.250	1.383E-01	4.363E-03	2.680	5.187E-03	2.448E-04
0.275	1.313E-01	4.762E-03	2.760	4.568E-03	2.203E-04
0.305	1.275E-01	4.701E-03	2.840	4.213E-03	2.093E-04
0.335	1.112E-01	4.118E-03	2.920	3.766E-03	1.903E-04
0.365	1.068E-01	3.804E-03	3.000	3.396E-03	1.812E-04
0.395	1.041E-01	3.520E-03	3.080	3.152E-03	1.765E-04
0.425	9.861E-02	2.729E-03	3.160	2.742E-03	1.635E-04
0.455	9.024E-02	2.345E-03	3.250	2.473E-03	1.500E-04
0.485	8.456E-02	2.140E-03	3.350	2.578E-03	1.554E-04
0.520	7.857E-02	2.126E-03	3.450	2.350E-03	1.402E-04
0.560	7.659E-02	2.116E-03	3.550	1.867E-03	1.260E-04
0.600	7.645E-02	2.064E-03	3.650	1.485E-03	1.078E-04
0.640	7.198E-02	1.952E-03	3.750	1.174E-03	9.818E-05
0.680	6.794E-02	1.825E-03	3.860	9.362E-04	8.615E-05
0.720	6.496E-02	1.687E-03	3.980	8.149E-04	7.963E-05
0.760	6.170E-02	1.555E-03	4.100	7.016E-04	6.943E-05
0.800	5.933E-02	1.439E-03	4.220	5.385E-04	5.990E-05
0.840	5.721E-02	1.352E-03	4.340	3.447E-04	4.594E-05
0.880	5.532E-02	1.293E-03	4.460	2.169E-04	3.635E-05
0.925	5.198E-02	1.185E-03	4.580	1.551E-04	2.908E-05
0.975	5.004E-02	1.160E-03	4.700	9.552E-05	2.173E-05
1.025	4.625E-02	1.128E-03	4.820	4.473E-05	1.363E-05
1.075	4.182E-02	1.068E-03	4.940	1.921E-05	8.774E-06
1.125	4.028E-02	1.012E-03	5.070	9.880E-06	6.302E-06
1.175	3.761E-02	9.610E-04	5.210	4.781E-06	5.168E-06
1.225	3.435E-02	8.810E-04	5.350	1.607E-06	4.298E-06
1.275	3.186E-02	8.229E-04	5.490	2.004E-07	4.262E-06
1.325	2.970E-02	7.820E-04	5.630	5.665E-07	4.121E-06
1.375	2.862E-02	7.505E-04	5.770	2.555E-06	4.268E-06
1.430	2.780E-02	7.083E-04	5.910	5.147E-06	4.375E-06
1.490	2.534E-02	6.901E-04	6.050	5.449E-06	4.463E-06
1.550	2.333E-02	6.498E-04	6.190	3.071E-06	4.380E-06
1.610	2.133E-02	6.036E-04	6.330	8.584E-07	4.248E-06
1.670	1.983E-02	5.726E-04	6.480	6.501E-07	4.097E-06
1.730	1.857E-02	5.837E-04	6.640	2.315E-06	4.215E-06
1.790	1.641E-02	5.371E-04	6.800	3.438E-06	4.448E-06
1.850	1.480E-02	5.056E-04	6.960	2.668E-06	4.400E-06
1.910	1.379E-02	5.020E-04	7.120	1.140E-06	4.594E-06
1.970	1.287E-02	4.811E-04	7.280	4.840E-07	4.708E-06
2.040	1.168E-02	4.257E-04	7.440	5.497E-07	4.903E-06
2.120	1.031E-02	4.092E-04	7.600	8.164E-07	5.002E-06
2.200	9.762E-03	3.968E-04	7.760	7.599E-07	5.430E-06
2.280	8.446E-03	3.543E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 3950 SEC AFTER END OF IRRADIATION
COUNT FOR 2000 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	2.343E-01	6.422E-03	2.360	8.238E-03	3.673E-04
0.190	2.075E-01	5.965E-03	2.440	7.300E-03	3.276E-04
0.210	1.970E-01	5.579E-03	2.520	5.858E-03	3.003E-04
0.230	1.855E-01	5.344E-03	2.600	5.316E-03	2.650E-04
0.250	1.741E-01	5.149E-03	2.680	5.154E-03	2.485E-04
0.275	1.723E-01	5.703E-03	2.760	4.846E-03	2.337E-04
0.305	1.522E-01	5.557E-03	2.840	4.366E-03	2.168E-04
0.335	1.466E-01	4.864E-03	2.920	4.077E-03	2.048E-04
0.365	1.411E-01	4.510E-03	3.000	3.708E-03	1.937E-04
0.395	1.324E-01	4.141E-03	3.080	3.090E-03	1.708E-04
0.425	1.225E-01	3.195E-03	3.160	2.705E-03	1.588E-04
0.455	1.129E-01	2.757E-03	3.250	2.465E-03	1.561E-04
0.485	1.076E-01	2.516E-03	3.350	2.110E-03	1.427E-04
0.520	1.035E-01	2.508E-03	3.450	1.988E-03	1.347E-04
0.560	9.781E-02	2.516E-03	3.550	1.812E-03	1.235E-04
0.600	9.650E-02	2.432E-03	3.650	1.426E-03	1.037E-04
0.640	8.888E-02	2.312E-03	3.750	1.113E-03	1.005E-04
0.680	8.475E-02	2.118E-03	3.860	9.173E-04	8.547E-05
0.720	8.166E-02	1.978E-03	3.980	8.651E-04	8.135E-05
0.760	7.682E-02	1.781E-03	4.100	7.214E-04	7.104E-05
0.800	7.777E-02	1.670E-03	4.220	4.611E-04	5.639E-05
0.840	7.533E-02	1.560E-03	4.340	3.522E-04	4.786E-05
0.880	7.147E-02	1.486E-03	4.460	3.043E-04	4.362E-05
0.925	6.637E-02	1.391E-03	4.580	2.023E-04	3.309E-05
0.975	6.053E-02	1.341E-03	4.700	1.211E-04	2.415E-05
1.025	5.774E-02	1.288E-03	4.820	7.443E-05	1.964E-05
1.075	5.310E-02	1.238E-03	4.940	3.904E-05	1.259E-05
1.125	4.976E-02	1.145E-03	5.070	1.693E-05	7.679E-06
1.175	4.627E-02	1.080E-03	5.210	8.009E-06	6.082E-06
1.225	4.261E-02	9.978E-04	5.350	3.360E-06	4.705E-06
1.275	4.018E-02	9.454E-04	5.490	8.753E-07	4.182E-06
1.325	3.748E-02	8.752E-04	5.630	1.729E-07	4.188E-06
1.375	3.499E-02	8.340E-04	5.770	1.009E-06	4.120E-06
1.430	3.299E-02	7.696E-04	5.910	2.231E-06	4.236E-06
1.490	3.104E-02	7.505E-04	6.050	2.952E-06	4.395E-06
1.550	2.847E-02	7.208E-04	6.190	3.106E-06	4.246E-06
1.610	2.547E-02	6.747E-04	6.330	3.160E-06	4.365E-06
1.670	2.377E-02	6.285E-04	6.480	3.153E-06	4.408E-06
1.730	2.154E-02	6.231E-04	6.640	2.132E-06	4.214E-06
1.790	1.927E-02	6.099E-04	6.800	1.492E-06	4.254E-06
1.850	1.770E-02	5.685E-04	6.960	2.399E-06	4.423E-06
1.910	1.634E-02	5.388E-04	7.120	3.421E-06	4.681E-06
1.970	1.507E-02	5.357E-04	7.280	2.753E-06	4.703E-06
2.040	1.307E-02	4.709E-04	7.440	1.243E-06	4.954E-06
2.120	1.134E-02	4.447E-04	7.600	5.742E-07	5.006E-06
2.200	1.005E-02	4.198E-04	7.760	5.336E-07	5.430E-06
2.280	8.451E-03	3.973E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 5950 SEC AFTER END OF IRRADIATION
COUNT FOR 4000 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	2.657E-01	7.293E-03	2.360	7.460E-03	3.697E-04
0.190	2.415E-01	6.769E-03	2.440	6.551E-03	3.197E-04
0.210	2.224E-01	6.333E-03	2.520	5.459E-03	3.003E-04
0.230	2.062E-01	5.988E-03	2.600	4.887E-03	2.747E-04
0.250	1.936E-01	5.806E-03	2.680	4.383E-03	2.496E-04
0.275	1.853E-01	6.435E-03	2.760	3.853E-03	2.157E-04
0.305	1.766E-01	6.236E-03	2.840	3.412E-03	1.959E-04
0.335	1.668E-01	5.539E-03	2.920	3.004E-03	1.813E-04
0.365	1.583E-01	5.091E-03	3.000	2.685E-03	1.663E-04
0.395	1.499E-01	4.665E-03	3.080	2.425E-03	1.554E-04
0.425	1.362E-01	3.626E-03	3.160	2.164E-03	1.405E-04
0.455	1.255E-01	3.151E-03	3.250	1.952E-03	1.331E-04
0.485	1.212E-01	2.872E-03	3.350	1.754E-03	1.246E-04
0.520	1.136E-01	2.841E-03	3.450	1.440E-03	1.185E-04
0.560	1.112E-01	2.837E-03	3.550	1.193E-03	9.508E-05
0.600	1.006E-01	2.725E-03	3.650	1.055E-03	9.658E-05
0.640	9.616E-02	2.566E-03	3.750	9.437E-04	8.271E-05
0.680	9.946E-02	2.379E-03	3.860	8.751E-04	8.315E-05
0.720	9.302E-02	2.174E-03	3.980	6.787E-04	6.906E-05
0.760	8.988E-02	2.003E-03	4.100	4.592E-04	6.170E-05
0.800	8.372E-02	1.864E-03	4.220	3.691E-04	5.083E-05
0.840	7.909E-02	1.745E-03	4.340	3.018E-04	4.673E-05
0.880	7.611E-02	1.671E-03	4.460	2.160E-04	3.802E-05
0.925	7.105E-02	1.539E-03	4.580	1.465E-04	2.574E-05
0.975	6.583E-02	1.494E-03	4.700	8.751E-05	2.037E-05
1.025	6.271E-02	1.434E-03	4.820	3.720E-05	1.387E-05
1.075	5.888E-02	1.362E-03	4.940	1.891E-05	1.040E-05
1.125	5.470E-02	1.278E-03	5.070	2.336E-05	8.940E-06
1.175	5.130E-02	1.199E-03	5.210	2.136E-05	7.711E-06
1.225	4.767E-02	1.091E-03	5.350	9.166E-06	6.741E-06
1.275	4.497E-02	1.011E-03	5.490	3.291E-06	4.588E-06
1.325	4.016E-02	9.396E-04	5.630	4.674E-06	4.205E-06
1.375	3.737E-02	8.784E-04	5.770	8.562E-06	4.265E-06
1.430	3.588E-02	8.118E-04	5.910	1.124E-05	4.606E-06
1.490	3.208E-02	7.803E-04	6.050	1.165E-05	5.352E-06
1.550	2.918E-02	7.532E-04	6.190	8.660E-06	5.311E-06
1.610	2.686E-02	6.955E-04	6.330	4.533E-06	4.583E-06
1.670	2.471E-02	6.658E-04	6.480	2.351E-06	4.270E-06
1.730	2.186E-02	6.578E-04	6.640	2.653E-06	4.345E-06
1.790	1.924E-02	6.272E-04	6.800	2.676E-06	4.461E-06
1.850	1.672E-02	5.815E-04	6.960	1.489E-06	4.444E-06
1.910	1.431E-02	5.619E-04	7.120	6.046E-07	4.583E-06
1.970	1.324E-02	5.375E-04	7.280	5.331E-07	4.704E-06
2.040	1.279E-02	4.860E-04	7.440	6.845E-07	4.904E-06
2.120	1.112E-02	4.389E-04	7.600	8.412E-07	5.007E-06
2.200	8.721E-03	4.161E-04	7.760	7.294E-07	5.430E-06
2.280	7.658E-03	3.850E-04			

SPECTRUM OF BETA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 9950 SEC AFTER END OF IRRADIATION
COUNT FOR 3550 SEC

E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)	E(BETA) MEV	Y(BETA) BETAS/MEV/FISSION	DELTA(Y)
0.170	1.665E-01	6.325E-03	2.360	3.772E-03	2.749E-04
0.190	1.488E-01	5.845E-03	2.440	3.133E-03	2.584E-04
0.210	1.330E-01	5.429E-03	2.520	2.998E-03	2.229E-04
0.230	1.298E-01	5.082E-03	2.600	2.554E-03	2.188E-04
0.250	1.319E-01	4.838E-03	2.680	2.070E-03	1.931E-04
0.275	1.201E-01	5.228E-03	2.760	2.105E-03	1.646E-04
0.305	1.043E-01	5.056E-03	2.840	2.191E-03	1.565E-04
0.335	1.039E-01	4.532E-03	2.920	1.852E-03	1.280E-04
0.365	9.512E-02	4.203E-03	3.000	1.450E-03	1.161E-04
0.395	9.023E-02	3.840E-03	3.080	1.268E-03	1.036E-04
0.425	7.973E-02	3.022E-03	3.160	1.111E-03	9.907E-05
0.455	7.555E-02	2.552E-03	3.250	8.631E-04	9.140E-05
0.485	7.479E-02	2.348E-03	3.350	7.294E-04	8.352E-05
0.520	7.101E-02	2.327E-03	3.450	6.774E-04	7.609E-05
0.560	6.992E-02	2.313E-03	3.550	5.416E-04	6.505E-05
0.600	6.673E-02	2.235E-03	3.650	4.125E-04	5.902E-05
0.640	6.367E-02	2.096E-03	3.750	3.268E-04	5.922E-05
0.680	5.736E-02	1.932E-03	3.860	2.793E-04	5.218E-05
0.720	5.705E-02	1.769E-03	3.980	2.527E-04	4.295E-05
0.760	5.297E-02	1.631E-03	4.100	1.666E-04	3.315E-05
0.800	5.045E-02	1.511E-03	4.220	1.086E-04	2.532E-05
0.840	4.989E-02	1.443E-03	4.340	1.224E-04	2.646E-05
0.880	4.690E-02	1.393E-03	4.460	1.154E-04	2.478E-05
0.925	4.456E-02	1.294E-03	4.580	7.335E-05	1.842E-05
0.975	3.879E-02	1.248E-03	4.700	4.063E-05	1.344E-05
1.025	3.771E-02	1.199E-03	4.820	2.346E-05	1.054E-05
1.075	3.591E-02	1.122E-03	4.940	1.185E-05	6.708E-06
1.125	3.269E-02	1.033E-03	5.070	6.577E-06	5.369E-06
1.175	3.048E-02	9.525E-04	5.210	6.880E-06	5.182E-06
1.225	2.781E-02	8.784E-04	5.350	7.452E-06	5.120E-06
1.275	2.602E-02	7.958E-04	5.490	5.134E-06	4.573E-06
1.325	2.448E-02	7.350E-04	5.630	2.707E-06	4.898E-06
1.375	2.273E-02	6.879E-04	5.770	2.857E-06	5.338E-06
1.430	2.054E-02	6.320E-04	5.910	2.671E-06	5.109E-06
1.490	1.835E-02	5.978E-04	6.050	2.669E-06	4.364E-06
1.550	1.704E-02	5.761E-04	6.190	3.380E-06	4.338E-06
1.610	1.592E-02	5.490E-04	6.330	4.561E-06	4.476E-06
1.670	1.444E-02	5.167E-04	6.480	5.056E-06	4.328E-06
1.730	1.239E-02	5.291E-04	6.640	4.767E-06	4.352E-06
1.790	1.117E-02	4.794E-04	6.800	5.456E-06	4.380E-06
1.850	1.033E-02	4.651E-04	6.960	5.201E-06	4.643E-06
1.910	9.067E-03	4.389E-04	7.120	3.108E-06	4.640E-06
1.970	7.940E-03	4.311E-04	7.280	1.138E-06	4.646E-06
2.040	7.031E-03	3.839E-04	7.440	3.412E-07	4.917E-06
2.120	5.999E-03	3.654E-04	7.600	6.421E-07	5.002E-06
2.200	5.213E-03	3.458E-04	7.760	7.750E-07	5.430E-06
2.280	4.723E-03	3.074E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1.7 SEC AFTER END OF IRRADIATION
COUNT FOR 1 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	6.644E-02	2.294E-02	1.940	2.661E-02	4.447E-03
0.065	5.819E-02	2.371E-02	1.980	2.375E-02	4.390E-03
0.075	8.902E-02	2.578E-02	2.020	2.073E-02	4.217E-03
0.085	8.564E-02	2.711E-02	2.060	2.106E-02	4.161E-03
0.095	1.875E-01	3.228E-02	2.100	1.958E-02	3.934E-03
0.105	1.999E-01	3.293E-02	2.140	1.634E-02	3.731E-03
0.115	3.303E-01	3.885E-02	2.180	1.581E-02	3.641E-03
0.125	3.502E-01	3.948E-02	2.220	1.593E-02	3.732E-03
0.135	2.101E-01	3.194E-02	2.260	1.631E-02	3.630E-03
0.145	2.212E-01	3.185E-02	2.300	1.790E-02	3.800E-03
0.155	2.146E-01	3.190E-02	2.340	1.842E-02	3.638E-03
0.165	1.861E-01	3.147E-02	2.380	1.666E-02	3.716E-03
0.177	1.744E-01	2.782E-02	2.425	1.370E-02	3.392E-03
0.192	1.504E-01	2.558E-02	2.475	1.428E-02	3.518E-03
0.207	1.440E-01	2.430E-02	2.525	1.614E-02	3.686E-03
0.222	9.575E-02	2.157E-02	2.575	1.424E-02	3.410E-03
0.237	9.042E-02	2.080E-02	2.625	8.989E-03	3.084E-03
0.252	1.302E-01	2.219E-02	2.675	1.019E-02	3.318E-03
0.267	1.356E-01	2.269E-02	2.725	1.370E-02	3.493E-03
0.282	1.453E-01	2.285E-02	2.775	7.259E-03	2.879E-03
0.297	1.770E-01	2.422E-02	2.825	7.410E-03	2.840E-03
0.313	1.115E-01	1.994E-02	2.875	1.364E-02	3.156E-03
0.327	7.916E-02	1.719E-02	2.925	9.374E-03	2.818E-03
0.342	9.228E-02	1.793E-02	2.975	8.227E-03	2.711E-03
0.357	1.009E-01	1.825E-02	3.030	1.123E-02	2.663E-03
0.372	1.077E-01	1.859E-02	3.090	8.185E-03	2.542E-03
0.387	1.337E-01	1.917E-02	3.150	7.076E-03	2.416E-03
0.402	1.540E-01	1.980E-02	3.210	1.178E-02	2.638E-03
0.417	1.289E-01	1.808E-02	3.270	5.259E-02	2.842E-03
0.432	1.051E-01	1.670E-02	3.330	1.125E-02	2.567E-03
0.447	1.201E-01	1.227E-02	3.390	1.289E-02	2.736E-03
0.462	1.732E-01	1.340E-02	3.450	8.719E-03	2.279E-03
0.477	1.782E-01	1.390E-02	3.510	4.352E-03	1.928E-03
0.492	1.552E-01	1.267E-02	3.570	4.901E-03	1.878E-03
0.507	1.565E-01	1.297E-02	3.630	4.056E-03	1.885E-03
0.523	1.895E-01	1.415E-02	3.690	1.984E-03	1.590E-03
0.540	2.437E-01	1.563E-02	3.750	3.353E-03	1.784E-03
0.556	2.149E-01	1.489E-02	3.810	4.129E-03	1.875E-03
0.580	1.589E-01	1.277E-02	3.870	2.687E-03	1.739E-03
0.600	1.616E-01	1.262E-02	3.935	2.998E-03	1.785E-03
0.620	1.224E-01	1.104E-02	4.005	2.645E-03	1.671E-03
0.640	8.841E-02	9.464E-03	4.075	2.566E-03	1.435E-03
0.660	7.754E-02	9.320E-03	4.145	4.681E-03	1.777E-03
0.680	6.201E-02	8.234E-03	4.215	4.367E-03	1.584E-03
0.700	5.975E-02	8.454E-03	4.285	4.120E-03	1.618E-03
0.720	5.620E-02	8.008E-03	4.355	5.539E-03	1.696E-03
0.740	5.855E-02	7.930E-03	4.425	6.104E-03	1.529E-03
0.760	6.558E-02	8.553E-03	4.495	3.299E-03	1.310E-03
0.780	7.649E-02	8.764E-03	4.565	8.540E-04	8.732E-04
0.800	1.165E-01	1.005E-02	4.635	6.829E-04	1.002E-03
0.820	1.382E-01	1.045E-02	4.705	1.284E-03	8.776E-04
0.840	1.028E-01	9.131E-03	4.775	1.291E-03	8.662E-04
0.860	6.998E-02	7.751E-03	4.845	6.989E-04	7.474E-04
0.880	6.356E-02	7.960E-03	4.915	1.546E-04	5.911E-04
0.900	5.671E-02	7.393E-03	4.985	4.240E-04	6.745E-04
0.920	5.635E-02	7.550E-03	5.060	1.217E-03	7.666E-04
0.940	7.137E-02	8.014E-03	5.140	1.589E-03	8.047E-04
0.962	8.254E-02	8.424E-03	5.220	1.111E-03	7.333E-04
0.987	7.295E-02	8.022E-03	5.300	4.972E-04	6.524E-04
1.013	6.109E-02	7.368E-03	5.380	3.978E-04	5.935E-04
1.037	5.538E-02	7.415E-03	5.460	6.213E-04	5.496E-04
1.062	5.841E-02	7.283E-03	5.540	4.726E-04	5.351E-04
1.086	7.811E-02	8.085E-03	5.620	3.490E-04	4.451E-04
1.112	8.971E-02	8.352E-03	5.700	2.413E-04	5.292E-04
1.138	7.471E-02	7.654E-03	5.780	3.819E-04	6.121E-04
1.162	5.264E-02	6.828E-03	5.860	3.316E-04	5.247E-04
1.187	4.898E-02	6.692E-03	5.945	7.530E-05	4.391E-04
1.215	6.253E-02	7.215E-03	6.035	-1.195E-04	3.279E-04
1.245	6.322E-02	7.249E-03	6.125	5.305E-05	3.217E-04
1.275	5.428E-02	6.806E-03	6.215	5.742E-04	4.353E-04
1.305	5.040E-02	6.833E-03	6.305	8.111E-04	4.756E-04
1.335	3.724E-02	5.786E-03	6.395	5.998E-04	3.840E-04
1.365	3.157E-02	5.917E-03	6.485	4.228E-04	3.273E-04
1.395	4.118E-02	5.958E-03	6.575	2.436E-04	2.627E-04
1.425	4.050E-02	6.068E-03	6.665	9.068E-05	2.144E-04
1.455	2.662E-02	5.010E-03	6.755	3.998E-05	2.079E-04
1.485	2.633E-02	5.123E-03	6.850	5.603E-05	1.814E-04
1.515	3.125E-02	5.061E-03	6.950	6.806E-05	1.753E-04
1.545	2.979E-02	4.909E-03	7.050	7.264E-05	1.697E-04
1.580	3.310E-02	5.226E-03	7.150	5.288E-05	1.613E-04
1.620	3.200E-02	5.109E-03	7.250	3.127E-05	1.587E-04
1.660	2.571E-02	4.712E-03	7.350	1.769E-05	1.470E-04
1.700	2.474E-02	4.637E-03	7.450	2.533E-05	1.380E-04
1.740	2.997E-02	4.841E-03	7.550	1.456E-05	1.395E-04
1.780	3.316E-02	5.164E-03	7.650	9.129E-06	1.394E-04
1.820	2.704E-02	4.631E-03	7.750	1.529E-05	1.338E-04
1.860	2.673E-02	4.667E-03	7.850	1.906E-05	1.278E-04
1.900	2.549E-02	4.481E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 2.7 SEC AFTER END OF IRRADIATION
COUNT FOR 1 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	5.238E-02	2.063E-02	1.940	2.131E-02	4.064E-03
0.065	2.545E-02	2.060E-02	1.980	2.173E-02	3.786E-03
0.075	5.566E-02	2.359E-02	2.020	2.097E-02	3.973E-03
0.085	1.015E-01	2.597E-02	2.060	1.522E-02	3.362E-03
0.095	1.283E-01	2.766E-02	2.100	1.217E-02	3.276E-03
0.105	2.013E-01	3.146E-02	2.140	1.157E-02	3.089E-03
0.115	2.260E-01	3.270E-02	2.180	1.192E-02	3.002E-03
0.125	2.869E-01	3.528E-02	2.220	1.107E-02	3.210E-03
0.135	2.053E-01	3.023E-02	2.260	1.165E-02	2.854E-03
0.145	1.610E-01	2.718E-02	2.300	1.272E-02	3.171E-03
0.155	1.903E-01	2.874E-02	2.340	7.689E-03	2.655E-03
0.165	1.867E-01	3.024E-02	2.380	5.747E-03	2.481E-03
0.177	1.089E-01	2.362E-02	2.425	9.311E-03	2.919E-03
0.192	1.468E-01	2.373E-02	2.475	9.635E-03	2.702E-03
0.207	1.112E-01	2.140E-02	2.525	1.130E-02	3.085E-03
0.222	9.372E-02	2.008E-02	2.575	1.244E-02	2.880E-03
0.237	8.028E-02	1.668E-02	2.625	1.423E-02	3.250E-03
0.252	1.057E-01	1.968E-02	2.675	1.097E-02	2.878E-03
0.267	1.229E-01	2.081E-02	2.725	8.848E-03	2.714E-03
0.282	1.180E-01	2.018E-02	2.775	1.020E-02	2.833E-03
0.297	1.266E-01	2.007E-02	2.825	7.333E-03	2.537E-03
0.313	9.102E-02	1.751E-02	2.875	6.358E-03	2.353E-03
0.327	6.235E-02	1.505E-02	2.925	8.545E-03	2.598E-03
0.342	7.628E-02	1.575E-02	2.975	7.356E-03	2.308E-03
0.357	7.762E-02	1.549E-02	3.030	4.733E-03	2.161E-03
0.372	7.643E-02	1.514E-02	3.090	4.724E-03	1.914E-03
0.387	9.140E-02	1.603E-02	3.150	6.212E-03	2.218E-03
0.402	1.161E-01	1.663E-02	3.210	5.693E-03	1.906E-03
0.417	1.140E-01	1.677E-02	3.270	6.821E-03	2.131E-03
0.432	9.255E-02	1.487E-02	3.330	7.485E-03	2.111E-03
0.447	1.146E-01	1.089E-02	3.390	9.027E-03	2.268E-03
0.462	1.433E-01	1.208E-02	3.450	9.157E-03	2.229E-03
0.477	1.352E-01	1.194E-02	3.510	5.436E-03	1.824E-03
0.492	1.261E-01	1.164E-02	3.570	3.221E-03	1.567E-03
0.507	1.322E-01	1.150E-02	3.630	2.535E-03	1.571E-03
0.522	1.543E-01	1.253E-02	3.690	2.461E-03	1.294E-03
0.540	1.908E-01	1.355E-02	3.750	3.271E-03	1.570E-03
0.560	1.731E-01	1.294E-02	3.810	2.736E-03	1.391E-03
0.580	1.341E-01	1.147E-02	3.870	3.005E-03	1.531E-03
0.600	1.375E-01	1.124E-02	3.935	3.386E-03	1.560E-03
0.620	9.496E-02	9.589E-03	4.005	1.605E-03	1.252E-03
0.640	6.521E-02	7.966E-03	4.075	2.317E-03	1.281E-03
0.660	6.326E-02	8.307E-03	4.145	2.939E-03	1.316E-03
0.680	5.269E-02	7.424E-03	4.215	1.842E-03	1.046E-03
0.700	5.708E-02	7.783E-03	4.285	2.538E-03	1.194E-03
0.720	5.291E-02	7.450E-03	4.355	3.548E-03	1.271E-03
0.740	5.037E-02	7.185E-03	4.425	3.321E-03	1.200E-03
0.760	6.243E-02	7.917E-03	4.495	2.232E-03	1.155E-03
0.780	6.419E-02	7.804E-03	4.565	1.364E-03	9.227E-04
0.800	7.297E-02	8.032E-03	4.635	1.162E-03	8.678E-04
0.820	6.741E-02	8.594E-03	4.705	5.526E-04	6.157E-04
0.840	8.172E-02	8.043E-03	4.775	-7.766E-05	4.644E-04
0.860	6.595E-02	7.585E-03	4.845	2.162E-05	3.857E-04
0.880	5.328E-02	6.883E-03	4.915	6.589E-04	5.991E-04
0.900	4.872E-02	6.699E-03	4.985	1.300E-03	6.644E-04
0.920	5.160E-02	6.754E-03	5.060	1.557E-03	7.346E-04
0.940	5.489E-02	7.046E-03	5.140	1.103E-03	6.814E-04
0.962	5.799E-02	6.799E-03	5.220	2.986E-04	4.713E-04
0.987	5.413E-02	6.977E-03	5.300	3.222E-05	4.200E-04
1.013	3.965E-02	6.001E-03	5.380	1.707E-04	4.516E-04
1.037	3.991E-02	6.143E-03	5.460	3.096E-04	4.834E-04
1.062	5.340E-02	6.659E-03	5.540	3.253E-04	4.362E-04
1.088	6.273E-02	6.982E-03	5.620	4.163E-04	4.853E-04
1.112	6.125E-02	6.791E-03	5.700	4.321E-04	4.528E-04
1.138	4.851E-02	6.336E-03	5.780	3.244E-04	3.751E-04
1.162	4.037E-02	5.708E-03	5.860	1.754E-04	3.022E-04
1.187	3.985E-02	6.190E-03	5.945	9.125E-05	1.994E-04
1.215	4.297E-02	5.870E-03	6.035	1.590E-04	2.430E-04
1.245	4.833E-02	6.414E-03	6.125	3.796E-04	3.507E-04
1.275	3.951E-02	5.737E-03	6.215	3.247E-04	3.606E-04
1.305	3.519E-02	5.443E-03	6.305	1.499E-04	2.812E-04
1.335	3.279E-02	5.519E-03	6.395	5.744E-05	2.120E-04
1.365	2.368E-02	4.807E-03	6.485	-3.548E-05	1.754E-04
1.395	2.795E-02	5.107E-03	6.575	-5.203E-05	1.961E-04
1.425	3.276E-02	5.128E-03	6.665	1.096E-04	2.125E-04
1.455	2.591E-02	4.916E-03	6.755	3.134E-04	2.296E-04
1.485	2.489E-02	4.952 E-03	6.850	3.461E-04	2.187E-04
1.515	2.665E-02	4.719E-03	6.950	1.745E-04	1.777E-04
1.545	2.622E-02	4.112E-03	7.050	3.558E-05	1.449E-04
1.580	3.087E-02	4.830E-03	7.150	2.055E-05	1.592E-04
1.620	2.567E-02	4.379E-03	7.250	6.238E-05	1.636E-04
1.660	2.045E-02	4.127E-03	7.350	7.831E-05	1.679E-04
1.700	2.390E-02	4.235E-03	7.450	6.756E-05	1.551E-04
1.740	2.918E-02	4.523E-03	7.550	3.500E-05	1.290E-04
1.780	2.418E-02	4.295E-03	7.650	1.537E-05	1.402E-04
1.820	1.629E-02	3.745E-03	7.750	6.644E-06	1.335E-04
1.860	2.133E-02	3.984E-03	7.850	3.475E-06	1.239E-04
1.900	2.414E-02	4.054E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 3.7 SEC AFTER END OF IRRADIATION
COUNT FOR 1 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.669E-02	8.142E-03	1.940	1.488E-02	1.765E-03
0.065	5.655E-02	9.342E-03	1.980	1.591E-02	1.765E-03
0.075	5.730E-02	9.348E-03	2.020	1.652E-02	1.804E-03
0.085	7.630E-02	1.015E-02	2.060	1.458E-02	1.656E-03
0.095	1.206E-01	1.139E-02	2.100	1.248E-02	1.611E-03
0.105	1.635E-01	1.239E-02	2.140	9.805E-03	1.471E-03
0.115	1.752E-01	1.271E-02	2.180	9.775E-03	1.477E-03
0.125	1.893E-01	1.301E-02	2.220	1.112E-02	1.579E-03
0.135	1.404E-01	1.175E-02	2.260	1.204E-02	1.535E-03
0.145	1.453E-01	1.187E-02	2.300	1.116E-02	1.527E-03
0.155	1.472E-01	1.215E-02	2.340	9.095E-03	1.370E-03
0.165	1.254E-01	1.169E-02	2.380	9.094E-03	1.501E-03
0.177	9.558E-02	9.455E-03	2.425	9.732E-03	1.447E-03
0.192	8.491E-02	8.597E-03	2.475	9.858E-03	1.457E-03
0.207	8.253E-02	8.291E-03	2.525	8.226E-03	1.367E-03
0.222	8.555E-02	8.309E-03	2.575	7.573E-03	1.269E-03
0.237	6.416E-02	7.615E-03	2.625	8.623E-03	1.315E-03
0.252	6.626E-02	7.450E-03	2.675	6.080E-03	1.203E-03
0.267	8.473E-02	8.054E-03	2.725	6.564E-03	1.281E-03
0.282	1.102E-01	8.891E-03	2.775	9.118E-03	1.323E-03
0.297	1.202E-01	9.085E-03	2.825	7.533E-03	1.242E-03
0.313	7.241E-02	7.505E-03	2.875	7.372E-03	1.188E-03
0.327	4.271E-02	6.144E-03	2.925	7.633E-03	1.224E-03
0.342	3.564E-02	5.847E-03	2.975	5.721E-03	1.109E-03
0.357	4.242E-02	5.849E-03	3.030	5.607E-03	1.094E-03
0.372	6.999E-02	6.905E-03	3.090	5.009E-03	1.003E-03
0.387	8.016E-02	7.117E-03	3.150	4.150E-03	9.618E-04
0.402	9.390E-02	7.490E-03	3.210	4.715E-03	9.549E-04
0.417	9.041E-02	7.081E-03	3.270	6.955E-03	1.070E-03
0.432	7.923E-02	6.732E-03	3.330	6.553E-03	1.049E-03
0.447	9.123E-02	6.855E-03	3.390	5.566E-03	9.598E-04
0.462	1.057E-01	5.197E-03	3.450	6.389E-03	1.023E-03
0.477	1.112E-01	5.204E-03	3.510	4.872E-03	8.888E-04
0.492	1.062E-01	5.098E-03	3.570	3.114E-03	8.001E-04
0.507	1.039E-01	4.955E-03	3.630	2.797E-03	7.379E-04
0.522	1.346E-01	5.783E-03	3.690	2.936E-03	7.718E-04
0.540	1.688E-01	6.326E-03	3.750	2.967E-03	7.127E-04
0.560	1.474E-01	5.962E-03	3.810	2.975E-03	7.462E-04
0.580	1.174E-01	5.295E-03	3.870	2.872E-03	6.833E-04
0.600	9.976E-02	4.813E-03	3.935	2.897E-03	6.844E-04
0.620	8.436E-02	4.406E-03	4.005	2.021E-03	6.079E-04
0.640	6.802E-02	3.963E-03	4.075	1.602E-03	5.697E-04
0.660	4.987E-02	3.513E-03	4.145	1.972E-03	6.279E-04
0.680	4.233E-02	3.254E-03	4.215	1.913E-03	5.825E-04
0.700	4.219E-02	3.267E-03	4.285	1.985E-03	5.900E-04
0.720	3.859E-02	3.162E-03	4.355	1.510E-03	5.233E-04
0.740	3.986E-02	3.181E-03	4.425	8.002E-04	4.547E-04
0.760	4.009E-02	3.205E-03	4.495	4.885E-04	4.335E-04
0.780	4.371E-02	3.211E-03	4.565	8.423E-04	4.339E-04
0.800	6.162E-02	3.528E-03	4.635	1.453E-03	4.736E-04
0.820	6.855E-02	3.712E-03	4.705	1.324E-03	4.559E-04
0.840	5.681E-02	3.396E-03	4.775	9.164E-04	4.038E-04
0.860	4.668E-02	3.208E-03	4.845	8.527E-04	3.768E-04
0.880	4.153E-02	3.026E-03	4.915	9.444E-04	3.802E-04
0.900	3.886E-02	2.988E-03	4.985	9.985E-04	3.571E-04
0.920	3.943E-02	2.928E-03	5.060	7.280E-04	3.446E-04
0.940	4.260E-02	3.010E-03	5.140	5.413E-04	3.028E-04
0.962	4.389E-02	3.016E-03	5.220	7.988E-04	2.957E-04
0.987	4.000E-02	2.931E-03	5.300	5.576E-04	2.647E-04
1.013	3.973E-02	2.853E-03	5.380	2.243E-04	2.175E-04
1.037	3.524E-02	2.788E-03	5.460	2.564E-04	2.463E-04
1.062	3.063E-02	2.635E-03	5.540	1.756E-04	2.479E-04
1.088	4.229E-02	2.914E-03	5.620	1.071E-04	1.832E-04
1.112	4.958E-02	3.071E-03	5.700	2.162E-04	1.823E-04
1.138	4.032E-02	2.775E-03	5.780	2.183E-04	1.680E-04
1.162	3.135E-02	2.558E-03	5.860	1.974E-04	1.808E-04
1.187	2.978E-02	2.539E-03	5.945	3.574E-04	1.949E-04
1.215	3.381E-02	2.653E-03	6.035	4.183E-04	1.985E-04
1.245	3.498E-02	2.669E-03	6.125	1.518E-04	1.399E-04
1.275	3.190E-02	2.542E-03	6.215	-5.713E-05	1.198E-04
1.305	3.362E-02	2.640E-03	6.305	-5.218E-05	1.029E-04
1.335	3.163E-02	2.515E-03	6.395	6.415E-05	1.222E-04
1.365	2.521E-02	2.461E-03	6.485	1.260E-04	1.222E-04
1.395	2.318E-02	2.300E-03	6.575	1.033E-04	9.066E-05
1.425	2.479E-02	2.334E-03	6.665	8.696E-05	1.076E-04
1.455	2.247E-02	2.204E-03	6.755	1.269E-04	1.028E-04
1.485	2.060E-02	2.146E-03	6.850	1.804E-04	8.997E-05
1.515	2.086E-02	2.013E-03	6.950	1.182E-04	6.816E-05
1.545	2.046E-02	2.133E-03	7.050	2.447E-05	4.250E-05
1.580	2.108E-02	2.157E-03	7.150	-7.140E-06	4.183E-05
1.620	2.289E-02	2.185E-03	7.250	7.193E-07	4.180E-05
1.660	1.970E-02	2.071E-03	7.350	8.677E-06	4.229E-05
1.700	1.909E-02	2.020E-03	7.450	1.570E-05	3.819E-05
1.740	1.871E-02	1.991E-03	7.550	1.637E-05	3.163E-05
1.780	1.861E-02	1.954E-03	7.650	2.010E-05	3.340E-05
1.820	1.736E-02	1.892E-03	7.750	1.275E-05	3.189E-05
1.860	1.512E-02	1.774E-03	7.850	3.302E-06	2.912E-05
1.900	1.542E-02	1.768E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF Z35-U

START COUNT 4.7 SEC AFTER END OF IRRADIATION
COUNT FOR 2 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	5.169E-02	9.806E-03	1.940	2.493E-02	2.291E-03
0.065	7.197E-02	1.116E-02	1.980	2.299E-02	2.193E-03
0.075	1.064E-01	1.183E-02	2.020	2.559E-02	2.252E-03
0.085	1.125E-01	1.246E-02	2.060	2.480E-02	2.164E-03
0.095	2.141E-01	1.445E-02	2.100	2.076E-02	2.034E-03
0.105	2.679E-01	1.540E-02	2.140	1.894E-02	1.960E-03
0.115	2.821E-01	1.560E-02	2.180	1.852E-02	2.002E-03
0.125	2.549E-01	1.500E-02	2.220	1.941E-02	1.969E-03
0.135	2.423E-01	1.473E-02	2.260	1.835E-02	2.004E-03
0.145	2.326E-01	1.467E-02	2.300	1.635E-02	1.806E-03
0.155	2.347E-01	1.425E-02	2.340	1.773E-02	1.921E-03
0.165	1.969E-01	1.413E-02	2.380	1.644E-02	1.863E-03
0.177	1.712E-01	1.191E-02	2.425	1.462E-02	1.794E-03
0.192	1.468E-01	1.074E-02	2.475	1.531E-02	1.798E-03
0.207	1.531E-01	1.057E-02	2.525	1.424E-02	1.750E-03
0.222	1.465E-01	1.026E-02	2.575	1.223E-02	1.625E-03
0.237	1.131E-01	9.580E-03	2.625	1.365E-02	1.667E-03
0.252	1.321E-01	9.753E-03	2.675	1.440E-02	1.684E-03
0.267	1.675E-01	1.080E-02	2.725	1.284E-02	1.601E-03
0.282	1.698E-01	1.087E-02	2.775	1.047E-02	1.516E-03
0.297	1.761E-01	1.086E-02	2.825	1.120E-02	1.520E-03
0.313	1.145E-01	9.270E-03	2.875	1.137E-02	1.483E-03
0.327	6.958E-02	7.921E-03	2.925	9.437E-03	1.440E-03
0.342	5.733E-02	7.423E-03	2.975	7.044E-03	1.319E-03
0.357	7.065E-02	7.563E-03	3.030	5.881E-03	1.243E-03
0.372	1.061E-01	8.558E-03	3.090	6.958E-03	1.242E-03
0.387	1.328E-01	9.173E-03	3.150	7.605E-03	1.252E-03
0.402	1.430E-01	9.325E-03	3.210	7.670E-03	1.180E-03
0.417	1.50[illegible]	9.064E-03	3.270	1.017E-02	1.339E-03
0.432	[illegible]	8.828E-03	3.330	1.026E-02	1.275E-03
0.447	1.390E-01	6.111E-03	3.390	1.161E-02	1.378E-03
0.462	1.658E-01	6.447E-03	3.450	8.643E-03	1.159E-03
0.477	1.689E-01	6.429E-03	3.510	7.419E-03	1.172E-03
0.492	1.583E-01	6.195E-03	3.570	7.131E-03	1.077E-03
0.507	1.678E-01	6.274E-03	3.630	5.023E-03	1.025E-03
0.522	2.058E-01	7.327E-03	3.690	2.677E-03	8.428E-04
0.540	2.550E-01	7.900E-03	3.750	3.559E-03	9.915E-04
0.560	2.302E-01	7.424E-03	3.810	4.683E-03	9.473E-04
0.580	1.668E-01	6.350E-03	3.870	3.593E-03	8.444E-04
0.600	1.571E-01	6.050E-03	3.935	3.451E-03	8.805E-04
0.620	1.346E-01	5.625E-0[illegible]	4.005	3.324E-03	8.201E-04
0.640	1.080E-01	5.016E[illegible]	4.075	2.883E-03	8.038E-04
0.660	8.030E-02	4.537E-03	4.145	2.517E-03	7.254E-04
0.680	6.430E-02	4.116E-03	4.215	3.603E-03	7.798E-04
0.700	6.591E-02	4.152E-03	4.285	3.764E-03	7.738E-04
0.720	6.760E-02	4.145E-03	4.355	2.658E-03	6.883E-04
0.740	6.346E-02	4.079E-03	4.425	2.799E-03	6.416E-04
0.760	6.455E-02	4.332E-03	4.495	3.011E-03	6.823E-04
0.780	8.348E-02	4.358E-03	4.565	2.365E-03	5.753E-04
0.800	1.002E-01	4.564E-03	4.635	1.788E-03	5.610E-04
0.820	9.928E-02	4.558E-03	4.705	1.333E-03	4.870E-04
0.840	9.060E-02	4.333E-03	4.775	1.531E-03	5.150E-04
0.860	7.753E-02	4.142E-03	4.845	1.286E-03	4.629E-04
0.880	6.653E-02	3.866E-03	4.915	5.155E-04	3.825E-04
0.900	6.608E-02	3.873E-03	4.985	4.477E-04	4.136E-04
0.920	6.763E-02	3.818E-03	5.060	6.656E-04	4.107E-04
0.940	6.787E-02	3.897E-03	5.140	6.441E-04	3.985E-04
0.962	7.175E-02	3.879E-03	5.220	6.380E-04	3.476E-04
0.987	6.809E-02	3.788E-03	5.300	5.582E-04	3.599E-04
1.013	5.545E-02	3.466E-03	5.380	3.556E-04	3.520E-04
1.037	5.301E-02	3.404E-03	5.460	8.832E-04	3.463E-04
1.062	5.666E-02	3.500E-03	5.540	1.447E-03	3.763E-04
1.088	6.444E-02	3.636E-03	5.620	8.294E-04	3.514E-04
1.112	7.184E-02	3.718E-03	5.700	2.595E-04	2.652E-04
1.138	6.394E-02	3.468E-03	5.780	4.752E-04	2.670E-04
1.162	4.936E-02	3.229E-03	5.860	5.310E-04	2.347E-04
1.187	4.695E-02	3.195E-03	5.945	2.754E-04	1.963E-04
1.215	5.159E-02	3.321E-03	6.035	1.959E-04	1.953E-04
1.245	4.959E-02	3.209E-03	6.125	3.803E-04	2.425E-04
1.275	5.004E-02	3.295E-03	6.215	4.113E-04	1.906E-04
1.305	5.095E-02	3.257E-03	6.305	1.130E-04	1.356E-04
1.335	4.564E-02	3.150E-03	6.395	-4.152E-05	8.808E-05
1.365	4.059E-02	2.990E-03	6.485	7.461E-05	1.117E-04
1.395	4.478E-02	3.154E-03	6.575	1.344E-04	1.246E-04
1.425	4.533E-02	3.054E-03	6.665	4.365E-05	8.490E-05
1.455	4.137E-02	2.938E-03	6.755	2.595E-05	6.739E-05
1.485	3.705E-02	2.748E-03	6.850	1.109E-04	8.302E-05
1.515	3.741E-02	2.682E-03	6.950	1.541E-04	8.033E-05
1.545	3.817E-02	2.902E-03	7.050	1.202E-04	7.776E-05
1.580	3.415E-02	2.801E-03	7.150	4.285E-05	5.963E-05
1.620	3.592E-02	2.741E-03	7.250	1.106E-05	4.182E-05
1.660	2.998E-02	2.616E-03	7.350	1.653E-05	4.169E-05
1.700	2.695E-02	2.459E-03	7.450	2.084E-05	4.098E-05
1.740	3.169E-02	2.609E-03	7.550	1.418E-05	4.199E-05
1.780	3.095E-02	2.520E-03	7.650	6.490E-06	4.072E-05
1.820	2.819E-02	2.448E-03	7.750	-2.186E-07	3.353E-05
1.860	2.320E-02	2.230E-03	7.850	4.161E-07	2.975E-05
1.900	2.444E-02	2.236E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF Z35-U

START COUNT 6.7 SEC AFTER END OF IRRADIATION
COUNT FOR 3 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.202E-02	1.015E-02	1.940	2.602E-02	2.332E-03
0.065	9.874E-02	1.196E-02	1.980	2.776E-02	2.379E-03
0.075	1.279E-01	1.309E-02	2.020	2.922E-02	2.379E-03
0.085	1.400E-01	1.349E-02	2.060	2.482E-02	2.193E-03
0.095	2.595E-01	1.532E-02	2.100	2.116E-02	2.075E-03
0.105	3.267E-01	1.658E-02	2.140	2.180E-02	2.104E-03
0.115	3.077E-01	1.612E-02	2.180	1.823E-02	1.979E-03
0.125	2.827E-01	1.546E-02	2.220	1.760E-02	1.974E-03
0.135	2.658E-01	1.564E-02	2.260	2.139E-02	2.059E-03
0.145	2.780E-01	1.563E-02	2.300	2.061E-02	2.001E-03
0.155	2.260E-01	1.482E-02	2.340	1.965E-02	1.965E-03
0.165	2.250E-01	1.483E-02	2.380	1.967E-02	2.048E-03
0.177	1.965E-01	1.260E-02	2.425	1.882E-02	1.931E-03
0.192	1.531E-01	1.106E-02	2.475	1.818E-02	1.984E-03
0.207	1.781E-01	1.106E-02	2.525	1.560E-02	1.800E-03
0.222	1.568E-01	1.056E-02	2.575	1.632E-02	1.817E-03
0.237	1.232E-01	1.005E-02	2.625	1.592E-02	1.810E-03
0.252	1.544E-01	1.036E-02	2.675	1.513E-02	1.739E-03
0.267	1.770E-01	1.114E-02	2.725	1.186E-02	1.573E-03
0.282	1.951E-01	1.140E-02	2.775	1.017E-02	1.493E-03
0.297	2.072E-01	1.170E-02	2.825	1.105E-02	1.529E-03
0.313	1.180E-01	9.389E-03	2.875	1.184E-02	1.583E-03
0.327	7.367E-02	8.139E-03	2.925	1.188E-02	1.565E-03
0.342	6.930E-02	7.739E-03	2.975	1.109E-02	1.532E-03
0.357	9.292E-02	8.406E-03	3.030	8.107E-03	1.327E-03
0.372	1.120E-01	8.919E-03	3.090	7.978E-03	1.330E-03
0.387	1.527E-01	9.731E-03	3.150	7.116E-03	1.262E-03
0.402	1.844E-01	1.013E-02	3.210	8.998E-03	1.336E-03
0.417	1.797E-01	1.004E-02	3.270	9.094E-03	1.311E-03
0.432	1.588E-01	9.067E-03	3.330	1.008E-02	1.307E-03
0.447	1.528E-01	6.420E-03	3.390	1.320E-02	1.467E-03
0.462	1.547E-01	6.262E-03	3.450	8.571E-03	1.188E-03
0.477	1.583E-01	6.347E-03	3.510	6.828E-03	1.144E-03
0.492	1.613E-01	6.299E-03	3.570	7.114E-03	1.099E-03
0.507	1.811E-01	6.535E-03	3.630	6.648E-03	1.114E-03
0.522	2.314E-01	7.727E-03	3.690	6.580E-03	1.068E-03
0.540	2.831E-01	8.318E-03	3.750	5.971E-03	1.028E-03
0.560	2.452E-01	7.630E-03	3.810	4.301E-03	9.035E-04
0.580	1.848E-01	6.711E-03	3.870	4.647E-03	9.182E-04
0.600	1.738E-01	6.335E-03	3.935	5.477E-03	9.365E-04
0.620	1.510E-01	5.901E-03	4.005	4.089E-03	8.009E-04
0.640	1.068E-01	5.055E-03	4.075	3.683E-03	8.137E-04
0.660	8.439E-02	4.646E-03	4.145	3.745E-03	7.666E-04
0.680	7.897E-02	4.518E-03	4.215	4.477E-03	8.064E-04
0.700	8.369E-02	4.558E-03	4.285	3.414E-03	7.573E-04
0.720	7.851E-02	4.525E-03	4.355	2.519E-03	6.662E-04
0.740	6.914E-02	4.197E-03	4.425	1.826E-03	6.310E-04
0.760	8.000E-02	4.415E-03	4.495	5.185E-04	5.213E-04
0.780	9.773E-02	4.646E-03	4.565	1.234E-03	5.130E-04
0.800	1.101E-01	4.816E-03	4.635	1.684E-03	5.629E-04
0.820	1.074E-01	4.694E-03	4.705	8.835E-04	4.940E-04
0.840	9.460E-02	4.494E-03	4.775	1.323E-03	5.279E-04
0.860	8.365E-02	4.349E-03	4.845	1.702E-03	5.677E-04
0.880	7.762E-02	4.142E-03	4.915	1.690E-03	5.067E-04
0.900	7.842E-02	4.167E-03	4.985	1.677E-03	5.311E-04
0.920	7.980E-02	4.064E-03	5.060	7.676E-04	4.307E-04
0.940	7.650E-02	4.106E-03	5.140	5.447E-04	3.939E-04
0.962	7.779E-02	3.979E-03	5.220	1.223E-03	4.247E-04
0.987	7.891E-02	4.015E-03	5.300	1.171E-03	4.227E-04
1.013	6.603E-02	3.763E-03	5.380	1.039E-03	3.673E-04
1.037	6.321E-02	3.656E-03	5.460	9.446E-04	3.728E-04
1.062	6.474E-02	3.745E-03	5.540	6.181E-04	3.053E-04
1.088	7.441E-02	3.894E-03	5.620	9.342E-04	3.263E-04
1.112	8.133E-02	3.976E-03	5.700	6.802E-04	2.757E-04
1.138	6.381E-02	3.521E-03	5.780	7.971E-04	2.772E-04
1.162	5.469E-02	3.374E-03	5.860	4.603E-04	2.441E-04
1.187	5.492E-02	3.437E-03	5.945	2.175E-04	1.960E-04
1.215	5.588E-02	3.365E-03	6.035	4.035E-04	2.204E-04
1.245	5.910E-02	3.520E-03	6.125	3.546E-04	1.768E-04
1.275	5.807E-02	3.473E-03	6.215	5.575E-05	1.281E-04
1.305	6.547E-02	3.654E-03	6.305	-1.205E-05	1.090E-04
1.335	5.386E-02	3.379E-03	6.395	1.575E-04	1.054E-04
1.365	4.553E-02	3.183E-03	6.485	2.616E-04	1.193E-04
1.395	5.186E-02	3.318E-03	6.575	1.403E-04	9.675E-05
1.425	5.549E-02	3.328E-03	6.665	3.714E-05	8.578E-05
1.455	5.228E-02	3.147E-03	6.755	4.577E-05	7.664E-05
1.485	4.317E-02	2.899E-03	6.850	6.117E-05	6.700E-05
1.515	4.318E-02	2.786E-03	6.950	5.706E-05	7.158E-05
1.545	4.143E-02	3.023E-03	7.050	3.588E-05	5.932E-05
1.580	3.871E-02	2.935E-03	7.150	1.724E-05	4.132E-05
1.620	3.633E-02	2.792E-03	7.250	2.358E-05	4.089E-05
1.660	2.964E-02	2.563E-03	7.350	4.909E-05	4.928E-05
1.700	2.719E-02	2.474E-03	7.450	4.688E-05	5.273E-05
1.740	3.302E-02	2.672E-03	7.550	1.866E-05	4.472E-05
1.780	3.910E-02	2.850E-03	7.650	-3.940E-06	3.454E-05
1.820	3.297E-02	2.656E-03	7.750	-3.240E-05	3.150E-05
1.860	2.604E-02	2.366E-03	7.850	8.767E-06	3.478E-05
1.900	2.558E-02	2.337E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 9.7 SEC AFTER END OF IRRADIATION
COUNT FOR 5 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	8.005E-02	1.157E-02	1.940	3.408E-02	2.600E-03
0.065	1.079E-01	1.303E-02	1.980	3.068E-02	2.483E-03
0.075	1.519E-01	1.385E-02	2.020	3.133E-02	2.496E-03
0.085	1.941E-01	1.466E-02	2.060	3.341E-02	2.440E-03
0.095	3.157E-01	1.675E-02	2.100	2.532E-02	2.262E-03
0.105	3.948E-01	1.780E-02	2.140	2.197E-02	2.144E-03
0.115	3.422E-01	1.670E-02	2.180	2.575E-02	2.266E-03
0.125	3.015E-01	1.599E-02	2.220	2.443E-02	2.236E-03
0.135	3.513E-01	1.697E-02	2.260	2.463E-02	2.227E-03
0.145	3.338E-01	1.689E-02	2.300	2.426E-02	2.204E-03
0.155	2.972E-01	1.637E-02	2.340	2.180E-02	2.061E-03
0.165	2.582E-01	1.581E-02	2.380	2.147E-02	2.186E-03
0.177	2.064E-01	1.286E-02	2.425	2.082E-02	2.068E-03
0.192	1.869E-01	1.187E-02	2.475	2.183E-02	2.135E-03
0.207	2.438E-01	1.238E-02	2.525	2.112E-02	2.092E-03
0.222	1.966E-01	1.151E-02	2.575	1.935E-02	1.892E-03
0.237	1.614E-01	1.097E-02	2.625	2.018E-02	1.992E-03
0.252	1.814E-01	1.110E-02	2.675	1.693E-02	1.880E-03
0.267	2.240E-01	1.193E-02	2.725	1.785E-02	1.979E-03
0.282	2.481E-01	1.239E-02	2.775	1.648E-02	1.851E-03
0.297	2.236E-01	1.190E-02	2.825	1.563E-02	1.710E-03
0.313	1.300E-01	1.002E-02	2.875	1.731E-02	1.768E-03
0.327	8.241E-02	8.594E-03	2.925	1.114E-02	1.546E-03
0.342	9.595E-02	8.795E-03	2.975	7.397E-03	1.441E-03
0.357	1.074E-01	9.119E-03	3.030	9.702E-03	1.460E-03
0.372	1.480E-01	9.822E-03	3.090	7.728E-03	1.367E-03
0.387	2.110E-01	1.116E-02	3.150	6.284E-03	1.244E-03
0.402	2.266E-01	1.134E-02	3.210	9.841E-03	1.363E-03
0.417	2.038E-01	1.066E-02	3.270	1.302E-02	1.512E-03
0.432	1.905E-01	9.934E-03	3.330	1.203E-02	1.460E-03
0.447	1.777E-01	6.887E-03	3.390	1.180E-02	1.495E-03
0.462	1.555E-01	8.444E-03	3.450	9.891E-03	1.307E-03
0.477	1.494E-01	6.277E-03	3.510	8.326E-03	1.224E-03
0.492	1.623E-01	6.411E-03	3.570	8.571E-03	1.262E-03
0.507	2.064E-01	6.890E-03	3.630	6.960E-03	1.156E-03
0.522	2.631E-01	8.336E-03	3.690	6.152E-03	1.108E-03
0.540	2.912E-01	8.497E-03	3.750	7.086E-03	1.103E-03
0.560	2.545E-01	7.794E-03	3.810	7.280E-03	1.065E-03
0.580	1.925E-01	6.908E-03	3.870	6.342E-03	1.042E-03
0.600	1.936E-01	6.714E-03	3.935	5.215E-03	9.244E-04
0.620	1.696E-01	6.341E-03	4.005	5.525E-03	9.535E-04
0.640	1.324E-01	5.665E-03	4.075	4.954E-03	8.974E-04
0.660	1.068E-01	5.213E-03	4.145	5.356E-03	8.978E-04
0.680	9.774E-02	5.022E-03	4.215	4.532E-03	8.727E-04
0.700	1.013E-01	5.055E-03	4.285	2.947E-03	7.093E-04
0.720	9.907E-02	4.961E-03	4.355	2.578E-03	7.241E-04
0.740	9.084E-02	4.828E-03	4.425	1.584E-03	6.094E-04
0.760	1.021E-01	4.904E-03	4.495	1.647E-03	6.727E-04
0.780	1.256E-01	5.195E-03	4.565	2.026E-03	6.001E-04
0.800	1.357E-01	5.255E-03	4.635	2.076E-03	6.426E-04
0.820	1.284E-01	5.098E-03	4.705	1.982E-03	5.538E-04
0.840	1.096E-01	4.888E-03	4.775	1.688E-03	5.677E-04
0.860	9.312E-02	4.622E-03	4.845	1.292E-03	5.133E-04
0.880	8.870E-02	4.491E-03	4.915	1.695E-03	5.139E-04
0.900	9.063E-02	4.543E-03	4.985	1.720E-03	5.208E-04
0.920	9.376E-02	4.418E-03	5.060	1.170E-03	4.120E-04
0.940	9.702E-02	4.588E-03	5.140	1.366E-03	4.625E-04
0.962	9.728E-02	4.464E-03	5.220	1.145E-03	4.058E-04
0.987	9.635E-02	4.497E-03	5.300	8.232E-04	3.620E-04
1.013	8.714E-02	4.206E-03	5.380	9.475E-04	3.979E-04
1.037	7.239E-02	3.997E-03	5.460	7.969E-04	3.443E-04
1.062	7.409E-02	3.988E-03	5.540	3.694E-04	3.207E-04
1.088	8.754E-02	4.285E-03	5.620	4.596E-04	2.802E-04
1.112	9.494E-02	4.233E-03	5.700	6.279E-04	2.902E-04
1.138	8.500E-02	4.024E-03	5.780	3.136E-04	2.420E-04
1.162	7.048E-02	3.712E-03	5.860	1.849E-04	1.992E-04
1.187	6.459E-02	3.724E-03	5.945	5.057E-04	2.185E-04
1.215	6.323E-02	3.620E-03	6.035	6.455E-04	2.281E-04
1.245	7.065E-02	3.809E-03	6.125	3.938E-04	2.066E-04
1.275	7.247E-02	3.885E-03	6.215	1.032E-04	1.442E-04
1.305	7.159E-02	3.927E-03	6.305	-7.515E-06	1.006E-04
1.335	6.712E-02	3.833E-03	6.395	7.452E-05	1.232E-04
1.365	5.812E-02	3.589E-03	6.485	1.474E-04	1.079E-04
1.395	6.597E-02	3.672E-03	6.575	6.591E-05	9.251E-05
1.425	7.206E-02	3.754E-03	6.665	1.178E-05	7.215E-05
1.455	6.399E-02	3.451E-03	6.755	9.082E-05	8.747E-05
1.485	5.552E-02	3.236E-03	6.850	1.281E-04	8.915E-05
1.515	5.329E-02	3.108E-03	6.950	4.677E-05	6.055E-05
1.545	4.787E-02	3.258E-03	7.050	3.012E-05	4.386E-05
1.580	4.244E-02	3.111E-03	7.150	6.864E-05	5.164E-05
1.620	4.670E-02	3.087E-03	7.250	7.949E-05	5.655E-05
1.660	3.816E-02	2.898E-03	7.350	3.833E-05	4.508E-05
1.700	3.869E-02	2.907E-03	7.450	7.852E-06	3.870E-05
1.740	4.292E-02	3.026E-03	7.550	3.967E-06	3.420E-05
1.780	4.153E-02	2.958E-03	7.650	1.581E-05	3.845E-05
1.820	3.740E-02	2.755E-03	7.750	1.581E-05	3.765E-05
1.860	3.425E-02	2.645E-03	7.850	3.701E-06	2.987E-05
1.900	3.333E-02	2.589E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 14.7 SEC AFTER END OF IRRADIATION
COUNT FOR 4 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	5.131E-02	9.675E-03	1.940	2.206E-02	2.120E-03
0.065	7.778E-02	1.084E-02	1.980	2.227E-02	2.145E-03
0.075	1.185E-01	1.224E-02	2.020	2.336E-02	2.115E-03
0.085	1.316E-01	1.255E-02	2.060	2.243E-02	2.035E-03
0.095	2.430E-01	1.457E-02	2.100	1.903E-02	1.980E-03
0.105	2.810E-01	1.525E-02	2.140	1.776E-02	1.932E-03
0.115	2.751E-01	1.498E-02	2.180	2.089E-02	1.962E-03
0.125	2.493E-01	1.454E-02	2.220	1.953E-02	2.026E-03
0.135	2.833E-01	1.526E-02	2.260	1.791E-02	1.900E-03
0.145	2.704E-01	1.500E-02	2.300	1.821E-02	1.922E-03
0.155	2.317E-01	1.358E-02	2.340	1.599E-02	1.706E-03
0.165	2.061E-01	1.360E-02	2.380	1.674E-02	1.949E-03
0.177	1.542E-01	1.117E-02	2.425	1.731E-02	1.759E-03
0.192	1.291E-01	1.023E-02	2.475	1.709E-02	1.842E-03
0.207	1.997E-01	1.108E-02	2.525	1.486E-02	1.694E-03
0.222	1.749E-01	1.060E-02	2.575	1.530E-02	1.689E-03
0.237	1.224E-01	9.483E-03	2.625	1.464E-02	1.618E-03
0.252	1.319E-01	9.416E-03	2.675	1.303E-02	1.608E-03
0.267	1.581E-01	1.014E-02	2.725	1.095E-02	1.598E-03
0.282	1.517E-01	1.020E-02	2.775	1.111E-02	1.654E-03
0.297	1.439E-01	9.737E-03	2.825	1.300E-02	1.479E-03
0.313	9.188E-02	8.379E-03	2.875	1.151E-02	1.413E-03
0.327	6.989E-02	7.709E-03	2.925	9.239E-03	1.312E-03
0.342	8.465E-02	7.836E-03	2.975	8.526E-03	1.323E-03
0.357	7.916E-02	7.815E-03	3.030	6.928E-03	1.203E-03
0.372	1.104E-01	8.352E-03	3.090	6.832E-03	1.218E-03
0.387	1.720E-01	9.783E-03	3.150	5.176E-03	1.095E-03
0.402	1.757E-01	9.728E-03	3.210	5.170E-03	1.067E-03
0.417	1.439E-01	8.728E-03	3.270	4.804E-03	1.041E-03
0.432	1.331E-01	8.209E-03	3.330	6.796E-03	1.075E-03
0.447	1.163E-01	5.486E-03	3.390	8.197E-03	1.144E-03
0.462	8.985E-02	4.946E-03	3.450	6.700E-03	1.030E-03
0.477	9.314E-02	4.992E-03	3.510	6.882E-03	1.059E-03
0.492	1.067E-01	5.195E-03	3.570	7.330E-03	1.043E-03
0.507	1.325E-01	5.562E-03	3.630	7.151E-03	1.069E-03
0.522	1.792E-01	6.675E-03	3.690	5.512E-03	9.259E-04
0.540	2.098E-01	7.081E-03	3.750	4.646E-03	9.175E-04
0.560	1.710E-01	6.332E-03	3.810	3.798E-03	8.335E-04
0.580	1.418E-01	5.801E-03	3.870	3.049E-03	7.737E-04
0.600	1.377E-01	5.622E-03	3.935	3.045E-03	7.788E-04
0.620	1.341E-01	5.501E-03	4.005	2.923E-03	7.072E-04
0.640	1.029E-01	4.858E-03	4.075	3.813E-03	8.063E-04
0.660	7.791E-02	4.391E-03	4.145	2.868E-03	7.259E-04
0.680	7.615E-02	4.349E-03	4.215	2.071E-03	6.299E-04
0.700	7.946E-02	4.358E-03	4.285	2.786E-03	6.555E-04
0.720	7.472E-02	4.242E-03	4.355	2.372E-03	6.200E-04
0.740	6.996E-02	4.139E-03	4.425	1.889E-03	5.562E-04
0.760	7.501E-02	4.242E-03	4.495	1.982E-03	5.763E-04
0.780	9.308E-02	4.400E-03	4.565	2.439E-03	5.567E-04
0.800	1.009E-01	4.552E-03	4.635	2.737E-03	5.715E-04
0.820	9.119E-02	4.309E-03	4.705	1.427E-03	4.767E-04
0.840	7.720E-02	4.100E-03	4.775	5.309E-04	3.869E-04
0.860	7.147E-02	4.048E-03	4.845	8.507E-04	4.221E-04
0.880	6.608E-02	3.833E-03	4.915	8.762E-04	3.948E-04
0.900	6.586E-02	3.736E-03	4.985	6.674E-04	3.614E-04
0.920	6.988E-02	3.827E-03	5.060	5.012E-04	3.485E-04
0.940	7.001E-02	3.827E-03	5.140	5.688E-04	2.938E-04
0.962	7.113E-02	3.818E-03	5.220	9.759E-04	3.389E-04
0.987	7.044E-02	3.788E-03	5.300	8.575E-04	3.281E-04
1.013	6.446E-02	3.597E-03	5.380	6.688E-04	3.230E-04
1.037	5.819E-02	3.462E-03	5.460	8.009E-04	3.066E-04
1.062	5.508E-02	3.432E-03	5.540	6.536E-04	2.624E-04
1.088	6.649E-02	3.600E-03	5.620	2.254E-04	2.187E-04
1.112	7.601E-02	3.754E-03	5.700	1.152E-04	1.809E-04
1.138	6.637E-02	3.542E-03	5.780	2.447E-04	2.021E-04
1.162	5.473E-02	3.258E-03	5.860	2.322E-04	1.754E-04
1.187	5.376E-02	3.301E-03	5.945	1.433E-04	1.859E-04
1.215	5.233E-02	3.231E-03	6.035	1.168E-04	1.611E-04
1.245	4.806E-02	3.141E-03	6.125	1.818E-04	1.432E-04
1.275	5.676E-02	3.366E-03	6.215	2.462E-04	1.432E-04
1.305	6.020E-02	3.420E-03	6.305	2.219E-04	1.588E-04
1.335	4.934E-02	3.271E-03	6.395	9.326E-05	1.409E-04
1.365	4.630E-02	3.197E-03	6.485	2.540E-05	1.237E-04
1.395	5.651E-02	3.283E-03	6.575	2.040E-05	1.077E-04
1.425	6.151E-02	3.334E-03	6.665	1.096E-05	9.383E-05
1.455	5.061E-02	2.996E-03	6.755	1.019E-05	8.880E-05
1.485	4.089E-02	2.776E-03	6.850	1.079E-04	9.787E-05
1.515	3.840E-02	2.616E-03	6.950	1.480E-04	1.046E-04
1.545	3.497E-02	2.778E-03	7.050	9.438E-05	8.531E-05
1.580	3.017E-02	2.602E-03	7.150	4.611E-05	6.856E-05
1.620	3.066E-02	2.506E-03	7.250	2.938E-05	6.149E-05
1.660	2.421E-02	2.371E-03	7.350	2.431E-05	5.581E-05
1.700	2.721E-02	2.452E-03	7.450	1.969E-05	5.610E-05
1.740	3.372E-02	2.660E-03	7.550	2.587E-05	4.959E-05
1.780	3.413E-02	2.587E-03	7.650	3.050E-05	4.654E-05
1.820	3.016E-02	2.406E-03	7.750	2.119E-05	4.092E-05
1.860	2.146E-02	2.160E-03	7.850	8.414E-06	3.703E-05
1.900	2.273E-02	2.145E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 19.7 SEC AFTER END OF IRRADIATION
COUNT FOR 5 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.079E-02	9.581E-03	1.940	1.861E-02	1.910E-03
0.065	6.419E-02	1.006E-02	1.980	2.092E-02	1.951E-03
0.075	9.898E-02	1.131E-02	2.020	1.995E-02	1.921E-03
0.085	1.318E-01	1.197E-02	2.060	1.861E-02	1.843E-03
0.095	2.228E-01	1.351E-02	2.100	1.565E-02	1.765E-03
0.105	2.298E-01	1.374E-02	2.140	1.274E-02	1.628E-03
0.115	1.934E-01	1.277E-02	2.180	1.466E-02	1.695E-03
0.125	1.932E-01	1.291E-02	2.220	1.553E-02	1.760E-03
0.135	2.260E-01	1.359E-02	2.260	1.346E-02	1.660E-03
0.145	1.834E-01	1.282E-02	2.300	1.367E-02	1.584E-03
0.155	1.607E-01	1.205E-02	2.340	1.316E-02	1.586E-03
0.165	1.428E-01	1.195E-02	2.380	1.132E-02	1.547E-03
0.177	1.273E-01	1.006E-02	2.425	1.309E-02	1.563E-03
0.192	1.174E-01	9.304E-03	2.475	1.275E-02	1.564E-03
0.207	1.480E-01	9.807E-03	2.525	1.161E-02	1.476E-03
0.222	1.381E-01	9.519E-03	2.575	1.325E-02	1.488E-03
0.237	1.014E-01	8.755E-03	2.625	1.093E-02	1.412E-03
0.252	9.339E-02	8.288E-03	2.675	1.018E-02	1.502E-03
0.267	1.257E-01	8.946E-03	2.725	1.177E-02	1.494E-03
0.282	1.080E-01	8.649E-03	2.775	1.031E-02	1.394E-03
0.297	1.047E-01	8.394E-03	2.825	7.471E-03	1.213E-03
0.313	7.557E-02	7.506E-03	2.875	8.010E-03	1.202E-03
0.327	5.716E-02	6.769E-03	2.925	6.237E-03	1.092E-03
0.342	5.515E-02	6.649E-03	2.975	4.711E-03	1.026E-03
0.357	7.471E-02	6.996E-03	3.030	5.848E-03	1.058E-03
0.372	1.036E-01	8.063E-03	3.090	4.260E-03	9.687E-04
0.387	1.324E-01	8.631E-03	3.150	4.321E-03	9.254E-04
0.402	1.637E-01	9.091E-03	3.210	4.325E-03	9.649E-04
0.417	1.409E-01	8.294E-03	3.270	3.801E-03	9.075E-04
0.432	1.079E-01	7.178E-03	3.330	5.374E-03	9.587E-04
0.447	9.232E-02	6.761E-03	3.390	6.050E-03	9.696E-04
0.462	7.050E-02	6.409E-03	3.450	4.891E-03	8.995E-04
0.477	5.125E-02	6.274E-03	3.510	4.273E-03	8.715E-04
0.492	6.905E-02	6.276E-03	3.570	5.104E-03	8.786E-04
0.507	9.259E-02	6.622E-03	3.630	5.218E-03	9.115E-04
0.522	1.265E-01	5.565E-03	3.690	3.594E-03	7.657E-04
0.540	1.448E-01	5.832E-03	3.750	3.967E-03	7.892E-04
0.560	1.226E-01	5.337E-03	3.810	4.327E-03	7.810E-04
0.580	1.048E-01	5.049E-03	3.870	3.032E-03	7.061E-04
0.600	1.177E-01	5.107E-03	3.935	2.720E-03	6.760E-04
0.620	9.735E-02	4.673E-03	4.005	2.575E-03	6.977E-04
0.640	6.983E-02	4.076E-03	4.075	2.235E-03	6.817E-04
0.660	6.049E-02	3.862E-03	4.145	2.430E-03	6.598E-04
0.680	6.308E-02	3.893E-03	4.215	1.698E-03	5.519E-04
0.700	8.691E-02	3.945E-03	4.285	1.473E-03	5.395E-04
0.720	6.132E-02	3.765E-03	4.355	1.543E-03	5.137E-04
0.740	6.237E-02	3.826E-03	4.425	1.259E-03	4.854E-04
0.760	6.274E-02	3.795E-03	4.495	1.237E-03	5.338E-04
0.780	6.877E-02	3.833E-03	4.565	1.906E-03	5.014E-04
0.800	8.241E-02	4.033E-03	4.635	2.001E-03	5.115E-04
0.820	8.051E-02	3.960E-03	4.705	1.166E-03	4.142E-04
0.840	6.628E-02	3.751E-03	4.775	8.299E-04	4.087E-04
0.860	5.615E-02	3.549E-03	4.845	6.626E-04	3.889E-04
0.880	5.590E-02	3.491E-03	4.915	6.172E-04	3.575E-04
0.900	5.696E-02	3.500E-03	4.985	9.338E-04	3.901E-04
0.920	5.274E-02	3.286E-03	5.060	9.158E-04	3.405E-04
0.940	5.270E-02	3.382E-03	5.140	5.073E-04	3.343E-04
0.962	5.847E-02	3.403E-03	5.220	5.401E-04	2.907E-04
0.987	5.501E-02	3.357E-03	5.300	8.429E-04	3.314E-04
1.013	4.761E-02	3.137E-03	5.380	8.066E-04	3.330E-04
1.037	4.460E-02	3.049E-03	5.460	6.825E-04	2.891E-04
1.062	4.360E-02	3.059E-03	5.540	3.341E-04	2.210E-04
1.088	5.136E-02	3.726E-03	5.620	1.023E-04	1.464E-04
1.112	5.839E-02	3.386E-03	5.700	1.841E-04	1.551E-04
1.138	5.398E-02	3.152E-03	5.780	1.963E-04	1.375E-04
1.162	4.611E-02	2.995E-03	5.860	1.442E-04	1.466E-04
1.187	4.367E-02	2.993E-03	5.945	1.759E-04	1.350E-04
1.215	4.408E-02	2.918E-03	6.035	2.485E-04	1.462E-04
1.245	3.821E-02	2.830E-03	6.125	1.619E-04	1.280E-04
1.275	4.280E-02	2.933E-03	6.215	3.812E-06	9.779E-05
1.305	5.091E-02	3.223E-03	6.305	-7.671E-05	8.947E-05
1.335	4.170E-02	3.051E-03	6.395	-4.367E-05	8.459E-05
1.365	3.866E-02	2.864E-03	6.485	4.525E-05	7.330E-05
1.395	4.933E-02	3.132E-03	6.575	1.385E-04	8.976E-05
1.425	5.306E-02	3.000E-03	6.665	1.480E-04	8.983E-05
1.455	4.391E-02	2.822E-03	6.755	9.450E-05	7.851E-05
1.485	3.124E-02	2.395E-03	6.850	3.476E-05	5.946E-05
1.515	2.799E-02	2.258E-03	6.950	1.162E-05	4.413E-05
1.545	2.888E-02	2.438E-03	7.050	5.822E-06	5.125E-05
1.580	2.778E-02	2.396E-03	7.150	9.014E-06	5.035E-05
1.620	2.389E-02	2.176E-03	7.250	1.238E-05	4.391E-05
1.660	1.960E-02	2.071E-03	7.350	1.028E-05	4.352E-05
1.700	2.480E-02	2.237E-03	7.450	1.727E-05	3.471E-05
1.740	2.797E-02	2.326E-03	7.550	4.026E-05	3.895E-05
1.780	2.495E-02	2.241E-03	7.650	5.270E-05	3.834E-05
1.820	2.079E-02	2.132E-03	7.750	3.079E-05	3.331E-05
1.860	1.930E-02	1.947E-03	7.850	7.821E-06	3.423E-05
1.900	1.850E-02	1.890E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 24.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	6.178E-02	1.086E-02	1.940	2.534E-02	2.307E-03
0.065	1.166E-01	1.284E-02	1.980	2.805E-02	2.309E-03
0.075	1.492E-01	1.379E-02	2.020	3.094E-02	2.425E-03
0.085	2.048E-01	1.451E-02	2.060	2.840E-02	2.318E-03
0.095	3.222E-01	1.646E-02	2.100	2.477E-02	2.212E-03
0.105	3.176E-01	1.632E-02	2.140	1.945E-02	2.075E-03
0.115	2.759E-01	1.520E-02	2.180	1.964E-02	2.030E-03
0.125	2.780E-01	1.562E-02	2.220	2.285E-02	2.140E-03
0.135	3.244E-01	1.610E-02	2.260	2.024E-02	2.133E-03
0.145	2.848E-01	1.557E-02	2.300	2.058E-02	2.062E-03
0.155	1.939E-01	1.381E-02	2.340	2.049E-02	1.996E-03
0.165	1.965E-01	1.397E-02	2.380	1.988E-02	2.085E-03
0.177	2.012E-01	1.251E-02	2.425	2.018E-02	2.018E-03
0.192	2.138E-01	1.185E-02	2.475	1.974E-02	2.044E-03
0.207	2.358E-01	1.203E-02	2.525	2.121E-02	2.004E-03
0.222	2.166E-01	1.171E-02	2.575	2.134E-02	1.929E-03
0.237	1.404E-01	1.043E-02	2.625	1.930E-02	1.824E-03
0.252	1.569E-01	1.039E-02	2.675	1.916E-02	1.962E-03
0.267	1.359E-01	1.035E-02	2.725	1.782E-02	1.928E-03
0.282	1.582E-01	1.030E-02	2.775	1.411E-02	1.817E-03
0.297	1.585E-01	1.020E-02	2.825	1.224E-02	1.564E-03
0.313	1.039E-01	8.891E-03	2.875	1.326E-02	1.551E-03
0.327	8.553E-02	8.494E-03	2.925	1.205E-02	1.472E-03
0.342	1.009E-01	8.809E-03	2.975	9.329E-03	1.408E-03
0.357	1.280E-01	9.313E-03	3.030	7.353E-03	1.264E-03
0.372	1.565E-01	9.971E-03	3.090	7.187E-03	1.237E-03
0.387	2.269E-01	1.127E-02	3.150	7.656E-03	1.239E-03
0.402	2.594E-01	1.164E-02	3.210	8.353E-03	1.253E-03
0.417	2.064E-01	1.034E-02	3.270	8.558E-03	1.241E-03
0.432	1.555E-01	8.767E-03	3.330	7.342E-03	1.188E-03
0.447	1.288E-01	5.810E-03	3.390	7.250E-03	1.181E-03
0.462	1.012E-01	5.365E-03	3.450	8.540E-03	1.189E-03
0.477	9.157E-02	5.307E-03	3.510	8.395E-03	1.192E-03
0.492	9.226E-02	5.149E-03	3.570	7.560E-03	1.145E-03
0.507	1.170E-01	5.543E-03	3.630	6.892E-03	1.088E-03
0.522	1.713E-01	6.675E-03	3.690	6.042E-03	1.036E-03
0.540	2.049E-01	7.108E-03	3.750	5.152E-03	9.609E-04
0.560	1.798E-01	6.581E-03	3.810	5.610E-03	9.345E-04
0.580	1.686E-01	6.417E-03	3.870	4.851E-03	9.138E-04
0.600	1.738E-01	6.377E-03	3.935	3.164E-03	7.799E-04
0.620	1.557E-01	6.032E-03	4.005	5.124E-03	8.900E-04
0.640	1.229E-01	5.440E-03	4.075	5.675E-03	9.108E-04
0.660	9.238E-02	4.888E-03	4.145	4.856E-03	8.355E-04
0.680	9.720E-02	4.849E-03	4.215	3.914E-03	7.522E-04
0.700	1.106E-01	5.098E-03	4.285	2.340E-03	6.397E-04
0.720	1.061E-01	4.973E-03	4.355	1.756E-03	6.225E-04
0.740	9.578E-02	4.782E-03	4.425	1.378E-03	6.493E-04
0.760	9.697E-02	4.743E-03	4.495	1.801E-03	5.877E-04
0.780	1.146E-01	4.985E-03	4.565	2.598E-03	6.210E-04
0.800	1.264E-01	5.095E-03	4.635	2.118E-03	5.747E-04
0.820	1.199E-01	4.949E-03	4.705	8.982E-04	4.900E-04
0.840	1.084E-01	4.810E-03	4.775	7.852E-04	4.973E-04
0.860	9.579E-02	4.625E-03	4.845	1.551E-03	4.892E-04
0.880	8.371E-02	4.370E-03	4.915	1.627E-03	4.798E-04
0.900	8.273E-02	4.285E-03	4.985	1.268E-03	4.659E-04
0.920	8.538E-02	4.191E-03	5.060	1.190E-03	4.417E-04
0.940	8.344E-02	4.291E-03	5.140	1.273E-03	4.384E-04
0.962	9.014E-02	4.264E-03	5.220	1.232E-03	3.884E-04
0.987	9.706E-02	4.458E-03	5.300	9.453E-04	3.729E-04
1.013	7.861E-02	3.960E-03	5.380	9.063E-04	3.028E-04
1.037	6.364E-02	3.748E-03	5.460	6.567E-04	2.970E-04
1.062	6.569E-02	3.766E-03	5.540	1.049E-04	2.354E-04
1.088	8.179E-02	4.106E-03	5.620	9.450E-05	2.029E-04
1.112	9.237E-02	4.252E-03	5.700	2.820E-04	2.043E-04
1.138	8.451E-02	3.954E-03	5.780	1.341E-04	1.843E-04
1.162	6.817E-02	3.715E-03	5.860	2.663E-05	1.928E-04
1.187	6.137E-02	3.641E-03	5.945	3.024E-04	1.936E-04
1.215	6.751E-02	3.739E-03	6.035	5.838E-04	2.149E-04
1.245	6.093E-02	3.620E-03	6.125	3.262E-04	1.689E-04
1.275	6.656E-02	3.681E-03	6.215	3.105E-05	1.202E-04
1.305	8.222E-02	4.088E-03	6.305	3.773E-05	1.105E-04
1.335	6.713E-02	3.797E-03	6.395	1.392E-04	1.270E-04
1.365	6.237E-02	3.764E-03	6.485	1.012E-04	1.030E-04
1.395	7.982E-02	3.842E-03	6.575	5.494E-05	7.666E-05
1.425	8.748E-02	3.957E-03	6.665	8.506E-05	8.385E-05
1.455	7.438E-02	3.524E-03	6.755	9.374E-05	8.159E-05
1.485	5.543E-02	3.076E-03	6.850	1.009E-04	9.214E-05
1.515	4.858E-02	2.935E-03	6.950	7.785E-05	7.973E-05
1.545	4.756E-02	3.149E-03	7.050	2.111E-05	5.860E-05
1.580	4.229E-02	3.082E-03	7.150	-1.050E-05	4.329E-05
1.620	3.760E-02	2.777E-03	7.250	-2.925E-06	4.326E-05
1.660	3.422E-02	2.696E-03	7.350	1.785E-05	5.215E-05
1.700	3.476E-02	2.699E-03	7.450	6.272E-05	4.316E-05
1.740	3.959E-02	2.845E-03	7.550	7.414E-05	3.885E-05
1.780	3.871E-02	2.851E-03	7.650	3.475E-05	3.434E-05
1.820	3.410E-02	2.697E-03	7.750	-4.288E-07	3.260E-05
1.860	2.827E-02	2.468E-03	7.850	4.793E-06	3.534E-05
1.900	2.415E-02	2.325E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 34.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	5.357E-02	1.039E-02	1.940	2.217E-02	2.097E-03
0.065	8.331E-02	1.104E-02	1.980	2.481E-02	2.121E-03
0.075	1.021E-01	1.191E-02	2.020	2.393E-02	2.135E-03
0.085	1.349E-01	1.240E-02	2.060	2.181E-02	1.973E-03
0.095	2.179E-01	1.394E-02	2.100	2.155E-02	1.976E-03
0.105	2.182E-01	1.360E-02	2.140	1.646E-02	1.906E-03
0.115	1.814E-01	1.299E-02	2.180	1.511E-02	1.827E-03
0.125	1.895E-01	1.319E-02	2.220	1.883E-02	1.920E-03
0.135	2.017E-01	1.334E-02	2.260	1.745E-02	1.814E-03
0.145	1.791E-01	1.292E-02	2.300	1.526E-02	1.808E-03
0.155	1.193E-01	1.148E-02	2.340	1.402E-02	1.652E-03
0.165	1.221E-01	1.151E-02	2.380	1.493E-02	1.786E-03
0.177	1.539E-01	1.079E-02	2.425	1.621E-02	1.789E-03
0.192	1.436E-01	1.305E-02	2.475	1.476E-02	1.740E-03
0.207	1.917E-01	1.066E-02	2.525	1.680E-02	1.790E-03
0.222	1.850E-01	1.058E-02	2.575	1.486E-02	1.627E-03
0.237	1.189E-01	9.328E-03	2.625	1.356E-02	1.624E-03
0.252	9.648E-02	8.327E-03	2.675	1.335E-02	1.709E-03
0.267	1.080E-01	8.725E-03	2.725	1.096E-02	1.735E-03
0.282	1.169E-01	9.204E-03	2.775	9.816E-03	1.714E-03
0.297	1.336E-01	9.443E-03	2.825	1.013E-02	1.464E-03
0.313	8.149E-02	7.954E-03	2.875	1.009E-02	1.313E-03
0.327	7.522E-02	7.624E-03	2.925	1.031E-02	1.317E-03
0.342	8.022E-02	7.784E-03	2.975	8.494E-03	1.261E-03
0.357	8.736E-02	8.009E-03	3.030	6.546E-03	1.155E-03
0.372	1.283E-01	8.573E-03	3.090	5.663E-03	1.094E-03
0.387	1.799E-01	9.877E-03	3.150	3.896E-03	9.522E-04
0.402	1.850E-01	9.558E-03	3.210	5.472E-03	1.035E-03
0.417	1.489E-01	8.576E-03	3.270	5.812E-03	1.042E-03
0.432	1.041E-01	7.248E-03	3.330	5.674E-03	1.008E-03
0.447	9.110E-02	6.894E-03	3.390	6.200E-03	1.024E-03
0.462	7.416E-02	6.649E-03	3.450	5.067E-03	9.512E-04
0.477	6.067E-02	6.416E-03	3.510	6.757E-03	1.033E-03
0.492	5.305E-02	6.365E-03	3.570	8.064E-03	1.113E-03
0.507	7.818E-02	6.564E-03	3.630	6.788E-03	9.378E-04
0.522	1.145E-01	5.377E-03	3.690	3.678E-03	8.280E-04
0.540	1.455E-01	5.871E-03	3.750	4.559E-03	8.530E-04
0.560	1.313E-01	5.601E-03	3.810	4.292E-03	8.122E-04
0.580	1.226E-01	5.410E-03	3.870	4.069E-03	7.919E-04
0.600	1.342E-01	5.468E-03	3.935	3.843E-03	7.491E-04
0.620	1.186E-01	5.267E-03	4.005	3.586E-03	7.431E-04
0.640	8.432E-02	4.464E-03	4.075	3.568E-03	7.170E-04
0.660	7.063E-02	4.252E-03	4.145	3.105E-03	7.090E-04
0.680	7.156E-02	4.164E-03	4.215	2.559E-03	5.995E-04
0.700	8.856E-02	4.479E-03	4.285	2.398E-03	5.967E-04
0.720	8.607E-02	4.424E-03	4.355	1.639E-03	5.260E-04
0.740	7.215E-02	4.118E-03	4.425	1.308E-03	5.082E-04
0.760	7.350E-02	4.191E-03	4.495	7.993E-04	4.869E-04
0.780	8.639E-02	4.291E-03	4.565	5.537E-04	4.327E-04
0.800	9.021E-02	4.394E-03	4.635	1.272E-03	4.724E-04
0.820	8.455E-02	4.212E-03	4.705	1.114E-03	4.451E-04
0.840	8.415E-02	4.270E-03	4.775	4.278E-04	3.708E-04
0.860	7.683E-02	4.106E-03	4.845	6.542E-04	4.178E-04
0.880	6.582E-02	3.918E-03	4.915	1.354E-03	3.970E-04
0.900	6.414E-02	3.680E-03	4.985	1.610E-03	4.427E-04
0.920	6.373E-02	3.672E-03	5.060	1.304E-03	3.601E-04
0.940	6.152E-02	3.688E-03	5.140	1.117E-03	3.687E-04
0.962	6.557E-02	3.633E-03	5.220	8.465E-04	3.123E-04
0.987	6.589E-02	3.666E-03	5.300	8.855E-04	3.058E-04
1.013	5.657E-02	3.481E-03	5.380	6.890E-04	2.796E-04
1.037	4.973E-02	3.344E-03	5.460	3.088E-04	1.839E-04
1.062	4.691E-02	3.295E-03	5.540	1.948E-04	1.893E-04
1.088	5.631E-02	3.422E-03	5.620	1.348E-04	1.507E-04
1.112	6.889E-02	3.888E-03	5.700	1.520E-04	1.787E-04
1.138	6.582E-02	3.539E-03	5.780	2.867E-04	1.798E-04
1.162	5.359E-02	3.290E-03	5.860	3.510E-04	1.603E-04
1.187	5.004E-02	3.165E-03	5.945	2.002E-04	1.301E-04
1.215	5.123E-02	3.182E-03	6.035	1.046E-04	1.082E-04
1.245	5.080E-02	3.212E-03	6.125	1.305E-04	1.346E-04
1.275	5.764E-02	3.422E-03	6.215	7.017E-05	1.282E-04
1.305	6.190E-02	3.583E-03	6.305	-1.602E-05	6.709E-05
1.335	5.646E-02	3.561E-03	6.395	-2.200E-05	4.280E-05
1.365	5.251E-02	3.512E-03	6.485	3.764E-05	5.223E-05
1.395	6.650E-02	3.542E-03	6.575	1.089E-04	6.516E-05
1.425	7.922E-02	3.630E-03	6.665	1.362E-04	6.884E-05
1.455	6.296E-02	3.159E-03	6.755	1.109E-04	6.280E-05
1.485	3.964E-02	2.629E-03	6.850	7.011E-05	4.684E-05
1.515	3.620E-02	2.579E-03	6.950	3.590E-05	3.958E-05
1.545	3.787E-02	2.760E-03	7.050	2.101E-05	3.966E-05
1.580	3.172E-02	2.630E-03	7.150	2.565E-05	3.977E-05
1.620	2.880E-02	2.501E-03	7.250	5.301E-05	4.752E-05
1.660	2.390E-02	2.304E-03	7.350	6.212E-05	4.871E-05
1.700	2.590E-02	2.379E-03	7.450	3.690E-05	3.813E-05
1.740	2.830E-02	2.461E-03	7.550	4.990E-06	3.281E-05
1.780	2.585E-02	2.362E-03	7.650	-3.167E-06	3.287E-05
1.820	2.507E-02	2.337E-03	7.750	3.660E-06	3.611E-05
1.860	2.224E-02	2.096E-03	7.850	4.155E-06	3.346E-05
1.900	2.234E-02	2.109E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 44.7 SEC AFTER END OF IRRADIATION
COUNT FOR 15 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.575E-02	1.071E-02	1.940	2.216E-02	2.189E-03
0.065	1.081E-01	1.287E-02	1.980	2.651E-02	2.272E-03
0.075	1.505E-01	1.343E-02	2.020	2.966E-02	2.326E-03
0.085	1.559E-01	1.323E-02	2.060	2.731E-02	2.298E-03
0.095	2.738E-01	1.501E-02	2.100	2.306E-02	2.084E-03
0.105	2.217E-01	1.366E-02	2.140	2.181E-02	2.065E-03
0.115	1.707E-01	1.271E-02	2.180	2.131E-02	2.053E-03
0.125	2.079E-01	1.420E-02	2.220	1.931E-02	1.984E-03
0.135	2.085E-01	1.376E-02	2.260	2.062E-02	2.129E-03
0.145	1.617E-01	1.261E-02	2.300	2.087E-02	2.042E-03
0.155	1.211E-01	1.185E-02	2.340	1.969E-02	1.891E-03
0.165	1.282E-01	1.234E-02	2.380	2.007E-02	2.129E-03
0.177	1.561E-01	1.171E-02	2.425	1.572E-02	1.831E-03
0.192	1.792E-01	1.124E-02	2.475	1.495E-02	1.860E-03
0.207	1.894E-01	1.131E-02	2.525	1.732E-02	1.865E-03
0.222	2.123E-01	1.152E-02	2.575	1.638E-02	1.716E-03
0.237	1.145E-01	9.558E-03	2.625	1.522E-02	1.699E-03
0.252	1.037E-01	9.364E-03	2.675	1.612E-02	1.967E-03
0.267	1.144E-01	9.131E-03	2.725	1.457E-02	2.003E-03
0.282	1.367E-01	9.762E-03	2.775	1.249E-02	1.782E-03
0.297	1.347E-01	9.652E-03	2.825	1.116E-02	1.520E-03
0.313	9.235E-02	8.494E-03	2.875	1.112E-02	1.430E-03
0.327	8.614E-02	8.265E-03	2.925	1.046E-02	1.334E-03
0.342	1.060E-01	8.752E-03	2.975	7.999E-03	1.282E-03
0.357	1.055E-01	8.779E-03	3.030	6.242E-03	1.177E-03
0.372	1.496E-01	9.771E-03	3.090	6.737E-03	1.200E-03
0.387	2.150E-01	1.074E-02	3.150	5.352E-03	1.170E-03
0.402	2.379E-01	1.112E-02	3.210	5.521E-03	1.117E-03
0.417	1.862E-01	9.695E-03	3.270	6.852E-03	1.132E-03
0.432	1.284E-01	7.982E-03	3.330	8.488E-03	1.206E-03
0.447	1.035E-01	5.304E-03	3.390	6.264E-03	1.073E-03
0.462	8.481E-02	4.940E-03	3.450	4.784E-03	1.023E-03
0.477	6.950E-02	4.722E-03	3.510	6.553E-03	1.105E-03
0.492	6.271E-02	4.599E-03	3.570	6.804E-03	1.112E-03
0.507	7.821E-02	4.713E-03	3.630	7.111E-03	1.068E-03
0.522	1.147E-01	5.656E-03	3.690	6.685E-03	1.040E-03
0.540	1.468E-01	6.016E-03	3.750	4.801E-03	9.183E-04
0.560	1.483E-01	6.026E-03	3.810	4.595E-03	9.138E-04
0.580	1.452E-01	5.977E-03	3.870	4.506E-03	8.594E-04
0.600	1.633E-01	6.107E-03	3.935	4.463E-03	8.455E-04
0.620	1.414E-01	5.825E-03	4.005	5.614E-03	8.679E-04
0.640	9.899E-02	4.967E-03	4.075	5.423E-03	8.623E-04
0.660	8.091E-02	4.503E-03	4.145	3.825E-03	7.626E-04
0.680	8.592E-02	4.676E-03	4.215	3.210E-03	7.039E-04
0.700	1.024E-01	4.898E-03	4.285	2.552E-03	6.414E-04
0.720	1.019E-01	4.855E-03	4.355	1.631E-03	5.866E-04
0.740	8.779E-02	4.664E-03	4.425	1.070E-03	5.411E-04
0.760	8.727E-02	4.528E-03	4.495	1.620E-03	5.410E-04
0.780	9.050E-02	4.591E-03	4.565	1.961E-03	5.467E-04
0.800	9.974E-02	4.661E-03	4.635	1.191E-03	4.886E-04
0.820	1.027E-01	4.728E-03	4.705	1.167E-03	4.492E-04
0.840	9.500E-02	4.676E-03	4.775	1.558E-03	4.955E-04
0.860	8.975E-02	4.564E-03	4.845	1.465E-03	4.629E-04
0.880	8.154E-02	4.364E-03	4.915	1.602E-03	4.997E-04
0.900	7.587E-02	4.173E-03	4.985	1.325E-03	4.592E-04
0.920	7.218E-02	4.300E-03	5.060	1.034E-03	3.665E-04
0.940	7.160E-02	4.057E-03	5.140	1.106E-03	3.772E-04
0.962	8.157E-02	4.103E-03	5.220	5.738E-04	3.270E-04
0.987	7.900E-02	3.985E-03	5.300	3.384E-04	3.171E-04
1.013	6.122E-02	3.686E-03	5.380	7.623E-04	3.301E-04
1.037	5.563E-02	3.594E-03	5.460	8.798E-04	3.457E-04
1.062	6.163E-02	3.734E-03	5.540	4.329E-04	2.325E-04
1.088	6.698E-02	3.912E-03	5.620	2.112E-04	2.359E-04
1.112	7.751E-02	3.982E-03	5.700	2.880E-04	1.795E-04
1.138	7.834E-02	3.851E-03	5.780	2.617E-04	1.657E-04
1.162	6.350E-02	3.580E-03	5.860	2.418E-04	1.571E-04
1.187	5.763E-02	3.525E-03	5.945	2.853E-04	1.399E-04
1.215	6.128E-02	3.502E-03	6.035	1.543E-04	1.400E-04
1.245	5.490E-02	3.470E-03	6.125	1.652E-05	1.222E-04
1.275	6.892E-02	3.766E-03	6.215	4.908E-05	1.142E-04
1.305	7.509E-02	4.015E-03	6.305	1.113E-04	1.133E-04
1.335	6.388E-02	3.879E-03	6.395	4.402E-05	5.848E-05
1.365	6.531E-02	3.933E-03	6.485	2.176E-06	4.815E-05
1.395	8.292E-02	3.970E-03	6.575	6.489E-05	5.740E-05
1.425	9.626E-02	4.027E-03	6.665	1.610E-04	7.801E-05
1.455	7.817E-02	3.539E-03	6.755	1.476E-04	9.219E-05
1.485	5.155E-02	2.957E-03	6.850	5.159E-05	7.340E-05
1.515	4.368E-02	2.869E-03	6.950	2.265E-05	4.834E-05
1.545	4.317E-02	3.002E-03	7.050	6.850E-06	4.821E-05
1.580	4.166E-02	2.947E-03	7.150	-3.156E-05	6.467E-05
1.620	2.825E-02	2.590E-03	7.250	-3.839E-05	7.508E-05
1.660	2.829E-02	2.492E-03	7.350	2.049E-05	5.407E-05
1.700	3.103E-02	2.623E-03	7.450	9.838E-05	5.221E-05
1.740	3.086E-02	2.670E-03	7.550	9.663E-05	4.793E-05
1.780	3.426E-02	2.657E-03	7.650	3.056E-05	4.208E-05
1.820	2.806E-02	2.597E-03	7.750	-6.090E-06	3.904E-05
1.860	2.450E-02	2.312E-03	7.850	-1.236E-06	3.536E-05
1.900	2.700E-02	2.335E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 59.7 SEC AFTER END OF IRRADIATION
COUNT FOR 15 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	2.817E-02	4.396E-03	1.940	2.020E-02	1.967E-03
0.065	9.419E-02	1.112E-02	1.980	2.247E-02	1.981E-03
0.075	1.213E-01	1.213E-02	2.020	2.284E-02	2.066E-03
0.085	1.298E-01	1.203E-02	2.060	1.961E-02	1.904E-03
0.095	1.822E-01	1.293E-02	2.100	1.849E-02	1.849E-03
0.105	1.423E-01	1.176E-02	2.140	1.619E-02	1.810E-03
0.115	1.139E-01	1.085E-02	2.180	1.234E-02	1.731E-03
0.125	1.481E-01	1.215E-02	2.220	1.406E-02	1.835E-03
0.135	1.251E-01	1.122E-02	2.260	1.719E-02	1.857E-03
0.145	1.013E-01	1.051E-02	2.300	1.809E-02	1.881E-03
0.155	9.714E-02	1.055E-02	2.340	1.649E-02	1.734E-03
0.165	1.006E-01	1.092E-02	2.380	1.250E-02	1.669E-03
0.177	1.195E-01	9.883E-03	2.425	1.062E-02	1.564E-03
0.192	1.313E-01	9.847E-03	2.475	1.254E-02	1.631E-03
0.207	1.466E-01	9.853E-03	2.525	1.327E-02	1.643E-03
0.222	1.719E-01	1.030E-02	2.575	1.493E-02	1.659E-03
0.237	9.143E-02	8.600E-03	2.625	1.174E-02	1.531E-03
0.252	8.742E-02	8.418E-03	2.675	1.033E-02	1.646E-03
0.267	7.784E-02	8.242E-03	2.725	1.232E-02	1.813E-03
0.282	1.018E-01	8.697E-03	2.775	1.243E-02	1.642E-03
0.297	1.010E-01	8.461E-03	2.825	1.050E-02	1.371E-03
0.313	9.740E-02	7.794E-03	2.875	7.910E-03	1.209E-03
0.327	6.509E-02	7.302E-03	2.925	7.120E-03	1.150E-03
0.342	9.284E-02	7.948E-03	2.975	5.709E-03	1.087E-03
0.357	9.547E-02	8.318E-03	3.030	4.551E-03	1.028E-03
0.372	1.122E-01	8.370E-03	3.090	4.680E-03	1.025E-03
0.387	1.675E-01	9.404E-03	3.150	5.334E-03	1.009E-03
0.402	1.927E-01	9.840E-03	3.210	5.816E-03	1.025E-03
0.417	1.326E-01	8.027E-03	3.270	4.516E-03	9.384E-04
0.432	8.789E-02	6.844E-03	3.330	4.433E-03	9.363E-04
0.447	7.462E-02	6.555E-03	3.390	4.077E-03	9.071E-04
0.462	6.362E-02	6.483E-03	3.450	3.852E-03	8.796E-04
0.477	5.263E-02	6.123E-03	3.510	6.412E-03	1.007E-03
0.492	4.938E-02	6.050E-03	3.570	7.225E-03	1.064E-03
0.507	5.336E-02	6.230E-03	3.630	5.091E-03	9.346E-04
0.522	7.956E-02	6.852E-03	3.690	3.982E-03	8.608E-04
0.540	1.082E-01	5.292E-03	3.750	3.132E-03	7.774E-04
0.560	1.094E-01	5.137E-03	3.810	2.683E-03	7.203E-04
0.580	1.209E-01	5.331E-03	3.870	3.297E-03	7.616E-04
0.600	1.285E-01	5.498E-03	3.935	3.551E-03	7.002E-04
0.620	1.112E-01	5.092E-03	4.005	3.421E-03	7.589E-04
0.640	7.833E-02	4.458E-03	4.075	3.586E-03	7.095E-04
0.660	6.368E-02	4.017E-03	4.145	4.149E-03	7.383E-04
0.680	6.877E-02	4.164E-03	4.215	2.406E-03	6.195E-04
0.700	7.892E-02	4.312E-03	4.285	1.910E-03	6.157E-04
0.720	8.142E-02	4.273E-03	4.355	2.197E-03	5.571E-04
0.740	6.983E-02	4.136E-03	4.425	7.595E-04	4.785E-04
0.760	6.244E-02	3.933E-03	4.495	4.762E-04	4.684E-04
0.780	6.515E-02	3.860E-03	4.565	1.401E-03	4.671E-04
0.800	7.215E-02	4.048E-03	4.635	1.303E-03	4.548E-04
0.820	8.100E-02	4.158E-03	4.705	5.547E-04	4.087E-04
0.840	8.219E-02	4.294E-03	4.775	7.805E-04	4.125E-04
0.860	7.419E-02	4.185E-03	4.845	7.993E-04	3.903E-04
0.880	6.669E-02	3.848E-03	4.915	1.221E-03	4.022E-04
0.900	6.117E-02	3.693E-03	4.985	1.503E-03	4.342E-04
0.920	5.699E-02	3.610E-03	5.060	9.817E-04	3.445E-04
0.940	5.648E-02	3.649E-03	5.140	7.699E-04	3.549E-04
0.962	6.250E-02	3.600E-03	5.220	9.177E-04	3.325E-04
0.987	6.371E-02	3.629E-03	5.300	7.631E-04	2.729E-04
1.013	4.835E-02	3.291E-03	5.380	6.949E-04	2.483E-04
1.037	3.846E-02	3.227E-03	5.460	7.301E-04	2.407E-04
1.062	4.554E-02	3.194E-03	5.540	3.961E-04	1.962E-04
1.088	5.518E-02	3.475E-03	5.620	8.378E-05	1.761E-04
1.112	5.700E-02	3.462E-03	5.700	-1.990E-05	1.443E-04
1.138	5.625E-02	3.409E-03	5.780	-7.441E-05	1.398E-04
1.162	4.877E-02	3.205E-03	5.860	-7.285E-05	1.469E-04
1.187	4.300E-02	3.107E-03	5.945	7.800E-06	1.343E-04
1.215	4.712E-02	2.983E-03	6.035	1.176E-04	1.153E-04
1.245	4.316E-02	3.132E-03	6.125	2.172E-04	1.266E-04
1.275	5.207E-02	3.285E-03	6.215	2.115E-04	1.285E-04
1.305	6.764E-02	3.794E-03	6.305	2.316E-04	1.143E-04
1.335	5.474E-02	3.823E-03	6.395	2.370E-04	1.098E-04
1.365	5.259E-02	3.513E-03	6.485	1.153E-04	7.500E-05
1.395	6.542E-02	3.633E-03	6.575	2.635E-05	4.801E-05
1.425	7.482E-02	3.639E-03	6.665	3.712E-05	5.834E-05
1.455	6.473E-02	3.217E-03	6.755	6.705E-05	6.567E-05
1.485	4.125E-02	2.692E-03	6.850	4.791E-05	6.421E-05
1.515	3.468E-02	2.503E-03	6.950	1.207E-05	5.557E-05
1.545	3.905E-02	2.774E-03	7.050	-4.534E-07	4.421E-05
1.580	3.226E-02	2.577E-03	7.150	1.365E-05	3.956E-05
1.620	2.393E-02	2.315E-03	7.250	3.167E-05	4.898E-05
1.660	2.285E-02	2.245E-03	7.350	3.581E-05	4.979E-05
1.700	2.244E-02	2.193E-03	7.450	2.872E-05	4.937E-05
1.740	2.181E-02	2.260E-03	7.550	2.881E-05	4.232E-05
1.780	1.908E-02	2.270E-03	7.650	2.522E-05	3.454E-05
1.820	2.132E-02	2.222E-03	7.750	2.975E-05	3.661E-05
1.860	2.038E-02	2.042E-03	7.850	1.218E-05	3.469E-05
1.900	1.873E-02	1.947E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 75.0 SEC AFTER END OF IRRADIATION
COUNT FOR 15 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.534E-02	8.688E-03	1.940	1.782E-02	1.839E-03
0.065	7.514E-02	1.107E-02	1.980	1.737E-02	1.846E-03
0.075	1.005E-01	1.105E-02	2.020	1.578E-02	1.765E-03
0.085	8.136E-02	1.051E-02	2.060	1.424E-02	1.693E-03
0.095	1.531E-01	1.168E-02	2.100	1.248E-02	1.613E-03
0.105	8.809E-02	1.037E-02	2.140	9.999E-03	1.641E-03
0.115	8.808E-02	9.577E-03	2.180	9.447E-03	1.585E-03
0.125	1.019E-01	1.042E-02	2.220	1.279E-02	1.619E-03
0.135	9.333E-02	1.020E-02	2.260	1.432E-02	1.796E-03
0.145	9.132E-02	9.840E-03	2.300	1.226E-02	1.632E-03
0.155	5.455E-02	8.994E-03	2.340	1.091E-02	1.468E-03
0.165	7.428E-02	9.434E-03	2.380	1.066E-02	1.506E-03
0.177	9.602E-02	9.207E-03	2.425	1.364E-02	1.447E-03
0.192	9.869E-02	8.791E-03	2.475	9.361E-03	1.421E-03
0.207	1.321E-01	9.446E-03	2.525	9.461E-03	1.418E-03
0.222	1.244E-01	9.091E-03	2.575	1.184E-02	1.445E-03
0.237	7.646E-02	8.085E-03	2.625	1.200E-02	1.406E-03
0.252	6.746E-02	7.441E-03	2.675	9.373E-03	1.510E-03
0.267	6.358E-02	7.325E-03	2.725	8.182E-03	1.673E-03
0.282	7.847E-02	7.700E-03	2.775	9.433E-03	1.649E-03
0.297	9.140E-02	8.021E-03	2.825	8.671E-03	1.275E-03
0.313	6.007E-02	7.094E-03	2.875	7.590E-03	1.121E-03
0.327	5.216E-02	6.726E-03	2.925	6.991E-03	1.060E-03
0.342	6.497E-02	6.808E-03	2.975	4.896E-03	9.748E-04
0.357	7.028E-02	7.160E-03	3.030	3.643E-03	8.855E-04
0.372	9.231E-02	7.424E-03	3.090	4.340E-03	9.114E-04
0.387	1.351E-01	8.439E-03	3.150	3.415E-03	8.435E-04
0.402	1.457E-01	8.464E-03	3.210	3.150E-03	8.472E-04
0.417	1.204E-01	7.411E-03	3.270	3.499E-03	8.365E-04
0.432	7.821E-02	6.359E-03	3.330	4.362E-03	8.597E-04
0.447	6.162E-02	6.178E-03	3.390	4.094E-03	8.076E-04
0.462	5.800E-02	6.089E-03	3.450	3.931E-03	8.164E-04
0.477	4.325E-02	5.938E-03	3.510	4.114E-03	8.476E-04
0.492	3.711E-02	5.735E-03	3.570	4.399E-03	8.607E-04
0.507	4.593E-02	5.716E-03	3.630	3.886E-03	8.433E-04
0.522	6.168E-02	4.289E-03	3.690	3.528E-03	7.698E-04
0.540	8.202E-02	4.549E-03	3.750	4.006E-03	8.026E-04
0.560	8.580E-02	4.597E-03	3.810	2.913E-03	6.828E-04
0.580	8.455E-02	4.637E-03	3.870	1.707E-03	6.229E-04
0.600	1.340E-01	4.988E-03	3.935	1.827E-03	5.900E-04
0.620	8.283E-02	4.952E-03	4.005	2.482E-03	6.365E-04
0.640	5.634E-02	3.800E-03	4.075	2.662E-03	6.564E-04
0.660	5.191E-02	3.585E-03	4.145	2.537E-03	6.301E-04
0.680	5.194E-02	3.606E-03	4.215	2.413E-03	5.965E-04
0.700	6.770E-02	3.951E-03	4.285	2.352E-03	5.914E-04
0.720	7.135E-02	3.945E-03	4.355	1.796E-03	5.308E-04
0.740	5.867E-02	3.718E-03	4.425	1.307E-03	4.658E-04
0.760	5.231E-02	3.517E-03	4.495	1.068E-03	4.566E-04
0.780	5.585E-02	3.503E-03	4.565	1.095E-03	4.356E-04
0.800	5.880E-02	3.631E-03	4.635	1.059E-03	4.411E-04
0.820	6.060E-02	3.700E-03	4.705	8.570E-04	4.052E-04
0.840	6.467E-02	3.842E-03	4.775	6.112E-04	4.051E-04
0.860	5.939E-02	3.661E-03	4.845	7.771E-04	3.612E-04
0.880	5.188E-02	3.449E-03	4.915	9.760E-04	3.773E-04
0.900	4.889E-02	3.349E-03	4.985	7.303E-04	3.063E-04
0.920	4.557E-02	3.165E-03	5.060	7.850E-04	2.920E-04
0.940	4.751E-02	3.269E-03	5.140	7.506E-04	2.865E-04
0.962	5.385E-02	3.323E-03	5.220	3.813E-04	2.420E-04
0.987	4.884E-02	3.163E-03	5.300	5.791E-04	2.444E-04
1.013	3.796E-02	3.029E-03	5.380	8.124E-04	3.070E-04
1.037	3.639E-02	2.980E-03	5.460	3.597E-04	2.227E-04
1.062	3.745E-02	2.960E-03	5.540	2.424E-05	1.131E-04
1.088	3.998E-02	3.079E-03	5.620	4.421E-05	1.671E-04
1.112	4.502E-02	3.090E-03	5.700	8.162E-05	1.765E-04
1.138	4.166E-02	3.018E-03	5.780	1.344E-04	1.565E-04
1.162	3.674E-02	2.847E-03	5.860	2.465E-04	1.471E-04
1.187	3.558E-02	2.821E-03	5.945	2.199E-04	1.323E-04
1.215	3.027E-02	2.719E-03	6.035	7.644E-05	1.115E-04
1.245	3.047E-02	2.676E-03	6.125	2.405E-05	1.013E-04
1.275	4.183E-02	3.014E-03	6.215	7.253E-05	9.690E-05
1.305	5.162E-02	3.443E-03	6.305	9.538E-05	8.499E-05
1.335	4.897E-02	3.305E-03	6.395	7.850E-05	6.921E-05
1.365	4.533E-02	3.363E-03	6.485	7.182E-05	6.112E-05
1.395	5.844E-02	3.405E-03	6.575	9.477E-05	6.992E-05
1.425	6.615E-02	3.349E-03	6.665	1.004E-04	8.171E-05
1.455	4.841E-02	2.805E-03	6.755	4.046E-05	7.163E-05
1.485	3.153E-02	2.386E-03	6.850	-3.907E-05	7.448E-05
1.515	3.082E-02	2.307E-03	6.950	-6.400E-05	6.837E-05
1.545	3.097E-02	2.485E-03	7.050	-1.896E-05	6.480E-05
1.580	2.621E-02	2.346E-03	7.150	1.741E-05	5.841E-05
1.620	2.389E-02	2.220E-03	7.250	1.585E-05	6.699E-05
1.660	1.825E-02	1.967E-03	7.350	2.096E-05	5.169E-05
1.700	1.804E-02	1.947E-03	7.450	5.984E-05	6.109E-05
1.740	1.779E-02	2.016E-03	7.550	6.326E-05	5.306E-05
1.780	1.864E-02	2.094E-03	7.650	3.302E-05	4.671E-05
1.820	1.830E-02	2.018E-03	7.750	2.898E-06	3.616E-05
1.860	1.633E-02	1.795E-03	7.850	9.169E-06	3.597E-05
1.900	1.616E-02	1.744E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
1-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 90.3 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	2.085E-02	9.045E-03	1.940	1.672E-02	1.880E-03
0.065	7.949E-02	1.134E-02	1.980	1.958E-02	1.902E-03
0.075	8.961E-02	1.154E-02	2.020	1.852E-02	1.905E-03
0.085	1.086E-01	1.150E-02	2.060	1.486E-02	1.748E-03
0.095	1.411E-01	1.194E-02	2.100	1.400E-02	1.730E-03
0.105	1.096E-01	1.097E-02	2.140	1.250E-02	1.642E-03
0.115	8.467E-02	1.021E-02	2.180	1.278E-02	1.740E-03
0.125	1.115E-01	1.115E-02	2.220	1.311E-02	1.716E-03
0.135	7.986E-02	1.017E-02	2.260	1.393E-02	1.735E-03
0.145	8.234E-02	1.024E-02	2.300	1.495E-02	1.734E-03
0.155	6.990E-02	9.175E-03	2.340	1.170E-02	1.588E-03
0.165	7.361E-02	1.028E-02	2.380	9.532E-03	1.505E-03
0.177	1.014E-01	9.719E-03	2.425	1.082E-02	1.524E-03
0.192	1.134E-01	9.719E-03	2.475	1.094E-02	1.494E-03
0.207	1.252E-01	9.477E-03	2.525	1.254E-02	1.623E-03
0.222	1.338E-01	9.504E-03	2.575	1.102E-02	1.446E-03
0.237	9.056E-02	8.530E-03	2.625	9.875E-03	1.529E-03
0.252	6.675E-02	7.566E-03	2.675	6.666E-03	1.603E-03
0.267	7.300E-02	7.784E-03	2.725	5.157E-03	1.794E-03
0.282	7.446E-02	7.818E-03	2.775	7.879E-03	1.711E-03
0.297	9.698E-02	8.694E-03	2.825	8.581E-03	1.340E-03
0.313	8.409E-02	8.073E-03	2.875	7.687E-03	1.130E-03
0.327	5.175E-02	7.385E-03	2.925	5.527E-03	1.035E-03
0.342	8.023E-02	7.551E-03	2.975	3.911E-03	9.679E-04
0.357	9.057E-02	8.012E-03	3.030	4.316E-03	9.887E-04
0.372	8.916E-02	7.812E-03	3.090	3.425E-03	9.407E-04
0.387	1.420E-01	8.667E-03	3.150	3.639E-03	9.232E-04
0.402	1.526E-01	8.721E-03	3.210	4.176E-03	9.183E-04
0.417	1.168E-01	7.493E-03	3.270	4.808E-03	8.966E-04
0.432	8.584E-02	6.629E-03	3.330	5.145E-03	9.633E-04
0.447	7.670E-02	4.540E-03	3.390	3.063E-03	8.165E-04
0.462	5.742E-02	4.367E-03	3.450	3.865E-03	8.709E-04
0.477	4.631E-02	4.074E-03	3.510	6.220E-03	9.111E-04
0.492	3.855E-02	3.830E-03	3.570	6.916E-03	1.023E-03
0.507	4.737E-02	4.054E-03	3.630	5.052E-03	8.593E-04
0.522	5.986E-02	4.419E-03	3.690	2.991E-03	7.690E-04
0.540	7.814E-02	4.655E-03	3.750	2.604E-03	7.117E-04
0.560	9.219E-02	4.743E-03	3.810	3.397E-03	7.477E-04
0.580	9.957E-02	5.016E-03	3.870	2.548E-03	6.504E-04
0.600	1.092E-01	5.164E-03	3.935	1.285E-03	5.968E-04
0.620	8.699E-02	4.700E-03	4.005	2.360E-03	6.052E-04
0.640	5.823E-02	3.958E-03	4.075	3.389E-03	6.517E-04
0.660	4.514E-02	3.791E-03	4.145	2.671E-03	6.280E-04
0.680	5.544E-02	3.823E-03	4.215	2.331E-03	5.566E-04
0.700	7.579E-02	4.133E-03	4.285	2.329E-03	6.138E-04
0.720	7.696E-02	4.158E-03	4.355	1.774E-03	5.378E-04
0.740	6.707E-02	3.963E-03	4.425	1.610E-03	4.753E-04
0.760	5.791E-02	3.622E-03	4.495	7.649E-04	4.078E-04
0.780	5.526E-02	3.774E-03	4.565	3.799E-04	3.584E-04
0.800	5.981E-02	3.737E-03	4.635	8.136E-04	3.877E-04
0.820	7.309E-02	4.057E-03	4.705	9.147E-04	3.617E-04
0.840	8.300E-02	4.246E-03	4.775	9.171E-04	4.024E-04
0.860	7.222E-02	4.054E-03	4.845	7.587E-04	3.409E-04
0.880	6.533E-02	3.806E-03	4.915	4.380E-04	3.249E-04
0.900	5.949E-02	3.688E-03	4.985	3.895E-04	2.738E-04
0.920	5.498E-02	3.473E-03	5.060	8.055E-04	3.394E-04
0.940	5.596E-02	3.531E-03	5.140	7.689E-04	2.808E-04
0.962	5.908E-02	3.513E-03	5.220	3.097E-04	2.410E-04
0.987	5.963E-02	3.567E-03	5.300	4.896E-04	2.238E-04
1.013	4.580E-02	3.176E-03	5.380	8.709E-04	2.460E-04
1.037	3.698E-02	3.088E-03	5.460	5.391E-04	2.102E-04
1.062	4.045E-02	3.086E-03	5.540	1.464E-04	1.687E-04
1.088	4.463E-02	3.283E-03	5.620	7.350E-05	1.378E-04
1.112	5.237E-02	3.369E-03	5.700	4.959E-05	1.266E-04
1.138	4.981E-02	3.298E-03	5.780	2.984E-05	1.194E-04
1.162	4.017E-02	3.058E-03	5.860	6.146E-05	1.166E-04
1.187	3.877E-02	2.946E-03	5.945	1.172E-04	1.146E-04
1.215	3.758E-02	2.866E-03	6.035	1.293E-04	1.166E-04
1.245	3.770E-02	2.885E-03	6.125	1.477E-04	1.189E-04
1.275	4.683E-02	3.323E-03	6.215	1.454E-04	1.108E-04
1.305	5.602E-02	3.627E-03	6.305	8.438E-05	7.385E-05
1.335	5.421E-02	3.617E-03	6.395	1.592E-05	5.943E-05
1.365	4.605E-02	3.549E-03	6.485	-7.151E-06	7.322E-05
1.395	5.847E-02	3.547E-03	6.575	4.902E-06	6.288E-05
1.425	6.678E-02	3.427E-03	6.665	1.185E-05	7.492E-05
1.455	5.397E-02	2.990E-03	6.755	-2.663E-06	8.051E-05
1.485	4.112E-02	2.628E-03	6.850	2.747E-06	5.987E-05
1.515	3.164E-02	2.316E-03	6.950	2.041E-05	5.629E-05
1.545	2.972E-02	2.455E-03	7.050	3.306E-05	6.035E-05
1.580	2.740E-02	2.344E-03	7.150	2.535E-05	6.446E-05
1.620	2.055E-02	2.151E-03	7.250	2.059E-05	6.469E-05
1.660	2.433E-02	2.151E-03	7.350	3.021E-05	6.346E-05
1.700	2.152E-02	2.100E-03	7.450	4.685E-05	5.828E-05
1.740	1.847E-02	2.129E-03	7.550	4.116E-05	5.017E-05
1.780	1.851E-02	2.259E-03	7.650	1.825E-05	4.457E-05
1.820	1.643E-02	2.061E-03	7.750	5.238E-06	3.983E-05
1.860	1.474E-02	1.835E-03	7.850	4.725E-06	3.678E-05
1.900	1.537E-02	1.827E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 10.7 SEC AFTER END OF IRRADIATION
COUNT FOR 5 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.881E-02	9.205E-03	1.940	2.382E-02	1.902E-03
0.065	9.987E-02	1.074E-02	1.980	2.888E-02	1.970E-03
0.075	1.582E-01	1.213E-02	2.020	3.218E-02	2.058E-03
0.085	1.584E-01	1.214E-02	2.060	2.585E-02	1.850E-03
0.095	2.975E-01	1.426E-02	2.100	2.353E-02	1.757E-03
0.105	3.762E-01	1.529E-02	2.140	2.155E-02	1.762E-03
0.115	2.754E-01	1.362E-02	2.180	1.982E-02	1.703E-03
0.125	2.801E-01	1.362E-02	2.220	2.185E-02	1.774E-03
0.135	3.217E-01	1.454E-02	2.260	2.091E-02	1.719E-03
0.145	2.905E-01	1.395E-02	2.300	1.988E-02	1.654E-03
0.155	2.478E-01	1.334E-02	2.340	1.880E-02	1.581E-03
0.165	2.283E-01	1.305E-02	2.380	1.838E-02	1.697E-03
0.177	2.034E-01	1.103E-02	2.425	1.994E-02	1.667E-03
0.192	1.839E-01	1.025E-02	2.475	1.960E-02	1.702E-03
0.207	2.309E-01	1.051E-02	2.525	1.798E-02	1.603E-03
0.222	1.954E-01	9.969E-03	2.575	1.770E-02	1.532E-03
0.237	1.339E-01	8.960E-03	2.625	1.627E-02	1.477E-03
0.252	1.669E-01	9.309E-03	2.675	1.573E-02	1.527E-03
0.267	1.873E-01	9.801E-03	2.725	1.543E-02	1.474E-03
0.282	1.902E-01	9.851E-03	2.775	1.267E-02	1.371E-03
0.297	1.715E-01	9.442E-03	2.825	1.088E-02	1.266E-03
0.313	1.075E-01	8.038E-03	2.875	1.112E-02	1.247E-03
0.327	9.161E-02	7.669E-03	2.925	1.053E-02	1.228E-03
0.342	9.957E-02	7.815E-03	2.975	8.966E-03	1.195E-03
0.357	1.112E-01	7.973E-03	3.030	6.553E-03	1.030E-03
0.372	1.344E-01	8.557E-03	3.090	8.042E-03	1.107E-03
0.387	1.994E-01	9.463E-03	3.150	7.062E-03	1.042E-03
0.402	2.243E-01	1.000E-02	3.210	7.635E-03	1.048E-03
0.417	2.006E-01	9.139E-03	3.270	8.643E-03	1.048E-03
0.432	1.601E-01	8.095E-03	3.330	1.076E-02	1.135E-03
0.447	1.316E-01	5.191E-03	3.390	9.484E-03	1.059E-03
0.462	1.182E-01	5.000E-03	3.450	7.842E-03	9.872E-04
0.477	1.055E-01	4.799E-03	3.510	6.477E-03	9.124E-04
0.492	1.170E-01	4.876E-03	3.570	7.091E-03	9.229E-04
0.507	1.581E-01	5.433E-03	3.630	7.906E-03	9.706E-04
0.522	2.091E-01	6.589E-03	3.690	6.035E-03	8.731E-04
0.540	2.330E-01	6.746E-03	3.750	5.502E-03	8.284E-04
0.560	1.968E-01	6.063E-03	3.810	4.903E-03	7.923E-04
0.580	1.716E-01	5.673E-03	3.870	3.981E-03	7.369E-04
0.600	1.690E-01	5.528E-03	3.935	4.718E-03	7.635E-04
0.620	1.577E-01	5.288E-03	4.005	4.215E-03	7.130E-04
0.640	1.163E-01	4.685E-03	4.075	3.884E-03	6.860E-04
0.660	9.263E-02	4.277E-03	4.145	3.944E-03	6.915E-04
0.680	9.027E-02	4.191E-03	4.215	3.045E-03	6.055E-04
0.700	9.409E-02	4.208E-03	4.285	3.112E-03	6.225E-04
0.720	9.135E-02	4.160E-03	4.355	2.546E-03	5.711E-04
0.740	8.386E-02	4.034E-03	4.425	1.909E-03	5.430E-04
0.760	8.766E-02	4.000E-03	4.495	1.770E-03	5.099E-04
0.780	1.091E-01	4.223E-03	4.565	1.843E-03	4.882E-04
0.800	1.226E-01	4.389E-03	4.635	1.670E-03	4.557E-04
0.820	1.137E-01	4.265E-03	4.705	1.486E-03	4.640E-04
0.840	9.976E-02	4.048E-03	4.775	1.547E-03	4.492E-04
0.860	8.654E-02	3.899E-03	4.845	1.532E-03	4.431E-04
0.880	7.876E-02	3.727E-03	4.915	1.530E-03	4.127E-04
0.900	8.222E-02	3.738E-03	4.985	1.497E-03	4.168E-04
0.920	8.350E-02	3.668E-03	5.060	9.641E-04	3.576E-04
0.940	8.170E-02	3.681E-03	5.140	6.340E-04	3.150E-04
0.962	8.292E-02	3.624E-03	5.220	9.269E-04	3.215E-04
0.987	7.930E-02	3.605E-03	5.300	1.006E-03	3.121E-04
1.013	7.404E-02	3.406E-03	5.380	8.640E-04	3.027E-04
1.037	6.503E-02	3.296E-03	5.460	7.936E-04	2.848E-04
1.062	6.170E-02	3.250E-03	5.540	5.928E-04	2.831E-04
1.088	7.305E-02	3.437E-03	5.620	3.822E-04	2.558E-04
1.112	8.366E-02	3.494E-03	5.700	3.324E-04	2.255E-04
1.138	7.819E-02	3.370E-03	5.780	2.469E-04	2.002E-04
1.162	6.258E-02	3.090E-03	5.860	1.149E-04	1.950E-04
1.187	6.139E-02	3.114E-03	5.945	5.032E-05	1.890E-04
1.215	6.266E-02	3.130E-03	6.035	1.957E-04	1.731E-04
1.245	5.671E-02	3.036E-03	6.125	3.856E-04	1.762E-04
1.275	6.522E-02	3.284E-03	6.215	3.002E-04	1.585E-04
1.305	6.919E-02	3.332E-03	6.305	1.272E-04	1.248E-04
1.335	6.091E-02	3.141E-03	6.395	9.171E-05	1.339E-04
1.365	5.466E-02	3.011E-03	6.485	1.310E-04	9.346E-05
1.395	6.528E-02	3.168E-03	6.575	2.049E-04	1.099E-04
1.425	6.834E-02	3.168E-03	6.665	1.385E-04	9.419E-05
1.455	6.042E-02	2.901E-03	6.755	9.233E-05	7.157E-05
1.485	5.006E-02	2.639E-03	6.850	1.420E-04	8.493E-05
1.515	4.426E-02	2.422E-03	6.950	9.141E-05	5.454E-05
1.545	4.515E-02	2.643E-03	7.050	2.837E-05	3.011E-05
1.580	3.971E-02	2.549E-03	7.150	2.449E-05	2.971E-05
1.620	3.588E-02	2.345E-03	7.250	4.329E-05	3.685E-05
1.660	3.449E-02	2.285E-03	7.350	4.349E-05	3.729E-05
1.700	2.939E-02	2.228E-03	7.450	1.595E-05	2.820E-05
1.740	3.596E-02	2.575E-03	7.550	-1.481E-05	2.511E-05
1.780	4.239E-02	2.619E-03	7.650	-1.540E-05	2.517E-05
1.820	3.147E-02	2.296E-03	7.750	7.519E-07	2.690E-05
1.860	2.643E-02	2.044E-03	7.850	7.048E-06	2.426E-05
1.900	2.830E-02	2.028E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 16.7 SEC AFTER END OF IRRADIATION
COUNT FOR 8 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.892E-02	9.253E-03	1.940	2.680E-02	1.965E-03
0.065	1.059E-01	1.089E-02	1.980	2.762E-02	1.922E-03
0.075	1.460E-01	1.177E-02	2.020	2.801E-02	1.948E-03
0.085	1.787E-01	1.230E-02	2.060	2.654E-02	1.835E-03
0.095	3.028E-01	1.416E-02	2.100	2.427E-02	1.786E-03
0.105	3.142E-01	1.614E-02	2.140	2.120E-02	1.745E-03
0.115	2.702E-01	1.336E-02	2.180	2.100E-02	1.727E-03
0.125	2.636E-01	1.332E-02	2.220	2.192E-02	1.717E-03
0.135	3.102E-01	1.404E-02	2.260	2.208E-02	1.746E-03
0.145	2.697E-01	1.354E-02	2.300	1.991E-02	1.605E-03
0.155	1.922E-01	1.218E-02	2.340	1.852E-02	1.592E-03
0.165	2.030E-01	1.237E-02	2.380	1.678E-02	1.619E-03
0.177	1.951E-01	1.080E-02	2.425	1.707E-02	1.574E-03
0.192	1.706E-01	9.887E-03	2.475	1.831E-02	1.644E-03
0.207	2.296E-01	1.040E-02	2.525	1.673E-02	1.569E-03
0.222	2.085E-01	1.011E-02	2.575	1.658E-02	1.479E-03
0.237	1.417E-01	8.895E-03	2.625	1.766E-02	1.505E-03
0.252	1.483E-01	8.929E-03	2.675	1.581E-02	1.471E-03
0.267	1.739E-01	9.372E-03	2.725	1.531E-02	1.507E-03
0.282	1.682E-01	9.364E-03	2.775	1.354E-02	1.391E-03
0.297	1.581E-01	9.078E-03	2.825	1.165E-02	1.253E-03
0.313	1.017E-01	7.899E-03	2.875	1.150E-02	1.252E-03
0.327	8.776E-02	7.479E-03	2.925	1.043E-02	1.191E-03
0.342	9.644E-02	7.555E-03	2.975	9.306E-03	1.165E-03
0.357	1.071E-01	7.937E-03	3.030	7.838E-03	1.079E-03
0.372	1.412E-01	8.458E-03	3.090	7.647E-03	1.073E-03
0.387	2.057E-01	9.616E-03	3.150	6.838E-03	1.013E-03
0.402	2.293E-01	9.925E-03	3.210	6.176E-03	9.759E-04
0.417	1.962E-01	9.019E-03	3.270	7.051E-03	9.927E-04
0.432	1.487E-01	7.792E-03	3.330	7.556E-03	1.003E-03
0.447	1.244E-01	4.996E-03	3.390	8.554E-03	1.021E-03
0.462	1.016E-01	4.682E-03	3.450	7.260E-03	9.666E-04
0.477	8.514E-02	4.416E-03	3.510	6.970E-03	9.400E-04
0.492	1.026E-01	4.570E-03	3.570	8.709E-03	1.015E-03
0.507	1.333E-01	5.028E-03	3.630	6.310E-03	8.773E-04
0.522	1.738E-01	5.962E-03	3.690	5.780E-03	8.902E-04
0.540	2.067E-01	6.290E-03	3.750	5.377E-03	8.374E-04
0.560	1.777E-01	5.742E-03	3.810	4.645E-03	7.658E-04
0.580	1.613E-01	5.454E-03	3.870	5.019E-03	7.885E-04
0.600	1.652E-01	5.401E-03	3.935	4.418E-03	7.149E-04
0.620	1.587E-01	5.271E-03	4.005	5.095E-03	7.578E-04
0.640	1.154E-01	4.609E-03	4.075	4.438E-03	6.988E-04
0.660	9.044E-02	4.181E-03	4.145	4.170E-03	6.643E-04
0.680	9.327E-02	4.246E-03	4.215	4.767E-03	6.697E-04
0.700	1.009E-01	4.305E-03	4.285	2.493E-03	5.823E-04
0.720	9.563E-02	4.206E-03	4.355	1.747E-03	5.144E-04
0.740	8.530E-02	4.027E-03	4.425	1.509E-03	5.065E-04
0.760	9.978E-02	4.122E-03	4.495	1.393E-03	4.696E-04
0.780	1.151E-01	4.292E-03	4.565	1.614E-03	4.697E-04
0.800	1.160E-01	4.231E-03	4.635	1.467E-03	4.638E-04
0.820	1.098E-01	4.179E-03	4.705	1.668E-03	4.346E-04
0.840	9.656E-02	3.994E-03	4.775	1.637E-03	4.466E-04
0.860	8.268E-02	3.824E-03	4.845	1.382E-03	4.125E-04
0.880	7.511E-02	3.626E-03	4.915	1.686E-03	4.406E-04
0.900	7.935E-02	3.641E-03	4.985	1.640E-03	4.063E-04
0.920	8.353E-02	3.647E-03	5.060	1.338E-03	3.584E-04
0.940	8.088E-02	3.622E-03	5.140	6.803E-04	3.294E-04
0.962	8.268E-02	3.584E-03	5.220	7.216E-04	2.998E-04
0.987	8.241E-02	3.622E-03	5.300	7.605E-04	2.830E-04
1.013	7.348E-02	3.397E-03	5.380	8.071E-04	2.882E-04
1.037	6.711E-02	3.315E-03	5.460	7.346E-04	3.002E-04
1.062	6.273E-02	3.265E-03	5.540	5.810E-04	2.910E-04
1.088	7.089E-02	3.364E-03	5.620	5.090E-04	2.441E-04
1.112	8.426E-02	3.475E-03	5.700	4.813E-04	2.085E-04
1.138	7.928E-02	3.380E-03	5.780	3.642E-04	2.041E-04
1.162	6.632E-02	3.111E-03	5.860	2.408E-04	1.733E-04
1.187	6.142E-02	3.132E-03	5.945	2.660E-04	1.979E-04
1.215	6.100E-02	3.093E-03	6.035	3.044E-04	1.902E-04
1.245	5.787E-02	3.048E-03	6.125	2.873E-04	1.723E-04
1.275	6.192E-02	3.166E-03	6.215	1.490E-04	1.408E-04
1.305	6.991E-02	3.361E-03	6.305	8.866E-06	1.216E-04
1.335	6.285E-02	3.179E-03	6.395	-1.058E-05	9.414E-05
1.365	5.904E-02	3.137E-03	6.485	7.833E-05	1.075E-04
1.395	7.142E-02	3.238E-03	6.575	1.443E-04	1.138E-04
1.425	8.125E-02	3.351E-03	6.665	1.473E-04	9.798E-05
1.455	6.880E-02	3.015E-03	6.755	1.263E-04	9.826E-05
1.485	5.237E-02	2.666E-03	6.850	9.154E-05	7.908E-05
1.515	4.399E-02	2.401E-03	6.950	6.043E-05	5.554E-05
1.545	3.971E-02	2.513E-03	7.050	4.511E-05	4.192E-05
1.580	3.704E-02	2.452E-03	7.150	3.006E-05	3.113E-05
1.620	3.317E-02	2.281E-03	7.250	2.117E-05	2.876E-05
1.660	3.130E-02	2.211E-03	7.350	2.803E-05	3.042E-05
1.700	3.084E-02	2.264E-03	7.450	2.491E-05	3.637E-05
1.740	4.021E-02	2.577E-03	7.550	-2.249E-06	3.103E-05
1.780	3.978E-02	2.670E-03	7.650	-2.519E-05	2.500E-05
1.820	3.130E-02	2.349E-03	7.750	-1.817E-05	2.232E-05
1.860	2.729E-02	2.061E-03	7.850	-1.228E-06	2.124E-05
1.900	2.626E-02	1.925E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 24.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.648E-02	8.933E-03	1.940	2.366E-02	1.857E-03
0.065	1.135E-01	1.094E-02	1.980	2.364E-02	1.814E-03
0.075	1.465E-01	1.154E-02	2.020	2.526E-02	1.871E-03
0.085	1.571E-01	1.163E-02	2.060	2.297E-02	1.766E-03
0.095	2.736E-01	1.354E-02	2.100	2.372E-02	1.774E-03
0.105	2.644E-01	1.310E-02	2.140	2.232E-02	1.760E-03
0.115	2.274E-01	1.241E-02	2.180	1.962E-02	1.691E-03
0.125	2.567E-01	1.296E-02	2.220	2.066E-02	1.689E-03
0.135	2.733E-01	1.334E-02	2.260	2.103E-02	1.675E-03
0.145	2.294E-01	1.256E-02	2.300	2.025E-02	1.623E-03
0.155	1.639E-01	1.136E-02	2.340	1.835E-02	1.544E-03
0.165	1.558E-01	1.138E-02	2.380	1.674E-02	1.592E-03
0.177	1.716E-01	1.011E-02	2.425	1.720E-02	1.540E-03
0.192	1.558E-01	9.479E-03	2.475	1.647E-02	1.590E-03
0.207	2.218E-01	1.010E-02	2.525	1.777E-02	1.573E-03
0.222	2.052E-01	9.937E-03	2.575	1.959E-02	1.537E-03
0.237	1.207E-01	8.616E-03	2.625	1.627E-02	1.439E-03
0.252	1.139E-01	8.202E-03	2.675	1.462E-02	1.415E-03
0.267	1.391E-01	8.656E-03	2.725	1.487E-02	1.495E-03
0.282	1.358E-01	8.578E-03	2.775	1.190E-02	1.374E-03
0.297	1.323E-01	8.452E-03	2.825	1.074E-02	1.241E-03
0.313	8.064E-02	7.284E-03	2.875	1.202E-02	1.216E-03
0.327	8.326E-02	7.179E-03	2.925	1.105E-02	1.201E-03
0.342	1.063E-01	7.794E-03	2.975	8.500E-03	1.109E-03
0.357	1.072E-01	7.809E-03	3.030	8.413E-03	1.070E-03
0.372	1.405E-01	8.360E-03	3.090	6.684E-03	1.015E-03
0.387	2.062E-01	9.524E-03	3.150	5.788E-03	9.471E-04
0.402	2.282E-01	9.876E-03	3.210	7.982E-03	1.013E-03
0.417	1.841E-01	8.654E-03	3.270	6.445E-03	9.339E-04
0.432	1.347E-01	7.341E-03	3.330	6.674E-03	9.366E-04
0.447	1.137E-01	4.721E-03	3.390	6.508E-03	8.876E-04
0.462	8.814E-02	4.385E-03	3.450	7.349E-03	9.351E-04
0.477	6.728E-02	4.080E-03	3.510	7.166E-03	9.225E-04
0.492	7.864E-02	4.139E-03	3.570	7.483E-03	9.105E-04
0.507	1.002E-01	4.515E-03	3.630	8.393E-03	9.591E-04
0.522	1.300E-01	5.259E-03	3.690	4.716E-03	7.880E-04
0.540	1.676E-01	5.591E-03	3.750	3.822E-03	7.290E-04
0.560	1.574E-01	5.425E-03	3.810	4.871E-03	7.510E-04
0.580	1.428E-01	5.166E-03	3.870	3.442E-03	6.653E-04
0.600	1.541E-01	5.210E-03	3.935	3.329E-03	6.590E-04
0.620	1.445E-01	5.070E-03	4.005	3.901E-03	6.796E-04
0.640	1.017E-01	4.387E-03	4.075	3.819E-03	6.858E-04
0.660	8.093E-02	4.015E-03	4.145	3.978E-03	6.435E-04
0.680	8.708E-02	4.095E-03	4.215	3.503E-03	6.203E-04
0.700	9.407E-02	4.145E-03	4.285	2.504E-03	5.309E-04
0.720	9.628E-02	4.126E-03	4.355	1.670E-03	5.066E-04
0.740	8.643E-02	3.988E-03	4.425	1.061E-03	4.796E-04
0.760	8.717E-02	3.946E-03	4.495	1.787E-03	4.698E-04
0.780	1.023E-01	4.082E-03	4.565	2.041E-03	4.950E-04
0.800	1.072E-01	4.112E-03	4.635	1.530E-03	4.239E-04
0.820	1.023E-01	3.983E-03	4.705	1.632E-03	4.270E-04
0.840	9.040E-02	3.889E-03	4.775	1.325E-03	4.078E-04
0.860	8.081E-02	3.769E-03	4.845	1.007E-03	3.737E-04
0.880	7.960E-02	3.687E-03	4.915	1.224E-03	4.030E-04
0.900	7.848E-02	3.609E-03	4.985	1.191E-03	3.702E-04
0.920	7.904E-02	3.540E-03	5.060	9.912E-04	3.246E-04
0.940	8.019E-02	3.605E-03	5.140	7.481E-04	3.044E-04
0.962	7.745E-02	3.496E-03	5.220	6.339E-04	2.913E-04
0.987	7.684E-02	3.509E-03	5.300	7.857E-04	2.829E-04
1.013	7.115E-02	3.319E-03	5.380	1.181E-03	2.790E-04
1.037	6.080E-02	3.158E-03	5.460	1.053E-03	2.938E-04
1.062	5.934E-02	3.179E-03	5.540	2.861E-04	2.447E-04
1.088	6.878E-02	3.305E-03	5.620	1.710E-04	1.827E-04
1.112	7.526E-02	3.395E-03	5.700	3.515E-04	1.823E-04
1.138	7.227E-02	3.265E-03	5.780	1.411E-04	1.360E-04
1.162	6.701E-02	3.122E-03	5.860	1.819E-05	1.371E-04
1.187	6.117E-02	3.093E-03	5.945	2.476E-04	1.686E-04
1.215	5.753E-02	3.015E-03	6.035	4.906E-04	1.653E-04
1.245	5.321E-02	2.941E-03	6.125	3.783E-04	1.256E-04
1.275	6.064E-02	3.141E-03	6.215	1.227E-04	8.983E-05
1.305	6.956E-02	3.364E-03	6.305	1.465E-05	4.884E-05
1.335	6.152E-02	3.191E-03	6.395	5.917E-05	5.960E-05
1.365	5.991E-02	3.126E-03	6.485	8.820E-05	6.860E-05
1.395	7.251E-02	3.252E-03	6.575	3.736E-05	4.884E-05
1.425	8.159E-02	3.366E-03	6.665	-8.675E-06	3.839E-05
1.455	6.912E-02	2.964E-03	6.755	-6.520E-06	3.248E-05
1.485	5.283E-02	2.620E-03	6.850	4.279E-05	4.369E-05
1.515	4.367E-02	2.349E-03	6.950	8.218E-05	5.398E-05
1.545	3.976E-02	2.464E-03	7.050	3.170E-05	3.802E-05
1.580	4.093E-02	2.461E-03	7.150	4.511E-06	2.835E-05
1.620	3.313E-02	2.261E-03	7.250	1.497E-05	2.757E-05
1.660	2.961E-02	2.135E-03	7.350	2.116E-05	3.071E-05
1.700	3.146E-02	2.254E-03	7.450	1.636E-05	2.854E-05
1.740	3.296E-02	2.537E-03	7.550	1.213E-05	2.702E-05
1.780	3.700E-02	2.697E-03	7.650	1.360E-05	2.334E-05
1.820	2.975E-02	2.342E-03	7.750	1.591E-05	2.487E-05
1.860	2.258E-02	1.927E-03	7.850	1.022E-05	2.410E-05
1.900	2.256E-02	1.862E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 34.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.134E-02	7.886E-03	1.940	1.983E-02	1.653E-03
0.065	8.028E-02	9.551E-03	1.980	2.024E-02	1.659E-03
0.075	1.116E-01	1.026E-02	2.020	2.077E-02	1.692E-03
0.085	1.222E-01	1.040E-02	2.060	1.873E-02	1.588E-03
0.095	2.198E-01	1.189E-02	2.100	1.528E-02	1.494E-03
0.105	1.764E-01	1.104E-02	2.140	1.318E-02	1.460E-03
0.115	1.673E-01	1.081E-02	2.180	1.368E-02	1.462E-03
0.125	1.707E-01	1.100E-02	2.220	1.685E-02	1.516E-03
0.135	1.702E-01	1.104E-02	2.260	1.773E-02	1.529E-03
0.145	1.489E-01	1.056E-02	2.300	1.566E-02	1.461E-03
0.155	1.034E-01	9.494E-03	2.340	1.509E-02	1.406E-03
0.165	1.197E-01	9.988E-03	2.380	1.367E-02	1.440E-03
0.177	1.149E-01	8.826E-03	2.425	1.146E-02	1.336E-03
0.192	1.407E-01	8.763E-03	2.475	1.363E-02	1.394E-03
0.207	1.703E-01	9.215E-03	2.525	1.486E-02	1.412E-03
0.222	1.708E-01	9.015E-03	2.575	1.542E-02	1.352E-03
0.237	9.720E-02	7.782E-03	2.625	1.354E-02	1.282E-03
0.252	9.256E-02	7.278E-03	2.675	1.268E-02	1.313E-03
0.267	8.869E-02	7.271E-03	2.725	1.122E-02	1.280E-03
0.282	1.056E-01	7.540E-03	2.775	1.058E-02	1.242E-03
0.297	9.790E-02	7.257E-03	2.825	1.116E-02	1.136E-03
0.313	7.086E-02	6.698E-03	2.875	1.016E-02	1.091E-03
0.327	6.468E-02	6.370E-03	2.925	7.384E-03	9.780E-04
0.342	8.112E-02	6.750E-03	2.975	6.238E-03	9.671E-04
0.357	8.466E-02	6.876E-03	3.030	5.127E-03	8.866E-04
0.372	1.099E-01	7.444E-03	3.090	4.279E-03	8.469E-04
0.387	1.661E-01	8.362E-03	3.150	4.915E-03	8.755E-04
0.402	1.873E-01	8.635E-03	3.210	5.405E-03	8.561E-04
0.417	1.368E-01	7.328E-03	3.270	5.467E-03	8.404E-04
0.432	9.456E-02	6.244E-03	3.330	4.305E-03	7.560E-04
0.447	8.294E-02	6.097E-03	3.390	4.888E-03	7.861E-04
0.462	6.529E-02	5.761E-03	3.450	5.751E-03	8.091E-04
0.477	5.527E-02	5.605E-03	3.510	6.326E-03	8.425E-04
0.492	5.340E-02	5.567E-03	3.570	6.891E-03	8.513E-04
0.507	6.445E-02	5.693E-03	3.630	6.311E-03	8.240E-04
0.522	9.406E-02	6.408E-03	3.690	4.168E-03	7.430E-04
0.540	1.227E-01	4.771E-03	3.750	2.655E-03	6.186E-04
0.560	1.233E-01	4.672E-03	3.810	2.586E-03	6.316E-04
0.580	1.119E-01	4.540E-03	3.870	2.770E-03	6.231E-04
0.600	1.260E-01	4.679E-03	3.935	3.452E-03	6.282E-04
0.620	1.130E-01	4.416E-03	4.005	3.283E-03	5.962E-04
0.640	7.647E-02	3.805E-03	4.075	2.904E-03	5.687E-04
0.660	6.551E-02	3.607E-03	4.145	3.245E-03	5.782E-04
0.680	7.028E-02	3.628E-03	4.215	2.988E-03	5.499E-04
0.700	8.008E-02	3.746E-03	4.285	2.504E-03	5.159E-04
0.720	8.131E-02	3.721E-03	4.355	2.405E-03	5.003E-04
0.740	7.154E-02	3.574E-03	4.425	1.695E-03	4.529E-04
0.760	6.479E-02	3.450E-03	4.495	5.050E-04	3.636E-04
0.780	7.307E-02	3.488E-03	4.565	4.654E-04	3.630E-04
0.800	8.146E-02	3.607E-03	4.635	9.587E-04	3.702E-04
0.820	7.623E-02	3.559E-03	4.705	8.913E-04	3.651E-04
0.840	7.333E-02	3.504E-03	4.775	8.516E-04	3.529E-04
0.860	7.154E-02	3.435E-03	4.845	9.510E-04	3.545E-04
0.880	6.211E-02	3.284E-03	4.915	1.079E-03	3.667E-04
0.900	5.916E-02	3.111E-03	4.985	1.158E-03	3.450E-04
0.920	6.065E-02	3.114E-03	5.060	1.075E-03	3.273E-04
0.940	5.643E-02	3.088E-03	5.140	7.489E-04	2.675E-04
0.962	5.793E-02	3.061E-03	5.220	5.053E-04	2.555E-04
0.987	6.143E-02	3.082E-03	5.300	7.037E-04	2.336E-04
1.013	5.125E-02	2.885E-03	5.380	1.005E-03	2.659E-04
1.037	4.500E-02	2.756E-03	5.460	7.518E-04	2.608E-04
1.062	4.485E-02	2.794E-03	5.540	4.386E-04	2.176E-04
1.088	5.044E-02	2.908E-03	5.620	3.333E-04	1.679E-04
1.112	5.700E-02	2.943E-03	5.700	1.513E-04	1.256E-04
1.138	5.366E-02	2.845E-03	5.780	4.575E-05	8.414E-05
1.162	4.646E-02	2.670E-03	5.860	7.534E-05	1.098E-04
1.187	4.433E-02	2.627E-03	5.945	1.543E-04	1.090E-04
1.215	4.586E-02	2.647E-03	6.035	2.379E-04	1.137E-04
1.245	4.683E-02	2.727E-03	6.125	1.619E-04	1.039E-04
1.275	5.226E-02	2.893E-03	6.215	4.673E-05	9.615E-05
1.305	5.991E-02	3.067E-03	6.305	1.374E-05	8.380E-05
1.335	5.416E-02	2.948E-03	6.395	5.492E-06	8.166E-05
1.365	5.081E-02	2.847E-03	6.485	1.277E-05	7.396E-05
1.395	6.319E-02	3.032E-03	6.575	6.542E-05	7.733E-05
1.425	6.872E-02	3.021E-03	6.665	6.254E-05	7.303E-05
1.455	5.551E-02	2.647E-03	6.755	5.456E-05	7.357E-05
1.485	4.254E-02	2.303E-03	6.850	9.357E-05	6.305E-05
1.515	3.817E-02	2.199E-03	6.950	1.118E-04	6.067E-05
1.545	3.589E-02	2.265E-03	7.050	8.829E-05	4.258E-05
1.580	3.289E-02	2.187E-03	7.150	3.981E-05	4.418E-05
1.620	2.549E-02	1.944E-03	7.250	1.095E-05	4.290E-05
1.660	2.282E-02	1.878E-03	7.350	5.153E-06	3.122E-05
1.700	2.271E-02	2.000E-03	7.450	-2.694E-06	2.910E-05
1.740	2.545E-02	2.191E-03	7.550	-2.234E-05	2.205E-05
1.780	2.237E-02	2.306E-03	7.650	-5.768E-06	2.964E-05
1.820	2.203E-02	2.030E-03	7.750	3.373E-06	2.667E-05
1.860	1.919E-02	1.763E-03	7.850	2.188E-06	2.144E-05
1.900	1.950E-02	1.645E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 44.7 SEC AFTER END OF IRRADIATION
COUNT FOR 10 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.628E-02	7.114E-03	1.940	1.620E-02	1.603E-03
0.065	7.559E-02	9.167E-03	1.980	1.944E-02	1.674E-03
0.075	1.109E-01	9.637E-03	2.020	1.745E-02	1.631E-03
0.085	9.549E-02	9.256E-03	2.060	1.448E-02	1.495E-03
0.095	1.456E-01	1.013E-02	2.100	1.522E-02	1.529E-03
0.105	1.445E-01	9.991E-03	2.140	1.167E-02	1.449E-03
0.115	1.031E-01	9.224E-03	2.180	1.089E-02	1.363E-03
0.125	1.370E-01	9.878E-03	2.220	1.328E-02	1.503E-03
0.135	1.324E-01	9.770E-03	2.260	1.250E-02	1.413E-03
0.145	9.752E-02	8.720E-03	2.300	1.123E-02	1.362E-03
0.155	8.111E-02	8.587E-03	2.340	1.005E-02	1.280E-03
0.165	8.141E-02	8.766E-03	2.380	1.024E-02	1.349E-03
0.177	1.142E-01	9.363E-03	2.425	1.155E-02	1.329E-03
0.192	1.064E-01	7.866E-03	2.475	1.076E-02	1.342E-03
0.207	1.350E-01	8.176E-03	2.525	1.035E-02	1.268E-03
0.222	1.225E-01	8.046E-03	2.575	1.213E-02	1.270E-03
0.237	7.692E-02	6.924E-03	2.625	1.089E-02	1.221E-03
0.252	6.605E-02	6.437E-03	2.675	9.045E-03	1.199E-03
0.267	7.325E-02	6.609E-03	2.725	1.042E-02	1.218E-03
0.282	8.073E-02	6.865E-03	2.775	1.017E-02	1.227E-03
0.297	8.432E-02	6.897E-03	2.825	9.124E-03	1.112E-03
0.313	6.494E-02	6.280E-03	2.875	8.661E-03	1.070E-03
0.327	5.410E-02	5.971E-03	2.925	6.705E-03	9.632E-04
0.342	6.703E-02	6.159E-03	2.975	4.908E-03	9.136E-04
0.357	7.415E-02	6.415E-03	3.030	2.862E-03	8.141E-04
0.372	1.046E-01	6.936E-03	3.090	3.194E-03	8.312E-04
0.387	1.456E-01	7.790E-03	3.150	4.042E-03	8.651E-04
0.402	1.451E-01	7.782E-03	3.210	2.528E-03	7.493E-04
0.417	1.174E-01	6.685E-03	3.270	3.567E-03	7.958E-04
0.432	8.866E-02	5.935E-03	3.330	3.553E-03	7.648E-04
0.447	6.696E-02	5.697E-03	3.390	3.398E-03	7.482E-04
0.462	5.346E-02	5.552E-03	3.450	4.491E-03	7.793E-04
0.477	4.136E-02	5.337E-03	3.510	4.658E-03	7.814E-04
0.492	3.714E-02	5.249E-03	3.570	5.910E-03	8.306E-04
0.507	4.615E-02	5.316E-03	3.630	6.871E-03	8.769E-04
0.522	7.186E-02	5.943E-03	3.690	4.874E-03	7.569E-04
0.540	1.002E-01	4.383E-03	3.750	3.024E-03	6.853E-04
0.560	9.640E-02	4.263E-03	3.810	3.024E-03	6.412E-04
0.580	9.618E-02	4.275E-03	3.870	3.073E-03	6.644E-04
0.600	1.052E-01	4.408E-03	3.935	1.934E-03	5.590E-04
0.620	9.008E-02	4.071E-03	4.005	2.143E-03	5.645E-04
0.640	6.378E-02	3.535E-03	4.075	2.672E-03	6.160E-04
0.660	5.099E-02	3.234E-03	4.145	2.072E-03	5.195E-04
0.680	5.806E-02	3.321E-03	4.215	2.289E-03	5.379E-04
0.700	6.747E-02	3.515E-03	4.285	2.247E-03	5.296E-04
0.720	6.835E-02	3.500E-03	4.355	1.645E-03	4.847E-04
0.740	5.972E-02	3.380E-03	4.425	9.657E-04	4.226E-04
0.760	5.273E-02	3.183E-03	4.495	1.064E-03	3.912E-04
0.780	6.043E-02	3.271E-03	4.565	1.392E-03	3.857E-04
0.800	6.790E-02	3.375E-03	4.635	1.014E-03	3.784E-04
0.820	6.865E-02	3.404E-03	4.705	8.455E-04	3.612E-04
0.840	6.653E-02	3.371E-03	4.775	1.115E-03	3.358E-04
0.860	5.955E-02	3.212E-03	4.845	1.013E-03	3.813E-04
0.880	5.197E-02	3.066E-03	4.915	7.109E-04	3.273E-04
0.900	4.737E-02	2.897E-03	4.985	5.465E-04	3.352E-04
0.920	4.541E-02	2.870E-03	5.060	2.463E-04	2.369E-04
0.940	4.748E-02	2.874E-03	5.140	2.040E-04	2.450E-04
0.962	5.148E-02	2.912E-03	5.220	3.090E-04	2.053E-04
0.987	5.097E-02	2.849E-03	5.300	2.727E-04	1.987E-04
1.013	4.433E-02	2.739E-03	5.380	5.018E-04	2.045E-04
1.037	3.544E-02	2.525E-03	5.460	8.302E-04	2.389E-04
1.062	3.551E-02	2.589E-03	5.540	7.143E-04	2.037E-04
1.088	4.287E-02	2.675E-03	5.620	3.841E-04	1.755E-04
1.112	5.240E-02	2.821E-03	5.700	1.687E-04	1.333E-04
1.138	5.080E-02	2.766E-03	5.780	1.252E-04	1.109E-04
1.162	3.820E-02	2.552E-03	5.860	9.096E-05	8.960E-05
1.187	3.683E-02	2.533E-03	5.945	3.240E-05	5.686E-05
1.215	4.220E-02	2.578E-03	6.035	3.208E-05	6.900E-05
1.245	3.651E-02	2.535E-03	6.125	7.724E-05	6.119E-05
1.275	3.902E-02	2.618E-03	6.215	1.251E-04	7.735E-05
1.305	4.593E-02	2.821E-03	6.305	7.235E-05	5.501E-05
1.335	4.419E-02	2.716E-03	6.395	2.608E-05	3.735E-05
1.365	4.481E-02	2.738E-03	6.485	2.183E-05	4.338E-05
1.395	5.437E-02	2.789E-03	6.575	2.239E-05	4.582E-05
1.425	6.006E-02	2.917E-03	6.665	1.377E-05	3.955E-05
1.455	4.865E-02	2.538E-03	6.755	1.105E-05	3.902E-05
1.485	3.446E-02	2.242E-03	6.850	1.619E-05	3.536E-05
1.515	3.032E-02	2.093E-03	6.950	1.681E-05	3.523E-05
1.545	3.096E-02	2.316E-03	7.050	1.374E-05	3.968E-05
1.580	2.656E-02	2.153E-03	7.150	3.824E-06	3.081E-05
1.620	2.277E-02	1.941E-03	7.250	3.652E-06	2.992E-05
1.660	1.864E-02	1.832E-03	7.350	1.768E-05	2.970E-05
1.700	1.708E-02	1.858E-03	7.450	2.378E-05	3.515E-05
1.740	2.028E-02	2.141E-03	7.550	1.360E-05	3.383E-05
1.780	1.785E-02	2.241E-03	7.650	-6.102E-07	2.788E-05
1.820	1.939E-02	2.101E-03	7.750	-1.339E-06	2.539E-05
1.860	1.835E-02	1.781E-03	7.850	2.179E-06	2.266E-05
1.900	1.467E-02	1.558E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 54.7 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.630E-02	9.188E-03	1.940	2.400E-02	1.990E-03
0.065	1.369E-01	1.146E-02	1.980	3.034E-02	2.113E-03
0.075	1.409E-01	1.151E-02	2.020	2.869E-02	2.077E-03
0.085	1.538E-01	1.184E-02	2.060	2.328E-02	1.900E-03
0.095	2.470E-01	1.292E-02	2.100	2.222E-02	1.833E-03
0.105	1.646E-01	1.135E-02	2.140	1.815E-02	1.823E-03
0.115	1.567E-01	1.120E-02	2.180	1.600E-02	1.707E-03
0.125	1.914E-01	1.181E-02	2.220	1.921E-02	1.816E-03
0.135	1.668E-01	1.134E-02	2.260	1.932E-02	1.787E-03
0.145	1.349E-01	1.073E-02	2.300	1.809E-02	1.781E-03
0.155	1.014E-01	1.012E-02	2.340	1.607E-02	1.629E-03
0.165	1.272E-01	1.089E-02	2.380	1.499E-02	1.651E-03
0.177	1.658E-01	1.032E-02	2.425	1.483E-02	1.638E-03
0.192	1.619E-01	9.797E-03	2.475	1.439E-02	1.581E-03
0.207	2.147E-01	1.023E-02	2.525	1.931E-02	1.705E-03
0.222	2.056E-01	1.004E-02	2.575	1.858E-02	1.628E-03
0.237	1.237E-01	8.717E-03	2.625	1.497E-02	1.496E-03
0.252	1.193E-01	8.442E-03	2.675	1.588E-02	1.562E-03
0.267	1.107E-01	8.242E-03	2.725	1.819E-02	1.663E-03
0.282	1.278E-01	8.518E-03	2.775	1.512E-02	1.555E-03
0.297	1.390E-01	8.628E-03	2.825	1.310E-02	1.375E-03
0.313	1.013E-01	7.900E-03	2.875	1.200E-02	1.311E-03
0.327	9.330E-02	7.642E-03	2.925	9.675E-03	1.214E-03
0.342	1.043E-01	7.905E-03	2.975	7.723E-03	1.184E-03
0.357	1.182E-01	8.159E-03	3.030	4.592E-03	1.024E-03
0.372	1.694E-01	8.906E-03	3.090	5.615E-03	1.073E-03
0.387	2.273E-01	1.017E-02	3.150	5.234E-03	1.011E-03
0.402	2.418E-01	1.010E-02	3.210	6.229E-03	9.932E-04
0.417	1.858E-01	8.646E-03	3.270	7.761E-03	1.066E-03
0.432	1.188E-01	7.121E-03	3.330	6.780E-03	9.799E-04
0.447	1.034E-01	6.732E-03	3.390	8.232E-03	1.048E-03
0.462	8.758E-02	6.531E-03	3.450	6.598E-03	9.416E-04
0.477	7.031E-02	6.194E-03	3.510	7.372E-03	1.026E-03
0.492	6.364E-02	6.167E-03	3.570	9.881E-03	1.101E-03
0.507	6.508E-02	6.206E-03	3.630	8.764E-03	1.031E-03
0.522	9.601E-02	6.796E-03	3.690	4.517E-03	8.139E-04
0.540	1.406E-01	5.281E-03	3.750	2.898E-03	7.662E-04
0.560	1.451E-01	5.325E-03	3.810	4.265E-03	7.677E-04
0.580	1.444E-01	5.333E-03	3.870	3.320E-03	7.266E-04
0.600	1.672E-01	5.539E-03	3.935	2.049E-03	6.174E-04
0.620	1.407E-01	5.207E-03	4.005	5.055E-03	7.552E-04
0.640	9.104E-02	4.363E-03	4.075	5.423E-03	7.726E-04
0.660	7.629E-02	4.157E-03	4.145	3.671E-03	6.806E-04
0.680	8.617E-02	4.204E-03	4.215	3.096E-03	6.347E-04
0.700	1.091E-01	4.560E-03	4.285	2.724E-03	6.220E-04
0.720	1.086E-01	4.477E-03	4.355	2.223E-03	5.724E-04
0.740	9.788E-02	4.302E-03	4.425	1.349E-03	5.001E-04
0.760	8.459E-02	4.056E-03	4.495	1.023E-03	4.919E-04
0.780	8.671E-02	3.992E-03	4.565	1.556E-03	4.688E-04
0.800	9.679E-02	4.157E-03	4.635	2.046E-03	4.942E-04
0.820	9.953E-02	4.248E-03	4.705	1.224E-03	4.353E-04
0.840	1.042E-01	4.309E-03	4.775	7.731E-04	3.847E-04
0.860	9.758E-02	4.221E-03	4.845	9.842E-04	4.226E-04
0.880	8.507E-02	3.926E-03	4.915	1.098E-03	3.721E-04
0.900	7.891E-02	3.847E-03	4.985	1.538E-03	4.179E-04
0.920	7.744E-02	3.677E-03	5.060	1.304E-03	3.945E-04
0.940	8.181E-02	3.825E-03	5.140	7.784E-04	2.807E-04
0.962	8.636E-02	3.768E-03	5.220	9.798E-04	3.045E-04
0.987	8.369E-02	3.768E-03	5.300	9.856E-04	3.145E-04
1.013	6.969E-02	3.522E-03	5.380	8.311E-04	3.005E-04
1.037	5.994E-02	3.330E-03	5.460	7.380E-04	2.607E-04
1.062	5.951E-02	3.340E-03	5.540	3.680E-04	2.257E-04
1.088	6.524E-02	3.429E-03	5.620	8.251E-05	1.413E-04
1.112	7.626E-02	3.567E-03	5.700	6.479E-05	1.316E-04
1.138	7.102E-02	3.372E-03	5.780	9.277E-05	1.365E-04
1.162	5.848E-02	3.180E-03	5.860	2.500E-04	1.438E-04
1.187	5.659E-02	3.185E-03	5.945	4.045E-04	1.501E-04
1.215	5.838E-02	3.188E-03	6.035	2.151E-04	1.201E-04
1.245	5.969E-02	3.244E-03	6.125	4.838E-05	8.718E-05
1.275	6.633E-02	3.431E-03	6.215	9.782E-05	1.002E-04
1.305	7.997E-02	3.694E-03	6.305	1.108E-04	9.929E-05
1.335	7.766E-02	3.667E-03	6.395	2.670E-05	7.208E-05
1.365	6.885E-02	3.507E-03	6.485	4.572E-06	5.063E-05
1.395	8.711E-02	3.640E-03	6.575	5.999E-05	6.019E-05
1.425	1.012E-01	3.812E-03	6.665	8.074E-05	6.655E-05
1.455	8.520E-02	3.355E-03	6.755	4.262E-05	4.587E-05
1.485	5.737E-02	2.888E-03	6.850	2.647E-05	3.521E-05
1.515	4.513E-02	2.601E-03	6.950	3.219E-05	3.908E-05
1.545	4.990E-02	2.970E-03	7.050	2.288E-05	3.129E-05
1.580	4.226E-02	2.759E-03	7.150	1.122E-06	3.590E-05
1.620	3.497E-02	2.471E-03	7.250	-3.744E-06	3.850E-05
1.660	2.871E-02	2.325E-03	7.350	2.217E-05	3.315E-05
1.700	2.888E-02	2.418E-03	7.450	3.574E-05	3.412E-05
1.740	2.990E-02	2.718E-03	7.550	1.324E-05	2.981E-05
1.780	3.267E-02	2.982E-03	7.650	-7.884E-06	3.019E-05
1.820	3.352E-02	2.725E-03	7.750	-5.663E-06	3.190E-05
1.860	2.660E-02	2.156E-03	7.850	5.606E-06	2.843E-05
1.900	2.350E-02	1.974E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 75 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.636E-02	8.384E-03	1.940	1.949E-02	1.771E-03
0.065	1.057E-01	1.017E-02	1.980	2.266E-02	1.833E-03
0.075	1.165E-01	1.053E-02	2.020	2.274E-02	1.835E-03
0.085	1.005E-01	1.013E-02	2.060	1.708E-02	1.645E-03
0.095	1.712E-01	1.089E-02	2.100	1.415E-02	1.546E-03
0.105	1.175E-01	9.698E-03	2.140	1.630E-02	1.633E-03
0.115	1.141E-01	9.644E-03	2.180	1.673E-02	1.594E-03
0.125	1.238E-01	1.022E-02	2.220	1.434E-02	1.606E-03
0.135	9.600E-02	9.524E-03	2.260	1.561E-02	1.618E-03
0.145	9.130E-02	9.135E-03	2.300	1.561E-02	1.618E-03
0.155	6.354E-02	8.796E-03	2.340	1.303E-02	1.403E-03
0.165	8.312E-02	9.275E-03	2.380	1.306E-02	1.548E-03
0.177	1.297E-01	9.152E-03	2.425	1.017E-02	1.332E-03
0.192	1.356E-01	8.847E-03	2.475	1.089E-02	1.386E-03
0.207	1.659E-01	9.076E-03	2.525	1.408E-02	1.444E-03
0.222	1.480E-01	8.756E-03	2.575	1.440E-02	1.485E-03
0.237	9.877E-02	7.846E-03	2.625	1.130E-02	1.273E-03
0.252	8.140E-02	7.145E-03	2.675	1.249E-02	1.390E-03
0.267	9.012E-02	7.295E-03	2.725	1.358E-02	1.465E-03
0.282	9.821E-02	7.472E-03	2.775	1.230E-02	1.334E-03
0.297	1.116E-01	7.861E-03	2.825	8.965E-03	1.136E-03
0.313	8.101E-02	7.099E-03	2.875	7.959E-03	1.076E-03
0.327	7.085E-02	6.690E-03	2.925	6.780E-03	1.012E-03
0.342	8.813E-02	7.175E-03	2.975	5.199E-03	9.784E-04
0.357	9.548E-02	7.231E-03	3.030	5.535E-03	9.562E-04
0.372	1.141E-01	7.539E-03	3.090	4.975E-03	9.311E-04
0.387	1.667E-01	8.459E-03	3.150	5.073E-03	9.225E-04
0.402	1.765E-01	8.552E-03	3.210	4.001E-03	8.533E-04
0.417	1.322E-01	7.317E-03	3.270	4.488E-03	8.663E-04
0.432	9.664E-02	6.341E-03	3.330	5.534E-03	8.935E-04
0.447	8.235E-02	4.137E-03	3.390	4.521E-03	8.165E-04
0.462	7.126E-02	4.022E-03	3.450	5.293E-03	8.720E-04
0.477	5.259E-02	3.769E-03	3.510	5.355E-03	8.372E-04
0.492	4.371E-02	3.544E-03	3.570	6.200E-03	9.020E-04
0.507	5.243E-02	3.676E-03	3.630	6.192E-03	8.749E-04
0.522	7.490E-02	4.226E-03	3.690	3.973E-03	7.842E-04
0.540	9.856E-02	4.565E-03	3.750	2.770E-03	7.124E-04
0.560	9.952E-02	4.467E-03	3.810	3.120E-03	7.137E-04
0.580	1.099E-01	4.671E-03	3.870	3.190E-03	6.823E-04
0.600	1.254E-01	4.895E-03	3.935	3.087E-03	6.311E-04
0.620	1.032E-01	4.459E-03	4.005	2.880E-03	6.413E-04
0.640	6.756E-02	3.780E-03	4.075	2.912E-03	5.995E-04
0.660	5.182E-02	3.440E-03	4.145	3.877E-03	6.604E-04
0.680	6.466E-02	3.633E-03	4.215	3.507E-03	6.173E-04
0.700	8.261E-02	3.926E-03	4.285	2.932E-03	5.838E-04
0.720	8.648E-02	3.960E-03	4.355	2.316E-03	5.275E-04
0.740	7.559E-02	3.780E-03	4.425	1.055E-03	4.253E-04
0.760	6.179E-02	3.512E-03	4.495	6.567E-04	3.915E-04
0.780	6.440E-02	3.507E-03	4.565	8.804E-04	3.954E-04
0.800	7.072E-02	3.618E-03	4.635	9.073E-04	3.878E-04
0.820	7.446E-02	3.763E-03	4.705	8.772E-04	3.877E-04
0.840	8.045E-02	3.798E-03	4.775	9.839E-04	3.735E-04
0.860	7.692E-02	3.697E-03	4.845	1.172E-03	3.702E-04
0.880	6.464E-02	3.466E-03	4.915	1.102E-03	3.563E-04
0.900	5.983E-02	3.330E-03	4.985	9.540E-04	3.181E-04
0.920	6.076E-02	3.291E-03	5.060	1.114E-03	3.136E-04
0.940	6.019E-02	3.293E-03	5.140	9.977E-04	2.974E-04
0.962	6.363E-02	3.311E-03	5.220	6.838E-04	2.494E-04
0.987	6.537E-02	3.303E-03	5.300	7.560E-04	2.295E-04
1.013	5.328E-02	2.986E-03	5.380	7.215E-04	2.252E-04
1.037	4.424E-02	2.838E-03	5.460	3.951E-04	1.681E-04
1.062	4.410E-02	2.951E-03	5.540	9.337E-05	1.538E-04
1.088	4.879E-02	3.004E-03	5.620	-7.834E-06	1.049E-04
1.112	5.176E-02	3.022E-03	5.700	4.343E-05	1.156E-04
1.138	4.840E-02	2.943E-03	5.780	3.958E-05	1.018E-04
1.162	4.592E-02	2.850E-03	5.860	1.251E-05	7.813E-05
1.187	4.444E-02	2.802E-03	5.945	1.216E-04	9.719E-05
1.215	4.459E-02	2.736E-03	6.035	2.746E-04	1.144E-04
1.245	4.663E-02	2.882E-03	6.125	2.438E-04	1.032E-04
1.275	5.162E-02	3.105E-03	6.215	1.076E-04	7.913E-05
1.305	6.188E-02	3.266E-03	6.305	5.352E-05	6.837E-05
1.335	5.496E-02	3.168E-03	6.395	7.938E-05	5.001E-05
1.365	5.551E-02	3.072E-03	6.485	7.515E-05	4.730E-05
1.395	7.239E-02	3.370E-03	6.575	5.097E-05	4.002E-05
1.425	7.876E-02	3.402E-03	6.665	3.916E-05	4.300E-05
1.455	6.379E-02	2.910E-03	6.755	7.589E-06	3.476E-05
1.485	4.021E-02	2.406E-03	6.850	-1.691E-05	2.609E-05
1.515	3.026E-02	2.234E-03	6.950	-1.646E-05	2.910E-05
1.545	3.360E-02	2.448E-03	7.050	9.563E-06	3.805E-05
1.580	3.209E-02	2.396E-03	7.150	1.017E-05	3.638E-05
1.620	2.328E-02	2.061E-03	7.250	7.926E-06	3.062E-05
1.660	2.556E-02	2.076E-03	7.350	1.109E-05	1.973E-05
1.700	2.068E-02	2.073E-03	7.450	2.235E-05	2.253E-05
1.740	2.398E-02	2.516E-03	7.550	1.932E-05	3.101E-05
1.780	2.211E-02	2.650E-03	7.650	1.043E-06	3.243E-05
1.820	2.319E-02	2.352E-03	7.750	-3.319E-06	2.499E-05
1.860	2.084E-02	1.958E-03	7.850	7.444E-06	2.087E-05
1.900	1.813E-02	1.768E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 95 SEC AFTER END OF IRRADIATION
COUNT FOR 20 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	8.176E-04	7.031E-03	1.940	1.502E-02	1.550E-03
0.065	9.052E-02	9.133E-03	1.980	1.573E-02	1.562E-03
0.075	9.732E-02	9.681E-03	2.020	1.303E-02	1.482E-03
0.085	7.968E-02	9.307E-03	2.060	1.230E-02	1.439E-03
0.095	1.137E-01	9.575E-03	2.100	1.184E-02	1.441E-03
0.105	7.969E-02	8.641E-03	2.140	8.841E-03	1.351E-03
0.115	7.425E-02	8.341E-03	2.180	1.090E-02	1.398E-03
0.125	8.608E-02	8.904E-03	2.220	1.294E-02	1.455E-03
0.135	7.710E-02	8.673E-03	2.260	1.156E-02	1.390E-03
0.145	7.934E-02	8.476E-03	2.300	1.051E-02	1.344E-03
0.155	5.583E-02	7.953E-03	2.340	1.025E-02	1.290E-03
0.165	8.102E-02	8.751E-03	2.380	1.006E-02	1.310E-03
0.177	8.669E-02	7.947E-03	2.425	1.009E-02	1.253E-03
0.192	1.116E-01	8.166E-03	2.475	9.288E-03	1.200E-03
0.207	1.184E-01	7.883E-03	2.525	9.062E-03	1.249E-03
0.222	9.369E-02	7.669E-03	2.575	1.09.-02	1.279E-03
0.237	7.291E-02	6.840E-03	2.625	9.598E-03	1.198E-03
0.252	5.644E-02	6.484E-03	2.675	9.393E-03	1.215E-03
0.267	7.135E-02	6.653E-03	2.725	9.726E-03	1.276E-03
0.282	7.038E-02	6.769E-03	2.775	8.533E-03	1.264E-03
0.297	9.199E-02	7.131E-03	2.825	6.828E-03	1.045E-03
0.313	7.549E-02	6.661E-03	2.875	5.793E-03	9.141E-04
0.327	6.310E-02	6.277E-03	2.925	5.098E-03	8.705E-04
0.342	8.111E-02	6.784E-03	2.975	3.628E-03	8.415E-04
0.357	8.147E-02	6.476E-03	3.030	2.722E-03	7.683E-04
0.372	9.112E-02	6.840E-03	3.090	3.441E-03	8.007E-04
0.387	1.170E-01	7.190E-03	3.150	2.828E-03	7.257E-04
0.402	1.317E-01	7.460E-03	3.210	3.279E-03	7.481E-04
0.417	9.813E-02	6.375E-03	3.270	3.800E-03	7.481E-04
0.432	7.410E-02	5.635E-03	3.330	3.575E-03	7.118E-04
0.447	6.610E-02	5.812E-03	3.390	4.106E-03	7.127E-04
0.462	5.835E-02	5.621E-03	3.450	4.413E-03	7.454E-04
0.477	4.714E-02	5.464E-03	3.510	5.007E-03	7.612E-04
0.492	3.449E-02	5.209E-03	3.570	5.939E-03	8.478E-04
0.507	3.618E-02	5.235E-03	3.630	4.531E-03	7.421E-04
0.522	5.592E-02	5.611E-03	3.690	2.877E-03	6.359E-04
0.540	7.370E-02	5.931E-03	3.750	2.311E-03	6.002E-04
0.560	7.083E-02	5.862E-03	3.810	1.643E-03	5.619E-04
0.580	8.958E-02	4.177E-03	3.870	1.239E-03	5.275E-04
0.600	1.046E-01	4.413E-03	3.935	1.304E-03	4.809E-04
0.620	8.492E-02	4.071E-03	4.005	1.950E-03	5.197E-04
0.640	5.573E-02	3.483E-03	4.075	2.910E-03	5.581E-04
0.660	5.035E-02	3.347E-03	4.145	2.829E-03	5.455E-04
0.680	5.289E-02	3.348E-03	4.215	1.653E-03	4.819E-04
0.700	6.831E-02	3.522E-03	4.285	2.051E-03	4.782E-04
0.720	7.756E-02	3.670E-03	4.355	2.634E-03	4.976E-04
0.740	6.531E-02	3.409E-03	4.425	1.298E-03	4.021E-04
0.760	5.471E-02	3.237E-03	4.495	6.638E-04	3.554E-04
0.780	4.866E-02	3.114E-03	4.565	9.601E-04	3.704E-04
0.800	5.530E-02	3.200E-03	4.635	7.551E-04	3.136E-04
0.820	6.542E-02	3.453E-03	4.705	5.153E-04	3.157E-04
0.840	6.723E-02	3.461E-03	4.775	3.335E-04	2.932E-04
0.860	6.498E-02	3.402E-03	4.845	2.891E-04	2.743E-04
0.880	5.956E-02	3.259E-03	4.915	4.844E-04	3.041E-04
0.900	5.473E-02	3.148E-03	4.985	5.547E-04	2.934E-04
0.920	5.032E-02	2.957E-03	5.060	5.356E-04	2.965E-04
0.940	4.851E-02	3.012E-03	5.140	6.025E-04	2.431E-04
0.962	5.437E-02	3.025E-03	5.220	5.939E-04	2.362E-04
0.987	5.316E-02	2.915E-03	5.300	5.106E-04	2.177E-04
1.013	4.189E-02	2.688E-03	5.380	7.806E-04	2.312E-04
1.037	3.626E-02	2.671E-03	5.460	9.215E-04	2.364E-04
1.062	3.540E-02	2.646E-03	5.540	5.236E-04	1.844E-04
1.088	4.199E-02	2.712E-03	5.620	1.228E-04	1.674E-04
1.112	4.373E-02	2.786E-03	5.700	1.018E-05	1.111E-04
1.138	3.844E-02	2.609E-03	5.780	4.025E-05	1.231E-04
1.162	3.381E-02	2.483E-03	5.860	3.752E-05	9.327E-05
1.187	3.228E-02	2.466E-03	5.945	6.717E-06	6.179E-05
1.215	3.470E-02	2.509E-03	6.035	1.127E-05	5.690E-05
1.245	3.340E-02	2.532E-03	6.125	9.367E-05	7.904E-05
1.275	4.380E-02	2.774E-03	6.215	1.906E-04	9.094E-05
1.305	5.291E-02	3.050E-03	6.305	1.691E-04	8.086E-05
1.335	4.351E-02	2.848E-03	6.395	6.620E-05	6.139E-05
1.365	4.281E-02	2.759E-03	6.485	-1.956E-06	4.906E-05
1.395	5.871E-02	2.971E-03	6.575	-2.134E-05	4.692E-05
1.425	6.838E-02	3.094E-03	6.665	-2.784E-05	4.010E-05
1.455	5.137E-02	2.628E-03	6.755	-2.536E-05	4.783E-05
1.485	3.318E-02	2.204E-03	6.850	7.246E-06	4.905E-05
1.515	2.698E-02	2.069E-03	6.950	4.290E-05	4.546E-05
1.545	2.803E-02	2.215E-03	7.050	3.210E-05	4.228E-05
1.580	2.411E-02	2.060E-03	7.150	2.285E-05	3.751E-05
1.620	2.025E-02	1.833E-03	7.250	3.872E-05	3.970E-05
1.660	1.929E-02	1.755E-03	7.350	4.483E-05	4.261E-05
1.700	1.575E-02	1.926E-03	7.450	1.708E-05	3.454E-05
1.740	1.783E-02	2.210E-03	7.550	-8.004E-06	3.723E-05
1.780	1.821E-02	2.363E-03	7.650	-2.896E-06	3.338E-05
1.820	1.564E-02	2.128E-03	7.750	1.195E-05	3.084E-05
1.860	1.512E-02	1.676E-03	7.850	1.101E-05	2.960E-05
1.900	1.528E-02	1.587E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 115 SEC AFTER END OF IRRADIATION
COUNT FOR 40 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.736E-02	9.693E-03	1.940	2.309E-02	1.965E-03
0.065	1.085E-01	1.128E-02	1.980	2.185E-02	1.885E-03
0.075	1.337E-01	1.231E-02	2.020	2.176E-02	1.867E-03
0.085	1.298E-01	1.184E-02	2.060	1.874E-02	1.760E-03
0.095	1.856E-01	1.185E-02	2.100	1.765E-02	1.774E-03
0.105	1.244E-01	1.048E-02	2.140	1.661E-02	1.730E-03
0.115	1.040E-01	1.014E-02	2.180	1.408E-02	1.644E-03
0.125	1.027E-01	1.050E-02	2.220	1.699E-02	1.700E-03
0.135	1.067E-01	1.084E-02	2.260	1.795E-02	1.753E-03
0.145	1.131E-01	1.042E-02	2.300	1.618E-02	1.599E-03
0.155	8.019E-02	1.026E-02	2.340	1.748E-02	1.594E-03
0.165	1.108E-01	1.074E-02	2.380	1.465E-02	1.561E-03
0.177	1.436E-01	1.011E-02	2.425	1.164E-02	1.417E-03
0.192	1.544E-01	9.741E-03	2.475	1.389E-02	1.540E-03
0.207	1.783E-01	9.591E-03	2.525	1.556E-02	1.558E-03
0.222	1.544E-01	9.479E-03	2.575	1.721E-02	1.480E-03
0.237	1.048E-01	8.528E-03	2.625	1.408E-02	1.376E-03
0.252	9.713E-02	8.114E-03	2.675	9.774E-03	1.447E-03
0.267	1.109E-01	8.274E-03	2.725	1.181E-02	1.600E-03
0.282	1.171E-01	8.559E-03	2.775	1.457E-02	1.569E-03
0.297	1.357E-01	8.898E-03	2.825	1.071E-02	1.296E-03
0.313	1.205E-01	8.426E-03	2.875	9.006E-03	1.150E-03
0.327	1.047E-01	8.189E-03	2.925	7.442E-03	1.057E-03
0.342	1.079E-01	8.029E-03	2.975	5.182E-03	1.023E-03
0.357	1.112E-01	8.015E-03	3.030	3.869E-03	9.300E-04
0.372	1.344E-01	8.228E-03	3.090	3.003E-03	9.165E-04
0.387	1.640E-01	8.852E-03	3.150	3.074E-03	8.551E-04
0.402	1.808E-01	8.893E-03	3.210	5.291E-03	9.226E-04
0.417	1.582E-01	8.150E-03	3.270	6.259E-03	9.528E-04
0.432	1.225E-01	7.184E-03	3.330	7.001E-03	9.511E-04
0.447	1.044E-01	6.828E-03	3.390	6.673E-03	9.247E-04
0.462	8.928E-02	6.584E-03	3.450	5.551E-03	8.569E-04
0.477	7.257E-02	6.300E-03	3.510	7.751E-03	9.603E-04
0.492	6.038E-02	6.242E-03	3.570	8.966E-03	1.014E-03
0.507	6.194E-02	6.262E-03	3.63	6.949E-03	9.450E-04
0.522	7.564E-02	6.613E-03	3.690	3.911E-03	8.165E-04
0.540	8.954E-02	6.663E-03	3.750	2.894E-03	7.234E-04
0.560	1.019E-01	6.734E-03	3.810	3.037E-03	7.627E-04
0.580	1.314E-01	5.068E-03	3.870	2.250E-03	6.919E-04
0.600	1.603E-01	5.535E-03	3.935	1.923E-03	6.727E-04
0.620	1.276E-01	4.985E-03	4.005	2.958E-03	6.258E-04
0.640	8.096E-02	4.225E-03	4.075	4.180E-03	6.976E-04
0.660	6.506E-02	4.000E-03	4.145	4.773E-03	7.236E-04
0.680	7.949E-02	4.111E-03	4.215	3.344E-03	6.227E-04
0.700	1.067E-01	4.419E-03	4.285	2.867E-03	5.734E-04
0.720	1.224E-01	4.770E-03	4.355	3.627E-03	6.049E-04
0.740	1.020E-01	4.354E-03	4.425	2.207E-03	4.812E-04
0.760	7.815E-02	3.956E-03	4.495	7.403E-04	4.131E-04
0.780	7.163E-02	3.797E-03	4.565	7.818E-04	3.734E-04
0.800	8.325E-02	3.966E-03	4.635	1.265E-03	3.956E-04
0.820	1.017E-01	4.404E-03	4.705	1.288E-03	3.910E-04
0.840	1.107E-01	4.504E-03	4.775	9.139E-04	3.534E-04
0.860	1.004E-01	4.312E-03	4.845	4.425E-04	3.596E-04
0.880	8.698E-02	4.036E-03	4.915	3.179E-04	3.185E-04
0.900	8.373E-02	3.913E-03	4.985	6.616E-04	3.624E-04
0.920	8.057E-02	3.794E-03	5.060	9.063E-04	3.145E-04
0.940	8.536E-02	3.896E-03	5.140	6.716E-04	2.960E-04
0.962	9.381E-02	3.903E-03	5.220	3.946E-04	2.378E-04
0.987	8.330E-02	3.719E-03	5.300	7.883E-04	2.523E-04
1.013	6.823E-02	3.368E-03	5.380	1.075E-03	2.504E-04
1.037	5.231E-02	3.274E-03	5.460	6.935E-04	2.023E-04
1.062	5.557E-02	3.245E-03	5.540	3.227E-04	1.940E-04
1.088	6.633E-02	3.516E-03	5.620	2.674E-04	1.596E-04
1.112	6.371E-02	3.390E-03	5.700	1.850E-04	1.480E-04
1.138	5.579E-02	3.208E-03	5.780	1.199E-05	1.008E-04
1.162	4.835E-02	3.069E-03	5.860	-6.229E-05	7.548E-05
1.187	4.723E-02	3.098E-03	5.945	-4.988E-05	7.723E-05
1.215	5.119E-02	3.023E-03	6.035	1.451E-05	9.202E-05
1.245	5.668E-02	3.203E-03	6.125	6.943E-05	9.979E-05
1.275	6.444E-02	3.564E-03	6.215	7.480E-05	9.772E-05
1.305	7.250E-02	3.763E-03	6.305	5.880E-05	5.675E-05
1.335	6.693E-02	3.707E-03	6.395	9.079E-05	5.456E-05
1.365	6.120E-02	3.538E-03	6.485	1.207E-04	7.301E-05
1.395	8.181E-02	3.678E-03	6.575	7.609E-05	6.037E-05
1.425	9.485E-02	3.755E-03	6.665	2.747E-05	6.926E-05
1.455	7.119E-02	3.182E-03	6.755	-3.653E-05	6.184E-05
1.485	4.658E-02	2.611E-03	6.850	-4.222E-05	4.890E-05
1.515	3.680E-02	2.396E-03	6.950	7.469E-06	3.394E-05
1.545	3.869E-02	2.619E-03	7.050	3.328E-05	3.849E-05
1.580	3.607E-02	2.519E-03	7.150	3.120E-05	3.521E-05
1.620	2.740E-02	2.234E-03	7.250	3.283E-05	3.633E-05
1.660	2.919E-02	2.208E-03	7.350	3.468E-05	3.508E-05
1.700	2.515E-02	2.303E-03	7.450	2.583E-05	3.120E-05
1.740	2.710E-02	2.872E-03	7.550	1.301E-05	3.436E-05
1.780	2.676E-02	3.039E-03	7.650	3.626E-06	3.210E-05
1.820	2.620E-02	2.660E-03	7.750	5.400E-06	2.993E-05
1.860	1.923E-02	2.050E-03	7.850	1.104E-05	2.702E-05
1.900	1.882E-02	1.800E-03			

HC0021 STOP 0

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 155 SEC AFTER END OF IRRADIATION
COUNT FOR 60 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.095E-02	5.508E-03	1.940	1.917E-02	1.860E-03
0.065	1.120E-01	6.702E-03	1.980	2.468E-02	1.960E-03
0.075	1.336E-01	7.002E-03	2.020	1.980E-02	1.863E-03
0.085	1.104E-01	6.724E-03	2.060	1.883E-02	1.790E-03
0.095	1.679E-01	6.918E-03	2.100	1.890E-02	1.797E-03
0.105	1.385E-01	6.392E-03	2.140	1.580E-02	1.758E-03
0.115	9.501E-02	5.998E-03	2.180	1.604E-02	1.833E-03
0.125	1.038E-01	6.145E-03	2.220	1.735E-02	1.746E-03
0.135	1.120E-01	6.196E-03	2.260	1.455E-02	1.709E-03
0.145	1.125E-01	6.305E-03	2.300	1.436E-02	1.673E-03
0.155	1.023E-01	6.220E-03	2.340	1.590E-02	1.570E-03
0.165	1.077E-01	6.477E-03	2.380	1.380E-02	1.570E-03
0.177	1.504E-01	6.022E-03	2.425	1.155E-02	1.538E-03
0.192	1.912E-01	6.111E-03	2.475	9.834E-03	1.468E-03
0.207	1.682E-01	5.738E-03	2.525	1.365E-02	1.528E-03
0.222	1.497E-01	5.547E-03	2.575	1.464E-02	1.520E-03
0.237	1.066E-01	5.196E-03	2.625	1.262E-02	1.459E-03
0.252	1.063E-01	4.964E-03	2.675	1.317E-02	1.640E-03
0.267	1.262E-01	5.107E-03	2.725	1.187E-02	1.770E-03
0.282	1.283E-01	5.186E-03	2.775	9.347E-03	1.743E-03
0.297	1.569E-01	5.475E-03	2.825	9.731E-03	1.390E-03
0.313	1.509E-01	5.387E-03	2.875	8.149E-03	1.115E-03
0.327	1.174E-01	5.099E-03	2.925	5.730E-03	1.008E-03
0.342	1.118E-01	4.988E-03	2.975	4.473E-03	1.008E-03
0.357	1.188E-01	5.002E-03	3.030	3.389E-03	9.335E-04
0.372	1.178E-01	4.998E-03	3.090	4.382E-03	9.498E-04
0.387	1.362E-01	5.249E-03	3.150	4.258E-03	8.858E-04
0.402	1.617E-01	5.429E-03	3.210	4.593E-03	8.887E-04
0.417	1.462E-01	5.295E-03	3.270	5.092E-03	8.642E-04
0.432	1.305E-01	4.956E-03	3.330	7.159E-03	9.356E-04
0.447	1.257E-01	4.116E-03	3.390	6.361E-03	9.024E-04
0.462	1.122E-01	3.823E-03	3.450	5.273E-03	8.172E-04
0.477	8.460E-02	3.521E-03	3.510	7.392E-03	9.213E-04
0.492	6.698E-02	3.354E-03	3.570	7.020E-03	9.404E-04
0.507	6.428E-02	3.378E-03	3.630	4.311E-03	8.168E-04
0.522	7.139E-02	3.655E-03	3.690	3.532E-03	7.293E-04
0.540	8.439E-02	3.659E-03	3.750	2.758E-03	6.856E-04
0.560	9.716E-02	3.642E-03	3.810	1.923E-03	6.413E-04
0.580	1.270E-01	4.024E-03	3.870	2.469E-03	6.829E-04
0.600	1.541E-01	4.286E-03	3.935	1.997E-03	6.021E-04
0.620	1.310E-01	4.027E-03	4.005	2.030E-03	6.097E-04
0.640	8.808E-02	3.460E-03	4.075	4.830E-03	6.951E-04
0.660	6.921E-02	3.206E-03	4.145	4.213E-03	6.858E-04
0.680	8.139E-02	3.341E-03	4.215	2.085E-03	5.729E-04
0.700	1.189E-01	3.695E-03	4.285	2.835E-03	6.118E-04
0.720	1.371E-01	3.947E-03	4.355	3.213E-03	5.841E-04
0.740	1.214E-01	3.738E-03	4.425	2.162E-03	5.055E-04
0.760	9.492E-02	3.295E-03	4.495	7.296E-04	4.134E-04
0.780	8.449E-02	3.157E-03	4.565	1.218E-03	3.809E-04
0.800	9.210E-02	3.262E-03	4.635	1.852E-03	4.595E-04
0.820	1.130E-01	3.530E-03	4.705	6.283E-04	3.849E-04
0.840	1.299E-01	3.772E-03	4.775	4.259E-04	3.912E-04
0.860	1.235E-01	3.712E-03	4.845	8.505E-04	3.852E-04
0.880	1.081E-01	3.431E-03	4.915	6.831E-04	3.402E-04
0.900	1.029E-01	3.211E-03	4.985	5.397E-04	3.439E-04
0.920	9.994E-02	3.175E-03	5.060	3.760E-04	3.039E-04
0.940	9.734E-02	3.143E-03	5.140	8.400E-04	3.045E-04
0.962	1.041E-01	3.182E-03	5.220	1.427E-03	3.189E-04
0.987	1.001E-01	3.116E-03	5.300	8.748E-04	2.724E-04
1.013	8.169E-02	2.857E-03	5.380	5.345E-04	2.591E-04
1.037	6.909E-02	2.656E-03	5.460	6.332E-04	2.259E-04
1.062	6.206E-02	2.649E-03	5.540	3.214E-04	2.159E-04
1.088	6.647E-02	2.741E-03	5.620	5.241E-05	1.741E-04
1.112	6.699E-02	2.722E-03	5.700	3.799E-05	1.561E-04
1.138	6.280E-02	2.567E-03	5.780	-2.399E-05	1.397E-04
1.162	5.914E-02	2.511E-03	5.860	-5.805E-05	1.178E-04
1.187	5.394E-02	2.499E-03	5.945	1.298E-04	1.130E-04
1.215	5.344E-02	2.429E-03	6.035	3.827E-04	1.389E-04
1.245	5.319E-02	2.523E-03	6.125	3.384E-04	1.133E-04
1.275	5.720E-02	2.719E-03	6.215	1.740E-04	8.855E-05
1.305	7.067E-02	2.985E-03	6.305	8.327E-05	8.522E-05
1.335	6.490E-02	2.985E-03	6.395	-9.327E-06	8.396E-05
1.365	5.906E-02	2.787E-03	6.485	-2.971E-05	7.611E-05
1.395	8.032E-02	2.840E-03	6.575	1.660E-05	1.066E-04
1.425	9.389E-02	2.939E-03	6.665	-1.406E-05	6.543E-05
1.455	7.685E-02	2.571E-03	6.755	-1.663E-05	3.950E-05
1.485	4.448E-02	2.127E-03	6.850	8.563E-05	8.344E-05
1.515	3.489E-02	2.473E-03	6.950	1.406E-04	7.743E-05
1.545	4.055E-02	2.668E-03	7.050	8.108E-05	7.891E-05
1.580	3.353E-02	2.484E-03	7.150	-2.908E-05	6.529E-05
1.620	2.649E-02	2.301E-03	7.250	-7.006E-05	3.406E-05
1.660	2.872E-02	2.243E-03	7.350	-2.251E-05	7.775E-05
1.700	2.598E-02	2.386E-03	7.450	-2.542E-07	4.969E-05
1.740	2.912E-02	2.982E-03	7.550	-6.619E-06	3.942E-05
1.780	2.952E-02	3.365E-03	7.650	-5.071E-06	3.512E-05
1.820	2.523E-02	2.754E-03	7.750	6.799E-07	3.247E-05
1.860	2.326E-02	2.162E-03	7.850	4.221E-06	2.841E-05
1.900	1.652E-02	1.784E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 215 SEC AFTER END OF IRRADIATION
COUNT FOR 80 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	2.448E-02	5.382E-03	1.940	1.760E-02	1.787E-03
0.065	9.929E-02	6.591E-03	1.980	2.192E-02	1.833E-03
0.075	1.256E-01	6.973E-03	2.020	2.163E-02	1.838E-03
0.085	1.039E-01	6.530E-03	2.060	1.460E-02	1.717E-03
0.095	1.280E-01	6.433E-03	2.100	1.445E-02	1.752E-03
0.105	1.134E-01	6.199E-03	2.140	1.813E-02	1.802E-03
0.115	8.140E-02	5.794E-03	2.180	1.554E-02	1.754E-03
0.125	7.367E-02	5.872E-03	2.220	1.404E-02	1.703E-03
0.135	1.002E-01	6.160E-03	2.260	1.317E-02	1.735E-03
0.145	1.080E-01	6.339E-03	2.300	1.133E-02	1.568E-03
0.155	1.001E-01	6.281E-03	2.340	1.254E-02	1.511E-03
0.165	1.098E-01	6.387E-03	2.380	1.243E-02	1.551E-03
0.177	1.502E-01	6.165E-03	2.425	1.035E-02	1.435E-03
0.192	1.986E-01	6.082E-03	2.475	8.845E-03	1.363E-03
0.207	1.460E-01	5.600E-03	2.525	1.089E-02	1.380E-03
0.222	1.258E-01	5.361E-03	2.575	1.378E-02	1.408E-03
0.237	9.891E-02	5.058E-03	2.625	1.214E-02	1.483E-03
0.252	1.138E-01	5.012E-03	2.675	1.287E-02	1.669E-03
0.267	1.299E-01	5.116E-03	2.725	1.319E-02	1.416E-03
0.282	1.270E-01	5.157E-03	2.775	7.504E-03	1.705E-03
0.297	1.508E-01	5.404E-03	2.825	5.934E-03	1.347E-03
0.313	1.642E-01	5.518E-03	2.875	4.911E-03	1.021E-03
0.327	1.282E-01	5.177E-03	2.925	4.110E-03	9.426E-04
0.342	1.137E-01	5.114E-03	2.975	3.707E-03	9.139E-04
0.357	1.106E-01	5.027E-03	3.030	2.998E-03	8.438E-04
0.372	9.320E-02	4.748E-03	3.090	3.738E-03	8.733E-04
0.387	9.671E-02	4.659E-03	3.150	3.637E-03	8.178E-04
0.402	1.206E-01	4.913E-03	3.210	4.015E-03	8.556E-04
0.417	1.238E-01	4.939E-03	3.270	4.773E-03	8.425E-04
0.432	1.265E-01	4.927E-03	3.330	6.490E-03	8.768E-04
0.447	1.258E-01	4.165E-03	3.390	6.702E-03	8.748E-04
0.462	1.126E-01	3.898E-03	3.450	4.124E-03	8.141E-04
0.477	8.593E-02	3.627E-03	3.510	5.845E-03	8.479E-04
0.492	6.543E-02	3.448E-03	3.570	7.333E-03	9.264E-04
0.507	6.381E-02	3.496E-03	3.630	3.510E-03	7.288E-04
0.522	6.510E-02	3.690E-03	3.690	1.667E-03	6.345E-04
0.540	6.119E-02	3.395E-03	3.750	1.819E-03	6.337E-04
0.560	7.727E-02	3.424E-03	3.810	1.793E-03	5.974E-04
0.580	1.181E-01	3.881E-03	3.870	2.297E-03	6.211E-04
0.600	1.462E-01	4.232E-03	3.935	2.051E-03	5.861E-04
0.620	1.202E-01	4.007E-03	4.005	2.987E-03	6.220E-04
0.640	8.393E-02	3.465E-03	4.075	4.523E-03	6.915E-04
0.660	7.013E-02	3.225E-03	4.145	3.424E-03	5.906E-04
0.680	8.443E-02	3.370E-03	4.215	2.775E-03	5.634E-04
0.700	1.199E-01	3.700E-03	4.285	2.630E-03	4.887E-04
0.720	1.434E-01	3.973E-03	4.355	2.047E-03	5.090E-04
0.740	1.233E-01	3.743E-03	4.425	1.339E-03	4.336E-04
0.760	8.764E-02	3.225E-03	4.495	1.169E-03	4.324E-04
0.780	8.086E-02	3.128E-03	4.565	7.624E-04	3.706E-04
0.800	9.645E-02	3.320E-03	4.635	3.284E-04	3.392E-04
0.820	1.168E-01	3.659E-03	4.705	1.242E-04	3.011E-04
0.840	1.286E-01	3.877E-03	4.775	2.481E-05	3.137E-04
0.860	1.262E-01	3.770E-03	4.845	3.182E-04	2.680E-04
0.880	1.153E-01	3.506E-03	4.915	6.656E-04	2.949E-04
0.900	1.071E-01	3.278E-03	4.985	3.991E-04	2.896E-04
0.920	1.049E-01	3.196E-03	5.060	3.274E-04	3.019E-04
0.940	1.063E-01	3.259E-03	5.140	8.791E-04	2.989E-04
0.962	1.057E-01	3.199E-03	5.220	1.038E-03	2.816E-04
0.987	9.617E-02	3.138E-03	5.300	7.256E-04	2.484E-04
1.013	7.819E-02	2.806E-03	5.380	3.496E-04	1.957E-04
1.037	6.550E-02	2.632E-03	5.460	2.291E-04	2.302E-04
1.062	6.173E-02	2.632E-03	5.540	3.388E-04	2.075E-04
1.088	6.256E-02	2.649E-03	5.620	3.327E-04	1.600E-04
1.112	6.258E-02	2.663E-03	5.700	1.089E-04	1.429E-04
1.138	5.838E-02	2.567E-03	5.780	-3.824E-05	1.326E-04
1.162	5.205E-02	2.421E-03	5.860	-4.042E-05	1.533E-04
1.187	4.888E-02	2.436E-03	5.945	6.222E-05	1.639E-04
1.215	4.962E-02	2.385E-03	6.035	1.827E-04	1.461E-04
1.245	4.988E-02	2.502E-03	6.125	1.270E-04	1.179E-04
1.275	5.474E-02	2.772E-03	6.215	6.540E-05	1.095E-04
1.305	6.101E-02	2.944E-03	6.305	1.369E-04	1.275E-04
1.335	5.555E-02	2.908E-03	6.395	1.651E-04	1.327E-04
1.365	5.299E-02	2.746E-03	6.485	5.746E-05	1.415E-04
1.395	6.386E-02	2.712E-03	6.575	-9.884E-05	8.692E-05
1.425	7.175E-02	2.707E-03	6.665	-1.331E-04	1.012E-04
1.455	5.626E-02	2.324E-03	6.755	-1.031E-04	9.633E-05
1.485	3.511E-02	1.984E-03	6.850	1.632E-05	1.130E-04
1.515	2.900E-02	2.302E-03	6.950	1.907E-05	7.776E-05
1.545	2.917E-02	2.364E-03	7.050	7.070E-07	6.697E-05
1.580	2.530E-02	2.246E-03	7.150	5.477E-05	6.448E-05
1.620	2.198E-02	2.178E-03	7.250	1.014E-04	5.655E-05
1.660	2.490E-02	2.277E-03	7.350	8.919E-05	6.860E-05
1.700	2.481E-02	2.430E-03	7.450	5.568E-05	6.004E-05
1.740	2.239E-02	2.882E-03	7.550	3.664E-05	4.713E-05
1.780	2.798E-02	3.170E-03	7.650	2.287E-05	4.359E-05
1.820	2.362E-02	2.640E-03	7.750	9.027E-06	3.438E-05
1.860	1.613E-02	2.030E-03	7.850	7.171E-06	3.014E-05
1.900	1.526E-02	1.815E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 295 SEC AFTER END OF IRRADIATION
COUNT FOR 100 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.980E-02	5.305E-03	1.940	1.578E-02	1.720E-03
0.065	7.767E-02	6.462E-03	1.980	1.817E-02	1.784E-03
0.075	1.085E-01	6.990E-03	2.020	1.700E-02	1.752E-03
0.085	9.315E-02	6.467E-03	2.060	1.277E-02	1.645E-03
0.095	9.007E-02	5.908E-03	2.100	1.202E-02	1.707E-03
0.105	9.887E-02	6.044E-03	2.140	1.348E-02	1.762E-03
0.115	6.910E-02	5.806E-03	2.180	1.407E-02	1.744E-03
0.125	6.900E-02	5.816E-03	2.220	1.343E-02	1.763E-03
0.135	9.181E-02	6.240E-03	2.260	1.170E-02	1.630E-03
0.145	1.042E-01	6.370E-03	2.300	8.783E-03	1.524E-03
0.155	1.091E-01	6.412E-03	2.340	8.072E-03	1.365E-03
0.165	1.118E-01	6.550E-03	2.380	8.628E-03	1.415E-03
0.177	1.420E-01	6.109E-03	2.425	7.680E-03	1.355E-03
0.192	1.900E-01	6.082E-03	2.475	8.212E-03	1.282E-03
0.207	1.281E-01	5.462E-03	2.525	1.080E-02	1.356E-03
0.222	1.094E-01	5.223E-03	2.575	1.142E-02	1.335E-03
0.237	9.014E-02	4.973E-03	2.625	1.036E-02	1.322E-03
0.252	1.154E-01	5.036E-03	2.675	1.257E-02	1.655E-03
0.267	1.266E-01	5.082E-03	2.725	1.021E-02	1.961E-03
0.282	1.205E-01	5.150E-03	2.775	4.276E-03	1.749E-03
0.297	1.552E-01	5.567E-03	2.825	5.417E-03	1.288E-03
0.313	1.771E-01	5.775E-03	2.875	4.320E-03	9.707E-04
0.327	1.305E-01	5.409E-03	2.925	2.988E-03	8.405E-04
0.342	1.088E-01	4.983E-03	2.975	2.599E-03	7.462E-04
0.357	9.862E-02	4.787E-03	3.030	1.970E-03	8.007E-04
0.372	7.912E-02	4.528E-03	3.090	2.451E-03	7.158E-04
0.387	6.957E-02	4.232E-03	3.150	2.734E-03	7.190E-04
0.402	9.256E-02	4.414E-03	3.210	2.560E-03	6.846E-04
0.417	1.048E-01	4.627E-03	3.270	3.439E-03	7.203E-04
0.432	1.149E-01	4.738E-03	3.330	4.621E-03	7.500E-04
0.447	1.129E-01	4.085E-03	3.390	4.265E-03	7.141E-04
0.462	1.027E-01	3.872E-03	3.450	4.189E-03	7.258E-04
0.477	8.438E-02	3.586E-03	3.510	5.182E-03	7.842E-04
0.492	6.350E-02	3.446E-03	3.570	5.274E-03	8.312E-04
0.507	5.426E-02	3.433E-03	3.630	3.379E-03	6.518E-04
0.522	5.588E-02	3.680E-03	3.690	1.814E-03	5.553E-04
0.540	5.381E-02	3.397E-03	3.750	1.216E-03	5.230E-04
0.560	6.618E-02	3.344E-03	3.810	1.135E-03	5.487E-04
0.580	1.109E-01	3.838E-03	3.870	9.315E-04	5.033E-04
0.600	1.305E-01	4.121E-03	3.935	4.311E-04	4.616E-04
0.620	1.067E-01	3.867E-03	4.005	1.269E-03	5.162E-04
0.640	7.303E-02	3.416E-03	4.075	2.444E-03	5.509E-04
0.660	6.106E-02	3.157E-03	4.145	2.284E-03	5.025E-04
0.680	7.704E-02	3.252E-03	4.215	1.740E-03	4.593E-04
0.700	1.094E-01	3.569E-03	4.285	1.628E-03	4.566E-04
0.720	1.298E-01	3.855E-03	4.355	1.836E-03	4.895E-04
0.740	1.139E-01	3.663E-03	4.425	9.139E-04	4.002E-04
0.760	8.897E-02	3.254E-03	4.495	3.559E-04	3.371E-04
0.780	8.201E-02	3.160E-03	4.565	7.297E-04	3.368E-04
0.800	9.322E-02	3.300E-03	4.635	7.483E-04	3.149E-04
0.820	1.148E-01	3.610E-03	4.705	5.562E-04	3.130E-04
0.840	1.267E-01	3.884E-03	4.775	5.585E-04	2.963E-04
0.860	1.255E-01	3.879E-03	4.845	6.431E-04	2.604E-04
0.880	1.193E-01	3.630E-03	4.915	5.154E-04	2.637E-04
0.900	1.100E-01	3.324E-03	4.985	4.179E-04	2.727E-04
0.920	1.054E-01	3.201E-03	5.060	5.311E-04	2.657E-04
0.940	1.070E-01	3.235E-03	5.140	5.324E-04	2.285E-04
0.962	1.064E-01	3.213E-03	5.220	3.558E-04	2.166E-04
0.987	9.194E-02	3.044E-03	5.300	1.545E-04	2.085E-04
1.013	7.581E-02	2.787E-03	5.380	-1.489E-05	1.574E-04
1.037	6.788E-02	2.622E-03	5.460	4.292E-05	1.339E-04
1.062	5.903E-02	2.600E-03	5.540	1.139E-04	1.026E-04
1.088	5.638E-02	2.639E-03	5.620	4.972E-05	9.402E-05
1.112	5.611E-02	2.671E-03	5.700	-6.954E-06	1.066E-04
1.138	5.011E-02	2.520E-03	5.780	4.192E-05	1.107E-04
1.162	4.559E-02	2.396E-03	5.860	6.889E-05	1.212E-04
1.187	4.507E-02	2.381E-03	5.945	-2.474E-05	1.205E-04
1.215	4.761E-02	2.325E-03	6.035	-1.250E-04	9.935E-05
1.245	4.814E-02	2.497E-03	6.125	-1.119E-04	8.114E-05
1.275	4.707E-02	2.701E-03	6.215	-1.342E-05	1.066E-04
1.305	4.828E-02	2.862E-03	6.305	1.323E-04	9.638E-05
1.335	4.614E-02	2.779E-03	6.395	2.060E-04	1.258E-04
1.365	4.283E-02	2.663E-03	6.485	1.482E-04	1.035E-04
1.395	4.966E-02	2.552E-03	6.575	1.132E-04	9.239E-05
1.425	5.134E-02	2.390E-03	6.665	8.537E-05	7.861E-05
1.455	3.895E-02	2.074E-03	6.755	1.062E-04	1.078E-04
1.485	2.764E-02	1.821E-03	6.850	5.470E-05	8.329E-05
1.515	2.462E-02	2.111E-03	6.950	-2.736E-07	1.079E-04
1.545	2.391E-02	2.270E-03	7.050	-2.048E-05	9.728E-05
1.580	1.843E-02	2.086E-03	7.150	-1.119E-05	5.240E-05
1.620	1.700E-02	2.006E-03	7.250	2.079E-05	6.992E-05
1.660	2.110E-02	2.104E-03	7.350	5.874E-06	5.437E-05
1.700	2.289E-02	2.325E-03	7.450	1.005E-05	7.060E-05
1.740	2.260E-02	2.766E-03	7.550	3.339E-05	6.181E-05
1.780	2.478E-02	2.959E-03	7.650	1.564E-05	4.542E-05
1.820	2.285E-02	2.535E-03	7.750	8.617E-07	3.517E-05
1.860	1.674E-02	1.933E-03	7.850	5.730E-06	2.954E-05
1.900	1.509E-02	1.723E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 395 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	8.567E-03	6.643E-03	1.940	1.874E-02	2.016E-03
0.065	7.564E-02	7.850E-03	1.980	2.307E-02	2.100E-03
0.075	1.536E-01	8.818E-03	2.020	2.425E-02	2.246E-03
0.085	1.399E-01	8.322E-03	2.060	1.980E-02	2.017E-03
0.095	8.727E-02	7.446E-03	2.100	1.620E-02	2.216E-03
0.105	1.189E-01	7.312E-03	2.140	2.143E-02	2.238E-03
0.115	9.225E-02	7.107E-03	2.180	2.401E-02	2.241E-03
0.125	9.409E-02	7.443E-03	2.220	2.000E-02	2.196E-03
0.135	1.258E-01	7.598E-03	2.260	1.592E-02	2.081E-03
0.145	1.459E-01	7.915E-03	2.300	1.117E-02	1.869E-03
0.155	1.498E-01	8.015E-03	2.340	1.004E-02	1.727E-03
0.165	1.566E-01	8.223E-03	2.380	1.042E-02	1.782E-03
0.177	2.339E-01	7.719E-03	2.425	8.520E-03	1.719E-03
0.192	2.981E-01	7.528E-03	2.475	1.076E-02	1.586E-03
0.207	1.822E-01	6.637E-03	2.525	1.445E-02	1.681E-03
0.222	1.413E-01	6.356E-03	2.575	1.232E-02	1.659E-03
0.237	1.352E-01	6.240E-03	2.625	8.092E-03	1.704E-03
0.252	1.596E-01	6.140E-03	2.675	7.154E-03	2.251E-03
0.267	1.759E-01	6.269E-03	2.725	7.799E-03	2.513E-03
0.282	1.891E-01	6.395E-03	2.775	1.008E-02	2.238E-03
0.297	2.422E-01	6.898E-03	2.825	8.646E-03	1.686E-03
0.313	2.930E-01	7.266E-03	2.875	5.731E-03	1.154E-03
0.327	2.079E-01	6.734E-03	2.925	5.611E-03	9.379E-04
0.342	1.605E-01	6.213E-03	2.975	3.838E-03	9.350E-04
0.357	1.465E-01	5.973E-03	3.030	2.535E-03	9.055E-04
0.372	1.130E-01	5.554E-03	3.090	2.868E-03	8.431E-04
0.387	9.515E-02	5.213E-03	3.150	4.496E-03	8.476E-04
0.402	1.180E-01	5.412E-03	3.210	3.726E-03	8.735E-04
0.417	1.424E-01	5.625E-03	3.270	4.290E-03	8.201E-04
0.432	1.515E-01	5.702E-03	3.330	6.378E-03	8.600E-04
0.447	1.640E-01	5.119E-03	3.390	4.031E-03	7.802E-04
0.462	1.559E-01	4.864E-03	3.450	2.638E-03	7.254E-04
0.477	1.240E-01	4.545E-03	3.510	6.161E-03	8.213E-04
0.492	9.047E-02	4.334E-03	3.570	7.069E-03	8.770E-04
0.507	7.778E-02	4.438E-03	3.630	3.937E-03	7.235E-04
0.522	8.133E-02	4.717E-03	3.690	1.887E-03	6.141E-04
0.540	7.814E-02	4.351E-03	3.750	9.877E-04	5.826E-04
0.560	9.705E-02	4.240E-03	3.810	9.538E-04	5.953E-04
0.580	1.623E-01	4.799E-03	3.870	1.465E-03	5.278E-04
0.600	1.798E-01	5.034E-03	3.935	1.338E-03	5.105E-04
0.620	1.438E-01	4.787E-03	4.005	1.403E-03	5.483E-04
0.640	1.154E-01	4.315E-03	4.075	2.058E-03	5.982E-04
0.660	9.945E-02	4.063E-03	4.145	2.142E-03	5.219E-04
0.680	1.064E-01	4.063E-03	4.215	1.465E-03	5.179E-04
0.700	1.445E-01	4.387E-03	4.285	2.158E-03	5.465E-04
0.720	1.698E-01	4.620E-03	4.355	2.794E-03	4.689E-04
0.740	1.489E-01	4.380E-03	4.425	1.391E-03	3.899E-04
0.760	1.226E-01	4.024E-03	4.495	2.516E-04	3.707E-04
0.780	1.227E-01	3.971E-03	4.565	4.956E-04	3.423E-04
0.800	1.424E-01	4.203E-03	4.635	6.829E-04	3.067E-04
0.820	1.670E-01	4.576E-03	4.705	3.566E-04	3.144E-04
0.840	1.867E-01	4.942E-03	4.775	1.146E-04	2.935E-04
0.860	1.935E-01	4.983E-03	4.845	3.278E-04	2.904E-04
0.880	1.889E-01	4.593E-03	4.915	6.274E-04	2.521E-04
0.900	1.836E-01	4.332E-03	4.985	5.745E-04	2.326E-04
0.920	1.753E-01	4.119E-03	5.060	2.592E-04	2.668E-04
0.940	1.651E-01	4.090E-03	5.140	7.389E-05	2.526E-04
0.962	1.523E-01	3.915E-03	5.220	8.210E-05	2.281E-04
0.987	1.278E-01	3.729E-03	5.300	2.024E-04	2.219E-04
1.013	1.145E-01	3.482E-03	5.380	3.169E-04	1.570E-04
1.037	1.014E-01	3.324E-03	5.460	3.095E-04	1.099E-04
1.062	8.182E-02	3.182E-03	5.540	2.387E-04	1.369E-04
1.088	7.448E-02	3.247E-03	5.620	1.454E-04	1.547E-04
1.112	7.573E-02	3.223E-03	5.700	1.817E-04	1.471E-04
1.138	6.874E-02	3.140E-03	5.780	8.340E-05	1.205E-04
1.162	6.214E-02	3.031E-03	5.860	-1.604E-04	1.403E-04
1.187	6.498E-02	2.961E-03	5.945	-3.866E-04	5.444E-05
1.215	7.061E-02	2.927E-03	6.035	-1.690E-04	1.087E-04
1.245	6.819E-02	3.160E-03	6.125	-4.698E-05	9.281E-05
1.275	6.432E-02	3.545E-03	6.215	-3.716E-05	9.884E-05
1.305	6.433E-02	3.646E-03	6.305	1.005E-04	1.224E-04
1.335	5.553E-02	3.552E-03	6.395	1.999E-04	1.040E-04
1.365	5.543E-02	3.416E-03	6.485	1.143E-04	7.751E-05
1.395	6.223E-02	3.087E-03	6.575	9.768E-06	9.469E-05
1.425	5.503E-02	2.736E-03	6.665	6.634E-06	8.206E-05
1.455	4.358E-02	2.349E-03	6.755	4.486E-05	9.567E-05
1.485	3.434E-02	2.088E-03	6.850	7.360E-05	9.725E-05
1.515	3.197E-02	2.628E-03	6.950	4.969E-06	8.948E-05
1.545	2.956E-02	2.737E-03	7.050	-1.396E-04	4.935E-05
1.580	2.488E-02	2.611E-03	7.150	-1.117E-04	9.797E-05
1.620	2.693E-02	2.482E-03	7.250	-3.676E-05	9.146E-05
1.660	3.126E-02	2.637E-03	7.350	3.806E-05	6.467E-05
1.700	3.370E-02	2.954E-03	7.450	5.718E-05	7.417E-05
1.740	3.390E-02	3.352E-03	7.550	5.213E-05	4.265E-05
1.780	3.312E-02	3.463E-03	7.650	2.763E-05	3.570E-05
1.820	2.517E-02	2.973E-03	7.750	1.399E-05	3.518E-05
1.860	2.095E-02	2.420E-03	7.850	6.238E-06	2.870E-05
1.900	1.981E-02	2.129E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
10-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 595 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	2.558E-03	6.352E-03	1.940	1.463E-02	1.826E-03
0.065	4.081E-02	7.535E-03	1.980	1.647E-02	1.854E-03
0.075	1.080E-01	8.734E-03	2.020	1.784E-02	1.881E-03
0.085	1.186E-01	7.969E-03	2.060	1.303E-02	1.908E-03
0.095	6.202E-02	6.964E-03	2.100	1.111E-02	2.000E-03
0.105	6.397E-02	6.918E-03	2.140	1.667E-02	2.060E-03
0.115	6.578E-02	6.891E-03	2.180	1.807E-02	2.032E-03
0.125	6.538E-02	7.167E-03	2.220	1.417E-02	1.984E-03
0.135	8.257E-02	7.378E-03	2.260	1.195E-02	1.884E-03
0.145	1.080E-01	7.475E-03	2.300	1.035E-02	1.640E-03
0.155	1.249E-01	7.646E-03	2.340	7.843E-03	1.510E-03
0.165	1.188E-01	7.760E-03	2.380	6.883E-03	1.573E-03
0.177	1.920E-01	7.305E-03	2.425	7.131E-03	1.487E-03
0.192	2.324E-01	7.160E-03	2.475	6.890E-03	1.485E-03
0.207	1.444E-01	6.157E-03	2.525	6.708E-03	1.494E-03
0.222	9.846E-02	5.840E-03	2.575	8.058E-03	1.527E-03
0.237	9.165E-02	5.840E-03	2.625	5.499E-03	1.638E-03
0.252	1.245E-01	5.811E-03	2.675	7.069E-03	2.037E-03
0.267	1.477E-01	5.893E-03	2.725	6.329E-03	2.368E-03
0.282	1.303E-01	5.969E-03	2.775	8.381E-03	2.048E-03
0.297	1.858E-01	6.574E-03	2.825	7.685E-03	1.453E-03
0.313	2.442E-01	6.961E-03	2.875	2.095E-03	9.005E-04
0.327	1.831E-01	6.397E-03	2.925	1.554E-03	7.455E-04
0.342	1.162E-01	5.923E-03	2.975	1.944E-03	7.040E-04
0.357	1.011E-01	5.591E-03	3.030	1.654E-03	7.060E-04
0.372	7.851E-02	5.048E-03	3.090	1.716E-03	7.237E-04
0.387	6.214E-02	4.557E-03	3.150	1.833E-03	6.528E-04
0.402	8.843E-02	4.697E-03	3.210	2.027E-03	6.677E-04
0.417	1.022E-01	4.932E-03	3.270	3.175E-03	6.447E-04
0.432	1.094E-01	5.075E-03	3.330	3.112E-03	7.210E-04
0.447	1.167E-01	4.462E-03	3.390	2.677E-03	6.296E-04
0.462	1.096E-01	4.349E-03	3.450	2.471E-03	6.254E-04
0.477	8.766E-02	4.186E-03	3.510	3.569E-03	6.774E-04
0.492	6.520E-02	4.024E-03	3.570	4.342E-03	7.300E-04
0.507	5.951E-02	4.097E-03	3.630	3.274E-03	5.968E-04
0.522	5.731E-02	4.317E-03	3.690	1.780E-03	4.581E-04
0.540	5.479E-02	3.944E-03	3.750	4.917E-04	4.041E-04
0.560	7.960E-02	3.855E-03	3.810	3.000E-04	3.760E-04
0.580	1.178E-01	4.354E-03	3.870	9.521E-04	4.031E-04
0.600	1.317E-01	4.603E-03	3.935	9.533E-04	4.038E-04
0.620	1.039E-01	4.276E-03	4.005	6.330E-04	3.913E-04
0.640	8.105E-02	3.874E-03	4.075	7.862E-04	4.356E-04
0.660	7.281E-02	3.644E-03	4.145	7.095E-04	3.992E-04
0.680	6.973E-02	3.567E-03	4.215	7.364E-04	3.751E-04
0.700	8.857E-02	3.637E-03	4.285	6.295E-04	3.876E-04
0.720	1.055E-01	3.797E-03	4.355	4.986E-04	2.946E-04
0.740	9.746E-02	3.722E-03	4.425	6.656E-04	3.068E-04
0.760	8.717E-02	3.538E-03	4.495	8.938E-04	2.926E-04
0.780	8.840E-02	3.603E-03	4.565	9.455E-04	3.099E-04
0.800	9.659E-02	3.724E-03	4.635	7.109E-04	3.172E-04
0.820	1.159E-01	4.097E-03	4.705	3.487E-04	2.728E-04
0.840	1.354E-01	4.482E-03	4.775	1.721E-04	2.366E-04
0.860	1.430E-01	4.521E-03	4.845	8.817E-05	1.823E-04
0.880	1.456E-01	4.157E-03	4.915	-8.176E-05	1.774E-04
0.900	1.470E-01	3.930E-03	4.985	-2.360E-04	1.302E-04
0.920	1.422E-01	3.746E-03	5.060	-9.706E-05	1.959E-04
0.940	1.286E-01	3.709E-03	5.140	1.153E-04	1.953E-04
0.962	1.088E-01	3.421E-03	5.220	2.189E-04	1.645E-04
0.987	9.332E-02	3.179E-03	5.300	8.768E-05	8.573E-05
1.013	8.623E-02	3.104E-03	5.380	-2.870E-05	7.640E-05
1.037	7.908E-02	3.070E-03	5.460	-1.996E-05	9.279E-05
1.062	6.208E-02	2.877E-03	5.540	6.583E-05	1.550E-04
1.088	4.945E-02	2.949E-03	5.620	9.122E-05	1.569E-04
1.112	4.865E-02	3.000E-03	5.700	-9.361E-06	1.355E-04
1.138	4.814E-02	2.936E-03	5.780	-1.321E-04	9.557E-05
1.162	4.794E-02	2.768E-03	5.860	-1.925E-04	1.073E-04
1.187	4.956E-02	2.694E-03	5.945	-1.963E-04	8.321E-05
1.215	5.239E-02	2.685E-03	6.035	-1.026E-05	1.430E-04
1.245	5.504E-02	2.927E-03	6.125	6.386E-05	1.355E-04
1.275	4.487E-02	3.293E-03	6.215	1.111E-05	1.149E-04
1.305	4.098E-02	3.348E-03	6.305	-1.769E-05	9.303E-05
1.335	3.958E-02	3.316E-03	6.395	8.635E-05	1.019E-04
1.365	3.871E-02	3.238E-03	6.485	2.209E-04	9.099E-05
1.395	4.257E-02	2.861E-03	6.575	2.290E-04	9.755E-05
1.425	3.769E-02	2.345E-03	6.665	7.670E-05	8.203E-05
1.455	2.638E-02	2.069E-03	6.755	9.755E-06	9.088E-05
1.485	2.358E-02	1.871E-03	6.850	5.683E-05	9.941E-05
1.515	2.298E-02	2.282E-03	6.950	9.937E-05	1.038E-04
1.545	2.084E-02	2.272E-03	7.050	-3.731E-05	7.291E-05
1.580	1.853E-02	2.238E-03	7.150	-1.331E-04	8.015E-05
1.620	1.666E-02	2.300E-03	7.250	-8.866E-05	8.651E-05
1.660	2.421E-02	2.351E-03	7.350	-1.467E-05	9.753E-05
1.700	2.343E-02	2.387E-03	7.450	-2.231E-05	9.082E-05
1.740	1.877E-02	2.642E-03	7.550	-2.825E-05	7.945E-05
1.780	1.967E-02	2.766E-03	7.650	-2.795E-05	6.681E-05
1.820	1.879E-02	2.448E-03	7.750	-3.310E-06	3.542E-05
1.860	1.561E-02	2.052E-03	7.850	7.574E-06	2.827E-05
1.900	1.398E-02	1.954E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 70 SEC AFTER END OF IRRADIATION
COUNT FOR 40 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.259E-02	2.094E-03	1.940	2.491E-02	9.565E-04
0.065	1.023E-01	2.556E-03	1.980	2.378E-02	9.164E-04
0.075	1.424E-01	2.693E-03	2.020	2.239E-02	8.993E-04
0.085	1.184E-01	2.506E-03	2.060	1.994E-02	8.315E-04
0.095	1.787E-01	2.725E-03	2.100	1.710E-02	7.882E-04
0.105	1.421E-01	2.511E-03	2.140	1.709E-02	7.909E-04
0.115	8.256E-02	2.157E-03	2.180	1.711E-02	7.871E-04
0.125	9.083E-02	2.191E-03	2.220	1.669E-02	7.839E-04
0.135	9.975E-02	2.273E-03	2.260	1.767E-02	7.850E-04
0.145	9.842E-02	2.264E-03	2.300	1.764E-02	7.625E-04
0.155	8.861E-02	2.270E-03	2.340	1.685E-02	7.524E-04
0.165	9.192E-02	2.335E-03	2.380	1.581E-02	7.946E-04
0.177	1.146E-01	2.202E-03	2.425	1.269E-02	7.096E-04
0.192	1.622E-01	2.276E-03	2.475	1.212E-02	7.289E-04
0.207	1.455E-01	2.180E-03	2.525	1.523E-02	7.476E-04
0.222	1.218E-01	2.073E-03	2.575	1.693E-02	7.374E-04
0.237	8.746E-02	1.896E-03	2.625	1.496E-02	7.091E-04
0.252	8.185E-02	1.791E-03	2.675	1.327E-02	6.808E-04
0.267	9.767E-02	1.856E-03	2.725	1.366E-02	6.904E-04
0.282	1.055E-01	1.888E-03	2.775	1.224E-02	6.567E-04
0.297	1.184E-01	1.950E-03	2.825	9.946E-03	6.014E-04
0.313	1.146E-01	1.924E-03	2.875	8.642E-03	5.600E-04
0.327	9.859E-02	1.847E-03	2.925	7.990E-03	5.490E-04
0.342	1.045E-01	1.892E-03	2.975	6.910E-03	5.323E-04
0.357	1.103E-01	1.924E-03	3.030	5.543E-03	4.985E-04
0.372	1.188E-01	2.009E-03	3.090	4.874E-03	4.874E-04
0.387	1.550E-01	2.228E-03	3.150	4.432E-03	4.624E-04
0.402	1.881E-01	2.458E-03	3.210	5.087E-03	4.579E-04
0.417	1.628E-01	2.351E-03	3.270	6.500E-03	4.853E-04
0.432	1.196E-01	2.057E-03	3.330	7.023E-03	4.939E-04
0.447	1.002E-01	1.893E-03	3.390	6.106E-03	4.612E-04
0.462	8.719E-02	1.565E-03	3.450	5.607E-03	4.496E-04
0.477	6.829E-02	1.465E-03	3.510	6.239E-03	4.577E-04
0.492	5.513E-02	1.393E-03	3.570	8.971E-03	5.287E-04
0.507	5.835E-02	1.453E-03	3.630	7.921E-03	5.091E-04
0.522	7.516E-02	1.640E-03	3.690	4.492E-03	4.198E-04
0.540	9.508E-02	1.671E-03	3.750	3.252E-03	3.760E-04
0.560	1.059E-01	1.634E-03	3.810	3.387E-03	3.814E-04
0.580	1.265E-01	1.710E-03	3.870	2.629E-03	3.408E-04
0.600	1.523E-01	1.806E-03	3.935	3.106E-03	3.302E-04
0.620	1.236E-01	1.694E-03	4.005	4.199E-03	3.446E-04
0.640	8.391E-02	1.462E-03	4.075	4.750E-03	3.555E-04
0.660	6.776E-02	1.354E-03	4.145	4.466E-03	3.432E-04
0.680	8.201E-02	1.469E-03	4.215	3.204E-03	2.931E-04
0.700	1.095E-01	1.624E-03	4.285	3.428E-03	2.955E-04
0.720	1.240E-01	1.721E-03	4.355	3.145E-03	2.624E-04
0.740	1.088E-01	1.616E-03	4.425	1.747E-03	2.328E-04
0.760	8.072E-02	1.422E-03	4.495	9.715E-04	2.056E-04
0.780	7.375E-02	1.325E-03	4.565	1.280E-03	2.087E-04
0.800	8.627E-02	1.390E-03	4.635	1.525E-03	2.209E-04
0.820	1.028E-01	1.478E-03	4.705	9.423E-04	1.921E-04
0.840	1.094E-01	1.544E-03	4.775	6.610E-04	1.798E-04
0.860	1.018E-01	1.498E-03	4.845	7.659E-04	1.851E-04
0.880	9.292E-02	1.380E-03	4.915	7.518E-04	1.828E-04
0.900	8.571E-02	1.315E-03	4.985	7.178E-04	1.746E-04
0.920	8.185E-02	1.288E-03	5.060	8.697E-04	1.639E-04
0.940	8.415E-02	1.354E-03	5.140	1.012E-03	1.615E-04
0.962	8.903E-02	1.381E-03	5.220	7.973E-04	1.455E-04
0.987	8.677E-02	1.367E-03	5.300	7.541E-04	1.365E-04
1.013	6.895E-02	1.201E-03	5.380	1.039E-03	1.439E-04
1.037	5.695E-02	1.121E-03	5.460	9.761E-04	1.266E-04
1.062	5.586E-02	1.124E-03	5.540	4.595E-04	1.002E-04
1.088	6.024E-02	1.148E-03	5.620	1.608E-04	7.587E-05
1.112	6.400E-02	1.127E-03	5.700	1.667E-04	8.273E-05
1.138	5.851E-02	1.079E-03	5.780	1.455E-04	6.204E-05
1.162	5.068E-02	1.029E-03	5.860	6.639E-05	5.745E-05
1.187	5.040E-02	1.068E-03	5.945	2.977E-05	4.748E-05
1.215	5.121E-02	1.037E-03	6.035	8.091E-05	5.116E-05
1.245	5.332E-02	1.106E-03	6.125	1.430E-04	5.301E-05
1.275	6.195E-02	1.192E-03	6.215	1.259E-04	4.953E-05
1.305	7.347E-02	1.253E-03	6.305	5.890E-05	3.851E-05
1.335	6.838E-02	1.167E-03	6.395	3.850E-05	3.047E-05
1.365	6.358E-02	1.121E-03	6.485	5.175E-05	2.920E-05
1.395	8.340E-02	1.223E-03	6.575	4.928E-05	2.776E-05
1.425	9.566E-02	1.333E-03	6.665	4.384E-05	2.501E-05
1.455	7.608E-02	1.164E-03	6.755	3.854E-05	2.512E-05
1.485	4.739E-02	9.276E-04	6.850	1.573E-05	1.819E-05
1.515	3.722E-02	1.163E-03	6.950	1.356E-06	1.272E-05
1.545	3.782E-02	1.238E-03	7.050	1.383E-06	7.792E-06
1.580	3.367E-02	1.201E-03	7.150	6.620E-06	8.394E-06
1.620	3.053E-02	1.111E-03	7.250	1.094E-05	1.123E-05
1.660	2.619E-02	1.069E-03	7.350	4.094E-06	7.788E-06
1.700	2.629E-02	1.193E-03	7.450	1.103E-07	4.434E-06
1.740	2.415E-02	1.417E-03	7.550	-2.222E-07	5.113E-06
1.780	2.957E-02	1.435E-03	7.650	2.058E-06	6.112E-06
1.820	2.753E-02	1.185E-03	7.750	3.073E-06	5.901E-06
1.860	2.064E-02	9.549E-04	7.850	1.910E-06	5.797E-06
1.900	2.132E-02	9.202E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 110 SEC AFTER END OF IRRADIATION
COUNT FOR 60 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.880E-02	2.105E-03	1.940	2.431E-02	9.341E-04
0.065	9.405E-02	2.482E-03	1.980	2.466E-02	9.111E-04
0.075	1.352E-01	2.645E-03	2.020	2.093E-02	8.694E-04
0.085	1.157E-01	2.474E-03	2.060	1.847E-02	7.951E-04
0.095	1.625E-01	2.624E-03	2.100	1.697E-02	7.748E-04
0.105	1.326E-01	2.458E-03	2.140	1.709E-02	7.860E-04
0.115	8.277E-02	2.157E-03	2.180	1.825E-02	7.957E-04
0.125	7.922E-02	2.135E-03	2.220	1.647E-02	7.668E-04
0.135	1.007E-01	2.265E-03	2.260	1.581E-02	7.529E-04
0.145	1.063E-01	2.331E-03	2.300	1.617E-02	7.235E-04
0.155	9.764E-02	2.331E-03	2.340	1.607E-02	7.310E-04
0.165	1.024E-01	2.396E-03	2.380	1.433E-02	7.540E-04
0.177	1.303E-01	2.260E-03	2.425	1.289E-02	6.979E-04
0.192	1.732E-01	2.314E-03	2.475	1.206E-02	7.107E-04
0.207	1.396E-01	2.159E-03	2.525	1.454E-02	7.257E-04
0.222	1.205E-01	2.063E-03	2.575	1.604E-02	7.134E-04
0.237	8.936E-02	1.907E-03	2.625	1.360E-02	6.733E-04
0.252	9.153E-02	1.836E-03	2.675	1.284E-02	6.621E-04
0.267	1.087E-01	1.909E-03	2.725	1.155E-02	6.573E-04
0.282	1.151E-01	1.934E-03	2.775	1.168E-02	6.460E-04
0.297	1.362E-01	2.041E-03	2.825	1.012E-02	5.957E-04
0.313	1.417E-01	2.068E-03	2.875	7.766E-03	5.403E-04
0.327	1.180E-01	1.945E-03	2.925	6.436E-03	5.092E-04
0.342	1.130E-01	1.945E-03	2.975	5.592E-03	5.001E-04
0.357	1.188E-01	1.961E-03	3.030	4.899E-03	4.822E-04
0.372	1.119E-01	1.961E-03	3.090	4.527E-03	4.720E-04
0.387	1.320E-01	2.095E-03	3.150	4.113E-03	4.489E-04
0.402	1.578E-01	2.266E-03	3.210	4.260E-03	4.379E-04
0.417	1.462E-01	2.244E-03	3.270	6.468E-03	4.722E-04
0.432	1.174E-01	2.047E-03	3.330	7.412E-03	4.903E-04
0.447	1.121E-01	1.769E-03	3.390	6.500E-03	4.634E-04
0.462	9.709E-02	1.643E-03	3.450	6.060E-03	4.472E-04
0.477	7.633E-02	1.526E-03	3.510	7.159E-03	4.649E-04
0.492	6.015E-02	1.443E-03	3.570	9.044E-03	5.217E-04
0.507	5.917E-02	1.476E-03	3.630	7.048E-03	4.893E-04
0.522	7.032E-02	1.630E-03	3.690	4.573E-03	4.142E-04
0.540	8.417E-02	1.619E-03	3.750	2.913E-03	3.611E-04
0.560	9.631E-02	1.589E-03	3.810	1.987E-03	3.360E-04
0.580	1.290E-01	1.721E-03	3.870	2.106E-03	3.299E-04
0.600	1.573E-01	1.817E-03	3.935	2.725E-03	3.194E-04
0.620	1.276E-01	1.710E-03	4.005	3.595E-03	3.298E-04
0.640	8.429E-02	1.461E-03	4.075	4.588E-03	3.455E-04
0.660	6.554E-02	1.333E-03	4.145	4.911E-03	3.502E-04
0.680	8.844E-02	1.504E-03	4.215	3.187E-03	2.914E-04
0.700	1.243E-01	1.683E-03	4.285	3.186E-03	2.852E-04
0.720	1.428E-01	1.828E-03	4.355	3.453E-03	2.870E-04
0.740	1.230E-01	1.683E-03	4.425	1.933E-03	2.305E-04
0.760	8.595E-02	1.445E-03	4.495	9.368E-04	2.009E-04
0.780	7.447E-02	1.317E-03	4.565	1.188E-03	2.014E-04
0.800	9.865E-02	1.448E-03	4.635	1.329E-03	2.087E-04
0.820	1.062E-01	1.519E-03	4.705	9.580E-04	1.870E-04
0.840	1.161E-01	1.582E-03	4.775	9.308E-04	1.825E-04
0.860	1.176E-01	1.576E-03	4.845	7.685E-04	1.656E-04
0.880	1.063E-01	1.448E-03	4.915	6.209E-04	1.595E-04
0.900	9.941E-02	1.378E-03	4.985	8.100E-04	1.680E-04
0.920	9.677E-02	1.363E-03	5.060	9.495E-04	1.549E-04
0.940	9.847E-02	1.435E-03	5.140	8.956E-04	1.492E-04
0.962	1.028E-01	1.458E-03	5.220	8.543E-04	1.302E-04
0.987	9.514E-02	1.413E-03	5.300	7.256E-04	1.257E-04
1.013	7.653E-02	1.237E-03	5.380	5.458E-04	1.019E-04
1.037	6.350E-02	1.149E-03	5.460	6.241E-04	1.043E-04
1.062	6.068E-02	1.142E-03	5.540	3.729E-04	8.519E-05
1.088	6.553E-02	1.171E-03	5.620	9.778E-05	6.154E-05
1.112	6.496E-02	1.127E-03	5.700	5.491E-05	5.848E-05
1.138	5.762E-02	1.067E-03	5.780	4.901E-05	4.718E-05
1.162	5.297E-02	1.036E-03	5.860	6.333E-05	4.224E-05
1.187	5.191E-02	1.074E-03	5.945	6.788E-05	4.523E-05
1.215	5.288E-02	1.051E-03	6.035	9.205E-05	4.433E-05
1.245	5.358E-02	1.115E-03	6.125	1.401E-04	4.864E-05
1.275	6.167E-02	1.206E-03	6.215	1.020E-04	4.305E-05
1.305	7.136E-02	1.256E-03	6.305	4.201E-05	2.408E-05
1.335	6.673E-02	1.160E-03	6.395	9.047E-06	2.099E-05
1.365	6.430E-02	1.117E-03	6.485	-1.211E-06	1.375E-05
1.395	8.062E-02	1.206E-03	6.575	1.834E-05	1.372E-05
1.425	9.121E-02	1.309E-03	6.665	4.273E-05	1.693E-05
1.455	7.102E-02	1.132E-03	6.755	2.557E-05	1.269E-05
1.485	4.544E-02	9.089E-04	6.850	1.003E-05	8.332E-06
1.515	3.480E-02	1.141E-03	6.950	1.542E-05	1.213E-05
1.545	3.392E-02	1.168E-03	7.050	1.603E-05	1.059E-05
1.580	3.232E-02	1.167E-03	7.150	3.801E-06	8.085E-06
1.620	2.954E-02	1.086E-03	7.250	-8.263E-06	9.408E-06
1.660	2.995E-02	1.098E-03	7.350	-1.026E-05	5.372E-06
1.700	2.676E-02	1.212E-03	7.450	3.866E-06	9.395E-06
1.740	2.760E-02	1.465E-03	7.550	4.624E-06	8.131E-06
1.780	2.667E-02	1.474E-03	7.650	-2.571E-06	6.361E-06
1.820	2.579E-02	1.203E-03	7.750	-3.371E-06	6.436E-06
1.860	2.116E-02	9.506E-04	7.850	-7.129E-07	6.305E-06
1.900	1.912E-02	8.860E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 170 SEC AFTER END OF IRRADIATION
COUNT FOR 80 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	2.454E-02	1.966E-03	1.940	1.984E-02	8.347E-04
0.065	8.809E-02	2.384E-03	1.980	2.023E-02	8.245E-04
0.075	1.134E-01	2.469E-03	2.020	1.690E-02	7.791E-04
0.085	1.021E-01	2.328E-03	2.060	1.533E-02	7.278E-04
0.095	1.152E-01	2.340E-03	2.100	1.456E-02	7.198E-04
0.105	1.140E-01	2.292E-03	2.140	1.488E-02	7.278E-04
0.115	7.374E-02	2.079E-03	2.180	1.566E-02	7.321E-04
0.125	6.838E-02	2.033E-03	2.220	1.464E-02	7.112E-04
0.135	9.456E-02	2.188E-03	2.260	1.298E-02	6.786E-04
0.145	1.029E-01	2.277E-03	2.300	1.270E-02	6.423E-04
0.155	9.401E-02	2.272E-03	2.340	1.268E-02	6.514E-04
0.165	9.654E-02	2.323E-03	2.380	1.154E-02	6.717E-04
0.177	1.385E-01	2.255E-03	2.425	1.020E-02	6.149E-04
0.192	1.806E-01	2.298E-03	2.475	9.915E-03	6.313E-04
0.207	1.325E-01	2.089E-03	2.525	1.272E-02	6.546E-04
0.222	1.052E-01	1.946E-03	2.575	1.258E-02	6.263E-04
0.237	8.047E-02	1.829E-03	2.625	1.271E-02	6.316E-04
0.252	9.395E-02	1.806E-03	2.675	1.036E-02	5.968E-04
0.267	1.122E-01	1.892E-03	2.725	9.321E-03	6.019E-04
0.282	1.157E-01	1.902E-03	2.775	9.577E-03	5.944E-04
0.297	1.401E-01	2.020E-03	2.825	7.834E-03	5.267E-04
0.313	1.610E-01	2.132E-03	2.875	6.087E-03	4.756E-04
0.327	1.272E-01	1.993E-03	2.925	4.972E-03	4.546E-04
0.342	1.098E-01	1.913E-03	2.975	3.588E-03	4.241E-04
0.357	1.047E-01	1.873E-03	3.030	3.356E-03	4.151E-04
0.372	9.299E-02	1.824E-03	3.090	3.792E-03	4.168E-04
0.387	9.122E-02	1.814E-03	3.150	3.907E-03	4.030E-04
0.402	1.043E-01	1.915E-03	3.210	4.196E-03	3.940E-04
0.417	1.133E-01	2.009E-03	3.270	5.812E-03	4.301E-04
0.432	1.063E-01	1.959E-03	3.330	6.636E-03	4.454E-04
0.447	1.082E-01	1.724E-03	3.390	5.975E-03	4.193E-04
0.462	9.708E-02	1.620E-03	3.450	5.248E-03	3.933E-04
0.477	7.514E-02	1.500E-03	3.510	6.256E-03	4.185E-04
0.492	5.808E-02	1.402E-03	3.570	6.876E-03	4.491E-04
0.507	5.579E-02	1.425E-03	3.630	4.674E-03	3.971E-04
0.522	5.839E-02	1.515E-03	3.690	2.642E-03	3.335E-04
0.540	6.321E-02	1.465E-03	3.750	1.936E-03	3.044E-04
0.560	8.237E-02	1.479E-03	3.810	1.634E-03	3.045E-04
0.580	1.167E-01	1.630E-03	3.870	1.427E-03	2.879E-04
0.600	1.411E-01	1.721E-03	3.935	1.544E-03	2.654E-04
0.620	1.111E-01	1.614E-03	4.005	2.387E-03	2.778E-04
0.640	8.015E-02	1.412E-03	4.075	3.814E-03	3.067E-04
0.660	6.697E-02	1.321E-03	4.145	3.697E-03	2.958E-04
0.680	8.559E-02	1.464E-03	4.215	2.547E-03	2.577E-04
0.700	1.236E-01	1.657E-03	4.285	2.765E-03	2.498E-04
0.720	1.410E-01	1.774E-03	4.355	3.330E-03	2.624E-04
0.740	1.179E-01	1.635E-03	4.425	1.821E-03	2.055E-04
0.760	8.910E-02	1.432E-03	4.495	8.884E-04	1.710E-04
0.780	8.190E-02	1.338E-03	4.565	7.823E-04	1.678E-04
0.800	9.518E-02	1.412E-03	4.635	6.656E-04	1.569E-04
0.820	1.150E-01	1.507E-03	4.705	6.130E-04	1.550E-04
0.840	1.266E-01	1.608E-03	4.775	4.729E-04	1.382E-04
0.860	1.213E-01	1.576E-03	4.845	4.199E-04	1.305E-04
0.880	1.122E-01	1.453E-03	4.915	4.197E-04	1.307E-04
0.900	1.071E-01	1.376E-03	4.985	4.401E-04	1.261E-04
0.920	1.032E-01	1.355E-03	5.060	6.088E-04	1.290E-04
0.940	1.018E-01	1.419E-03	5.140	7.335E-04	1.253E-04
0.962	1.037E-01	1.428E-03	5.220	6.264E-04	1.192E-04
0.987	9.432E-02	1.367E-03	5.300	4.278E-04	9.354E-05
1.013	7.549E-02	1.196E-03	5.380	4.360E-04	9.692E-05
1.037	6.186E-02	1.109E-03	5.460	4.003E-04	8.238E-05
1.062	5.694E-02	1.088E-03	5.540	2.477E-04	6.943E-05
1.088	5.957E-02	1.099E-03	5.620	1.378E-04	5.842E-05
1.112	5.894E-02	1.056E-03	5.700	1.150E-04	5.115E-05
1.138	5.353E-02	1.010E-03	5.780	8.565E-05	4.314E-05
1.162	4.911E-02	9.773E-04	5.860	1.613E-05	3.612E-05
1.187	4.866E-02	1.015E-03	5.945	4.306E-06	3.083E-05
1.215	4.854E-02	9.955E-04	6.035	4.838E-05	3.882E-05
1.245	4.727E-02	1.060E-03	6.125	2.382E-05	3.289E-05
1.275	5.152E-02	1.137E-03	6.215	-9.702E-06	2.398E-05
1.305	5.728E-02	1.156E-03	6.305	1.542E-05	2.740E-05
1.335	5.490E-02	1.073E-03	6.395	5.470E-05	2.940E-05
1.365	5.383E-02	1.030E-03	6.485	5.590E-05	2.442E-05
1.395	6.507E-02	1.100E-03	6.575	4.975E-05	2.253E-05
1.425	7.016E-02	1.155E-03	6.665	4.357E-05	2.026E-05
1.455	5.335E-02	9.832E-04	6.755	2.973E-05	1.730E-05
1.485	3.358E-02	8.010E-04	6.850	1.356E-05	1.387E-05
1.515	2.709E-02	1.000E-03	6.950	3.731E-06	8.194E-06
1.545	2.547E-02	1.033E-03	7.050	-2.943E-06	6.889E-06
1.580	2.451E-02	1.023E-03	7.150	-1.191E-05	7.371E-06
1.620	2.487E-02	9.912E-04	7.250	-9.158E-06	1.072E-05
1.660	2.546E-02	1.018E-03	7.350	-8.597E-07	7.778E-06
1.700	2.482E-02	1.152E-03	7.450	3.454E-06	6.824E-06
1.740	2.419E-02	1.397E-03	7.550	4.987E-06	7.542E-06
1.780	2.429E-02	1.380E-03	7.650	5.361E-06	6.908E-06
1.820	2.210E-02	1.106E-03	7.750	2.822E-06	6.740E-06
1.860	1.605E-02	8.379E-04	7.850	1.196E-06	5.271E-06
1.900	1.706E-02	8.106E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 250 SEC AFTER END OF IRRADIATION
COUNT FOR 100 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.730E-02	1.819E-03	1.940	1.551E-02	7.353E-04
0.065	6.826E-02	2.192E-03	1.980	1.586E-02	7.299E-04
0.075	9.761E-02	2.344E-03	2.020	1.622E-02	7.412E-04
0.085	8.738E-02	2.178E-03	2.060	1.447E-02	6.931E-04
0.095	8.225E-02	2.100E-03	2.100	1.233E-02	6.631E-04
0.105	9.363E-02	2.138E-03	2.140	1.503E-02	7.139E-04
0.115	6.346E-02	1.980E-03	2.180	1.467E-02	7.075E-04
0.125	6.221E-02	1.954E-03	2.220	1.204E-02	6.498E-04
0.135	8.325E-02	2.111E-03	2.260	1.162E-02	6.193E-04
0.145	9.552E-02	2.205E-03	2.300	1.005E-02	5.749E-04
0.155	9.043E-02	2.219E-03	2.340	8.616E-03	5.479E-04
0.165	1.010E-01	2.307E-03	2.380	8.334E-03	5.831E-04
0.177	1.331E-01	2.202E-03	2.425	7.503E-03	5.334E-04
0.192	1.642E-01	2.271E-03	2.475	8.633E-03	5.660E-04
0.207	1.170E-01	1.988E-03	2.525	1.008E-02	5.843E-04
0.222	9.266E-02	1.845E-03	2.575	1.039E-02	5.608E-04
0.237	7.851E-02	1.778E-03	2.625	1.073E-02	5.650E-04
0.252	9.248E-02	1.768E-03	2.675	8.918E-03	5.575E-04
0.267	1.109E-01	1.844E-03	2.725	9.015E-03	5.813E-04
0.282	1.129E-01	1.854E-03	2.775	7.808E-03	5.414E-04
0.297	1.434E-01	2.020E-03	2.825	5.622E-03	4.559E-04
0.313	1.658E-01	2.137E-03	2.875	4.562E-03	4.175E-04
0.327	1.326E-01	1.988E-03	2.925	2.664E-03	3.628E-04
0.342	1.040E-01	1.855E-03	2.975	2.244E-03	3.601E-04
0.357	9.408E-02	1.780E-03	3.030	3.079E-03	3.707E-04
0.372	7.520E-02	1.677E-03	3.090	3.212E-03	3.754E-04
0.387	6.844E-02	1.631E-03	3.150	2.002E-03	3.167E-04
0.402	8.085E-02	1.722E-03	3.210	3.570E-03	3.473E-04
0.417	8.993E-02	1.830E-03	3.270	5.090E-03	3.807E-04
0.432	9.513E-02	1.851E-03	3.330	5.378E-03	3.940E-04
0.447	1.013E-01	1.674E-03	3.390	5.171E-03	3.804E-04
0.462	9.099E-02	1.561E-03	3.450	3.897E-03	3.363E-04
0.477	7.295E-02	1.459E-03	3.510	4.539E-03	3.547E-04
0.492	5.513E-02	1.356E-03	3.570	5.339E-03	3.851E-04
0.507	5.079E-02	1.364E-03	3.630	3.734E-03	3.307E-04
0.522	5.015E-02	1.433E-03	3.690	1.476E-03	2.614E-04
0.540	5.166E-02	1.361E-03	3.750	8.987E-04	2.522E-04
0.560	7.164E-02	1.395E-03	3.810	1.416E-03	2.601E-04
0.580	1.084E-01	1.560E-03	3.870	1.404E-03	2.646E-04
0.600	1.260E-01	1.630E-03	3.935	9.739E-04	2.220E-04
0.620	9.590E-02	1.505E-03	4.005	1.572E-03	2.318E-04
0.640	7.287E-02	1.342E-03	4.075	2.409E-03	2.516E-04
0.660	6.198E-02	1.271E-03	4.145	2.559E-03	2.560E-04
0.680	7.896E-02	1.405E-03	4.215	1.876E-03	2.174E-04
0.700	1.121E-01	1.576E-03	4.285	2.315E-03	2.243E-04
0.720	1.291E-01	1.694E-03	4.355	2.595E-03	2.273E-04
0.740	1.064E-01	1.550E-03	4.425	1.555E-03	1.846E-04
0.760	8.368E-02	1.373E-03	4.495	6.248E-04	1.397E-04
0.780	8.030E-02	1.307E-03	4.565	7.082E-04	1.455E-04
0.800	9.315E-02	1.378E-03	4.635	8.274E-04	1.409E-04
0.820	1.133E-01	1.486E-03	4.705	4.744E-04	1.263E-04
0.840	1.254E-01	1.592E-03	4.775	1.746E-04	1.004E-04
0.860	1.218E-01	1.566E-03	4.845	2.153E-04	1.018E-04
0.880	1.144E-01	1.437E-03	4.915	2.905E-04	1.017E-04
0.900	1.106E-01	1.379E-03	4.985	3.302E-04	9.745E-05
0.920	1.083E-01	1.352E-03	5.060	3.950E-04	9.864E-05
0.940	1.057E-01	1.425E-03	5.140	4.383E-04	9.670E-05
0.962	1.011E-01	1.386E-03	5.220	4.361E-04	9.192E-05
0.987	8.928E-02	1.314E-03	5.300	3.332E-04	8.045E-05
1.013	7.322E-02	1.152E-03	5.380	2.764E-04	6.889E-05
1.037	6.030E-02	1.068E-03	5.460	2.117E-04	6.172E-05
1.062	5.362E-02	1.033E-03	5.540	9.103E-05	4.656E-05
1.088	5.416E-02	1.038E-03	5.620	4.265E-05	3.976E-05
1.112	5.125E-02	9.789E-04	5.700	5.057E-05	4.267E-05
1.138	4.885E-02	9.576E-04	5.780	4.322E-05	3.437E-05
1.162	4.483E-02	9.303E-04	5.860	2.817E-05	2.829E-05
1.187	4.374E-02	9.565E-04	5.945	2.377E-05	2.538E-05
1.215	4.458E-02	9.443E-04	6.035	1.487E-05	1.859E-05
1.245	4.221E-02	1.009E-03	6.125	8.832E-06	1.183E-05
1.275	4.232E-02	1.075E-03	6.215	1.556E-05	1.529E-05
1.305	4.313E-02	1.063E-03	6.305	1.351E-05	1.623E-05
1.335	4.128E-02	9.624E-04	6.395	6.964E-06	1.283E-05
1.365	4.269E-02	9.447E-04	6.485	2.943E-06	9.201E-06
1.395	4.758E-02	9.789E-04	6.575	4.637E-06	7.466E-06
1.425	4.752E-02	9.779E-04	6.665	8.952E-06	8.477E-06
1.455	3.597E-02	8.502E-04	6.755	1.047E-05	1.041E-05
1.485	2.515E-02	7.118E-04	6.850	3.239E-06	1.151E-05
1.515	2.183E-02	9.148E-04	6.950	1.623E-07	9.017E-06
1.545	2.158E-02	9.325E-04	7.050	-9.227E-07	6.320E-06
1.580	1.929E-02	9.287E-04	7.150	-3.058E-06	4.056E-06
1.620	2.033E-02	9.020E-04	7.250	-1.063E-06	5.979E-06
1.660	2.249E-02	9.533E-04	7.350	8.924E-06	9.387E-06
1.700	2.370E-02	1.081E-03	7.450	1.249E-05	9.815E-06
1.740	2.172E-02	1.291E-03	7.550	4.696E-06	5.297E-06
1.780	2.183E-02	1.284E-03	7.650	7.812E-08	3.412E-06
1.820	1.826E-02	1.014E-03	7.750	9.352E-07	5.998E-06
1.860	1.389E-02	7.738E-04	7.850	2.290E-06	5.679E-06
1.900	1.503E-02	7.460E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 350 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.677E-02	2.156E-03	1.940	2.168E-02	8.635E-04
0.065	7.651E-02	2.516E-03	1.980	2.210E-02	8.480E-04
0.075	1.395E-01	2.811E-03	2.020	2.212E-02	8.512E-04
0.085	1.368E-01	2.677E-03	2.060	1.807E-02	7.919E-04
0.095	8.844E-02	2.371E-03	2.100	1.728E-02	7.818E-04
0.105	1.133E-01	2.458E-03	2.140	2.133E-02	8.416E-04
0.115	8.824E-02	2.371E-03	2.180	2.131E-02	8.234E-04
0.125	8.668E-02	2.373E-03	2.220	1.895E-02	7.791E-04
0.135	1.159E-01	2.511E-03	2.260	1.462E-02	6.952E-04
0.145	1.382E-01	2.656E-03	2.300	1.181E-02	6.391E-04
0.155	1.412E-01	2.725E-03	2.340	1.125E-02	6.251E-04
0.165	1.522E-01	2.795E-03	2.380	1.076E-02	6.615E-04
0.177	2.165E-01	2.725E-03	2.425	9.727E-03	6.015E-04
0.192	2.833E-01	2.784E-03	2.475	9.241E-03	6.114E-04
0.207	1.726E-01	2.389E-03	2.525	1.175E-02	6.375E-04
0.222	1.353E-01	2.223E-03	2.575	1.408E-02	6.460E-04
0.237	1.130E-01	2.127E-03	2.625	1.379E-02	6.444E-04
0.252	1.437E-01	2.148E-03	2.675	1.170E-02	6.466E-04
0.267	1.686E-01	2.218E-03	2.725	1.047E-02	6.602E-04
0.282	1.673E-01	2.223E-03	2.775	9.724E-03	6.169E-04
0.297	2.209E-01	2.468E-03	2.825	7.359E-03	5.149E-04
0.313	2.814E-01	2.699E-03	2.875	5.117E-03	4.423E-04
0.327	2.127E-01	2.469E-03	2.925	3.513E-03	3.961E-04
0.342	1.556E-01	2.234E-03	2.975	2.774E-03	3.899E-04
0.357	1.312E-01	2.095E-03	3.030	2.644E-03	3.840E-04
0.372	1.069E-01	1.993E-03	3.090	2.873E-03	3.803E-04
0.387	8.191E-02	1.854E-03	3.150	2.986E-03	3.579E-04
0.402	9.944E-02	1.943E-03	3.210	3.878E-03	3.655E-04
0.417	1.196E-01	2.105E-03	3.270	5.483E-03	3.977E-04
0.432	1.360E-01	2.180E-03	3.330	6.280E-03	4.194E-04
0.447	1.480E-01	1.999E-03	3.390	5.066E-03	3.728E-04
0.462	1.349E-01	1.913E-03	3.450	3.899E-03	3.379E-04
0.477	1.098E-01	1.811E-03	3.510	6.394E-03	3.967E-04
0.492	8.390E-02	1.703E-03	3.570	7.113E-03	4.298E-04
0.507	7.204E-02	1.680E-03	3.630	5.995E-03	3.871E-04
0.522	6.996E-02	1.738E-03	3.690	1.742E-03	2.742E-04
0.540	7.403E-02	1.659E-03	3.750	9.941E-04	2.355E-04
0.560	1.033E-01	1.679E-03	3.810	1.357E-03	2.666E-04
0.580	1.581E-01	1.870E-03	3.870	1.266E-03	2.484E-04
0.600	1.800E-01	1.934E-03	3.935	9.471E-04	2.160E-04
0.620	1.362E-01	1.790E-03	4.005	1.646E-03	2.272E-04
0.640	1.062E-01	1.614E-03	4.075	2.377E-03	2.517E-04
0.660	9.343E-02	1.531E-03	4.145	2.529E-03	2.486E-04
0.680	1.066E-01	1.648E-03	4.215	1.937E-03	2.193E-04
0.700	1.469E-01	1.817E-03	4.285	2.074E-03	2.082E-04
0.720	1.671E-01	1.929E-03	4.355	2.504E-03	2.223E-04
0.740	1.440E-01	1.811E-03	4.425	1.339E-03	1.675E-04
0.760	1.212E-01	1.646E-03	4.495	6.563E-04	1.447E-04
0.780	1.190E-01	1.576E-03	4.565	8.071E-04	1.368E-04
0.800	1.369E-01	1.657E-03	4.635	8.284E-04	1.369E-04
0.820	1.644E-01	1.801E-03	4.705	3.269E-04	1.135E-04
0.840	1.849E-01	1.945E-03	4.775	1.129E-04	8.399E-05
0.860	1.886E-01	1.945E-03	4.845	2.974E-04	9.046E-05
0.880	1.856E-01	1.806E-03	4.915	2.973E-04	7.850E-05
0.900	1.825E-01	1.721E-03	4.985	1.955E-04	8.322E-05
0.920	1.735E-01	1.705E-03	5.060	2.689E-04	7.669E-05
0.940	1.623E-01	1.731E-03	5.140	4.420E-04	8.982E-05
0.962	1.467E-01	1.657E-03	5.220	3.555E-04	8.177E-05
0.987	1.240E-01	1.555E-03	5.300	2.016E-04	6.437E-05
1.013	1.082E-01	1.383E-03	5.380	1.121E-04	4.657E-05
1.037	8.957E-02	1.287E-03	5.460	2.701E-05	3.801E-05
1.062	7.543E-02	1.219E-03	5.540	-1.717E-05	2.883E-05
1.088	7.284E-02	1.209E-03	5.620	1.401E-05	3.445E-05
1.112	7.074E-02	1.139E-03	5.700	5.387E-05	3.913E-05
1.138	6.763E-02	1.117E-03	5.780	3.266E-05	2.339E-05
1.162	6.484E-02	1.102E-03	5.860	1.071E-06	2.187E-05
1.187	6.459E-02	1.156E-03	5.945	1.801E-05	2.245E-05
1.215	6.671E-02	1.138E-03	6.035	5.358E-05	3.020E-05
1.245	6.761E-02	1.231E-03	6.125	5.358E-05	2.964E-05
1.275	6.452E-02	1.292E-03	6.215	1.840E-05	2.916E-05
1.305	6.257E-02	1.247E-03	6.305	-7.187E-06	1.802E-05
1.335	5.515E-02	1.123E-03	6.395	-2.925E-06	2.368E-05
1.365	5.474E-02	1.083E-03	6.485	-8.839E-06	2.941E-05
1.395	5.451E-02	1.063E-03	6.575	-9.053E-06	2.927E-05
1.425	5.031E-02	1.033E-03	6.665	3.900E-06	1.586E-05
1.455	3.853E-02	9.100E-04	6.755	2.135E-05	1.576E-05
1.485	2.926E-02	7.828E-04	6.850	3.189E-05	2.027E-05
1.515	2.822E-02	1.031E-03	6.950	2.133E-05	1.927E-05
1.545	2.704E-02	1.066E-03	7.050	1.384E-05	1.551E-05
1.580	2.555E-02	1.061E-03	7.150	6.701E-06	1.350E-05
1.620	2.599E-02	1.039E-03	7.250	1.478E-06	1.423E-05
1.660	3.072E-02	1.095E-03	7.350	-1.757E-06	1.242E-05
1.700	3.146E-02	1.205E-03	7.450	5.336E-06	1.508E-05
1.740	2.831E-02	1.370E-03	7.550	4.465E-06	1.133E-05
1.780	2.678E-02	1.354E-03	7.650	1.047E-06	7.916E-06
1.820	2.296E-02	1.108E-03	7.750	8.489E-07	6.761E-06
1.860	1.989E-02	9.068E-04	7.850	8.534E-07	6.233E-06
1.900	1.943E-02	8.566E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 550 SEC AFTER END OF IRRADIATION
COUNT FOR 200 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.115E-02	1.831E-03	1.940	1.469E-02	7.150E-04
0.065	3.434E-02	2.082E-03	1.980	1.649E-02	7.235E-04
0.075	1.024E-01	2.444E-03	2.020	1.668E-02	7.337E-04
0.085	1.086E-01	2.397E-03	2.060	1.336E-02	6.674E-04
0.095	5.477E-02	2.004E-03	2.100	1.209E-02	6.701E-04
0.105	6.673E-02	2.042E-03	2.140	1.557E-02	7.267E-04
0.115	6.166E-02	2.074E-03	2.180	1.765E-02	7.449E-04
0.125	7.024E-02	2.112E-03	2.220	1.477E-02	6.770E-04
0.135	8.749E-02	2.238E-03	2.260	1.036E-02	5.820E-04
0.145	1.024E-01	2.343E-03	2.300	7.153E-03	5.093E-04
0.155	1.110E-01	2.426E-03	2.340	6.685E-03	4.891E-04
0.165	1.183E-01	2.479E-03	2.380	7.108E-03	5.265E-04
0.177	1.743E-01	2.442E-03	2.425	5.926E-03	4.675E-04
0.192	2.261E-01	2.463E-03	2.475	6.441E-03	4.939E-04
0.207	1.349E-01	2.100E-03	2.525	8.061E-03	5.121E-04
0.222	1.038E-01	1.936E-03	2.575	9.921E-03	5.321E-04
0.237	8.506E-02	1.852E-03	2.625	1.015E-02	5.467E-04
0.252	1.086E-01	1.871E-03	2.675	8.008E-03	5.376E-04
0.267	1.294E-01	1.945E-03	2.725	6.567E-03	5.355E-04
0.282	1.313E-01	1.972E-03	2.775	5.378E-03	4.893E-04
0.297	1.801E-01	2.212E-03	2.825	3.654E-03	3.980E-04
0.313	2.385E-01	2.469E-03	2.875	2.512E-03	3.233E-04
0.327	1.791E-01	2.260E-03	2.925	1.617E-03	2.875E-04
0.342	1.160E-01	1.961E-03	2.975	1.121E-03	2.864E-04
0.357	9.841E-02	1.834E-03	3.030	1.309E-03	3.005E-04
0.372	7.846E-02	1.731E-03	3.090	1.607E-03	3.050E-04
0.387	5.989E-02	1.599E-03	3.150	1.684E-03	2.726E-04
0.402	7.132E-02	1.677E-03	3.210	2.096E-03	2.726E-04
0.417	8.568E-02	1.799E-03	3.270	3.575E-03	3.113E-04
0.432	9.540E-02	1.857E-03	3.330	4.163E-03	3.269E-04
0.447	1.091E-01	1.715E-03	3.390	2.840E-03	2.762E-04
0.462	1.030E-01	1.648E-03	3.450	2.499E-03	2.604E-04
0.477	8.495E-02	1.553E-03	3.510	3.922E-03	2.992E-04
0.492	6.347E-02	1.439E-03	3.570	4.725E-03	3.363E-04
0.507	5.361E-02	1.426E-03	3.630	2.907E-03	2.595E-04
0.522	5.302E-02	1.491E-03	3.690	1.209E-03	1.991E-04
0.540	5.726E-02	1.427E-03	3.750	5.562E-04	1.749E-04
0.560	7.950E-02	1.459E-03	3.810	4.020E-04	1.662E-04
0.580	1.167E-01	1.624E-03	3.870	6.599E-04	1.752E-04
0.600	1.316E-01	1.683E-03	3.935	7.911E-04	1.643E-04
0.620	9.761E-02	1.550E-03	4.005	7.751E-04	1.534E-04
0.640	7.974E-02	1.414E-03	4.075	1.010E-03	1.584E-04
0.660	7.014E-02	1.335E-03	4.145	1.179E-03	1.699E-04
0.680	6.812E-02	1.364E-03	4.215	8.352E-04	1.499E-04
0.700	8.932E-02	1.463E-03	4.285	1.022E-03	1.471E-04
0.720	1.049E-01	1.556E-03	4.355	1.185E-03	1.408E-04
0.740	9.397E-02	1.489E-03	4.425	4.866E-04	1.047E-04
0.760	8.813E-02	1.409E-03	4.495	1.521E-04	8.122E-05
0.780	8.930E-02	1.360E-03	4.565	3.484E-04	8.652E-05
0.800	9.878E-02	1.433E-03	4.635	3.635E-04	8.906E-05
0.820	1.190E-01	1.555E-03	4.705	2.458E-04	7.336E-05
0.840	1.374E-01	1.699E-03	4.775	9.743E-05	5.523E-05
0.860	1.455E-01	1.715E-03	4.845	3.739E-05	4.789E-05
0.880	1.481E-01	1.608E-03	4.915	7.769E-05	4.910E-05
0.900	1.476E-01	1.534E-03	4.985	8.098E-05	5.298E-05
0.920	1.410E-01	1.518E-03	5.060	8.419E-05	5.284E-05
0.940	1.265E-01	1.507E-03	5.140	1.178E-04	5.448E-05
0.962	1.047E-01	1.395E-03	5.220	8.284E-05	4.989E-05
0.987	8.655E-02	1.288E-03	5.300	6.004E-05	4.372E-05
1.013	7.920E-02	1.183E-03	5.380	6.918E-05	3.477E-05
1.037	6.835E-02	1.113E-03	5.460	5.194E-05	3.558E-05
1.062	5.500E-02	1.033E-03	5.540	2.560E-05	3.095E-05
1.088	4.890E-02	9.928E-04	5.620	3.148E-05	2.772E-05
1.112	4.721E-02	9.554E-04	5.700	6.927E-05	2.237E-05
1.138	4.630E-02	9.431E-04	5.780	7.968E-05	2.220E-05
1.162	4.606E-02	9.367E-04	5.860	3.670E-05	1.325E-05
1.187	4.688E-02	9.725E-04	5.945	1.713E-06	1.537E-05
1.215	4.980E-02	9.800E-04	6.035	-7.415E-06	2.021E-05
1.245	4.947E-02	1.057E-03	6.125	-1.888E-05	1.611E-05
1.275	4.651E-02	1.107E-03	6.215	-3.919E-05	1.021E-05
1.305	4.219E-02	1.061E-03	6.305	-3.329E-05	1.333E-05
1.335	3.851E-02	9.656E-04	6.395	6.541E-07	1.478E-05
1.365	3.868E-02	9.319E-04	6.485	1.007E-05	1.368E-05
1.395	3.692E-02	8.790E-04	6.575	8.202E-06	1.629E-05
1.425	3.185E-02	8.256E-04	6.665	1.148E-05	1.198E-05
1.455	2.420E-02	7.219E-04	6.755	1.866E-05	7.502E-06
1.485	1.963E-02	6.466E-04	6.850	3.699E-05	1.839E-05
1.515	1.901E-02	8.502E-04	6.950	1.534E-05	1.618E-05
1.545	1.747E-02	8.731E-04	7.050	1.310E-06	1.273E-05
1.580	1.726E-02	8.870E-04	7.150	-1.173E-06	1.369E-05
1.620	1.909E-02	9.004E-04	7.250	1.870E-07	1.225E-05
1.660	2.061E-02	9.212E-04	7.350	-1.070E-05	1.055E-05
1.700	2.105E-02	9.544E-04	7.450	-2.554E-07	1.322E-05
1.740	2.008E-02	1.046E-03	7.550	8.220E-06	9.249E-06
1.780	1.832E-02	1.018E-03	7.650	8.303E-06	8.200E-06
1.820	1.451E-02	8.512E-04	7.750	3.062E-06	6.584E-06
1.860	1.376E-02	7.470E-04	7.850	7.114E-07	6.014E-06
1.900	1.398E-02	7.144E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 750 SEC AFTER END OF IRRADIATION
COUNT FOR 400 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.860E-03	2.214E-03	1.940	1.926E-02	8.438E-04
0.065	3.119E-02	2.509E-03	1.980	2.366E-02	8.867E-04
0.075	1.491E-01	3.067E-03	2.020	2.444E-02	8.902E-04
0.085	1.708E-01	3.019E-03	2.060	1.818E-02	8.181E-04
0.095	7.338E-02	2.475E-03	2.100	1.703E-02	8.357E-04
0.105	8.775E-02	2.473E-03	2.140	2.213E-02	8.929E-04
0.115	8.450E-02	2.543E-03	2.180	2.594E-02	9.111E-04
0.125	1.019E-01	2.631E-03	2.220	2.282E-02	8.395E-04
0.135	1.258E-01	2.747E-03	2.260	1.515E-02	6.920E-04
0.145	1.619E-01	2.912E-03	2.300	1.056E-02	6.181E-04
0.155	1.645E-01	2.992E-03	2.340	9.574E-03	5.861E-04
0.165	1.758E-01	3.057E-03	2.380	9.529E-03	6.208E-04
0.177	2.662E-01	2.998E-03	2.425	7.981E-03	5.533E-04
0.192	3.404E-01	3.046E-03	2.475	9.081E-03	5.903E-04
0.207	2.092E-01	2.586E-03	2.525	1.270E-02	6.311E-04
0.222	1.562E-01	2.394E-03	2.575	1.456E-02	6.455E-04
0.237	1.222E-01	2.255E-03	2.625	1.263E-02	6.241E-04
0.252	1.622E-01	2.287E-03	2.675	9.604E-03	6.242E-04
0.267	1.903E-01	2.373E-03	2.725	6.574E-03	6.403E-04
0.282	2.031E-01	2.447E-03	2.775	4.549E-03	5.834E-04
0.297	3.101E-01	2.821E-03	2.825	4.205E-03	4.419E-04
0.313	3.981E-01	3.131E-03	2.875	2.542E-03	3.412E-04
0.327	2.862E-01	2.805E-03	2.925	2.025E-03	3.268E-04
0.342	1.845E-01	2.431E-03	2.975	2.568E-03	3.481E-04
0.357	1.463E-01	2.234E-03	3.030	2.268E-03	3.447E-04
0.372	1.116E-01	2.063E-03	3.090	2.418E-03	3.428E-04
0.387	8.531E-02	1.905E-03	3.150	2.344E-03	3.137E-04
0.402	9.838E-02	1.976E-03	3.210	2.762E-03	3.090E-04
0.417	1.206E-01	2.137E-03	3.270	3.602E-03	3.250E-04
0.432	1.360E-01	2.228E-03	3.330	3.341E-03	3.118E-04
0.447	1.647E-01	2.111E-03	3.390	2.821E-03	2.877E-04
0.462	1.572E-01	2.052E-03	3.450	3.283E-03	2.856E-04
0.477	1.266E-01	1.940E-03	3.510	5.342E-03	3.452E-04
0.492	9.424E-02	1.812E-03	3.570	5.958E-03	3.627E-04
0.507	8.076E-02	1.807E-03	3.630	3.385E-03	2.728E-04
0.522	8.300E-02	1.901E-03	3.690	1.306E-03	2.042E-04
0.540	9.392E-02	1.824E-03	3.750	9.460E-04	1.850E-04
0.560	1.276E-01	1.822E-03	3.810	8.156E-04	1.781E-04
0.580	1.680E-01	1.956E-03	3.870	6.894E-04	1.671E-04
0.600	1.818E-01	1.993E-03	3.935	8.868E-04	1.726E-04
0.620	1.441E-01	1.892E-03	4.005	7.966E-04	1.505E-04
0.640	1.225E-01	1.753E-03	4.075	8.044E-04	1.580E-04
0.660	1.049E-01	1.639E-03	4.145	9.302E-04	1.484E-04
0.680	9.210E-02	1.615E-03	4.215	6.942E-04	1.377E-04
0.700	1.075E-01	1.671E-03	4.285	6.307E-04	1.227E-04
0.720	1.278E-01	1.753E-03	4.355	8.523E-04	1.295E-04
0.740	1.287E-01	1.753E-03	4.425	5.286E-04	1.023E-04
0.760	1.259E-01	1.694E-03	4.495	1.765E-04	8.130E-05
0.780	1.282E-01	1.646E-03	4.565	1.234E-04	7.777E-05
0.800	1.351E-01	1.731E-03	4.635	1.388E-04	7.486E-05
0.820	1.606E-01	1.849E-03	4.705	1.133E-04	7.200E-05
0.840	1.983E-01	2.073E-03	4.775	3.233E-05	5.756E-05
0.860	2.202E-01	2.121E-03	4.845	-3.965E-07	5.048E-05
0.880	2.278E-01	1.999E-03	4.915	3.777E-05	5.491E-05
0.900	2.294E-01	1.892E-03	4.985	7.705E-05	5.918E-05
0.920	2.198E-01	1.908E-03	5.060	8.267E-05	5.534E-05
0.940	1.916E-01	1.876E-03	5.140	8.167E-05	5.485E-05
0.962	1.453E-01	1.651E-03	5.220	1.234E-04	5.668E-05
0.987	1.178E-01	1.534E-03	5.300	9.916E-05	4.510E-05
1.013	1.154E-01	1.432E-03	5.380	2.909E-05	3.346E-05
1.037	9.949E-02	1.347E-03	5.460	6.423E-07	2.704E-05
1.062	7.603E-02	1.242E-03	5.540	4.520E-06	1.985E-05
1.088	6.553E-02	1.200E-03	5.620	1.254E-05	1.813E-05
1.112	6.493E-02	1.151E-03	5.700	8.912E-06	2.133E-05
1.138	6.507E-02	1.141E-03	5.780	2.573E-05	2.505E-05
1.162	6.448E-02	1.132E-03	5.860	3.044E-05	2.477E-05
1.187	6.656E-02	1.180E-03	5.945	1.309E-05	2.238E-05
1.215	7.222E-02	1.193E-03	6.035	-1.606E-05	1.972E-05
1.245	7.713E-02	1.276E-03	6.125	-2.980E-05	1.217E-05
1.275	6.950E-02	1.437E-03	6.215	2.237E-06	2.588E-05
1.305	6.143E-02	1.376E-03	6.305	2.046E-05	2.238E-05
1.335	5.637E-02	1.210E-03	6.395	3.387E-05	2.739E-05
1.365	5.928E-02	1.164E-03	6.485	7.389E-06	1.903E-05
1.395	5.772E-02	1.080E-03	6.575	-2.350E-05	2.099E-05
1.425	4.794E-02	9.939E-04	6.665	-3.669E-05	8.873E-06
1.455	3.573E-02	8.641E-04	6.755	1.082E-05	1.710E-05
1.485	2.943E-02	7.689E-04	6.850	8.039E-05	2.603E-05
1.515	2.855E-02	1.026E-03	6.950	5.428E-05	2.377E-05
1.545	2.676E-02	1.054E-03	7.050	2.545E-05	1.857E-05
1.580	2.443E-02	1.081E-03	7.150	7.999E-06	8.887E-06
1.620	2.643E-02	1.074E-03	7.250	6.341E-06	1.873E-05
1.660	3.054E-02	1.111E-03	7.350	-1.610E-05	1.184E-05
1.700	2.988E-02	1.113E-03	7.450	-1.515E-05	1.772E-05
1.740	2.752E-02	1.127E-03	7.550	-5.278E-06	1.531E-05
1.780	2.424E-02	1.114E-03	7.650	1.504E-06	1.040E-05
1.820	1.972E-02	9.944E-04	7.750	1.636E-06	7.071E-06
1.860	1.983E-02	9.015E-04	7.850	1.219E-06	6.050E-06
1.900	2.001E-02	8.641E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1150 SEC AFTER END OF IRRADIATION
COUNT FOR 400 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	4.328E-03	1.956E-03	1.940	1.360E-02	7.054E-04
0.065	1.110E-02	2.202E-03	1.980	1.615E-02	7.203E-04
0.075	1.066E-01	2.742E-03	2.020	1.702E-02	7.588E-04
0.085	1.284E-01	2.677E-03	2.060	1.250E-02	7.043E-04
0.095	5.010E-02	2.186E-03	2.100	1.076E-02	7.183E-04
0.105	5.354E-02	2.129E-03	2.140	1.544E-02	7.871E-04
0.115	6.295E-02	2.242E-03	2.180	1.884E-02	8.181E-04
0.125	7.612E-02	2.339E-03	2.220	1.602E-02	7.171E-04
0.135	9.793E-02	2.468E-03	2.260	9.253E-03	5.846E-04
0.145	1.298E-01	2.629E-03	2.300	6.081E-03	5.200E-04
0.155	1.303E-01	2.699E-03	2.340	7.181E-03	5.090E-04
0.165	1.286E-01	2.699E-03	2.380	7.562E-03	5.390E-04
0.177	1.920E-01	2.618E-03	2.425	6.113E-03	4.704E-04
0.192	2.559E-01	2.672E-03	2.475	6.509E-03	4.879E-04
0.207	1.571E-01	2.282E-03	2.525	8.351E-03	5.239E-04
0.222	1.101E-01	2.073E-03	2.575	9.563E-03	5.353E-04
0.237	8.177E-02	1.950E-03	2.625	9.408E-03	5.355E-04
0.252	1.105E-01	1.961E-03	2.675	6.488E-03	5.412E-04
0.267	1.364E-01	2.057E-03	2.725	4.218E-03	5.773E-04
0.282	1.609E-01	2.191E-03	2.775	4.046E-03	5.181E-04
0.297	2.536E-01	2.576E-03	2.825	2.891E-03	3.682E-04
0.313	3.199E-01	2.837E-03	2.875	1.375E-03	2.700E-04
0.327	2.241E-01	2.511E-03	2.925	1.036E-03	2.504E-04
0.342	1.395E-01	2.159E-03	2.975	1.427E-03	2.631E-04
0.357	1.090E-01	1.982E-03	3.030	1.929E-03	2.844E-04
0.372	8.265E-02	1.816E-03	3.090	1.576E-03	2.668E-04
0.387	6.264E-02	1.677E-03	3.150	1.619E-03	2.530E-04
0.402	6.768E-02	1.705E-03	3.210	1.847E-03	2.492E-04
0.417	8.695E-02	1.853E-03	3.270	1.863E-03	2.388E-04
0.432	1.016E-01	1.947E-03	3.330	1.721E-03	2.269E-04
0.447	1.286E-01	1.854E-03	3.390	1.409E-03	2.086E-04
0.462	1.247E-01	1.806E-03	3.450	1.750E-03	2.150E-04
0.477	1.011E-01	1.693E-03	3.510	3.339E-03	2.673E-04
0.492	7.196E-02	1.558E-03	3.570	3.743E-03	2.622E-04
0.507	6.038E-02	1.552E-03	3.630	1.932E-03	2.048E-04
0.522	6.683E-02	1.669E-03	3.690	6.512E-04	1.452E-04
0.540	7.551E-02	1.598E-03	3.750	4.351E-04	1.275E-04
0.560	9.231E-02	1.577E-03	3.810	4.642E-04	1.255E-04
0.580	1.147E-01	1.673E-03	3.870	6.787E-04	1.279E-04
0.600	1.195E-01	1.683E-03	3.935	6.635E-04	1.242E-04
0.620	1.061E-01	1.651E-03	4.005	3.553E-04	1.093E-04
0.640	9.822E-02	1.571E-03	4.075	2.610E-04	9.833E-05
0.660	8.309E-02	1.461E-03	4.145	2.021E-04	8.929E-05
0.680	6.266E-02	1.371E-03	4.215	2.215E-04	8.201E-05
0.700	6.613E-02	1.372E-03	4.285	2.611E-04	7.996E-05
0.720	8.162E-02	1.438E-03	4.355	1.797E-04	7.362E-05
0.740	8.905E-02	1.482E-03	4.425	1.546E-04	6.297E-05
0.760	9.133E-02	1.459E-03	4.495	1.904E-04	6.939E-05
0.780	8.981E-02	1.406E-03	4.565	1.099E-04	5.465E-05
0.800	9.003E-02	1.466E-03	4.635	4.192E-05	5.433E-05
0.820	1.098E-01	1.592E-03	4.705	4.558E-05	4.311E-05
0.840	1.422E-01	1.795E-03	4.775	3.074E-05	4.478E-05
0.860	1.642E-01	1.870E-03	4.845	-1.439E-05	3.356E-05
0.880	1.721E-01	1.758E-03	4.915	-1.036E-05	3.430E-05
0.900	1.745E-01	1.662E-03	4.985	1.623E-05	2.907E-05
0.920	1.656E-01	1.651E-03	5.060	2.044E-05	3.983E-05
0.940	1.386E-01	1.582E-03	5.140	-2.135E-06	4.044E-05
0.962	9.898E-02	1.378E-03	5.220	-4.788E-06	3.379E-05
0.987	7.984E-02	1.271E-03	5.300	2.110E-05	2.687E-05
1.013	8.148E-02	1.215E-03	5.380	3.831E-05	2.193E-05
1.037	6.988E-02	1.147E-03	5.460	4.039E-05	2.034E-05
1.062	5.268E-02	1.055E-03	5.540	4.116E-05	3.379E-05
1.088	4.460E-02	1.025E-03	5.620	6.294E-05	4.231E-05
1.112	4.485E-02	9.928E-04	5.700	5.075E-05	3.084E-05
1.138	4.501E-02	9.934E-04	5.780	1.625E-05	2.476E-05
1.162	4.433E-02	9.651E-04	5.860	-2.250E-05	2.215E-05
1.187	4.638E-02	1.009E-03	5.945	-3.719E-05	1.000E-05
1.215	5.052E-02	1.072E-03	6.035	2.689E-05	2.868E-05
1.245	5.375E-02	1.132E-03	6.125	6.261E-05	3.621E-05
1.275	4.877E-02	1.458E-03	6.215	6.029E-05	3.161E-05
1.305	4.504E-02	1.081E-03	6.305	3.377E-05	2.234E-05
1.335	4.294E-02	1.162E-03	6.395	7.107E-08	1.811E-05
1.365	4.678E-02	1.051E-03	6.485	-3.773E-05	1.324E-05
1.395	4.785E-02	9.870E-04	6.575	-3.066E-05	2.467E-05
1.425	3.956E-02	8.796E-04	6.665	-1.881E-05	2.873E-05
1.455	2.816E-02	7.529E-04	6.755	-4.856E-06	2.158E-05
1.485	2.226E-02	6.717E-04	6.850	1.943E-05	2.370E-05
1.515	2.194E-02	8.956E-04	6.950	1.536E-05	1.458E-05
1.545	2.032E-02	9.186E-04	7.050	6.560E-06	2.125E-05
1.580	1.725E-02	9.154E-04	7.150	8.467E-06	2.541E-05
1.620	1.777E-02	9.282E-04	7.250	1.974E-05	2.084E-05
1.660	2.203E-02	9.661E-04	7.350	2.772E-05	1.808E-05
1.700	2.106E-02	9.303E-04	7.450	3.432E-05	1.762E-05
1.740	1.786E-02	9.196E-04	7.550	2.500E-05	2.019E-05
1.780	1.537E-02	9.138E-04	7.650	1.327E-05	1.363E-05
1.820	1.395E-02	8.422E-04	7.750	3.936E-06	7.915E-06
1.860	1.408E-02	7.775E-04	7.850	1.506E-06	6.972E-06
1.900	1.394E-02	7.390E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1550 SEC AFTER END OF IRRADIATION
COUNT FOR 400 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.776E-03	1.826E-03	1.940	1.084E-02	6.361E-04
0.065	5.880E-03	2.029E-03	1.980	1.163E-02	6.225E-04
0.075	8.716E-02	2.516E-03	2.020	1.244E-02	6.541E-04
0.085	1.025E-01	2.487E-03	2.060	9.450E-03	6.237E-04
0.095	4.120E-02	1.999E-03	2.100	7.465E-03	6.529E-04
0.105	4.014E-02	1.943E-03	2.140	1.146E-02	6.995E-04
0.115	4.594E-02	2.048E-03	2.180	1.456E-02	7.262E-04
0.125	5.622E-02	2.136E-03	2.220	1.210E-02	6.519E-04
0.135	7.737E-02	2.235E-03	2.260	8.242E-03	5.362E-04
0.145	1.079E-01	2.439E-03	2.300	4.908E-03	4.588E-04
0.155	1.104E-01	2.469E-03	2.340	4.632E-03	4.462E-04
0.165	9.650E-02	2.445E-03	2.380	5.817E-03	4.885E-04
0.177	1.524E-01	2.378E-03	2.425	5.217E-03	4.364E-04
0.192	1.945E-01	2.383E-03	2.475	4.884E-03	4.265E-04
0.207	1.256E-01	2.073E-03	2.525	7.035E-03	4.798E-04
0.222	8.524E-02	1.869E-03	2.575	7.882E-03	4.849E-04
0.237	5.967E-02	1.742E-03	2.625	6.529E-03	4.635E-04
0.252	7.682E-02	1.744E-03	2.675	5.517E-03	4.807E-04
0.267	1.047E-01	1.841E-03	2.725	3.178E-03	5.074E-04
0.282	1.331E-01	2.004E-03	2.775	2.161E-03	4.563E-04
0.297	2.134E-01	2.373E-03	2.825	1.751E-03	3.318E-04
0.313	2.638E-01	2.608E-03	2.875	1.073E-03	2.469E-04
0.327	1.752E-01	2.276E-03	2.925	1.004E-03	2.187E-04
0.342	1.042E-01	1.947E-03	2.975	1.059E-03	2.300E-04
0.357	8.028E-02	1.772E-03	3.030	9.651E-04	2.344E-04
0.372	5.861E-02	1.603E-03	3.090	9.905E-04	2.151E-04
0.387	4.532E-02	1.492E-03	3.150	1.052E-03	2.036E-04
0.402	5.132E-02	1.521E-03	3.210	1.203E-03	2.034E-04
0.417	6.555E-02	1.648E-03	3.270	1.279E-03	2.094E-04
0.432	8.005E-02	1.747E-03	3.330	9.991E-04	1.770E-04
0.447	1.054E-01	1.691E-03	3.390	1.002E-03	1.767E-04
0.462	1.062E-01	1.661E-03	3.450	1.452E-03	1.870E-04
0.477	8.334E-02	1.523E-03	3.510	2.390E-03	2.217E-04
0.492	5.666E-02	1.369E-03	3.570	2.526E-03	2.220E-04
0.507	4.623E-02	1.375E-03	3.630	1.410E-03	1.580E-04
0.522	5.325E-02	1.500E-03	3.690	5.291E-04	1.290E-04
0.540	6.248E-02	1.446E-03	3.750	2.158E-04	1.005E-04
0.560	7.104E-02	1.410E-03	3.810	2.025E-04	9.746E-05
0.580	8.163E-02	1.461E-03	3.870	3.993E-04	9.985E-05
0.600	8.416E-02	1.470E-03	3.935	5.347E-04	1.113E-04
0.620	8.063E-02	1.478E-03	4.005	3.663E-04	8.887E-05
0.640	8.097E-02	1.438E-03	4.075	1.657E-04	8.350E-05
0.660	6.719E-02	1.336E-03	4.145	7.491E-05	6.852E-05
0.680	4.683E-02	1.217E-03	4.215	5.796E-05	6.703E-05
0.700	4.721E-02	1.208E-03	4.285	9.957E-05	5.960E-05
0.720	6.174E-02	1.258E-03	4.355	1.294E-04	6.371E-05
0.740	7.180E-02	1.327E-03	4.425	1.217E-04	5.329E-05
0.760	7.383E-02	1.314E-03	4.495	8.470E-05	5.335E-05
0.780	6.910E-02	1.264E-03	4.565	1.908E-05	5.369E-05
0.800	6.625E-02	1.303E-03	4.635	-6.519E-06	3.405E-05
0.820	8.614E-02	1.436E-03	4.705	3.315E-05	3.559E-05
0.840	1.195E-01	1.646E-03	4.775	3.778E-05	1.736E-05
0.860	1.364E-01	1.715E-03	4.845	1.666E-05	2.596E-05
0.880	1.403E-01	1.608E-03	4.915	2.819E-05	2.207E-05
0.900	1.411E-01	1.507E-03	4.985	5.760E-05	3.365E-05
0.920	1.300E-01	1.448E-03	5.060	4.304E-05	2.894E-05
0.940	1.068E-01	1.394E-03	5.140	7.819E-06	2.132E-05
0.962	7.511E-02	1.210E-03	5.220	-1.914E-05	1.619E-05
0.987	6.073E-02	1.118E-03	5.300	-4.030E-05	1.353E-05
1.013	6.046E-02	1.069E-03	5.380	-2.592E-05	2.286E-05
1.037	5.225E-02	1.010E-03	5.460	-1.299E-05	2.042E-05
1.062	3.955E-02	9.405E-04	5.540	-1.038E-05	2.399E-05
1.088	3.350E-02	9.191E-04	5.620	-1.681E-05	2.062E-05
1.112	3.425E-02	8.945E-04	5.700	-1.824E-05	8.800E-06
1.138	3.421E-02	8.983E-04	5.780	-4.840E-06	1.241E-05
1.162	3.263E-02	8.689E-04	5.860	-1.394E-05	1.659E-05
1.187	3.360E-02	8.902E-04	5.945	-2.325E-05	2.022E-05
1.215	3.603E-02	4.125E-04	6.035	1.803E-05	2.408E-05
1.245	3.920E-02	1.001E-03	6.125	5.211E-05	1.444E-05
1.275	3.638E-02	1.109E-03	6.215	5.084E-05	1.026E-05
1.305	3.467E-02	5.974E-04	6.305	2.725E-05	2.068E-05
1.335	3.040E-02	2.783E-03	6.395	-5.409E-06	1.718E-05
1.365	3.881E-02	9.800E-04	6.485	-1.496E-05	2.584E-05
1.395	4.077E-02	9.218E-04	6.575	-1.966E-05	1.834E-05
1.425	3.323E-02	8.122E-04	6.665	-9.013E-06	1.765E-05
1.455	2.447E-02	6.899E-04	6.755	1.350E-05	1.905E-05
1.485	1.867E-02	6.199E-04	6.850	1.770E-05	2.387E-05
1.515	1.597E-02	7.786E-04	6.950	1.328E-05	1.955E-05
1.545	1.521E-02	8.117E-04	7.050	1.881E-05	1.884E-05
1.580	1.430E-02	8.224E-04	7.150	2.160E-05	1.581E-05
1.620	1.308E-02	8.240E-04	7.250	1.013E-05	1.690E-05
1.660	1.501E-02	8.299E-04	7.350	1.405E-06	1.985E-05
1.700	1.520E-02	8.213E-04	7.450	-9.415E-07	1.567E-05
1.740	1.300E-02	8.112E-04	7.550	5.862E-06	1.337E-05
1.780	1.075E-02	8.192E-04	7.650	9.551E-06	1.040E-05
1.820	1.051E-02	7.559E-04	7.750	4.529E-06	7.521E-06
1.860	1.098E-02	7.023E-04	7.850	1.042E-06	6.278E-06
1.900	1.112E-02	6.487E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 1950 SEC AFTER END OF IRRADIATION
COUNT FOR 500 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	2.075E-03	1.895E-03	1.940	9.647E-03	6.151E-04
0.065	4.432E-03	2.153E-03	1.980	1.109E-02	6.310E-04
0.075	8.150E-02	2.646E-03	2.020	1.137E-02	6.599E-04
0.085	1.023E-01	2.595E-03	2.060	8.276E-03	6.475E-04
0.095	3.544E-02	2.094E-03	2.100	7.220E-03	6.559E-04
0.105	4.235E-02	2.022E-03	2.140	1.076E-02	7.210E-04
0.115	4.001E-02	2.113E-03	2.180	1.334E-02	7.331E-04
0.125	5.711E-02	2.203E-03	2.220	1.147E-02	6.450E-04
0.135	7.642E-02	2.303E-03	2.260	6.931E-03	5.469E-04
0.145	1.137E-01	2.528E-03	2.300	4.580E-03	4.791E-04
0.155	1.198E-01	2.570E-03	2.340	5.390E-03	4.599E-04
0.165	1.339E-01	2.500E-03	2.380	5.565E-03	4.908E-04
0.177	1.507E-01	2.405E-03	2.425	4.436E-03	4.312E-04
0.192	1.863E-01	2.383E-03	2.475	5.189E-03	4.665E-04
0.207	1.236E-01	2.111E-03	2.525	6.573E-03	4.843E-04
0.222	8.471E-02	1.906E-03	2.575	7.550E-03	4.790E-04
0.237	5.650E-02	1.761E-03	2.625	7.263E-03	4.743E-04
0.252	6.981E-02	1.726E-03	2.675	4.947E-03	5.168E-04
0.267	1.017E-01	1.860E-03	2.725	2.467E-03	5.462E-04
0.282	1.293E-01	2.047E-03	2.775	1.889E-03	4.868E-04
0.297	2.179E-01	2.437E-03	2.825	1.684E-03	3.611E-04
0.313	2.641E-01	2.634E-03	2.875	1.332E-03	2.684E-04
0.327	1.698E-01	2.282E-03	2.925	1.333E-03	2.419E-04
0.342	9.832E-02	1.939E-03	2.975	1.335E-03	2.351E-04
0.357	7.626E-02	1.804E-03	3.030	1.138E-03	2.324E-04
0.372	5.717E-02	1.638E-03	3.090	1.204E-03	2.094E-04
0.387	4.266E-02	1.509E-03	3.150	1.752E-03	2.099E-04
0.402	4.752E-02	1.544E-03	3.210	1.008E-03	1.855E-04
0.417	6.349E-02	1.653E-03	3.270	1.006E-03	1.829E-04
0.432	8.514E-02	1.799E-03	3.330	7.016E-04	1.788E-04
0.447	1.112E-01	1.747E-03	3.390	8.107E-04	1.765E-04
0.462	1.117E-01	1.715E-03	3.450	1.197E-03	1.797E-04
0.477	8.369E-02	1.543E-03	3.510	1.581E-03	2.035E-04
0.492	5.558E-02	1.399E-03	3.570	1.933E-03	1.934E-04
0.507	4.673E-02	1.413E-03	3.630	1.341E-03	1.526E-04
0.522	5.340E-02	1.536E-03	3.690	5.043E-04	1.327E-04
0.540	6.413E-02	1.487E-03	3.750	3.989E-04	1.138E-04
0.560	7.152E-02	1.434E-03	3.810	5.791E-04	1.202E-04
0.580	7.459E-02	1.442E-03	3.870	6.353E-04	1.110E-04
0.600	7.371E-02	1.444E-03	3.935	5.256E-04	1.110E-04
0.620	7.936E-02	1.488E-03	4.005	2.582E-04	9.742E-05
0.640	8.377E-02	1.475E-03	4.075	1.139E-04	8.397E-05
0.660	7.027E-02	1.367E-03	4.145	7.604E-05	7.019E-05
0.680	4.620E-02	1.237E-03	4.215	2.821E-05	5.770E-05
0.700	4.409E-02	1.197E-03	4.285	9.113E-06	5.775E-05
0.720	5.945E-02	1.258E-03	4.355	3.982E-05	5.440E-05
0.740	7.349E-02	1.338E-03	4.425	8.517E-05	5.479E-05
0.760	7.783E-02	1.352E-03	4.495	7.365E-05	5.404E-05
0.780	6.827E-02	1.281E-03	4.565	1.354E-05	4.804E-05
0.800	6.552E-02	1.329E-03	4.635	-9.231E-06	4.100E-05
0.820	8.990E-02	1.487E-03	4.705	1.318E-05	4.667E-05
0.840	1.295E-01	1.705E-03	4.775	5.108E-05	4.901E-05
0.860	1.493E-01	1.806E-03	4.845	6.443E-05	5.314E-05
0.880	1.503E-01	1.657E-03	4.915	4.277E-05	4.754E-05
0.900	1.440E-01	1.518E-03	4.985	1.811E-05	4.269E-05
0.920	1.270E-01	1.443E-03	5.060	1.341E-06	3.945E-05
0.940	1.011E-01	1.369E-03	5.140	-3.517E-06	3.628E-05
0.962	6.992E-02	1.192E-03	5.220	1.587E-05	4.528E-05
0.987	5.645E-02	1.097E-03	5.300	3.191E-05	4.450E-05
1.013	5.843E-02	1.067E-03	5.380	3.292E-05	3.932E-05
1.037	4.988E-02	1.009E-03	5.460	5.117E-05	4.092E-05
1.062	3.668E-02	9.410E-04	5.540	6.590E-05	4.217E-05
1.088	3.107E-02	9.357E-04	5.620	7.016E-05	2.746E-05
1.112	3.316E-02	9.154E-04	5.700	6.868E-05	3.013E-05
1.138	3.457E-02	9.234E-04	5.780	3.471E-05	3.053E-05
1.162	3.096E-02	8.812E-04	5.860	-6.607E-06	2.836E-05
1.187	3.026E-02	8.731E-04	5.945	-3.189E-05	2.348E-05
1.215	3.287E-02	8.886E-04	6.035	-3.628E-05	3.684E-05
1.245	3.712E-02	9.966E-04	6.125	-5.653E-05	1.229E-05
1.275	3.500E-02	1.595E-03	6.215	-9.910E-06	2.620E-05
1.305	3.569E-02	1.573E-03	6.305	2.364E-05	2.575E-05
1.335	3.361E-02	1.520E-03	6.395	4.563E-05	2.712E-05
1.365	4.227E-02	1.026E-03	6.485	3.624E-05	2.535E-05
1.395	4.495E-02	9.677E-04	6.575	1.596E-05	2.113E-05
1.425	3.709E-02	8.432E-04	6.665	-1.988E-07	1.610E-05
1.455	2.591E-02	7.112E-04	6.755	-2.521E-05	1.563E-05
1.485	1.940E-02	6.295E-04	6.850	-1.889E-05	2.352E-05
1.515	1.774E-02	8.063E-04	6.950	-1.206E-05	2.205E-05
1.545	1.524E-02	8.160E-04	7.050	-1.910E-05	2.069E-05
1.580	1.298E-02	6.154E-04	7.150	-2.216E-05	1.465E-05
1.620	1.310E-02	8.229E-04	7.250	-2.322E-06	1.255E-05
1.660	1.375E-02	8.229E-04	7.350	6.617E-06	1.677E-05
1.700	1.374E-02	7.978E-04	7.450	1.526E-05	1.355E-05
1.740	1.255E-02	8.293E-04	7.550	3.011E-05	1.572E-05
1.780	1.127E-02	8.264E-04	7.650	2.145E-05	1.064E-05
1.820	1.012E-02	7.749E-04	7.750	5.480E-06	7.365E-06
1.860	1.021E-02	7.038E-04	7.850	1.223E-06	6.265E-06
1.900	1.039E-02	6.618E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 2450 SEC AFTER END OF IRRADIATION
COUNT FOR 500 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.482E-03	1.814E-03	1.940	6.911E-03	5.377E-04
0.065	7.617E-03	2.[illegible]E-03	1.980	7.894E-03	5.501E-04
0.075	6.570E-02	2.[illegible]E-03	2.020	7.917E-03	5.748E-04
0.085	8.031E-02	2.382E-03	2.060	6.655E-03	5.876E-04
0.095	2.967E-02	2.004E-03	2.100	5.946E-03	6.196E-04
0.105	3.091E-02	1.903E-03	2.140	8.831E-03	6.345E-04
0.115	3.187E-02	1.972E-03	2.180	1.105E-02	6.606E-04
0.125	4.266E-02	2.048E-03	2.220	9.228E-03	5.797E-04
0.135	6.462E-02	2.173E-03	2.260	5.525E-03	4.954E-04
0.145	9.904E-02	2.385E-03	2.300	3.717E-03	4.417E-04
0.155	1.008E-01	2.432E-03	2.340	4.525E-03	4.351E-04
0.165	8.328E-02	2.318E-03	2.380	5.405E-03	4.538E-04
0.177	1.139E-01	2.191E-03	2.425	4.[illegible]E-03	4.189E-04
0.192	1.458E-01	2.180E-03	2.475	4.476E-03	4.416E-04
0.207	9.872E-02	1.920E-03	2.525	5.965E-03	4.550E-04
0.222	6.384E-02	1.736E-03	2.575	5.287E-03	4.299E-04
0.237	4.136E-02	1.638E-03	2.625	4.819E-03	4.235E-04
0.252	5.309E-02	1.595E-03	2.675	3.364E-03	4.878E-04
0.267	7.674E-02	1.702E-03	2.725	2.011E-03	5.238E-04
0.282	1.049E-01	1.879E-03	2.775	1.892E-03	4.594E-04
0.297	1.704E-01	2.223E-03	2.825	1.600E-03	3.301E-04
0.313	2.005E-01	2.362E-03	2.875	1.027E-03	2.198E-04
0.327	1.296E-01	2.057E-03	2.925	1.112E-03	2.071E-04
0.342	7.326E-02	1.774E-03	2.975	1.406E-03	2.222E-04
0.357	5.441E-02	1.618E-03	3.030	1.137E-03	2.040E-04
0.372	4.024E-02	1.481E-03	3.090	9.978E-04	1.925E-04
0.387	3.102E-02	1.373E-03	3.150	7.733E-04	1.713E-04
0.402	4.013E-02	1.425E-03	3.210	4.839E-04	1.608E-04
0.417	5.200E-02	1.535E-03	3.270	7.757E-04	1.676E-04
0.432	7.016E-02	1.659E-03	3.330	7.290E-04	1.530E-04
0.447	9.111E-02	1.598E-03	3.390	6.982E-04	1.581E-04
0.462	9.262E-02	1.564E-03	3.450	8.597E-04	1.514E-04
0.477	6.811E-02	1.406E-03	3.510	1.077E-03	1.580E-04
0.492	4.622E-02	1.265E-03	3.570	1.238E-03	1.673E-04
0.507	3.760E-02	1.285E-03	3.630	7.755E-04	1.351E-04
0.522	4.343E-02	1.411E-03	3.690	3.858E-04	1.177E-04
0.540	5.346E-02	1.379E-03	3.750	3.161E-04	1.112E-04
0.560	5.869E-02	1.328E-03	3.810	3.909E-04	1.059E-04
0.580	5.770E-02	1.317E-03	3.870	4.977E-04	1.120E-04
0.600	5.496E-02	1.304E-03	3.935	3.583E-04	9.985E-05
0.620	6.292E-02	1.365E-03	4.005	2.123E-04	7.868E-05
0.640	7.151E-02	1.380E-03	4.075	2.603E-04	7.439E-05
0.660	5.971E-02	1.279E-03	4.145	1.405E-04	6.048E-05
0.680	3.891E-02	1.141E-03	4.215	1.375E-05	5.971E-05
0.700	3.576E-02	1.093E-03	4.285	3.620E-05	5.282E-05
0.720	4.739E-02	1.140E-03	4.355	1.120E-04	5.111E-05
0.740	6.118E-02	1.237E-03	4.425	1.225E-04	5.268E-05
0.760	6.462E-02	1.247E-03	4.495	3.507E-05	4.621E-05
0.780	5.537E-02	1.181E-03	4.565	-1.231E-05	4.107E-05
0.800	5.429E-02	1.227E-03	4.635	2.968E-05	4.117E-05
0.820	7.839E-02	1.407E-03	4.705	7.354E-05	4.308E-05
0.840	1.148E-01	1.646E-03	4.775	2.894E-05	3.581E-05
0.860	1.349E-01	1.710E-03	4.845	-5.085E-05	3.624E-05
0.880	1.303E-01	1.550E-03	4.915	-8.209E-05	2.375E-05
0.900	1.175E-01	1.379E-03	4.985	-3.922E-05	3.093E-05
0.920	1.026E-01	1.308E-03	5.060	7.124E-06	2.874E-05
0.940	7.826E-02	1.209E-03	5.140	1.481E-05	4.145E-05
0.962	5.277E-02	1.049E-03	5.220	1.900E-05	4.187E-05
0.987	4.364E-02	9.741E-04	5.300	1.135E-05	3.727E-05
1.013	4.332E-02	9.415E-04	5.380	2.982E-05	3.595E-05
1.037	3.737E-02	9.089E-04	5.460	4.969E-05	2.752E-05
1.062	3.050E-02	8.689E-04	5.540	5.873E-05	2.906E-05
1.088	2.617E-02	8.689E-04	5.620	5.411E-05	4.345E-05
1.112	2.650E-02	8.502E-04	5.700	3.443E-05	3.744E-05
1.138	2.696E-02	8.443E-04	5.780	1.374E-05	3.495E-05
1.162	2.325E-02	7.941E-04	5.860	4.706E-06	2.836E-05
1.187	2.116E-02	7.860E-04	5.945	2.622E-06	2.955E-05
1.215	2.333E-02	7.818E-04	6.035	5.305E-06	2.392E-05
1.245	2.727E-02	8.785E-04	6.125	1.353E-05	3.067E-05
1.275	2.692E-02	9.330E-04	6.215	6.877E-06	2.309E-05
1.305	2.735E-02	9.431E-04	6.305	3.292E-06	1.789E-05
1.335	2.979E-02	8.138E-04	6.395	-2.624E-07	2.625E-05
1.365	3.746E-02	9.747E-04	6.485	-4.260E-06	2.302E-05
1.395	3.706E-02	9.052E-04	6.575	4.455E-06	2.700E-05
1.425	3.197E-02	7.919E-04	6.665	3.018E-05	2.780E-05
1.455	2.174E-02	6.621E-04	6.755	1.784E-05	1.605E-05
1.485	1.582E-02	5.867E-04	6.850	2.592E-06	2.152E-05
1.515	1.483E-02	7.369E-04	6.950	-1.259E-05	2.192E-05
1.545	1.149E-02	7.294E-04	7.050	-1.215E-05	2.362E-05
1.580	9.823E-03	7.221E-04	7.150	4.426E-06	2.601E-05
1.620	9.830E-03	7.280E-04	7.250	2.383E-05	1.914E-05
1.660	1.175E-02	7.380E-04	7.350	2.954E-05	1.874E-05
1.700	1.209E-02	7.438E-04	7.450	3.155E-05	2.055E-05
1.740	9.932E-03	7.579E-04	7.550	2.472E-05	1.904E-05
1.780	9.129E-03	7.653E-04	7.650	9.548E-06	1.318E-05
1.820	8.021E-03	7.192E-04	7.750	1.577E-06	7.672E-06
1.860	8.207E-03	6.294E-04	7.850	1.428E-06	6.389E-06
1.900	7.057E-03	5.746E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 2950 SEC AFTER END OF IRRADIATION
COUNT FOR 1000 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.017E-03	2.373E-03	1.940	1.036E-02	6.886E-04
0.065	1.537E-02	2.702E-03	1.980	1.050E-02	6.805E-04
0.075	8.928E-02	3.222E-03	2.020	1.075E-02	7.166E-04
0.085	1.195E-01	3.121E-03	2.060	8.623E-03	7.438E-04
0.095	4.725E-02	2.616E-03	2.100	7.321E-03	7.840E-04
0.105	4.759E-02	2.455E-03	2.140	1.251E-02	8.197E-04
0.115	5.179E-02	2.545E-03	2.180	1.634E-02	8.432E-04
0.125	6.164E-02	2.667E-03	2.220	1.257E-02	7.449E-04
0.135	9.639E-02	2.774E-03	2.260	7.007E-03	6.496E-04
0.145	1.593E-01	3.046E-03	2.300	5.323E-03	5.519E-04
0.155	1.628E-01	3.131E-03	2.340	7.712E-03	5.636E-04
0.165	1.411E-01	2.998E-03	2.380	8.726E-03	6.125E-04
0.177	1.761E-01	2.763E-03	2.425	6.692E-03	5.608E-04
0.192	2.017E-01	2.704E-03	2.475	6.867E-03	5.583E-04
0.207	1.521E-01	2.442E-03	2.525	8.076E-03	5.732E-04
0.222	9.168E-02	2.234E-03	2.575	8.283E-03	5.407E-04
0.237	5.696E-02	2.080E-03	2.625	6.883E-03	5.348E-04
0.252	7.133E-02	2.013E-03	2.675	4.162E-03	6.483E-04
0.267	1.070E-01	2.135E-03	2.725	3.619E-03	7.191E-04
0.282	1.462E-01	2.330E-03	2.775	3.611E-03	6.438E-04
0.297	2.346E-01	2.688E-03	2.825	2.099E-03	4.436E-04
0.313	2.728E-01	2.827E-03	2.875	1.604E-03	2.884E-04
0.327	1.701E-01	2.474E-03	2.925	2.066E-03	2.805E-04
0.342	9.957E-02	2.204E-03	2.975	2.024E-03	2.757E-04
0.357	7.715E-02	2.053E-03	3.030	1.786E-03	2.495E-04
0.372	5.664E-02	1.878E-03	3.090	1.512E-03	2.369E-04
0.387	4.450E-02	1.743E-03	3.150	8.985E-04	2.084E-04
0.402	5.685E-02	1.814E-03	3.210	1.114E-03	2.099E-04
0.417	7.882E-02	1.957E-03	3.270	1.196E-03	2.069E-04
0.432	1.070E-01	2.113E-03	3.330	1.066E-03	2.014E-04
0.447	1.452E-01	2.047E-03	3.390	9.944E-04	1.901E-04
0.462	1.423E-01	2.009E-03	3.450	9.983E-04	1.868E-04
0.477	1.040E-01	1.813E-03	3.510	1.363E-03	1.947E-04
0.492	6.505E-02	1.665E-03	3.570	1.475E-03	1.935E-04
0.507	5.442E-02	1.708E-03	3.630	1.100E-03	1.697E-04
0.522	7.049E-02	1.881E-03	3.690	6.204E-04	1.465E-04
0.540	8.801E-02	1.808E-03	3.750	4.334E-04	1.209E-04
0.560	8.706E-02	1.684E-03	3.810	6.466E-04	1.285E-04
0.580	8.249E-02	1.654E-03	3.870	8.801E-04	1.253E-04
0.600	8.349E-02	1.651E-03	3.935	6.572E-04	8.397E-05
0.620	1.011E-01	1.748E-03	4.005	1.701E-04	7.336E-05
0.640	1.179E-01	1.790E-03	4.075	-8.561E-06	4.117E-05
0.660	9.608E-02	1.638E-03	4.145	6.900E-05	4.071E-05
0.680	6.196E-02	1.467E-03	4.215	1.052E-04	4.183E-05
0.700	5.623E-02	1.411E-03	4.285	1.418E-05	4.218E-05
0.720	7.538E-02	1.467E-03	4.355	3.038E-05	5.601E-05
0.740	9.926E-02	1.597E-03	4.425	1.145E-04	5.311E-05
0.760	1.048E-01	1.610E-03	4.495	1.228E-04	6.510E-05
0.780	8.876E-02	1.520E-03	4.565	4.177E-05	5.826E-05
0.800	8.949E-02	1.622E-03	4.635	-2.885E-05	6.664E-05
0.820	1.393E-01	1.860E-03	4.705	-9.442E-05	2.904E-05
0.840	2.118E-01	2.191E-03	4.775	-3.883E-05	5.854E-05
0.860	2.415E-01	2.282E-03	4.845	-9.186E-06	5.728E-05
0.880	2.196E-01	1.999E-03	4.915	-3.255E-05	5.159E-05
0.900	1.878E-01	1.763E-03	4.985	-2.802E-05	4.319E-05
0.920	1.526E-01	1.608E-03	5.060	3.233E-05	4.785E-05
0.940	1.111E-01	1.496E-03	5.140	5.224E-05	3.987E-05
0.962	7.456E-02	1.303E-03	5.220	5.411E-05	5.771E-05
0.987	6.159E-02	1.218E-03	5.300	4.658E-05	5.595E-05
1.013	6.454E-02	1.191E-03	5.380	7.810E-05	6.215E-05
1.037	5.512E-02	1.148E-03	5.460	6.184E-05	4.867E-05
1.062	4.385E-02	1.112E-03	5.540	8.596E-05	3.881E-05
1.088	4.071E-02	1.140E-03	5.620	1.065E-04	5.815E-05
1.112	4.304E-02	1.143E-03	5.700	9.101E-05	5.004E-05
1.138	4.090E-02	1.102E-03	5.780	-3.023E-05	5.063E-05
1.162	3.301E-02	1.017E-03	5.860	-4.123E-05	4.238E-05
1.187	2.820E-02	9.848E-04	5.945	-3.467E-05	4.548E-05
1.215	2.826E-02	1.458E-03	6.035	-3.814E-05	2.641E-05
1.245	3.391E-02	1.108E-03	6.125	3.098E-05	4.423E-05
1.275	3.289E-02	1.234E-03	6.215	1.016E-04	3.930E-05
1.305	3.422E-02	1.774E-03	6.305	8.659E-05	3.130E-05
1.335	4.488E-02	1.231E-03	6.395	3.602E-05	4.515E-05
1.365	6.198E-02	1.265E-03	6.485	-2.481E-06	3.986E-05
1.395	6.403E-02	1.197E-03	6.575	-2.706E-05	3.463E-05
1.425	5.344E-02	1.031E-03	6.665	-5.763E-05	3.398E-05
1.455	3.579E-02	8.448E-04	6.755	-6.981E-05	2.315E-05
1.485	2.424E-02	7.299E-04	6.850	-1.677E-05	3.371E-05
1.515	2.104E-02	9.196E-04	6.950	3.770E-05	3.866E-05
1.545	1.687E-02	9.193E-04	7.050	6.308E-05	3.555E-05
1.580	1.343E-02	9.143E-04	7.150	4.125E-05	2.999E-05
1.620	1.371E-02	9.122E-04	7.250	7.048E-06	2.784E-05
1.660	1.673E-02	9.378E-04	7.350	2.508E-05	2.661E-05
1.700	1.609E-02	9.298E-04	7.450	1.158E-05	2.650E-05
1.740	1.443E-02	9.768E-04	7.550	5.333E-06	2.660E-05
1.780	1.356E-02	1.024E-03	7.650	1.011E-06	1.741E-05
1.820	1.208E-02	9.485E-04	7.750	5.723E-07	8.002E-06
1.860	1.160E-02	8.192E-04	7.850	1.966E-06	6.502E-06
1.900	1.111E-02	7.221E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 3950 SEC AFTER END OF IRRADIATION
COUNT FOR 2000 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	3.927E-03	3.127E-03	1.940	1.082E-02	7.959E-04
0.065	2.024E-02	3.584E-03	1.980	1.011E-02	7.805E-04
0.075	1.084E-01	4.187E-03	2.020	9.694E-03	8.554E-04
0.085	1.456E-01	3.986E-03	2.060	8.569E-03	8.883E-04
0.095	5.681E-02	3.357E-03	2.100	7.982E-03	9.763E-04
0.105	5.377E-02	3.119E-03	2.140	1.281E-02	9.923E-04
0.115	5.567E-02	3.235E-03	2.180	1.883E-02	9.688E-04
0.125	7.618E-02	3.349E-03	2.220	8.440E-03	8.796E-04
0.135	1.229E-01	3.479E-03	2.260	8.536E-03	8.061E-04
0.145	1.970E-01	3.719E-03	2.300	7.301E-03	7.086E-04
0.155	2.182E-01	3.815E-03	2.340	9.796E-03	6.967E-04
0.165	1.976E-01	3.719E-03	2.380	1.277E-02	7.914E-04
0.177	2.109E-01	3.318E-03	2.425	1.044E-02	7.451E-04
0.192	2.223E-01	3.163E-03	2.475	8.476E-03	7.390E-04
0.207	1.891E-01	2.960E-03	2.525	1.018E-02	7.270E-04
0.222	1.193E-01	2.795E-03	2.575	8.880E-03	6.567E-04
0.237	7.647E-02	2.654E-03	2.625	6.998E-03	6.675E-04
0.252	8.513E-02	2.503E-03	2.675	5.377E-03	8.834E-04
0.267	1.253E-01	2.576E-03	2.725	3.912E-03	9.545E-04
0.282	1.700E-01	2.768E-03	2.775	3.684E-03	8.516E-04
0.297	2.298E-01	2.998E-03	2.825	3.118E-03	6.019E-04
0.313	2.606E-01	3.073E-03	2.875	2.426E-03	3.842E-04
0.327	1.697E-01	2.800E-03	2.925	2.950E-03	3.312E-04
0.342	1.025E-01	2.649E-03	2.975	3.776E-03	3.657E-04
0.357	8.082E-02	2.504E-03	3.030	2.326E-03	3.080E-04
0.372	5.633E-02	2.270E-03	3.090	9.352E-04	2.626E-04
0.387	5.837E-02	2.176E-03	3.150	1.061E-03	2.415E-04
0.402	7.757E-02	2.246E-03	3.210	1.424E-03	2.510E-04
0.417	1.043E-01	2.437E-03	3.270	1.605E-03	2.582E-04
0.432	1.419E-01	2.576E-03	3.330	1.315E-03	2.419E-04
0.447	1.798E-01	2.447E-03	3.390	1.073E-03	2.438E-04
0.462	1.726E-01	2.373E-03	3.450	8.309E-04	2.082E-04
0.477	1.264E-01	2.207E-03	3.510	9.104E-04	2.018E-04
0.492	8.300E-02	2.098E-03	3.570	1.279E-03	2.272E-04
0.507	7.064E-02	2.182E-03	3.630	1.317E-03	2.079E-04
0.522	9.421E-02	2.367E-03	3.690	7.174E-04	1.777E-04
0.540	1.225E-01	2.250E-03	3.750	3.598E-04	1.635E-04
0.560	1.201E-01	2.089E-03	3.810	5.001E-04	1.764E-04
0.580	1.051E-01	2.024E-03	3.870	8.108E-04	1.842E-04
0.600	1.099E-01	2.041E-03	3.935	1.001E-03	1.791E-04
0.620	1.438E-01	2.186E-03	4.005	6.886E-04	1.496E-04
0.640	1.692E-01	2.196E-03	4.075	2.725E-04	1.240E-04
0.660	1.393E-01	2.031E-03	4.145	3.369E-05	1.030E-04
0.680	8.650E-02	1.826E-03	4.215	-4.473E-05	9.313E-05
0.700	7.123E-02	1.720E-03	4.285	-3.697E-05	9.438E-05
0.720	9.992E-02	1.758E-03	4.355	2.543E-05	1.059E-04
0.740	1.399E-01	1.945E-03	4.425	8.766E-05	1.106E-04
0.760	1.439E-01	1.961E-03	4.495	8.042E-05	9.747E-05
0.780	1.211E-01	1.865E-03	4.565	5.906E-05	9.132E-05
0.800	1.341E-01	2.057E-03	4.635	1.678E-05	9.881E-05
0.820	2.321E-01	2.431E-03	4.705	-6.215E-07	9.661E-05
0.840	3.660E-01	2.864E-03	4.775	1.076E-04	9.638E-05
0.860	4.029E-01	2.998E-03	4.845	2.138E-04	1.003E-04
0.880	3.474E-01	2.560E-03	4.915	1.351E-04	8.689E-05
0.900	2.671E-01	2.111E-03	4.985	-2.463E-05	8.628E-05
0.920	1.894E-01	1.865E-03	5.060	-6.227E-05	5.147E-05
0.940	1.305E-01	1.710E-03	5.140	4.772E-05	5.555E-05
0.962	8.713E-02	1.507E-03	5.220	1.216E-04	9.784E-05
0.987	7.351E-02	1.451E-03	5.300	5.071E-05	8.995E-05
1.013	7.826E-02	1.413E-03	5.380	-6.366E-05	7.505E-05
1.037	7.209E-02	1.394E-03	5.460	-5.928E-05	6.091E-05
1.062	6.244E-02	1.410E-03	5.540	-1.623E-05	6.189E-05
1.088	5.904E-02	1.468E-03	5.620	6.770E-05	6.804E-05
1.112	6.059E-02	1.467E-03	5.700	1.053E-04	9.171E-05
1.138	5.626E-02	1.403E-03	5.780	1.131E-04	8.886E-05
1.162	4.267E-02	1.275E-03	5.860	1.338E-04	7.591E-05
1.187	3.061E-02	1.194E-03	5.945	9.461E-05	6.550E-05
1.215	3.083E-02	1.176E-03	6.035	2.406E-05	5.643E-05
1.245	4.347E-02	1.296E-03	6.125	2.088E-05	6.247E-05
1.275	4.555E-02	1.825E-03	6.215	2.453E-05	7.624E-05
1.305	5.359E-02	1.238E-03	6.305	-1.349E-05	6.589E-05
1.335	7.598E-02	1.554E-03	6.395	-2.133E-05	6.722E-05
1.365	1.004E-01	1.626E-03	6.485	-9.448E-06	5.893E-05
1.395	9.662E-02	1.482E-03	6.575	1.482E-05	5.570E-05
1.425	7.232E-02	1.230E-03	6.665	8.017E-05	6.035E-05
1.455	4.646E-02	1.018E-03	6.755	1.280E-04	6.037E-05
1.485	3.223E-02	8.651E-04	6.850	8.591E-05	6.348E-05
1.515	2.427E-02	1.047E-03	6.950	3.499E-05	5.312E-05
1.545	1.865E-02	1.070E-03	7.050	4.335E-05	4.467E-05
1.580	1.899E-02	1.081E-03	7.150	4.251E-05	4.822E-05
1.620	2.056E-02	1.091E-03	7.250	9.723E-06	3.499E-05
1.660	1.791E-02	1.094E-03	7.350	1.688E-06	5.581E-05
1.700	1.824E-02	1.097E-03	7.450	1.092E-05	3.894E-05
1.740	1.860E-02	1.207E-03	7.550	1.725E-05	3.222E-05
1.780	1.455E-02	1.302E-03	7.650	1.289E-05	1.963E-05
1.820	1.489E-02	1.201E-03	7.750	2.330E-06	8.156E-06
1.860	1.464E-02	1.012E-03	7.850	8.819E-07	6.466E-06
1.900	1.267E-02	8.715E-04			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 5950 SEC AFTER END OF IRRADIATION
COUNT FOR 4000 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.409E-02	4.119E-03	1.940	9.855E-03	8.995E-04
0.065	2.577E-02	4.741E-03	1.980	1.116E-02	9.165E-04
0.075	1.014E-01	5.462E-03	2.020	1.091E-02	1.001E-03
0.085	1.360E-01	5.114E-03	2.060	8.272E-03	1.072E-03
0.095	6.385E-02	4.416E-03	2.100	7.032E-03	1.133E-03
0.105	6.375E-02	4.013E-03	2.140	1.109E-02	1.128E-03
0.115	7.213E-02	4.164E-03	2.180	1.567E-02	1.107E-03
0.125	8.447E-02	4.333E-03	2.220	1.352E-02	1.070E-03
0.135	1.223E-01	4.350E-03	2.260	7.329E-03	9.841E-04
0.145	1.909E-01	4.435E-03	2.300	6.340E-03	8.720E-04
0.155	2.138E-01	4.457E-03	2.340	1.224E-02	8.496E-04
0.165	2.155E-01	4.462E-03	2.380	1.556E-02	9.693E-04
0.177	2.008E-01	3.949E-03	2.425	1.183E-02	9.073E-04
0.192	1.830E-01	3.692E-03	2.475	9.916E-03	8.908E-04
0.207	1.863E-01	3.495E-03	2.525	1.185E-02	8.646E-04
0.222	1.365E-01	3.431E-03	2.575	9.254E-03	8.001E-04
0.237	8.744E-02	3.380E-03	2.625	6.639E-03	8.489E-04
0.252	1.028E-01	3.139E-03	2.675	6.571E-03	1.154E-03
0.267	1.224E-01	3.137E-03	2.725	6.403E-03	1.328E-03
0.282	1.404E-01	3.254E-03	2.775	4.915E-03	1.147E-03
0.297	1.532E-01	3.329E-03	2.825	2.910E-03	7.701E-04
0.313	1.547E-01	3.233E-03	2.875	2.716E-03	4.724E-04
0.327	1.195E-01	3.179E-03	2.925	3.464E-03	3.972E-04
0.342	8.425E-02	3.242E-03	2.975	4.241E-03	4.202E-04
0.357	6.677E-02	3.099E-03	3.030	3.595E-03	3.597E-04
0.372	5.319E-02	2.811E-03	3.090	1.443E-03	2.920E-04
0.387	6.413E-02	2.689E-03	3.150	9.762E-04	2.832E-04
0.402	8.895E-02	2.773E-03	3.210	1.469E-03	2.970E-04
0.417	1.194E-01	2.923E-03	3.270	1.685E-03	3.037E-04
0.432	1.469E-01	3.024E-03	3.330	1.728E-03	2.847E-04
0.447	1.560E-01	2.699E-03	3.390	8.176E-04	2.563E-04
0.462	1.477E-01	2.608E-03	3.450	5.491E-04	2.490E-04
0.477	1.084E-01	2.518E-03	3.510	1.035E-03	2.508E-04
0.492	8.253E-02	2.536E-03	3.570	1.341E-03	2.580E-04
0.507	9.443E-02	2.742E-03	3.630	1.237E-03	2.543E-04
0.522	1.262E-01	2.992E-03	3.690	8.004E-04	2.311E-04
0.540	1.591E-01	2.784E-03	3.750	5.806E-04	2.118E-04
0.560	1.464E-01	2.517E-03	3.810	7.122E-04	2.167E-04
0.580	1.223E-01	2.447E-03	3.870	8.911E-04	2.194E-04
0.600	1.308E-01	2.490E-03	3.935	7.311E-04	1.997E-04
0.620	1.805E-01	2.661E-03	4.005	2.944E-04	1.863E-04
0.640	2.191E-01	2.650E-03	4.075	3.049E-04	1.738E-04
0.660	1.890E-01	2.463E-03	4.145	4.796E-04	1.602E-04
0.680	1.164E-01	2.212E-03	4.215	2.298E-04	1.251E-04
0.700	8.939E-02	2.070E-03	4.285	4.185E-05	1.194E-04
0.720	1.189E-01	2.089E-03	4.355	8.922E-05	1.070E-04
0.740	1.613E-01	2.244E-03	4.425	1.280E-04	1.409E-04
0.760	1.606E-01	2.255E-03	4.495	6.106E-05	1.279E-04
0.780	1.308E-01	2.143E-03	4.565	-6.375E-06	1.356E-04
0.800	1.697E-01	2.474E-03	4.635	-1.942E-05	1.271E-04
0.820	3.292E-01	2.966E-03	4.705	2.029E-05	1.316E-04
0.840	5.224E-01	3.521E-03	4.775	7.652E-05	1.334E-04
0.860	5.656E-01	3.666E-03	4.845	8.498E-05	1.144E-04
0.880	4.590E-01	2.966E-03	4.915	5.916E-05	1.078E-04
0.900	3.171E-01	2.399E-03	4.985	5.036E-05	1.122E-04
0.920	2.003E-01	2.084E-03	5.060	9.761E-05	1.121E-04
0.940	1.278E-01	1.908E-03	5.140	9.612E-05	1.069E-04
0.962	8.864E-02	1.724E-03	5.220	1.588E-05	9.935E-05
0.987	7.682E-02	1.666E-03	5.300	2.156E-05	9.841E-05
1.013	8.278E-02	1.624E-03	5.380	1.052E-04	9.666E-05
1.037	8.442E-02	1.664E-03	5.460	1.290E-04	1.134E-04
1.062	8.124E-02	1.754E-03	5.540	1.502E-04	1.213E-04
1.088	7.802E-02	1.887E-03	5.620	1.339E-04	1.043E-04
1.112	7.965E-02	1.893E-03	5.700	-1.479E-05	9.724E-05
1.138	7.228E-02	1.735E-03	5.780	-2.448E-05	7.336E-05
1.162	4.605E-02	1.580E-03	5.860	1.172E-04	7.657E-05
1.187	3.041E-02	1.447E-03	5.945	2.256E-04	1.092E-04
1.215	3.630E-02	1.424E-03	6.035	1.035E-04	1.063E-04
1.245	6.620E-02	1.738E-03	6.125	1.417E-05	7.057E-05
1.275	8.069E-02	1.615E-03	6.215	1.220E-04	1.025E-04
1.305	9.004E-02	2.019E-03	6.305	1.362E-04	7.414E-05
1.335	1.122E-01	2.678E-03	6.395	6.084E-05	7.407E-05
1.365	1.448E-01	2.186E-03	6.485	-4.960E-05	8.029E-05
1.395	1.270E-01	1.849E-03	6.575	-8.766E-05	9.083E-05
1.425	7.999E-02	1.429E-03	6.665	-1.021E-04	7.222E-05
1.455	4.695E-02	1.118E-03	6.755	-1.342E-05	8.679E-05
1.485	3.124E-02	9.789E-04	6.850	1.007E-04	6.200E-05
1.515	2.094E-02	1.171E-03	6.950	1.592E-04	6.459E-05
1.545	1.562E-02	1.230E-03	7.050	1.135E-04	7.648E-05
1.580	1.882E-02	1.227E-03	7.150	6.163E-05	6.139E-05
1.620	1.962E-02	1.230E-03	7.250	2.347E-05	5.523E-05
1.660	2.089E-02	1.240E-03	7.350	-1.902E-05	4.210E-05
1.700	2.489E-02	1.337E-03	7.450	-1.882E-05	4.930E-05
1.740	2.403E-02	1.508E-03	7.550	-1.735E-06	3.096E-05
1.780	2.066E-02	1.597E-03	7.650	5.825E-06	1.993E-05
1.820	1.636E-02	1.458E-03	7.750	7.651E-07	8.296E-06
1.860	1.654E-02	1.172E-03	7.850	1.496E-06	6.560E-06
1.900	1.350E-02	1.001E-03			

SPECTRUM OF GAMMA RAYS FOLLOWING A
100-SEC THERMAL-NEUTRON IRRADIATION OF 235-U

START COUNT 9950 SEC AFTER END OF IRRADIATION
COUNT FOR 4000 SEC

E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)	E(GAMMA) MEV	Y(GAMMA) GAMMAS/MEV/FISSION	DELTA(Y)
0.055	1.880E-02	3.979E-03	1.940	5.357E-04	7.651E-04
0.065	2.195E-02	4.666E-03	1.980	5.162E-03	7.814E-04
0.075	5.751E-02	5.302E-03	2.020	5.717E-03	8.290E-04
0.085	5.210E-02	4.955E-03	2.060	5.108E-03	9.009E-04
0.095	3.777E-02	4.286E-03	2.100	4.074E-03	9.183E-04
0.105	3.935E-02	3.891E-03	2.140	6.242E-03	9.183E-04
0.115	4.644E-02	4.050E-03	2.180	6.216E-03	9.097E-04
0.125	4.418E-02	4.252E-03	2.220	3.633E-03	9.007E-04
0.135	6.612E-02	4.158E-03	2.260	2.526E-03	8.552E-04
0.145	8.545E-02	4.048E-03	2.300	3.506E-03	7.999E-04
0.155	8.944E-02	4.078E-03	2.340	6.159E-03	7.638E-04
0.165	1.070E-01	4.112E-03	2.380	7.844E-03	8.359E-04
0.177	8.867E-02	3.629E-03	2.425	6.156E-03	7.918E-04
0.192	8.023E-02	3.319E-03	2.475	5.143E-03	7.624E-04
0.207	8.848E-02	3.155E-03	2.525	6.390E-03	7.571E-04
0.222	8.317E-02	3.191E-03	2.575	5.809E-03	7.044E-04
0.237	6.455E-02	3.205E-03	2.625	4.624E-03	7.486E-04
0.252	5.906E-02	2.961E-03	2.675	4.446E-03	1.056E-03
0.267	6.959E-02	2.904E-03	2.725	4.857E-03	1.225E-03
0.282	7.031E-02	2.992E-03	2.775	3.969E-03	1.024E-03
0.297	6.735E-02	3.022E-03	2.825	1.809E-03	6.999E-04
0.313	6.080E-02	2.889E-03	2.875	1.113E-03	4.071E-04
0.327	5.195E-02	2.927E-03	2.925	2.137E-03	3.403E-04
0.342	3.896E-02	3.037E-03	2.975	2.834E-03	3.476E-04
0.357	3.410E-02	2.914E-03	3.030	1.891E-03	2.964E-04
0.372	2.500E-02	2.635E-03	3.090	6.443E-04	2.693E-04
0.387	3.611E-02	2.445E-03	3.150	4.690E-04	2.597E-04
0.402	5.296E-02	2.450E-03	3.210	5.433E-04	2.537E-04
0.417	6.358E-02	2.532E-03	3.270	8.200E-04	2.484E-04
0.432	7.065E-02	2.544E-03	3.330	8.695E-04	2.340E-04
0.447	6.248E-02	2.204E-03	3.390	5.137E-04	2.277E-04
0.462	5.269E-02	2.124E-03	3.450	4.473E-04	2.229E-04
0.477	4.414E-02	2.115E-03	3.510	7.083E-04	2.233E-04
0.492	4.844E-02	2.188E-03	3.570	1.009E-03	2.285E-04
0.507	6.691E-02	2.422E-03	3.630	1.039E-03	2.155E-04
0.522	9.659E-02	2.712E-03	3.690	5.069E-04	1.836E-04
0.540	1.175E-01	2.501E-03	3.750	1.368E-04	1.844E-04
0.560	9.672E-02	2.239E-03	3.810	2.925E-04	1.753E-04
0.580	7.456E-02	2.169E-03	3.870	4.114E-04	1.828E-04
0.600	7.084E-02	2.197E-03	3.935	1.885E-04	1.797E-04
0.620	1.083E-01	2.303E-03	4.005	-3.954E-05	1.592E-04
0.640	1.399E-01	2.308E-03	4.075	-1.496E-06	1.467E-04
0.660	1.266E-01	2.180E-03	4.145	9.321E-05	1.531E-04
0.680	7.954E-02	1.951E-03	4.215	2.362E-04	1.567E-04
0.700	5.792E-02	1.805E-03	4.285	2.957E-04	1.702E-04
0.720	7.176E-02	1.818E-03	4.355	1.975E-04	1.561E-04
0.740	9.805E-02	1.929E-03	4.425	8.100E-05	1.465E-04
0.760	9.227E-02	1.913E-03	4.495	5.078E-05	1.367E-04
0.780	7.074E-02	1.821E-03	4.565	2.451E-05	1.255E-04
0.800	9.451E-02	2.086E-03	4.635	-2.645E-06	1.251E-04
0.820	1.975E-01	2.469E-03	4.705	2.369E-05	1.263E-04
0.840	3.105E-01	2.859E-03	4.775	1.107E-04	1.551E-04
0.860	3.261E-01	2.923E-03	4.845	1.448E-04	1.497E-04
0.880	2.559E-01	2.389E-03	4.915	7.714E-05	1.453E-04
0.900	1.668E-01	1.929E-03	4.985	-2.656E-05	1.283E-04
0.920	1.023E-01	1.668E-03	5.060	-5.176E-05	1.183E-04
0.940	6.607E-02	1.571E-03	5.140	2.775E-05	1.226E-04
0.962	4.842E-02	1.491E-03	5.220	1.143E-04	1.449E-04
0.987	4.479E-02	1.452E-03	5.300	1.123E-04	1.435E-04
1.013	4.946E-02	1.428E-03	5.380	1.434E-04	1.391E-04
1.037	5.355E-02	1.471E-03	5.460	1.822E-04	1.284E-04
1.062	4.867E-02	1.573E-03	5.540	1.239E-04	1.150E-04
1.088	4.822E-02	1.693E-03	5.620	3.725E-05	1.213E-04
1.112	5.319E-02	1.720E-03	5.700	-1.566E-06	1.305E-04
1.138	4.630E-02	1.599E-03	5.780	3.933E-06	1.291E-04
1.162	2.948E-02	1.433E-03	5.860	2.154E-05	1.278E-04
1.187	1.982E-02	1.327E-03	5.945	3.281E-05	1.161E-04
1.215	2.738E-02	1.299E-03	6.035	7.026E-05	8.962E-05
1.245	5.507E-02	1.576E-03	6.125	1.854E-04	1.005E-04
1.275	7.140E-02	2.402E-03	6.215	2.667E-04	1.219E-04
1.305	7.478E-02	1.957E-03	6.305	1.739E-04	1.065E-04
1.335	8.778E-02	2.058E-03	6.395	4.346E-05	9.067E-05
1.365	1.073E-01	2.023E-03	6.485	-3.876E-05	8.651E-05
1.395	8.621E-02	1.577E-03	6.575	-6.525E-05	8.146E-05
1.425	4.732E-02	1.164E-03	6.665	-5.807E-05	8.810E-05
1.455	2.485E-02	9.661E-04	6.755	-4.762E-05	9.534E-05
1.485	1.612E-02	8.470E-04	6.850	-2.030E-05	8.100E-05
1.515	1.021E-02	9.734E-04	6.950	6.833E-05	7.552E-05
1.545	8.405E-03	9.932E-04	7.050	1.412E-04	7.557E-05
1.580	9.762E-03	9.817E-04	7.150	1.181E-04	6.901E-05
1.620	1.086E-02	9.853E-04	7.250	9.220E-05	6.795E-05
1.660	1.300E-02	1.031E-03	7.350	8.467E-05	6.634E-05
1.700	1.552E-02	1.150E-03	7.450	7.238E-05	5.917E-05
1.740	1.447E-02	1.288E-03	7.550	5.621E-05	3.989E-05
1.780	1.129E-02	1.329E-03	7.650	2.638E-05	2.137E-05
1.820	8.539E-03	1.150E-03	7.750	3.825E-06	8.417E-06
1.860	7.646E-03	9.694E-04	7.850	7.335E-07	6.579E-06
1.900	6.637E-03	8.282E-04			

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