

COMMONWEALTH EDISON COMPANY
ZION NUCLEAR POWER STATION UNITS 1 AND 2
SUCCESS CRITERIA NOTEBOOK
STEAM GENERATOR TUBE RUPTURE

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1.0 INTRODUCTION

A steam generator tube rupture (SGTR) event is the rupture of a single steam generator tube. For Zion, the rupture is considered to be a double-ended break of a single tube which results in a break area of 0.003276 sq. ft. This break area corresponds to the Model 51 steam generator tube diameter of 0.775 inches and is assumed to exist at the top of the tube sheet on the cold leg side of the steam generator.

Given this definition of a SGTR, the success criteria for the various systems and operator actions can be identified as summarized below.

2.0 SUCCESS CRITERIA

2.1 ECCS Injection

Question: What is the minimum ECCS injection capability required for successful core cooling during the initial phase of the accident, without any operator actions?

Answer: For cases with AFW available, ECCS injection is not needed to ensure adequate core cooling provided that the ruptured SG can be isolated. For cases with failure of AFW, at least one high pressure injection pump is necessary to provide successful core cooling.

Question: What is the mission time for the ECCS injection pumps?

Answer: The mission time for the ECCS injection pumps is 6 hours.

Discussion:

Analyses from Reference 1 (Cases 1 and 2) indicate that for the scenario in which all high pressure injection fails, the RCS pressure will decrease to near the secondary pressure, ultimately resulting in the reduction of the primary to secondary break flow. Provided that the ruptured SG can be isolated, the RCS and ruptured SG pressures will equilibrate and the break flow will be terminated. If feedwater flow control can also be provided, the decay heat is removed via primary to secondary heat transfer in the intact SGs, and the plant will be in an equilibrium condition.

Other analyses from Reference 1 (Cases 3 and 4) indicate that for the scenario in which AFW fails, the operation of one charging or one safety injection pump is sufficient to maintain adequate core cooling. The analyses indicate that the RCS pressure equilibrates at a pressure below the shutoff head of the respective ECCS high pressure injection pumps such that the outgoing break flow is matched by the incoming SI flow thereby removing the decay heat.

In all cases with ECCS injection, the ECCS pump(s) are required to deliver flow to two of the four cold legs; this is based on the ECCS injection success criteria developed for the large LOCA in Reference 8.

The requirements for ECCS injection for the SGTR event are identical whether the emergency AC power buses are energized from offsite sources or from the diesel generators. In the case of offsite power available, the pumps would deliver flow to the RCS within 5 seconds of the SI signal. In the case of offsite power not available, the pumps would deliver flow to the RCS within 25 second of the SI signal (Reference 2). Analyses (Reference 3) show that this delay in SI actuation would not

impact core cooling for a LOCA; this conclusion is also applicable to the SGTR event. Thus, the power source for the ECCS pumps is not significant for the SGTR event.

The mission time for the ECCS injection pumps is 6 hours. This is chosen as a representative time for the ECCS pumps since it is expected that operator action to stop ECCS flow should occur within this time frame for most SGTR sequences.

2.2 Manual SI Initiation

Question: What is the maximum time for operator action to manually initiate SI, in the event of a failure of automatic initiation in conjunction with failure of AFW, in order to prevent core damage?

Answer: If high pressure injection can be manually initiated within 2 hours, core damage is prevented. However, this may preclude other actions from being taken due to loss of heat sink considerations.

Discussion:

Analyses from Reference 1 (Cases 5 and 6) indicate that for the case with no ECCS and no AFW, the onset of core uncover begins at slightly greater than 2 hours. Thus if high pressure injection can be initiated before the onset of core uncover (i.e., 2 hours), core damage can be prevented.

The manual initiation of the ECCS pumps is not modeled in the SGTR plant response trees.

2.3 Auxiliary Feedwater

Question: What is the minimum number of auxiliary feedwater pumps required for successful core cooling?

Answer: Auxiliary feedwater is not required for core cooling following a SGTR provided that at least one high pressure injection pump is operating.

Question: What is the minimum number of Auxiliary Feedwater pumps required to prevent the operators from going to the Loss of Heat Sink Procedure (FR-H. 1)?

Answer: To prevent initiation of the Loss of Heat Sink EOP, one of the following configurations is required:

- a. 1 motor-driven or 1 turbine-driven AFW pump delivering flow to 4 out of 4 steam generators, or*
- b. 1 motor-driven or 1 turbine-driven AFW pump delivering flow to 3 out of 4 steam generators with operator action to open the throttle valves on the delivering AFW lines.*

Question: What is the minimum number of Auxiliary Feedwater pumps required for:

- 1. initial RCS cooldown per Step 14 of E-3,*
- 2. maintenance of a 100°F/hr RCS cooldown.*

Answer: To perform the initial RCS cooldown per Step 14 of E-3, the initial secondary inventory in the unisolated, intact SGs is sufficient to perform this limited cooldown; therefore no auxiliary feedwater is needed (physically) for this initial RCS cooldown. However, there must be

sufficient auxiliary feedwater (see Answer above) to maintain the natural progression through the EOPs to Step 14 of E-3.

For maintenance of a 100°F/hr RCS cooldown, either of the AFW configurations noted above are sufficient.

Question: What is the mission time for AFW pumps?

Answer: The mission time for the AFW pumps is 6 hours.

Discussion:

The unique success criteria which have been established for the operation of the Auxiliary Feedwater System (AFW) include: 1) the minimum AFW requirements for decay heat removal, 2) the minimum requirements to prevent implementation of the Loss of Heat Sink Procedure (FR-H.1), and 3) the minimum requirements to accomplish RCS cooldown. Each of these requirements are discussed in the following paragraphs.

Analyses of the SGTR using the TREAT computer code (Reference 1, Cases 3 and 4) show that auxiliary feedwater is not required in order to remove decay heat following a SGTR with at least one high pressure injection pump operational. For the cases without AFW to the steam generators, the RCS does not pressurize above the cutoff head of the charging or SI pumps; rather the RCS is maintained at an equilibrium pressure at which the decay heat is being removed via the combination of primary to secondary break flow and the addition of relatively cold SI water from the high pressure injection pump.

Per the Zion EOPs, the minimum AFW flow necessary to prevent implementation of the 'Loss of Heat Sink' procedure is 340 gpm. This flow is based on the assumed AFW flow for the main feedwater line break safety analysis for Zion. Evaluations of the AFW system performance (Reference 4) indicate that although the AFW throttle valves are set in a restricted position during startup testing (GOP-2) and monthly surveillance (PT-7), the total AFW flow will be greater than this 340 gpm minimum requirement for accident conditions provided that flow is delivered to all four steam generators. Therefore, the success criterion for AFW can be specified as 1 motor-driven or 1 turbine-driven AFW pump delivering flow to 4 out of 4 steam generators. [Note that with the AFW flow throttled, flow from 1 AFW pump to less than 4 out of 4 steam generators will not exceed 340 gpm.]

Alternately, a review of the Zion EOPs show that if AFW flow is not delivered to all 4 steam generators and the turbine-driven AFW pump is not operable, the operators would be required to open the AFW throttle valves to achieve the desired AFW flow (i.e., 340 gpm). Therefore, inclusion of the operator action to open the AFW throttle valve has been considered as an alternate success criterion for AFW flow. Although flow to 1 SG with the AFW throttle valve in the open position would be sufficient to achieve the desired 340 gpm flow rate, the success criterion is defined as 1 motor-driven or 1 turbine-driven AFW pump delivering flow to 3 out of 4 steam generators with operator action to open the throttle valves on the delivering AFW lines. AFW flow to at least 3 SGs is necessary since 2 SGs are needed for success of the initial RCS cooldown (Section 2.8), and the ruptured SG is not used for the cooldown. It is noted that the operator action to open the AFW throttle valves is modeled in the AFW fault tree (Reference 15).

Analyses presented in Reference 1, Case 7 show that the initial RCS cooldown of approximately 40°F can be completed without any AFW flow, provided that the inventory of the unisolated, intact SGs is utilized. However, such a scenario cannot

happen due to the structure of the Zion EOPs; specifically, with no AFW the operators are instructed to use FR-H.1, Loss of Heat Sink and no RCS cooldown would be initiated without verification of at least 340 gpm of AFW flow.

Other analyses (Reference 5) show that approximately 250 gpm of AFW flow is required to maintain a 100°F/hr RCS cooldown. Therefore, either of the configurations identified above to provide the minimum 340 gpm flow rate to avoid a 'Loss of Heat Sink' will be sufficient to maintain a 100°F/hr RCS cooldown.

The mission time for the AFW pumps is taken to be 6 hours. This time is consistent with the actions needed to terminate the primary-to-secondary break flow; all actions should be completed within this time.

2.4 Alternate Feedwater

Question: What is the minimum number of main feedwater pumps required to maintain steam generator inventory in a SGTR event with loss of AFW?

Answer: With one main feedwater pump delivering flow to 1 out of the 3 intact steam generators, the steam generator inventory can be maintained for all circumstances in a SGTR event with loss of AFW.

Question: What is the mission time for the main feedwater pumps?

Answer: The mission time for the main feedwater pumps is 6 hours.

Discussion:

One main feedwater pump can deliver over 1000 gpm and is not limited by the flow restrictors in the AFW lines. The action to implement main feedwater flow in the event of a loss of AFW flow is governed by FR-H.1, Loss of Heat Sink. In order to escape the FR-H.1 procedure, the water level in at least one steam generator must be restored to greater than 4% narrow range. Therefore, one main feedwater pump delivering flow to 1 out of the 3 intact steam generators will satisfy this requirement.

The mission time for the main feedwater pump(s) is taken to be identical to that for the AFW pumps, or 6 hours.

2.5 Operator Action to Implement Alternate Feedwater

Question: What is the maximum time available to implement alternate feedwater to the steam generators in the event of a loss of AFW?

Answer: In order to return to the SGTR recovery procedure and terminate the primary to secondary break flow prior to SG overfill, the maximum time after the initiation of the SGTR event to restore alternate feedwater to the steam generator(s) is:

- a. 25 minutes if all ECCS is available,*
- b. 2 hours if no ECCS is available.*

In order to avoid 'bleed and feed' cooling per FR-H.1, the maximum time after the initiation of the SGTR event to restore alternate feedwater to the steam generator(s) is:

- a. 2 hours if all ECCS is available,
- b. 1.5 hours if only one charging pump is available.

Discussion:

For the scenario in which AFW fails, the purpose of implementing an alternate feedwater supply to the steam generator(s) is to first return to the normal EOP used for recovery from a SGTR (E-3), or second to prevent the possibility of going to 'Bleed and Feed Cooling' per FR-H.1 due to loss of secondary heat sink.

Following alternate feedwater injection to the steam generator(s), level recovery will result in transfer to the original procedure and subsequent transfer to E-3 for mitigation of a SGTR event. A Reference 1 analysis (Case 8) shows that for the scenario with all ECCS but no AFW, the ruptured SG level reaches 100% at 30 minutes. Based on logic discussed in Section 2.7, isolation of the ruptured SG must occur by the time the level reaches 100% in order to successfully terminate the primary to secondary break flow prior to SG overfill. It is assumed that the operators will take approximately 5 minutes for the EOP transfer from FR-H.1 to E-0 to E-3 to obtain the instructions to isolate the ruptured SG; thus alternate feedwater injection to the steam generator(s) must be established by 25 minutes following the initiation of the SGTR event in order to stop the break flow prior to SG overfill.

For the scenario with no AFW and no ECCS (Reference 1, Case 5), adequate core cooling is provided if alternate feedwater can be restored by 2 hours. This will enable the use of the steam generators for decay heat removal; subsequent isolation of the ruptured SG will equilibrate the RCS and ruptured SG pressures thereby terminating the primary to secondary break flow. 'Bleed and Feed' would not be initiated for this case due to the lack of ECCS injection.

The implementation of 'Bleed and Feed' is triggered by a steam generator water level below 24% of wide range indication in any 3 steam generators. Thus, the time available to implement an alternate feedwater source is limited by the time at which 'Bleed and Feed' would be implemented due to low level. Based on analyses in Reference 1 (Cases 3 and 8), the time to 24% wide range level is dependent on the number of ECCS pumps which are operating. For the scenario in which all ECCS is available and operating (i.e., both charging pumps and both SI pumps), this time is slightly over 2 hours. Presuming that the indication to establish alternate feedwater is early in the event, on the order of 10 minutes, the operators would have approximately 2 hours to perform this action.

For the scenario in which only one charging pump is available, the time until the SG wide range level reached 24% is slightly over 1.5 hours. The time is slightly shorter for this scenario since the lower equilibrium break flow rate results in less energy transfer via the break flow and more SG heat transfer, causing quicker boiloff of the secondary inventory. Presuming that the indication to establish alternate feedwater is early in the event, on the order of 10 minutes, the operators would have approximately 1.5 hours to perform this action.

It must be noted that for the scenario in which only SI is available, the SG wide range level is not used as an indication to begin 'bleed and feed' cooling; rather 'bleed and feed' is initiated immediately upon failure of charging pumps in conjunction with failure of AFW.

2.6 Steam Generator Isolation

Question: What equipment is necessary to isolate steam flow from the ruptured SG?

Answer: To isolate steam flow from the ruptured SG, the main steam isolation valve (MSIV) for the ruptured SG must be closed.

Alternately, if the MSIV for the ruptured SG cannot be closed, isolation of the ruptured SG may be accomplished by performing the following actions:

- closing the MSIVs for the intact SGs*
- closing the steam dump valves*
- isolating steam flow to the moisture separator reheaters, the steam jet air ejectors and the main feed pumps.*

Question: What equipment is necessary to isolate feedwater flow to the ruptured SG?

Answer: Isolation of feedwater flow to the ruptured SG entails the closure of the flow regulating valves controlling feedwater flow to the ruptured SG. This includes feedwater via the auxiliary feedwater pump(s) or the main feedwater pump(s).

Question: What is the mission time for the equipment used to isolate the ruptured SG (i.e., MSIVs, feedwater regulating valves, etc.)?

Answer: The mission time for the MSIVs and the feedwater pump flow regulating valves is 24 hours.

Discussion:

Isolation of steam flow from the ruptured SG provides a pressure/temperature differential between the ruptured SG and the intact SGs as a necessary precursor to perform subsequent operator actions to terminate the primary to secondary break flow through the ruptured steam generator tube. This isolation is performed via closure of the MSIV for the ruptured SG. It is noted that the Zion EOPs instruct the operators to close the MSIV bypass valves on the ruptured SG, close the blowdown isolation valves on the ruptured SG and isolate steam flow to the turbine driven AFW pump as part of the isolation process. However, these actions are not included here in the success criteria since these paths are relatively small steam leak paths compared to the main steam line. Additionally, these steam leak paths would be expected to be isolated automatically or manually per procedural guidance. (Note that there is only a 50% chance that the steam supply line to the turbine driven AFW pump would be from the ruptured SG since these lines only originate from SGs A and D).

Should the ruptured SG MSIV fail to close, successful isolation of the ruptured SG can also be performed by closure of the MSIVs for the intact SGs, closure of the steam dump (to condenser) valves, and isolation of steam flow to the moisture separator reheaters, main feed pumps plus steam jet air ejectors. As with the previous case, steam flow via the MSIV bypass valves, the blowdown isolation valves and the steam supply line to the turbine driven AFW pump is not included in this success criteria.

It is noted that closure of the MSIV for the ruptured SG is preferable since this gives the operator the option of using steam dump to the condenser for the subsequent RCS cooldown.

Isolation of the ruptured SG also includes stopping all feedwater flow to the ruptured SG. Control of feedwater flow to the ruptured SG is necessary to prevent overfilling the ruptured SG. For AFW flow, this control is provided by using the AFW flow

regulating valves on the motor driven AFW pump line (FW0051, FW0053, FW0055, FW0057) and the turbine driven AFW pump line (FW0050, FW0052, FW0054, FW0056). For feedwater flow to the SGs with the main feedwater pumps, control is provided by using the main feedwater flow regulating valve on each line (1LCV-FW510, 1LCV-FW520, 1LCV-FW530, 1LCV-FW540) as well as the main feedwater bypass line flow regulating valve on each line (1LCV-FW510A, 1LCV-FW520A, 1LCV-FW530A, 1LCV-FW540A).

The mission time for the MSIVs and feedwater pump discharge valves is taken to be 24 hours since the ruptured SG must remain isolated for this duration of time.

2.7 Operator Action to Isolate Ruptured Steam Generator

Question: What is the maximum time available for operator action to isolate the ruptured steam generator for cases with ECCS available, and still recover via Zion procedure E-3?

Answer: The operators must initiate isolation of the ruptured steam generator by the time level indication is 100% in the ruptured steam generator; this corresponds to 20 minutes following initiation of the SGTR.

Question: What is the maximum time available for operator action to isolate the ruptured steam generator for cases with no ECCS injection, but with AFW or alternate feedwater available?

Answer: The operators must initiate isolation of the ruptured steam generator within 10 hours of the initiation of the SGTR event in order to avoid core damage.

Discussion:

The criteria to initiate isolation of the ruptured SG was chosen as such for two reasons (Reference 1, Cases 0, 9 and 10). First, by the time the level reaches 100% indication in the ruptured SG, the disparity between the intact SGs and the ruptured SG levels is approximately 25% for wide range and 90% for narrow range indications. This increasing disparity with time in conjunction with radiation alarms in the steam line is judged to give a clear indication of a SGTR event, such that the identification of a SGTR can be reasonably accomplished by the time the level reaches 100% in the ruptured SG. Second, with minimal equipment for the subsequent EOP recovery actions (i.e., RCS cooldown and RCS depressurization), the primary to secondary break flow can still be terminated prior to SG overfill IF the operator actions are started at this time. On these bases, isolation of the ruptured SG must be initiated by 20 minutes following initiation of the SGTR for the case with AFW available, in order to terminate the primary to secondary break flow prior to SG overfill via E-3. Otherwise, the operators will transfer to ECA-3.1 and it is assumed that the primary to secondary break flow cannot be terminated prior to SG overfill.

For the case with no AFW, successful implementation of alternate feedwater by 25 minutes (see Section 2.5) will result in the ability of the operator to terminate the primary to secondary break flow prior to SG overfill. In this scenario, the time to reach 100% level is extended by 10 minutes since break flow only is filling the ruptured SG. However, earlier implementation of alternate feedwater at 1000 gpm will rapidly fill the ruptured SG and negate any additional available time due to the failure of AFW. Therefore, the time to isolate the ruptured SG will remain as 20 minutes for the scenario in which AFW is not available but alternate feedwater is established within 25 minutes.

For the case with no ECCS but AFW or alternate feedwater to the steam generators, decay heat removal is provided by primary to secondary heat transfer. However, without isolation of the ruptured SG a pressure differential between the RCS and the ruptured SG will exist leading to continued primary to secondary break flow. Failure to isolate the ruptured SG will result in RCS drainage and eventual core uncover. The timing for core uncover for this scenario has been determined to be approximately 10 hours (Reference 1, Case 11); thus isolation of the ruptured SG must occur in this instance by 10 hours in order to stop the primary to secondary break flow prior to core damage.

2.8 RCS Cooldown

Question: What steam relief capacity (from the steam generators) is required to achieve RCS cooldown, per Step 14 of Zion procedure E-3, prior to SG overfill?

Answer: 2 out of 3 atmospheric relief valves on the intact SGs or 2 out of 3 steam dump valves will provide the necessary steam relief to achieve this initial RCS cooldown prior to SG overfill.

Question: What steam relief capacity (from the steam generators) is required for RCS cooldown of SG overfill sequences?

Answer: 2 out of 3 atmospheric relief valves on the intact SGs or 2 out of 3 steam dump valves will provide the necessary steam relief to achieve this RCS cooldown for SG overfill sequences.

Question: What is the mission time for atmospheric relief valves and the condenser steam dump valves?

Answer: The mission time for the atmospheric relief valves and the steam dump valves is 6 hours.

Discussion:

Following identification of a SGTR event, and subsequent isolation of the ruptured SG, the EOPs instruct the operators to initiate a RCS cooldown. The purpose of this initial RCS cooldown is to establish or maintain a temperature difference between the RCS and the intact SGs for decay heat removal, plus increase the subcooling in the RCS so that subcooling is maintained following the subsequent RCS depressurization. The amount of this initial RCS cooldown is dependent on the ruptured SG pressure; typically, following isolation of the ruptured SG, the pressure in this SG will be maintained at or near the ARV setpoint (1050 psia). To achieve the amount of RCS subcooling margin as directed in E-3 at this pressure requires a RCS cooldown of less than 40°F. Analyses from Reference 1 (Cases 9 and 10) show that this degree of RCS cooldown can be achieved with 1 ARV; however, SG overfill will occur before the RCS cooldown target temperature is attained. Since the RCS cooldown can successfully be accomplished prior to SG overfill with 2 ARVs, the requirement for steam relief as directed in E-3 is either 2 ARVs (steam relief to atmosphere) on intact SGs or 2 steam dump valves (steam relief to condenser). Note that AFW must be provided to the SGs used for the RCS cooldown (Section 2.3).

RCS cooldown is also considered for SG overfill sequences as a necessary precursor to ECCS reduction (Section 2.12). These scenarios include failure to isolate feed flow to the ruptured SG (per E-3), failure to isolate steam flow from the ruptured SG (per ECA-3.1), and cases in which the charging pumps are not available (per E-3). In these instances, the RCS cooldown capability required is not as constrictive as discussed above since these sequences, by definition, result in SG overfill. In fact, the 1 ARV or 1 steam dump valve steam relief capability for a 100°F/hr RCS cooldown

(Reference 7) would be sufficient. However, the success criteria for this action will remain '2 out of 3 ARVs or 2 out of 3 steam dump valves' needed to perform this RCS cooldown. No change was made to the success criteria defined previously for several reasons: to avoid complicating the tree (a second RCS cooldown node would have been needed), a minimal benefit would be realized on the calculation of the failure probability of RCS cooldown equipment for a revised success criteria, and to maintain a bounding success criteria definition of RCS cooldown for all SG overfill cases. Therefore, the RCS cooldown success criteria for SG overfill cases remains 2 out of 3 ARVs on the intact SGs or 2 out of 3 condenser steam dump valves.

The mission time for the steam relief equipment (ARVs and steam dump valves) is 6 hours; this time is consistent with the mission time for AFW.

2.9 Operator Action for RCS Cooldown

Question: What is the maximum time available for operator action to initiate RCS cooldown and still terminate primary to secondary breakflow prior to steam generator overfill via Zion procedure E-3?

Answer: The operators must begin to implement initial RCS cooldown by 25 minutes following initiation of the SGTR event.

Question: What is the maximum time available for operator action to initiate RCS cooldown for SG overfill sequences?

Answer: The latest time at which the operator can initiate RCS cooldown for SG overfill sequences is 1 hour.

Discussion:

The operator action to perform an RCS cooldown is the next major action following isolation of the ruptured SG. The timing for the initiation of this RCS cooldown is dependent on the time of ruptured SG isolation and the steam relief capacity available. To wit, the timing for the initial RCS cooldown can be delayed proportionately for early isolation of the ruptured SG and maximum steam relief capability. However, to avoid a complex matrix of operator action times to initiate RCS cooldown as a function of isolation time and steam relief capability, a time was chosen which will encompass all scenarios leading to termination of the primary to secondary break flow prior to SG overfill. Specifically, for the case with the latest 'successful' ruptured SG isolation (i.e., 20 minutes) and with the minimum successful steam relief capability (i.e., 2 ARVs), the primary to secondary break flow can be terminated prior to SG overfill IF the RCS cooldown is initiated by 25 minutes (Reference 1, Cases 9 and 10). Thus the operator has 5 minutes after isolating the ruptured SG to initiate the RCS cooldown.

It is also noted RCS cooldown is considered for SG overfill sequences as a necessary precursor to ECCS reduction (Section 2.12). These scenarios include failure to isolate feed flow to the ruptured SG (per E-3), failure to isolate steam flow from the ruptured SG (per ECA-3.1), and cases in which the charging pumps are not available (per E-3). In these instances, the operator action time for RCS cooldown is not as constrictive as discussed above since these sequences, by definition, result in SG overfill. Analyses from Reference 1 indicate that the operator would have several hours to perform the RCS cooldown step. However, the success criteria for this action is defined as less than 1 hour to perform this RCS cooldown. No change which requires recalculation of probability was made to the success criteria defined previously for several reasons: to avoid complicating the tree (a second RCS cooldown operator action node would have been needed), a minimal benefit would be realized on the

calculation of the failure probability of the operator action to perform the RCS cooldown for a longer than 1 hour success criteria time, and maintain a bounding success criteria definition of the operator action to perform the RCS cooldown for all SG overfill case. Therefore, the RCS cooldown operator action success criteria for SG overfill cases will be defined as less than 1 hour.

2.10 RCS Depressurization

Question: What RCS depressurization mechanism(s) are required, per Step 18 of Zion procedure E-3, in order to successfully terminate the primary to secondary break flow prior to SG overfill?

Answer: The primary to secondary break flow can be terminated prior to SG overfill with no initial RCS depressurization; thus no RCS depressurization mechanism(s) are necessary in this instance. However, RCS depressurization may be performed with either normal pressurizer spray, one pressurizer PORV or auxiliary pressurizer spray.

Question: What is the mission time for normal pressurizer spray, one pressurizer PORV or auxiliary spray?

Answer: The mission time for normal pressurizer spray, or auxiliary spray is 6 hours; the mission time for the pressurizer PORV is 24 hours.

Discussion:

The purpose of the RCS depressurization in the EOPs is primarily to reestablish level in the pressurizer for subsequent ease in RCS inventory control. RCS depressurization also serves to reduce or terminate the primary to secondary break flow as the RCS

pressure approaches the pressure in the ruptured SG; however, the ECCS flow must ultimately be stopped to permanently terminate this break flow. Reference 1 (Case 10) indicates that the primary to secondary break flow can be terminated prior to SG overfill without this RCS depressurization step. Specifically, if RCS depressurization cannot be performed, the operators will transfer to ECA-3.3 at which time the operators will be instructed to stop all ECCS flow if the ruptured SG narrow range level is greater than 70%. Successful termination of ECCS flow (Section 2.12) will result in RCS depressurization to the ruptured SG pressure and consequently termination of the primary to secondary break flow. Thus the Reference 1 analysis shows that the primary to secondary break flow can be stopped prior to SG overfill without this initial RCS depressurization via operator transfer to ECA-3.3.

The RCS depressurization step is included in the Plant Response Tree, however, since this is the natural progression through the EOPs. The equipment necessary for success of this RCS depressurization step is that equipment associated with normal pressurizer spray (including RCP operation and the valves in the piping between the cold leg and pressurizer steam space), or one pressurizer PORV and its associated block valve opening and remaining open upon demand, or that equipment associated with auxiliary spray (including operation of one centrifugal charging pump and the valves in the piping between the CVCS and the pressurizer steam space).

The mission time for normal pressurizer spray or auxiliary spray is 6 hours since the actions to terminate primary-to-secondary break flow should be completed within this time. The mission time for the pressurizer PORVs is 24 hours based on logic discussed in Section 2.14.

2.11 Operator Action to Depressurize the RCS

Question: What is the maximum time available for operator action to initiate initial RCS depressurization, per Step 14 of Zion procedure E-3, and still terminate primary to secondary breakflow prior to steam generator overfill?

Answer: As noted previously, initial RCS depressurization is not a necessary step in order to terminate the primary to secondary break flow prior to SG overfill. However, if the initial RCS depressurization is to be performed, the operators must begin this initial RCS depressurization by 40 minutes following initiation of the SGTR event.

Discussion:

As noted previously, the initial RCS depressurization is performed primarily to recover pressurizer level for easier inventory control in subsequent actions. Termination of primary to secondary break flow prior to SG overfill can still occur via transfer to ECA-3.3 if this RCS depressurization is not performed. However, if the initial RCS depressurization is to be performed, and considering the timing for the subsequent step to stop all ECCS pumps (Section 2.12), the operators must begin this initial RCS depressurization by 40 minutes following initiation of the SGTR event.

Similar to the timing for the initial RCS cooldown, the timing for the initial RCS depressurization is dependent on the time of ruptured SG isolation, the time of initiation of the RCS cooldown, and the steam relief capability of the secondary system. However, to avoid a complex matrix of operator action times to initiate RCS depressurization as a function of isolation time, etc., a time was chosen which will encompass all scenarios leading to termination of the primary to secondary break flow

prior to SG overfill. Specifically, for the case with the latest 'successful' ruptured SG isolation time (i.e., 20 minutes), the minimum successful steam relief capability (i.e., 2 ARVs) and the latest 'successful' time to initiate RCS cooldown (i.e., 25 minutes), the primary to secondary break flow can be terminated prior to SG overfill IF the RCS depressurization is initiated by 40 minutes (Reference 1, Cases 9 and 10). Thus the operator has 3 minutes after completion of the RCS cooldown to initiate the RCS depressurization.

2.12 Operator Action to Reduce ECCS Injection

Question: What is the maximum time available for operator action to reduce ECCS injection and still terminate primary to secondary break flow prior to SG overfill?

Answer: The latest time at which the operator can reduce ECCS injection and still terminate primary to secondary break flow prior to SG overfill is:

- a. 52 minutes following initiation of a SGTR for the case in which RCS depressurization is successful,*
- b. 45 minutes following initiation of a SGTR for the case in which RCS depressurization is not successful.*

Question: What is the maximum time available for operator action to reduce ECCS injection in order to prevent core damage for the first 24 hours of the event?

Answer: The latest time at which the operator can reduce ECCS injection in order to prevent core damage for the first 24 hours of the event is 1 hour.

Discussion:

There are two scenarios in which ECCS reduction is examined: 1) ECCS reduction as a precursor to establishing RCS inventory control and eventual termination of primary to secondary break flow PRIOR to SG overfill and 2) ECCS reduction for SG overfill cases in order to extend the availability of RWST water for ECCS injection.

For the scenarios in which isolation of the ruptured SG and cooldown of the RCS has succeeded per instructions in E-3, the next operator action to be performed is depressurization of the RCS (Sections 2.10 and 2.11). Regardless of whether RCS depressurization is successful, the operator must subsequently reduce ECCS flow to that from only 1 charging pump as a necessary precursor to establishing RCS inventory control and eventually terminating the primary to secondary break flow prior to SG overfill. The instructions to reduce ECCS to the flow from 1 charging pump are in E-3 if RCS depressurization is successful, and in ECA-3.3 if no pressurizer pressure control is available. The success/failure of RCS depressurization impacts the time available to perform the ECCS reduction, as discussed henceforth.

As with the other operator actions discussed previously, the operator action time to reduce ECCS flow to that from 1 charging pump is dependent upon the combination of previous operator action times and the steam relief capability for the PCS cooldown step. Similarly, the time to accomplish ECCS reduction was chosen in order to encompass all scenarios leading to termination of the primary to secondary break flow prior to SG overfill. Specifically, for the case with the latest 'successful' ruptured SG isolation time (i.e., 20 minutes), the minimum successful steam relief capability (i.e., 2 ARVs), the latest 'successful' time to initiate RCS cooldown (i.e., 25 minutes) and the latest 'successful' time to initiate RCS depressurization (i.e., 40 minutes), the primary to secondary break flow can be terminated prior to SG overfill

IF ECCS reduction is completed by 52 minutes. Thus, the operator has 3 minutes after completion of the RCS depressurization to reduce ECCS flow.

For the case with latest 'successful' ruptured SG isolation time (i.e., 20 minutes), the minimum successful steam relief capability (i.e., 2 ARVs), the latest 'successful' time to initiate RCS cooldown (i.e., 25 minutes), and no RCS depressurization, the primary to secondary break flow can be terminated prior to SG overfill IF ECCS reduction is completed by 45 minutes. For this case, the operator has 8 minutes following completion of the RCS cooldown to reduce ECCS flow.

ECCS reduction is also considered for SG overfill cases; reducing the ECCS flow to 1 charging pump or 1 SI pump will extend the availability of the RWST water for ECCS injection thereby preventing any core damage for the initial 24 hours of the event. These SG overfill scenarios in which ECCS reduction is considered include failure to isolate feed flow to the ruptured SG (per E-3), failure to isolate steam flow from the ruptured SG (per ECA-3.1), and cases in which the charging pumps are not available (per E-3). In all cases, success of RCS cooldown is necessary in order to meet the ECCS reduction criteria (Section 2.8).

For these scenarios, the time available for the operator to reduce ECCS flow is not as constrictive as the success criteria discussed above since these sequences, by definition, result in SG overfill. Analyses from Reference 1 indicate that the operator would have several hours to perform the ECCS reduction step. However, the success criteria for this action will remain 'less than 1 hour' to perform the ECCS reduction step. No change which requires recalculation of probability was made to the success criteria defined previously for several reasons: to avoid complicating the tree (a second ECCS reduction node would have been needed), a minimal benefit would be realized on the calculation of the failure probability of a ECCS reduction operator action for a longer success criteria time, and maintain a bounding success criteria

definition of ECCS reduction for all SG overfill cases. Therefore, the success criteria for ECCS reduction for these scenarios may be defined such that the operator has a maximum time of 1 hour to reduce ECCS.

2.13 Operator Action to Establish Normal Charging

Question: What is the maximum time available for operator action to establish normal charging flow and still terminate primary to secondary break flow prior to SG overfill?

Answer: The latest time at which the operator can establish normal charging flow and still terminate primary to secondary break flow prior to SG overfill is:

- a. 52 minutes following initiation of a SGTR for the case in which RCS depressurization is successful,*
- b. 45 minutes following initiation of a SGTR for the case in which RCS depressurization is not successful.*

Discussion:

The operator action to establish normal charging flow is considered for those cases in which ruptured SG isolation, RCS cooldown and ECCS reduction are successful. Since ECCS reduction and establishing normal charging flow are coupled together as necessary steps to establishing RCS inventory control (and terminating the primary to secondary break flow prior to SG overfill), the identical success criteria is used for establishing normal charging as was used for ECCS reduction. It is noted that a SAM end state is still possible if the operator action to establish normal charging fails, provided that ECCS reduction is successful.

2.14 Normal Charging

Question: What equipment is necessary to establish normal charging?

Answer: Establishing normal charging includes the operation of one centrifugal charging pump plus the associated hardware for alignment of flow from the VCT to the cold leg.

Question: What is the mission time for the normal charging equipment?

Answer: The mission time for normal charging equipment is 18 hours.

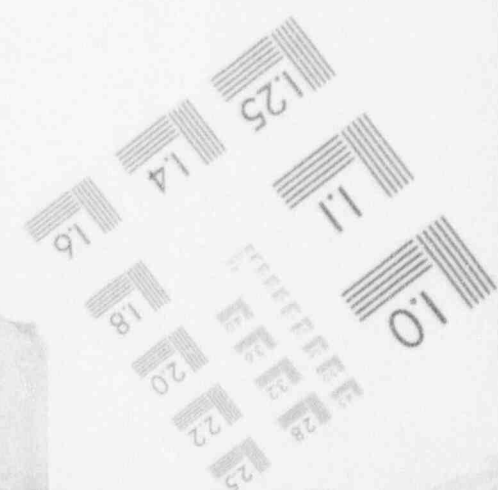
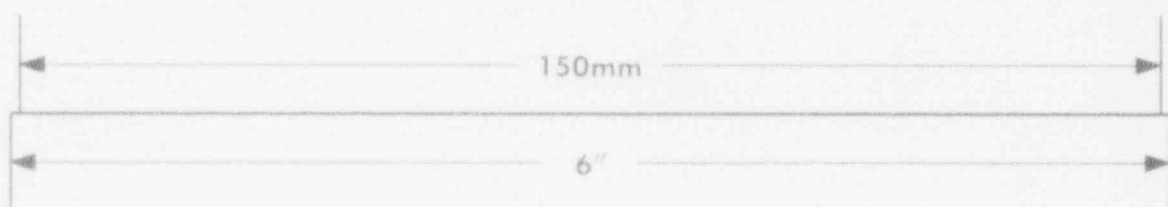
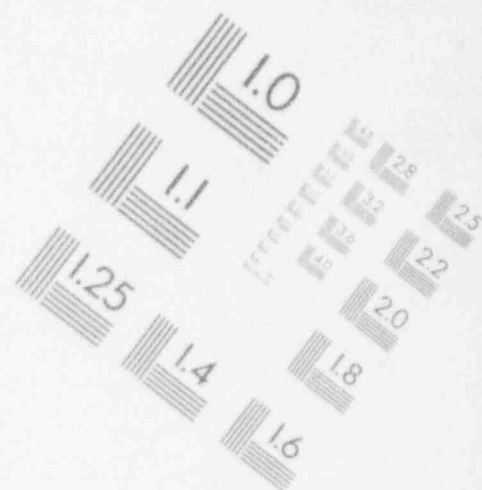
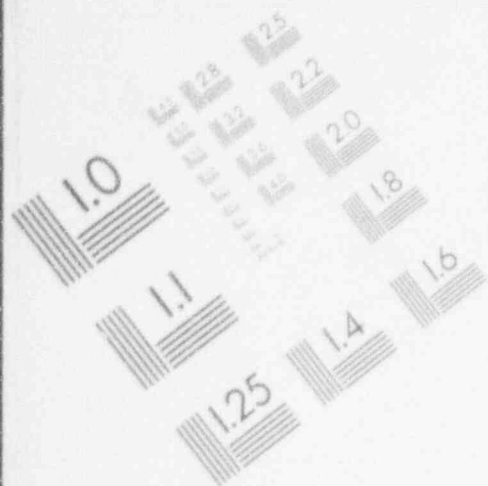
Discussion:

As noted previously, for all cases RCS inventory control must be established by terminating ECCS flow to the RCS and initiating normal charging flow. Normal charging is addressed for those sequences in which ruptured SG isolation, RCS Cooldown, ECCS reduction and operator action to establish normal charging are successful. It is noted that a SAM end state is still possible if normal charging fails, provided that ECCS reduction is successful.

Normal charging flow includes the operation of at least one of the centrifugal charging pumps taking suction from the VCT. The inventory in the VCT is replenished from the reactor makeup water storage tank. The valves in this configuration must also be realigned to provide flow via this path. These valves include opening MOV-VC8110, MOV-VC8111, MOV-VC8105, MOV-VC8106, AOV-VC8147 and MOV-VC8100 plus closing valves MOV-SI8803A (Unit 1 only), MOV-SI8803B (Unit 1 only), MOV-SI8801A and MOV-SI8801B.

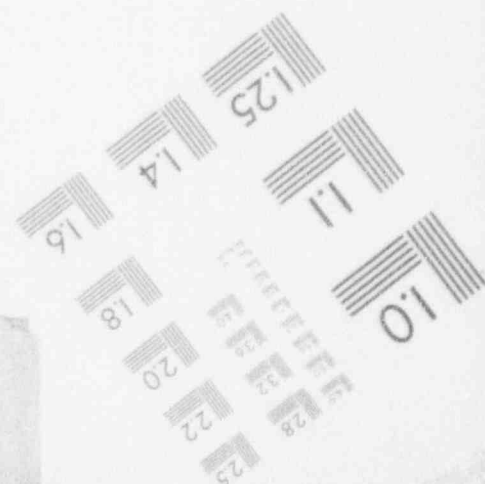
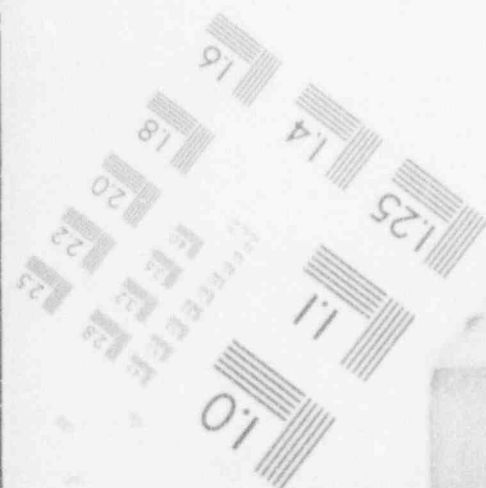
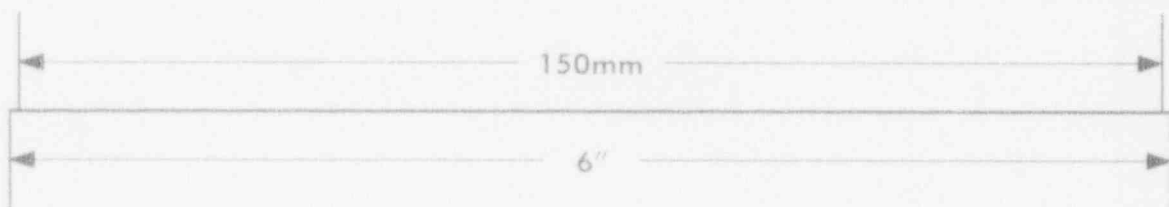
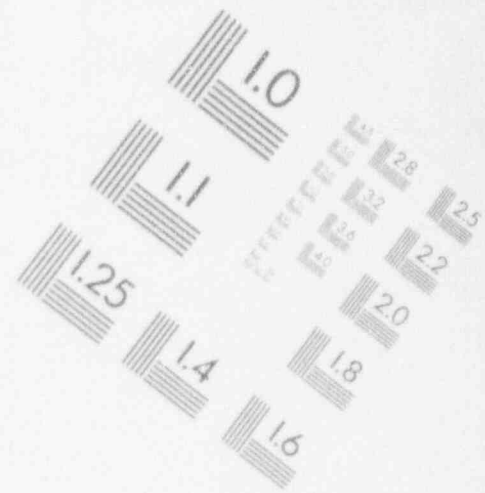
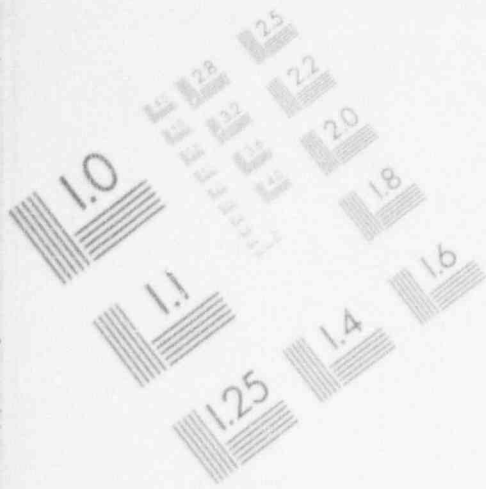
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IMAGE EVALUATION TEST TARGET (MT-3)



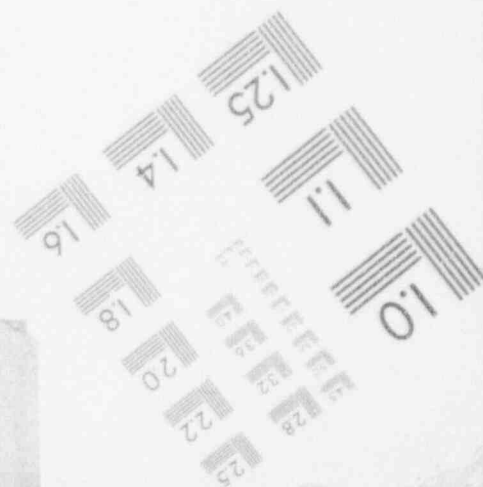
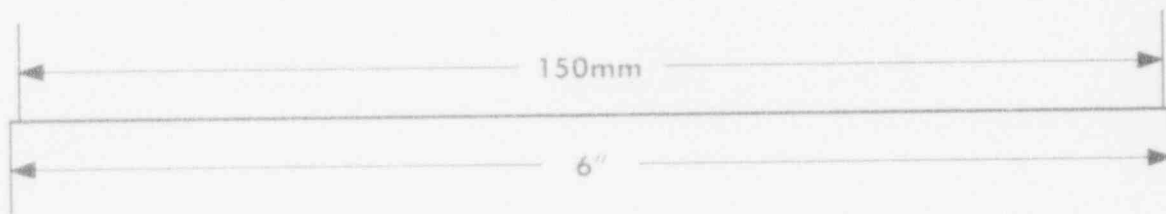
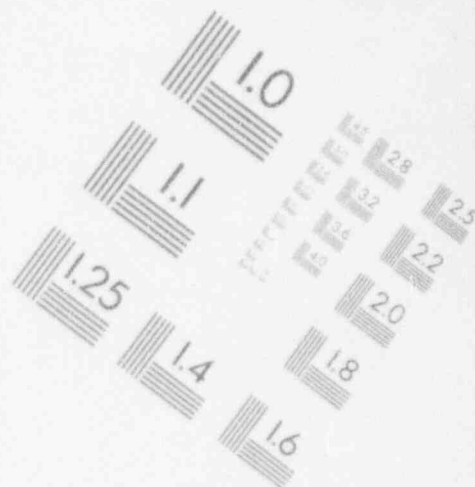
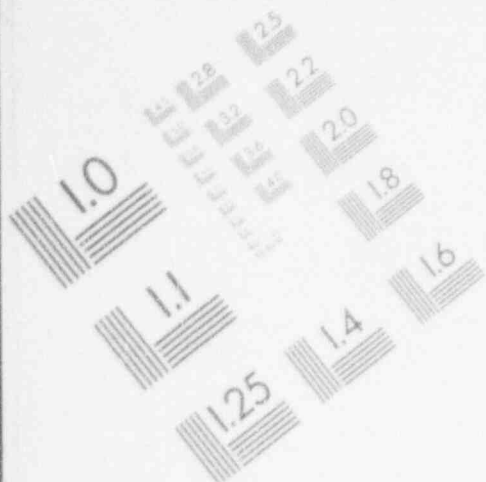
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IMAGE EVALUATION
TEST TARGET (MT-3)



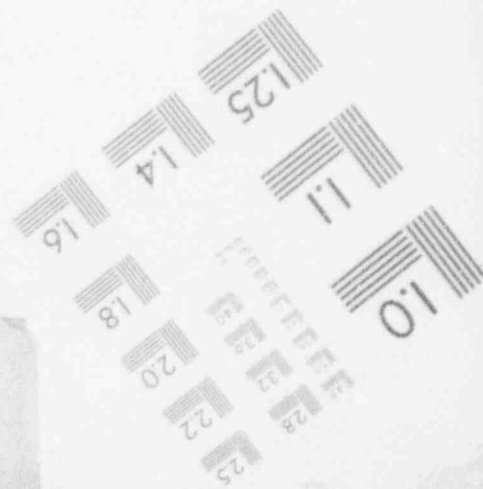
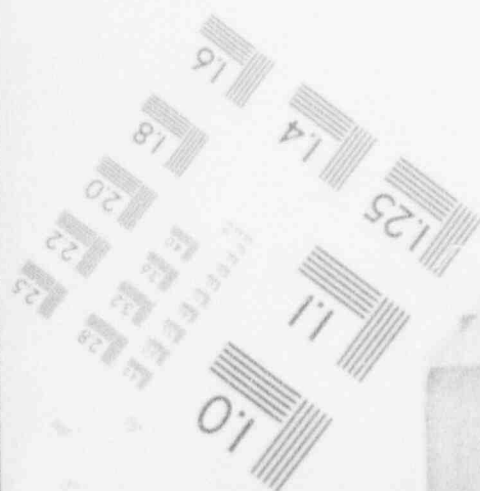
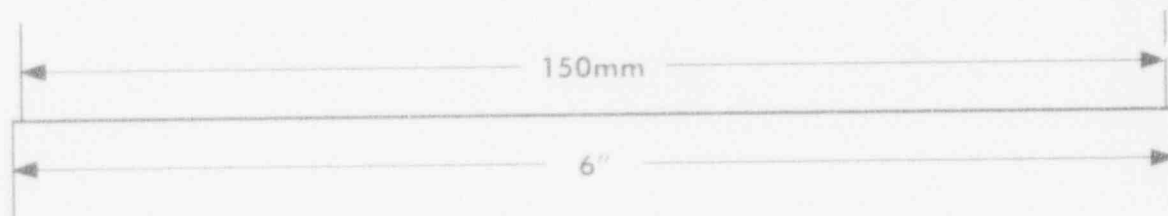
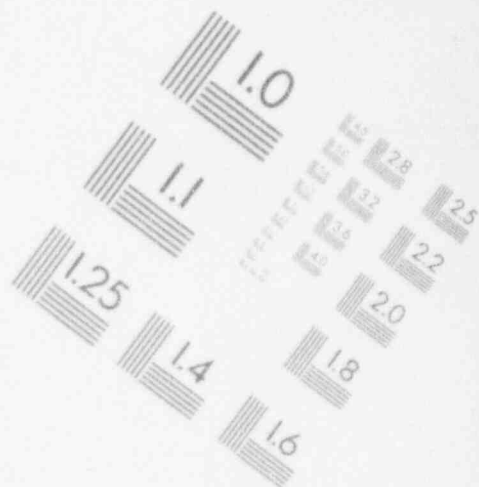
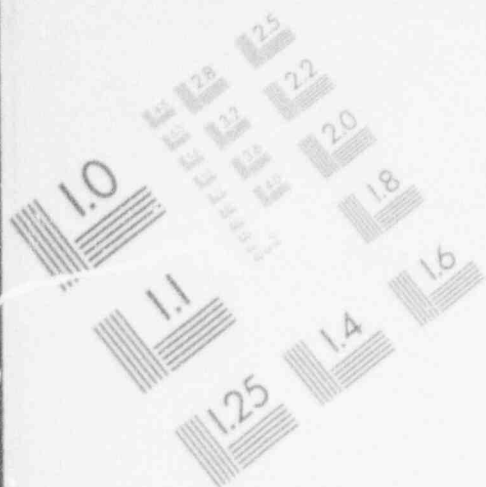
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IMAGE EVALUATION TEST TARGET (MT-3)



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IMAGE EVALUATION TEST TARGET (MT-3)



The mission time for the charging equipment is assumed to be 18 hours. Following termination of ECCS flow, normal charging will be necessary for RCS inventory control. Thus the mission time for ECCS injection and normal charging encompass the 24 hour period considered.

2.15 Bleed and Feed

Question: What pressure relief capability (from the pressurizer) is required for RCS depressurization in order for bleed and feed cooling to be successful?

Answer: 1 out of 2 pressurizer PORVs provides sufficient relief capacity to remove decay heat and maintain RCS pressure below the shutoff head of the high pressure injection ECCS pumps.

Question: What is the mission time for the pressurizer relief valves (PORVs)?

Answer: The mission time for the PORVs is 24 hours.

Discussion:

For the SGTR event scenario in which bleed and feed cooling is required, Reference 7 analyses for small LOCA show that the relief capacity of one pressurizer PORV is sufficient to remove decay heat and maintain the RCS pressure below the shutoff head of the high pressure injection ECCS pumps. This success criteria is carried over to SGTR event.

The mission time for the PORVs is 24 hours. To maintain ECCS recirculation, continued flow through the PORV(s) to the containment recirculation sump is necessary for the 24 hours considered.

2.16 Operator Action for Bleed and Feed

Question: What is the maximum time available for operator action to implement bleed and feed cooling using the pressurizer PORVs?

Answer: For those SGTR cases with no AFW, bleed and feed cooling must begin by 10 hours.

Discussion:

For SGTR scenarios with loss of AFW, the indication to initiate bleed and feed cooling is low level in 3 out of 4 SGs. This low SG level is predicted to be attained at approximately 2 hours (Reference 1, Case 8). However, analysis from Reference 1 (Cases 3 and 4) show that the ECCS flow will maintain adequate core cooling for this scenario until the RWST empties; bleed and feed must be initiated prior to this time to allow the accumulation of inventory in the containment sump to be used for ECCS recirculation. Since the RWST has been determined to empty at 15 hours for this scenario (Reference 1, Case 13) it is assumed that bleed and feed must be initiated by 10 hours in order for sufficient accumulation of RWST water in the containment sump.

2.17 RCFC Operation

Question: What is the minimum number of RCFC units which will prevent automatic containment spray actuation?

Answer: The only SGTR scenario in which containment spray may be actuated is the scenario where the pressurizer PORVs are opened for 'bleed and feed'

cooling. For this instance, 2 RCFC must be in operation to prevent containment spray operation.

Question: What is the minimum number of RCFC units operating which can substitute for the RHR heat exchanger during ECC recirculation?

Answer: With 1 out of 5 RCFCs operating in low speed, no RHR heat exchanger is required.

Question: What is the minimum number of RCFC units operating which will prevent containment failure due to overpressurization following core damage?

Answer: With 1 out of 5 RCFCs operating in low speed, no containment failure will occur.

Discussion:

Three unique success criteria have been established for the operation of the Reactor Containment Fan Coolers: 1) prevention of automatic containment spray actuation, 2) long term containment heat removal, and 3) substitution for the RHR heat exchangers for long term heat removal.

With respect to automatic containment spray actuation, analyses in Reference 15 show that the SGTR case in which the pressurizer PORVs are opened to facilitate bleed and feed following loss of heat sink conditions, operation of 2 out of 5 RCFC units will prevent automatic actuation of containment sprays.

With respect to long term heat removal, analyses in Reference 15 indicate that for a large LOCA, operation of 1 out of 5 RCFC is adequate to remove long term containment heat. This success criteria is applicable to the SGTR event.

Finally, with respect to substitution of the RCFCs for the RHR heat exchangers, analyses in Reference 15 show that 1 out of 5 RCFCs are capable of substituting for the RHR heat exchangers in preventing core damage during ECC recirculation; thus, with 1 out of 5 RCFCs operating during successful ECC recirculation, no RHR heat exchangers are needed.

The success criteria are based on the following assumptions:

1. The RCFC setpoint is the Safety Injection Signal, which activates the RCFC units in the LOW speed mode.
2. The RCFC units are supplied with service water at a temperature of less than 100 degrees Fahrenheit (Reference 9).

2.18 Containment Spray Actuation and Operation

Question: What is the minimum number of containment spray pumps required to operate to prevent containment failure?

Answer: Containment spray pumps cannot prevent containment failure.

Question: What is the minimum number of containment spray pumps required to scrub fission products from the containment atmosphere?

Answer: Scrubbing of containment fission products requires 1 out of 3 containment spray pumps operating at the time of core damage.

Discussion:

The containment spray actuation does not impact either the core damage success or the containment integrity success from the standpoint of containment heat removal. Only in the event of a failure of all feedwater to the steam generators, the subsequent 'bleed and feed' cooling plus a total loss of all RCFCs does the automatic actuation of the containment sprays impact the accident progression and consequences. In this case, the actuation and operation of the containment sprays provides for draining of the RWST water into the containment, thereby preventing core concrete interactions and the attendant additional fission product releases associated with this phenomena.

For the case of draining the RWST, one spray pump alone (2600 gpm) would take 144 minutes to completely drain the RWST; three spray pumps would take 48 minutes. This time difference is not significant in terms of preventing core concrete interactions after reactor vessel failure.

In terms of fission product removal from the containment atmosphere, the most efficient removal occurs during the first 10 to 30 minutes of spray; this is the time frame in which the larger, heavier fission product aerosols are removed. The difference in fission product removal at the end of 30 minutes is very nearly identical for the case of one spray pump or three spray pumps [unpublished analyses for Westinghouse AP600 Design concept].

Thus, the success criteria for containment sprays taking suction from the RWST is 1 out of 3 trains operating. The success of containment sprays for fission product removal also requires that the sprays be operating for at least 10 minutes following

the introduction of fission products into the containment following core damage. If containment sprays are actuated early in an accident and deplete the RWST water prior to core damage, they would not be available for fission product depletion.

As derived previously, the longest time to empty the RWST with only one spray pump operating is 144 minutes. Therefore, the mission time for the containment spray system is taken to be 2.4 hours.

These success criteria are based on the following assumptions:

1. The containment spray setpoint is the coincident safety injection and containment Hi-Hi pressure setpoint (23 psig), which activates all three trains of containment spray, taking suction from the RWST.
2. The emergency a.c. power buses are energized at, or before, the time the containment Hi-Hi pressure signal is received.

2.19 RHR Heat Exchanger Cooling

Question: What is the minimum RHR heat exchanger requirements to prevent core damage during ECCS recirculation?

Answer: The minimum RHR heat exchanger requirements to prevent core damage while on ECCS recirculation is:

- a) with at least 1 out of 5 RCFCs operational, no RHR heat exchangers are required to be functional to prevent core damage,*
- b) with 0 out of 5 RCFCs operational, 0 out of 2 RHR heat exchangers are required to be functional to prevent core damage in the first 24 hours after the accident initiation; however, 1 out of 2 RHR heat exchangers is required for long term heat removal.*

Discussion:

The success criteria for the RHR heat exchangers are carried over from the large LOCA success criteria (Reference 8). These results indicate that ECC recirculation with at least 1 RCFC is sufficient to prevent core damage. Additionally, successful recirculation with 0 RCFCs and 0 RHR heat exchangers will prevent core damage for the initial 24 hours after the initiation of the accident; however, success of at least 1 out of 2 RHR heat exchangers is required for long term heat removal.

2.20 Operator to Establish RHR Heat Exchanger Cooling

Question: What is the maximum time for the operators to establish CCW to the RHR heat exchanger, during ECCS recirculation, to prevent core damage?

Answer: The maximum time for the operators to align the CCW flow to the RHR heat exchanger to prevent core damage is 15 hours, if 0 RCFCs units are operating.

Discussion:

Based on analyses performed for the large LOCA (Reference 15), 1 out of 5 RCFC units can replace the RHR heat exchanger as a means of removing core decay heat during the ECC recirculation phase. Thus, for these cases, there is no requirement for operator action success in aligning the component cooling water to the RHR heat exchanger. For sequences in which 0 RCFCs are available, the operator must align the component cooling water to the RHR heat exchanger in order to remove decay heat. The time at which this alignment must be performed is approximately 15 hours. For the SGTR bleed and feed scenario, the earliest time to drain the RWST and initiate ECCS recirculation is approximately 2.5 hours (Reference 1, Case 15). Although there

is approximately a 2 hour difference between times to ECCS recirculation for SGTR vs. large LOCA, it is assumed that this time will not significantly affect the probability of establishing CCW to the RHR heat exchanger(s). Therefore, the success criteria from the large LOCA are applicable to the SGTR event.

Also, it is noted that there is no requirement to provide CCW water to the RHR heat exchanger to protect the RHR pumps in the recirculation mode since the maximum water temperature, as established in Reference 8, will be less than the temperature of the RCS during normal RHR heat removal operation.

2.21 ECCS Recirculation

Question: What is the minimum ECCS capability required during recirculation?

Answer: The minimum ECC capability during recirculation is 1 RHR pump aligned to either 1 charging pump or 1 SI pump and delivering flow to 2 of the 4 cold legs.

Discussion:

For the SGTR, the only scenario in which ECCS recirculation will be used for long term core cooling is the case in which 'bleed and feed' cooling has been initiated due to the lack of feedwater to the steam generators. For this case, the RCS pressure remains above the shutoff head of the low pressure injection pumps, thus high pressure recirculation is required.

Following the successful realignment of the RHR pumps to recirculation from the containment sump, the RHR pumps must restart and operate for the mission time of ECCS recirculation. During recirculation, 1 RHR pump aligned to either 1 charging

pump or 1 SI pump is sufficient to provide long term core cooling. This success criteria is derived from the core cooling injection phase analyses in Reference 8, wherein either 1 RHR pump alone, 1 charging pump alone, or 1 SI pump alone (delivering flow to 2 out of 4 cold legs) was sufficient to maintain core cooling during the injection phase. Since the decay heat levels are lower during the recirculation phase, the same pump requirements can be justified. In the high pressure recirculating phase, there is no possibility of using the SI or charging pumps in the recirculation mode without the operation of the RHR pumps; the RHR pumps act as booster pumps for the SI and/or charging pumps.

This success criteria is based on the following assumptions:

1. RHR pumps take suction from the containment sump which contains at least 111 inches of water per ES-1.3/Step 3. This provides a water depth which ensures the RHR pump NPSH requirements are met.
2. The SI pumps and charging pumps cannot take suction directly from the containment sump, based on NPSH requirements and piping arrangements.

2.22 Operator Action to Establish ECCS Recirculation

Question: What is the maximum time for the operators to accomplish the ECCS switchover to recirculation in order to prevent core damage?

Answer: The maximum time for the operators to accomplish the switchover procedure to prevent core damage is:

- a) *With containment spray in operation, the time for successful switchover to recirculation is 3 hours.*
- b) *With containment spray inoperable or not automatically actuated, the time for successful switchover to recirculation is 7 hours.*

Discussion:

The time for the operators to establish ECCS recirculation is specified as two distinct parameters; the time at which the switchover to recirculation is to be initiated and the time period within which the switchover operations must be completed.

Based on the structure of the SGTR plant response tree, the operator action time for ECCS recirculation must be established for the cases with and without operation of the containment spray system. For simplicity, the operator action time will be established for the cases in which maximum ECCS and maximum containment spray pumps are available. This gives the minimum operator action time for all SGTR scenarios.

The maximum time available for the operator to establish ECCS recirculation can be derived from the time at which the switchover to recirculation begins, using the following methodology developed in the large LOCA Success Criteria Notebook (Reference 8).

The time required to physically accomplish switchover to low pressure cold leg recirculation, following the steps in Zion procedure ES-1.3, is about 5 minutes. Based on the extra steps to implement high pressure recirculation, the time required to physically accomplish switchover to high pressure recirculation is estimated to be approximately 10 minutes. If switchover to recirculation cannot be accomplished, the

operator would recognize the loss of recirculation capability at about 15 minutes after the initiation of the ES-1.3 procedure in the case of high pressure recirculation.

The maximum operator action times to accomplish switchover to ECCS recirculation for the two cases developed below will be applied to all ECCS switchover actions. The operator action times for other scenarios may be longer than that calculated; however these times, as determined for the limiting cases, should be sufficiently long as to guarantee a high probability of success for these operator actions.

Maximum ECCS and Spray

For the SGTR scenario with maximum ECCS and containment spray actuation (i.e., no RCFCs), the RWST low level alarm is attained in approximately 2.5 hours, while the RWST is completely drained at approximately 2.8 hours (Reference 1, Case 15). At this point, there is no further addition of ECCS water to the RCS. The time to core damage for such a scenario has been determined to be approximately 3 hours following loss of all ECCS injection. Thus, including the time for the operator actions to physically accomplish the switchover, the total time available to the operators to accomplish ECCS switchover may be estimated to be 3 hours beginning at 2.5 hours after event initiation.

Maximum ECCS and No Spray

For the SGTR scenario with maximum ECCS but no containment spray, the RWST low level alarm setpoint is attained in approximately 3.5 hours, while the RWST is completely drained at approximately 5 hours (Reference 1, Case 16). At this point, there is no further addition of ECCS water to the RCS. The time to core damage for such a scenario has been determined to be approximately 6 hours following loss of all ECCS injection. Thus, including the time for the operator actions to physically

accomplish the switchover, the total time available to the operators to accomplish ECCS switchover may be estimated to be 7 hours beginning at 3.5 hours after event initiation.

2.23 RWST Refill

Question: What is the rate at which the RWST must be refilled in order to match ECCS injection flow?

Answer: The RWST refill rate must be:

- 1) 500 gpm for those scenarios with no 'bleed and feed',*
- 2) 400 gpm for those scenarios with 'bleed and feed'.*

Discussion:

For the SGTR event, there are two distinct scenarios in which RWST refill is necessary to maintain core cooling. The first scenario includes those cases in which ECCS flow is maintaining core cooling, such as following SG overfill and the consequential failure of a relief valve causing continued primary to secondary break flow. The first scenario also includes the case in which AFW is not available and 'bleed and feed' cooling should be initiated per FR-H.1 but cannot due to the failure of the pressurizer PORVs. For these cases, the RWST drain is controlled by the equilibrium pressure in the RCS; that is the pressure at which the ECCS flow and the breakflow are approximately equal. The second scenario includes the case in which 'bleed and feed' cooling is initiated via the opening of the pressurizer PORVs, and the RWST drain is controlled by the equilibrium between the flow through the open PORVs and the incoming ECCS flow.

For the first scenario, since the number of pumps will control the equilibrium pressure in the RCS the actual ECCS flow is a function of the number of operating ECCS pumps. A successful RWST refill rate will match the ECCS injection flow rate. To encompass all scenarios, the case with all ECCS pumps operating will be considered. For this instance, the equilibrium ECCS flow is approximately 65 lb/sec, or 500 gpm. Therefore, the RWST refill rate should be 500 gpm. Note that if RWST refill is started right away, a constant refill rate of 325 gpm would successfully maintain RWST inventory for 24 hours.

For the 'bleed and feed' scenario, the operation of both pressurizer PORVs will be considered. In this instance, the equilibrium ECCS flow is approximately 140 lb/sec, or 1100 gpm. Therefore, the RWST refill rate should be 1100 gpm. However, for this case, RWST refill is initiated via ECA-1.1. This procedure includes a graph of 'minimum ECCS flow rate' vs. time after trip. Should the operator utilize this graph, the necessary RWST refill rate can be reduced substantially based on the time after reactor trip. Specifically, for the case with 'bleed and feed' cooling, the latest possible time to initiate ECCS recirculation (or RWST refill) has been determined to be 10 hours after reactor trip with no containment sprays and 5 hours after reactor trip with containment sprays operable. Therefore, the RWST refill rate for these instances based on Figure 1 of ECA-1.1 would be 150 gpm and 175 gpm, respectively.

Realistically, since the flow from the ECCS pumps cannot be throttled, the operator would be expected to minimize ECCS flow by terminating the ECCS pumps. One charging pump delivering flow to the RCS at a pressure of approximately 1000 psia results in a total flow of approximately 400 gpm. Thus if the operators use Figure 1 of ECA-1.1, a successful RWST refill rate would be 400 gpm.

2.24 Operator Action to Establish RWST Refill

Question: What is the maximum time for the operators to initiate RWST refill in order to prevent core damage?

Answer: The maximum time for the operators to initiate RWST refill to prevent core damage is:

- 1) With no 'bleed and feed', the maximum time for the operators to initiate RWST refill is 14 hours,*
- 2) With 'bleed and feed', the maximum time for the operators to initiate RWST refill is 3 hours if containment sprays are operating and 7 hours if containment sprays are inoperable or not actuated.*

Discussion:

For the 'bleed and feed' case, the time for the operators to initiate RWST refill to prevent core damage can be obtained from an evaluation of the operator action time to initiate ECCS recirculation. Based on an evaluation presented in the Large Break LOCA Success Criteria Notebook (Reference 8), it is concluded that the operators would be likely to conclude that the plant was experiencing a loss of ECCS recirculation capability at about 10 minutes after initiation of switchover to recirculation is first attempted. In other words, the operators would transfer to Zion EOP ECA-1.1 (Loss of Emergency Coolant Recirculation) at about 10 minutes after they first enter the ES-1.3 procedure (Transfer to Cold Leg Recirculation).

Upon entry into the ECA-1.1 procedure 10 minutes following the RWST low level alarm, the operators are instructed to begin refill of the RWST. Therefore, it is

assumed that RWST refill operations are started at this time. As noted in Section 2.22, two cases will be examined - containment sprays operating and no containment sprays. For the case with containment sprays operating, the RWST low level alarm is attained at approximately 2.5 hours with the RWST emptied at 2.8 hours (Reference 1, Case 15). At this point, there is no further addition of ECCS water to the RCS. The time to core damage for such a scenario has been determined to be approximately 3 hours following loss of all ECCS injection. Thus, including the time for the operator actions to identify the lack of ECCS recirculation and transfer to ECA-1.1, the total time available to the operators to accomplish RWST refill may be estimated to be 3 hours beginning at 2.5 hours after event initiation.

For the case with no containment sprays, the RWST low level alarm setpoint is attained in approximately 3.5 hours, while the RWST is completely drained at approximately 5 hours (Reference 1, Case 14). At this point, there is no further addition of ECCS water to the RCS. The time to core damage for such a scenario has been determined to be approximately 6 hours following loss of all ECCS injection. Thus, including the time for the operator actions to identify the lack of ECCS recirculation and transfer to ECA-1.1, the total time available to the operators to accomplish RWST refill may be estimated to be 7 hours beginning at 3.5 hours after event initiation.

In the case of RWST refill which is initiated via ECA-3.2 due to low RWST level in conjunction with low containment level, RWST refill is initiated fairly early, when the RWST level is at 19.5 feet. With respect to timing, this level is attained at approximately 5 hours (recall no containment pressurization so no containment sprays), with the RWST emptied at approximately 10 hours. Core damage is not predicted to occur until approximately 20 hours. Therefore, the total time available to the operators to accomplish RWST refill may be estimated to be 14 hours beginning at 5 hours after event initiation.

3.0 SUMMARY

All of the possible success states have been determined for the SGTR plant response tree. These success states, arranged by system, are listed in Table 1. The mission time associated with each of the system success criteria are also summarized in Table 1. The success criteria is listed for each of these nodes as determined in Sections 2.1 through 2.24 of this Notebook. The success criteria for prevention of core damage as well as the success criteria for prevention of large releases of radioactivity are included on this table.

The identifiers for each column are:

AFW	Auxiliary Feedwater
TK	Refueling Water Storage Tank
CCP	ECCS Injection Using the Centrifugal Charging Pump(s)
SIP	ECCS Injection Using the Safety Injection Pump(s)
ORF	Operator Action to Establish Alternate Feedwater
ALT	Alternate Feedwater to Steam Generators
OBL	Operator Action to Initiate Bleed and Feed
BL	RCS Bleed via Two Pressurizer PORVs
OAI	Operator Action to Isolate the Ruptured SG
MSI	Closure of Ruptured SG MSIV (and associated steam paths) OR Closure of Intact SGs MSIVs (and associated steam paths)
OAF	Operator Action to Isolate Feedwater Flow to Ruptured SG
AFI	Closure or Throttling of Feedwater Pump(s) Discharge Valve(s)
ODS	Operator Action to Initiate RCS Cooldown via Intact SGs
DS	RCS Cooldown via Steam Dump from Intact SGs (to condenser or atmosphere)
ODP	Operator Action to Depressurize the RCS

DP	RCS Depressurization via Normal Pressurizer Spray OR Pressurizer PORV OR Auxiliary Pressurizer Spray
OIR	Operator Action to Reduce ECCS Injection
ONC	Operator Action to Establish Normal Charging
NC	Realign Centrifugal Charging Pumps to VCT for Normal Charging
FC	Reactor Containment Fan Coolers
CSI	Containment Spray Injection
OHX	Operator Action to Establish RHR Heat Exchanger Cooling
RHX	RHR Heat Exchanger
ORC	Operator Action to Establish ECCS Recirculation
HPR	High Pressure Recirculation
ORT	Operator Action to Refill the RWST
RTK	RWST Refill
CI	Containment Isolation

Table 1 [Page 1 of 6]
SGTR Success Criteria

Functional Requirement		System or Action	Success Criteria	Conditional Status of Other Systems	Mission Time	Comments	Note-book Sect.
Steam Generator Inventory: Initial Heat Sink	<u>or</u>	AFW	> 1/2 MD AFW Pumps or 1/1 TD AFW Pump to 4/4 SGs	AFW Throttle Valve - Restricted	6.0 Hours	Mission time based on expected max time to complete op acts.	2.3
			> 1/2 MD AFW Pumps to \geq 3/4 SGs	AFW Throttle Valve - Open	6.0 Hours		2.3
			or 1/1 TD AFW Pump to \geq 3/4 SGs	AFW Throttle Valve - Open	6.0 Hours		2.3
		ORF	< 25 minutes @ 5 minutes	AFW Failure All ECCS	N/A	Op Act time based on SG overfill prevention	2.4 2.5
			< 2 hours @ 5 minutes	AFW Failure No ECCS	N/A	Op Act time based on SG overfill prevention	2.4 2.5
		<u>and</u> ALT	1/3 Main Feed Pumps to \geq 1/4 SGs	Condensate Booster Pmps OK	6.0 Hours	Mission time based on expected max time to complete op acts	2.4 2.5
		<u>or</u> ORF	< 2 hours @ 5 minutes	AFW Failure All ECCS	N/A	Op Act time based on bleed & feed time	2.4 2.5
			< 1.5 hours @ 5 minutes	AFW Failure Only 1 CCP	N/A	Op Act time based on bleed & feed time	2.4 2.5
		<u>and</u> ALT	1/3 Main Feed Pumps to \geq 1/4 SGs	Condensate Booster Pmps OK	6.0 Hours	Mission time based on expected max time to complete op acts	2.4 2.5

Table 1 [Page 2 of 6]
SGTR Success Criteria

Functional Requirement		System or Action	Success Criteria	Conditional Status of Other Systems	Mission Time	Comments	Note-book Sect.
Water Source for Core and Containment Cooling		TK	> 224,890 gal and > 1700 ppm boron	None	24.0 Hours	Basis provided in 'Overall Criteria and Special Systems' SC	Ref. 16
Injection Phase Core Cooling	<u>or</u>	CCP	> 1/2 Charging Pmps to \geq 2/4 Cold Legs	TK Success	6.0 Hours	High Press Inject only required if AFW fails	2.1
		SIP	> 1/2 SI Pumps to \geq 2/4 Cold Legs	TK Success	6.0 Hours	Only required if CCP fails	2.1
Feed and Bleed Core Cooling	<u>and</u>	OBL	< 8 hours @ 2 hours	AFW, ORF and ALT Fail	N/A	Begin Bleed & Feed before RWST empty	2.15 & 2.16
		BL	1/2 Przr PORVs	CCP or SIP Success	24.0 Hours	Need water thru PORVs for ECCS Recirc	
Steam Generator Isolation	<u>and</u>	OAI	< 20 Minutes @ t=0	None	N/A	SGTR diagnosed; rupt. SG identified	2.6 & 2.7
		MSI	1/1 MSIV on ruptured SG <u>or</u> 3/3 MSIVs on intact SGs	None Rupt SG MSIV failure	24.0 Hours 24.0 Hours	 Mission time based on duration of SG Iso. All other steam paths should be isolated	
		OAF	< 20 minutes @ t=0	AFW or ORF/ALT Success	N/A	SGTR diagnosed; rupt. SG identified	
		AFI	1/1 MD AFW flow regulating valve and 1/1 TD AFW flow regulating valve, to ruptured SG	AFW Success	24.0 Hours	Mission time based on duration of SG iso	

Table 1 (Page 3 of 6)
SGTR Success Criteria

Functional Requirement		System or Action	Success Criteria	Conditional Status of Other Systems	Mission Time	Comments	Note-book Sect.
Steam Generator Isolation (cont)		AFI (cont)	<u>or</u> 1/1 MFW flow reg valve and 1/1 MFW flow bypass reg valve, to rupt SG	AFW Failure; ORF/ALT Success	24.0 Hours	Mission time based on duration of SG iso	
Initial RCS Cooldown	or	ODS <u>and</u> DS	< 5 minutes @ 20 minutes 2/3 steam dump valves <u>or</u> 2/3 intact SG ARVs	OAI/MSI Success Condenser Available Condenser Not Available	N/A 6.0 Hours	Prevent SG overfill Mission time based on expected max time to complete op acts.	2.8 & 2.9
		ODS <u>and</u>	≤ 1 hour	OAI/MSI Failure OAF/AFI Failure or CCP Failure SIP Success	N/A	100°F/hr cooldown to Reduce ECCS, extend RWST Availability	
		DS	2/3 steam dump valves <u>or</u> 2/3 intact SG ARVs	Condenser Available Condenser Not Available	6 Hours	Mission time consistent with that above	
RCS Depressurization		ODP <u>and</u> DP	< 5 minutes @ 35 minutes Przr Normal Spray <u>or</u> 1/2 Przr PORVs <u>or</u> Przr Aux Spray	ODS/DS Success RCPs Not Tripped Przr Normal Spray Unavail Przr Worm Spray & PORV Unavail	N/A 24.0 Hours 24.0 Hours 24.0 Hours	No RCS Depress until RCS Cooldown Complete Success End State can be attained without RCS Depressurization Mission times based on bleed and feed	2.10 & 2.11

Table 1 [Page 4 of 6]
SGTR Success Criteria

Functional Requirement		System or Action	Success Criteria	Conditional Status of Other Systems	Mission Time	Comments	Note-book Sect.
RCS Inventory Control		OIR	< 5 minutes @ 47 minutes	ODP/DP Success	N/A	Prevent SG overfill	2.12
			< 10 minutes @ 35 minutes	ODP/DP Failure	N/A		
		and ONC	< 5 minutes @ 47 minutes	ODP/DP Success OIR Success	N/A		2.13
			< 10 minutes @ 35 minutes	ODP/DP Failure OIR Success	N/A		
		and NC	1/2 Charging Pump in Normal Charging Mode.	OIR Success ONC Success	18 Hours	Mission time based on 6 hrs of initial injection	2.14
	or	OIR	< 1 Hour	OAI/MSI Failure or OAF/AFI Failure or CCP Failure SIP Success and ODS/DS Success	N/A	Extend RWST Availability	2.12
Recirculation Core Cooling		ORC	< 3 hours @ 2.5 hours	CSI Success All ECCS	N/A	Recirc only if AFW Failure and Bleed & Feed initiated	2.22 & 2.21
			< 7 hours @ 3.5 hours	CSI Failure All ECCS	N/A		
		and HPR	> 1/4 High Pressure pumps (CCP or SIP) to \geq 2/4 Cold Legs	RHR aligned to recirc sump; ORC Success	18.0 Hours	Mission time based on 6 hr initial injection	

Table 1 (Page 5 of 6)
SGTR Success Criteria

Functional Requirement		System or Action	Success Criteria	Conditional Status of Other Systems	Mission Time	Comments	Note-book Sect.
RWST Refill	or	ORT	< 14 hours @ 5 hours	No Bleed & Feed	N/A	Prevents Core Damage if ECCS Recirc is Unavailable	2.6 & 2.24
		and RTK	> 500 gpm refill		18 hours		
		ORT	<3 hours @ 2.5 hours	Bleed & Feed initiated; CSI Success	N/A	Mission time based on 6 hour initial injection	
		and RTK	< 7 hours @ 3.5 hours > 400 gpm refill	Bleed & Feed initiated; CSI Failure	N/A 18 Hours		
Long Term Heat Removal	or	FC	> 1/5 RCFC	None	24.0 Hours	Prevents Containment Failure for Core Damage Sequences	2.17 & 2.19
		FC and ORC	= 0/5 RCFC < 3 hours @ 2.5 hours	CSI Success All ECCS	24.0 Hours N/A	Success, but requires Accident Management	2.20 & 2.21
		and HPR	< 7 hours @ 3.5 hours > 1/4 high pressure pumps (CCP or SIP) to >2/4 cold legs	CSI Failure All ECCS RHR aligned to recirc sump; ORC Success	18.0 Hours	AFW, ORF/ALT Fail; init. bleed & feed in order for ECCS recirc	2.22
		or					

Table 1 (Page 6 of 6)
SGTR Success Criteria

Functional Requirement		System or Action	Success Criteria	Conditional Status of Other Systems	Mission Time	Comments	Note-book Sect.	
		FC	= 0/5 RCFC	None	24.0 Hours	Prevents Containment Failure for Core Damage Sequences		
		and DHX	< 15 hours		N/A			
		and RHX	> 1/2 RHR Hx	DHX Success	24.0 Hours	AFW, ORF/ALT Fail; init. bleed & feed in order for ECCS recirc Accident Management		
		and RPR	> 1/4 high pressure pumps (CCP or SIP) to $\geq 2/4$ cold legs	RHR aligned to recirc sump; ORC Success	18.0 Hours			
Short Term CTMT Heat Removal/ Fission Product Scrubbing		CSI	> 1/3 Spray Pumps	TK Success	2.4 Hours	Not Required for Pressure Reduction	2.18	
Containment Isolation		CI	All Lines > 2" Dia. "Isolated"	None	24.0 Hours		1.17	

4.0 ACCIDENT MANAGEMENT/IPE INSIGHTS

The IPE and accident management insights identified during the development of the SGTR success criteria are documented in this section.

RCP Trip

In Step 15 of ECA-3.1 and Step 10 of ECA-3.2, the operators are instructed to stop 2 of 4 RCPs. A note in the procedure instructs the operators to trip RCPs A and C, since normal pressurizer spray is supplied via loops B and D. However, analyses from Reference 1 show that reverse heat transfer may occur in the loop with the ruptured steam generator tube. This energy transfer to the RCS from the secondary is a competing effect to the RCS cooldown which should be in progress in ECA-3.1 or ECA-3.2. The same analyses show that RCP trip in the ruptured loop results in a reduction in this negative heat transfer, thus improving the RCS cooldown rate. Since the RCS cooldown rate may be important once the operators are completing actions via ECA-3.1 or ECA-3.2, it is suggested that these procedural steps include a note regarding reverse heat transfer and the tripping of the RCP in the loop with the ruptured steam generator tube.

RCS Cooldown and SG Level

One of the important operator actions following a SGTR event is the RCS cooldown. Step 14 of E-3 instructs the operators to cooldown the RCS at "maximum" rate until the target temperature is reached and then maintain a 100°F/hr cooldown rate. Step 8 of ECA-3.1 and step 4 of ECA-3.2 instructs the operators to cooldown the RCS at 100°F/hr until the RHR system can be placed in service. Now consider the AFW flow rate. Depending on the operability of the AFW pumps, the AFW flow to each SG will range from approximately 100-200 gpm. Analyses from Reference 1 show that

this AFW flow is not sufficient to maintain level in the SGs during the initial phase of the RCS cooldown. Calculations indicate that with all AFW pumps injecting, and with maximum cooldown rate, the SG level will not stabilize until approximately 100°F of cooling has been achieved. Therefore, it is suggested that a note or caution be included in the Zion EOPs to inform the operators of the expected SG level decrease during the RCS cooldown.

SG Overfill in all Steamlines

It is noted that for the scenario in which the ruptured SG cannot be isolated, there is a high probability that SG overfill will occur. Without isolation of the ruptured SG, the secondary water will spill into the steam lines of all steam generators. Although References 13 and 14 indicate that no failure of the steam lines will occur, this is still a highly undesirable situation. Therefore, the importance of closing the MSIV in the ruptured SG steamline should be highly stressed during operator training.

Bleed and Feed

Analyses from Reference 7 indicate that only 1 pressurizer PORV is needed for success of the 'bleed' portion of bleed and feed cooling, although the EOP instruct the operators to open both pressurizer PORVs. Perhaps the FR-H.1, Step 17 RNO column should be modified such that the operators continue with FR-H.1 procedure if only 1 PORV can be opened instead of being directed to open all head vent valves, depressurize a SG to atmospheric & align service water.

5.0 REFERENCES

1. Letter DFH-91-015 of March 6, 1992 from Doug Holderbaum (Westinghouse at CEC) to File, Subject: SGTR success criteria.

2. System Notebook for Electrical Power Distribution System, Zion Station Units 1 and 2, prepared by IPEP, Revision 0, April 1992.
3. CEC Co Letter of June 11, 1990 from George Klopp (CECo) to M. J. Loftus (Westinghouse).
4. Westinghouse letter of July 15, 1991 from Bob Lutz (Westinghouse) to Xavier Polanski (CECo), Subject: AFW system design.
5. "Background Document for FR-H.1: Response to Loss of Heat Sink, Westinghouse Owners Group Emergency Response Guidelines, Rev. 1A, September, 1989.
6. Letter DFH-91-001 of January 2, 1991 from Doug Holderbaum (Westinghouse at CEC Co) to Bob Lutz (Westinghouse), Subject: Zion Small LOCA MAAP Analyses
7. Small LOCA Success Criteria Notebook, Zion Station Units 1 and 2, prepared by IPEP, Revision 0, March 1992.
8. Large LOCA Success Criteria Notebook, Zion Station Units 1 and 2, prepared by IPEP, February 1992.
9. Letter from X. Polanski (CECo) to Bob Lutz dated June 7, 1990 regarding Zion MAAP parameter file.
10. Letter from X. Polanski (CECo) to M. Loftus (Westinghouse) dated February 5, 1991 regarding RHR pump operability.
11. Letter from George Klopp (CECo) to M. Loftus (Westinghouse) dated June 11, 1990.
12. Steam Generator Tube Rupture Plant Response Tree Notebook, Zion Station Units 1 and 2, prepared by IPEP, March 1992.
13. NUREG-0844, "NRC Integrated Program for the Resolution of Unresolved Safety Issues A-3, A-4 and A-5 Regarding Steam Generator Tube Integrity," Final Report, September 1988 (Division of Engineering and System Technology, Office of Nuclear Reactor Regulation, USNRC).
14. Letter dated October 20, 1982 from L.C. Shiek (Lawrence Livermore Laboratory) to K.R. Wichman (NRC); Subject: Maximum Stresses in Zion Unit 1 Main Steam Piping.

15. Letter from X. Polanski (CEG) to M. Loftus (Westinghouse) dated February 18, 1992 regarding Zion Success Criteria.
16. Overall Criteria and Special Systems Success Criteria Notebook, Zion Station Units 1 and 2, prepared by IPEP, April 1992.
17. Containment Isolation Notebook Zion Station Units 1 and 2, prepared by IPEP, February 1992.
18. Westinghouse Calculation Note CN-COA-92-106, SGTR Success Criteria Regarding ECCS Reduction.

NRC Information Request - Zion IPE
- HRA Phase II Validation Checklists

The checklists are included as Appendix A and B in the Phase II HRA Notebook, and are attached.

Appendix A:

Checklist for Identifying the
Performance Shaping Factors that
Apply to Each Operator Action

Performance Shaping Factor Checklist

Operator Action _____ Plant _____ Date _____
Initiating Event / Special Conditions _____

1. Diagnosis/ Situation Assessment PSFs

Does operator understand nature of event?

What is likelihood operators will misinterpret event?

Help: symptoms/indications strongly indicate initiating event

- ___ initiating event symptoms/indications are very clear and lead to single conclusion
 - ___ primary indication is alarmed
 - ___ primary indication is carefully monitored or routinely scanned
- ___ initiating event symptoms/indications are frequently practiced in the simulator within the context of this event
- ___ event is perceived by operators to be a high-likelihood event
- ___ operator workload is low when indications occur
- ___ other indicators are quiet (no other alarms) when indications occur
- ___ the procedure supports interpretation of initiating event well
- ___ the procedure has "catch" steps to detect errors in interpretation
- ___ other factors -

Hinder: symptoms/indications mask or obscure initiating event

- ___ there is no single initiating event
- ___ initiating event symptoms/indications are difficult to perceive (i.e., not salient)
- ___ operator workload is very high when indications occur
- ___ other malfunctions occur to obscure or mask primary event
- ___ other manual or automatic action occurs to obscure or mask primary indications (e.g., shrink and swell)
- ___ initiating event symptoms/indications are perceived but not easily interpreted
 - ___ indications are misleading
 - ___ indications are likely to be interpreted as something else (more familiar)
- ___ initiating event symptoms/indications are perceived but not given weight
 - ___ indications are likely to be explained away as "noise"
 - ___ indications are interpreted as false alarm
- ___ some critical indicator is available only to a single RC and is unlikely to be picked up by other control room personnel
- ___ event is perceived by operators to be a very low-likelihood event
- ___ other factors -

2. Procedure Selection PSFs

What is likelihood operator will identify and transfer to correct procedure?

Help: indications and procedure criteria are clear for transition to correct procedure

- ☐ criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- ☐ criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation
- ☐ other factors -

Hinder: indications may not be clear or criteria for transition may be ambiguous

- ☐ criterion for transition to correct procedure is not explicit in current procedure
- ☐ criterion for transition to correct procedure requires judgment or interpretation
- ☐ criterion for transition to correct procedure requires sustained monitoring to judge (e.g., trends over time)
- ☐ primary indications for transition may not be manifest when transition step is reached
- ☐ primary indications for transition may dissipate or disappear before transition step is reached
- ☐ other indications may result in transition to a different procedure before "desired" transition step is reached
- ☐ there are strong indications to transfer to a different procedure
- ☐ other factors -

3. Intention to Act PSFs - Specific Cues to Action

What is likelihood operator will perceive cues to action?

Help: indications for action are salient and unambiguous

- ☐ cues are identified in procedure in unambiguous way (i.e., objective, clear)
- ☐ cues are highly salient (i.e., position, discriminability)
- ☐ there is high redundancy in cues
- ☐ low operator workload when cues occur
- ☐ other factors -

Hinder: indications are obscured

- ___ cues are not reliable (given event)
- ___ cues are obscured by other indications
- ___ cues are not located near likely operator positions (hard to find)
- ___ cues require mental effort
 - comparison of several indications
 - calculation, determination of rate
 - knowledge of special context (e.g., setpoint shift)
- ___ high operator workload when cues occur
- ___ there are more familiar or frequent interpretations of cue
- ___ there is likely to be change in personnel (e.g., shift change) between initial event and time of action
- ___ other factors -

4. Intention to Act PSFs - Likelihood for Intentional Violation

What is likelihood operator will intentionally NOT take action or will delay action?

Help: Action is compatible with all goals

- ___ the action's effect is clearly understood and fits well with the goals of the current procedure
- ___ the operators are well trained on the goals of the action and of the larger procedure
- ___ training, procedures, and organizational climate (i.e., safety culture) instill and reinforce appropriate goal prioritization
- ___ other factors -

Hinder: other goals conflict with action or severe economic consequences will result and operator significantly delays or totally avoids action

- ___ taking action may violate standard operating practice (e.g., take operator out of optimal operating band but not into unsafe condition)
- ___ taking action may lead to reduced availability of safety systems, equipment, or instruments (may violate tech specs or design basis availability)
- ___ taking action may have a negative effect on some safety function (goal conflict)
- ___ there is a significant uncertainty or unknown risk associated with taking the action (e.g., PORV after being opened may stick open)
- ___ taking the action will adversely affect areas within plant and further burden recovery (e.g., contaminate aux building which will increase effort needed to do maintenance)

- ☐ taking the action will have severe consequences associated with cost (e.g., plant will be shut down for major cleanup after bleed and feed)
- ☐ taking the action will release radiation to environment
- ☐ other factors -

5. Intention to Act PSFs - Scheduling/Prioritizing the Action :

What is likelihood operator will not take action due to competition from other activities?

Help: The action takes precedence over other actions and can be executed immediately

- ☐ the action is very high in priority
- ☐ the action can be executed immediately; it does not rely on other actions
- ☐ the action is needed to allow other operators to continue working
- ☐ other factors -

Hinder: Other actions compete for resources or there is delay before action can occur

- ☐ there are other actions of greater importance or greater urgency
- ☐ the procedure is written to allow significant flexibility for sequencing of actions (e.g., words such as "as time permits...")
- ☐ the action may not be executed immediately because there is a need for another criterion to be satisfied first (e.g., wait till a parameter reaches value x)
- ☐ the action may not be executed immediately because operators are trying to achieve the goal through another (more preferred) action
- ☐ the action requires several operators to coordinate activities
- ☐ other factors -

6. Execution PSFs - Omission and Commission Errors

What is the likelihood operator will omit step or execute it incorrectly?

Help: Context, procedures, etc. lead to specific actions

- ☐ procedure is highly practiced and/or memorized
- ☐ action is logically required to proceed in procedure (e.g., interlock or permissive)
- ☐ controls are labelled or grouped to make them easily identified
- ☐ execution uses controls with only two settings; controls are clearly marked
- ☐ other factors -

Hinder: Specific actions are somehow incompatible with other aspects

- ☐ execution requires a difficult coordination between operators
- ☐ execution requires a control action to be taken outside of control room
- ☐ execution requires following procedures with an unusual or difficult logic

- ___ execution requires information found in a caution or note (not in procedural step)
- ___ specific information (e.g., valve control number) is not specified in procedure
- ___ actions required for procedure are severely underspecified
- ___ execution requires the use of more than one operating procedure
- ___ execution requires a long list of substeps
- ___ a major component of set of actions is strongly associated with another context and may, therefore, lead to inappropriate actions (capture-type slip)
- ___ execution is likely to be done in order different from procedure's order
- ___ controls are not placed near important indicators that determine execution
- ___ controls are likely to be confused with other similar controls
- ___ controls go against standard operational stereotype (e.g., flip a toggle up to turn off)
- ___ control can have many different settings, each of which has different meaning
- ___ execution requires some type of continuous control (e.g., tuning) where feedback is difficult to judge (e.g., delayed in time)
- ___ execution requires maintaining a parameter within a tight operating band (e.g., to avoid inadvertent trip or safety system activation)
- ___ other factors -

7. Execution PSFs - Detection of Errors

What is the likelihood operators will recognize error has been made?

Help: Formal checks to identify errors

- ___ procedure has explicit catch steps or verifications
- ___ other operators are likely to do careful checking of performance
- ___ there is a very salient indication when error is made or when action was successful (e.g., alarm, interlock)
- ___ other factors -

Hinder: Little or no feedback/indication error was made

- ___ other operators are all occupied in some other activity and will not check performance
- ___ there is very poor feedback on effect of control action
 - no direct indication
 - indication is not close to control
- ___ other factors -

8. Execution PSFs - Recovery from Error

What is likelihood operator can recovery from error?

Help: Formal procedure to recover

- ☐ there is procedure written for recovery from error
- ☐ other factors -

Hinder: Little or no indication of how error has changed situation; recovery actions unclear

- ☐ incorrect execution cannot be recovered due to damage done
- ☐ recovery requires a set of actions different from the set of actions done incorrectly
- ☐ there is severe time constraints for executing recovery actions
- ☐ other factors -

9. Local Stress Factor

Combine the following to determine stress on operator taking actions:

- workload
- time available vs time required
- potential severe consequences of actions being taken

Appendix B

Summary of Each Operator Action and Its Relevant PSFs

Establish ECC Recirculation

Operator Action: ORC1 and ORC2

Initiating Event: SB LOCA and in LB LOCA

Action:

During LOCA events, the operator is required to begin switchover to sump recirculation when the RWST low-level alarm is actuated. This switchover requires aligning the ECCS for recirculation and starting and stopping the ECCS pumps. ORC1 accounts for low-pressure recirculation and ORC2 is high-pressure recirculation.

Set of actions:

ORC1

- perceive RWST low-level alarm (1 out of 1 new)
- depress SI reset button (1/1)
- stop all but one containment spray pumps (2/3)
- stop RHR pump B (1/1)
- close RHR pump B suction isolation valve (1/1)
- simultaneously open recirc sump isol. valve (1/1)
 AND close RHR cross-over valves (2/2)
- start RHR pump B (1/1)

ORC2

- open CHG pump inlet valve (1/1)
- close makeup from RWST valves (2/2)
- close SI pump to RWST recirc (2/2)
- verify valve is closed (1/1)
- open cross-over valves (2/2)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

In general, operators should be well aware that they have a LOCA situation; especially in the case of the LB LOCA. The following factors apply:

Help:

- a. initiating event symptoms/indications are very clear and lead to single conclusion
- b. initiating event symptoms/indications are frequently practiced in the simulator within the context of this event
- c. event is perceived by operators to be a high-likelihood event
- d. operator workload is low when indications occur (except for SB LOCA with loss of feedwater)
- e. other indicators are quiet (no other alarms) when indications occur

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure. The following factors apply:

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient. The following factors apply:

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)
- c. low operator workload when cues occur

4. Intention to Act PSFs - Likelihood for Intentional Violation

There seems to be no threat of this occurring (except in case where cooldown is successful but RHR cannot be initiated due to mechanical problems. In this case, operator may delay ORC action in attempting to correct mechanical problems.)

Help:

- a. the action's effect is clearly understood and fits well with the goals of the current procedure
- b. the operators are well trained on the goals of the action and of the larger procedure

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There seem to be no concerns here

Help:

- a. the action is very high in priority
- b. the action can be executed immediately; it does not rely on other actions

6. Execution PSFs - Omission and Commission Errors

No major problems have been identified in this area. All control room actions are executed with buttons or controls that only have two positions, and all controls are well marked and segregated.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There is a concern here that unrecoverable damage can be done (actions in ORC1 are "Stop RHR pump B" and "Open sump valve")

Hinder:

- a. incorrect execution cannot be recovered due to damage done
(note: this applies to only two actions and a minute or two are available for recovery)

9. Local Stress Factor

Workload

workload on key operator is probably low. Nothing urgent is occupying his time and it is likely there will be no other alarms at time of key alarm.

Time available vs time required

no significant time pressure. There is probably 20-30 minutes for recovery after action in the case of the LB LOCA and 60-90 minutes for the SB LOCA.

Potential severe consequences of actions being taken

failure to establish recirculation has serious consequences for core damage. If recirculation cannot be established, the RWST must be used (see ORT)

Refill the RWST

Operator Action: ORT

Initiating Events: SB LOCA and LB LOCA

Special Conditions: reached from procedure (ORT-p)
reached from diagnosis of failure in ORC (ORT-d)

Action:

Entry to RWST refill is based on a diagnosis of the loss of RHR pump capability. This loss can occur under one of two conditions: 1) if RHR pumps are damaged early, or 2) if ECC recirculation cannot be aligned. Under one of these conditions, the operator is required to initiate makeup water to the RWST using all possible methods.

Note, the success of this action is especially critical when the switchover to sump recirculation action (ORC) is not carried out in response to the RWST low-level alarm (that is, ORC fails - the RHR pumps are not working). In this case the operator must transfer to the Loss of Emergency Coolant Recirculation procedure (ECA-1.1). Here, the operator is instructed to initiate makeup water to the RWST using whatever means available.

Set of actions:

-perceive RWST low-level alarm

Blender makeup

- close VCT isolation valve (1/2)
- close makeup injection valve (1/2)
- open blender to RWST (outside CR) (1/1)
- open blender isolation (outside CR) (1/1)
- adjust makeup controls for H₂O and boric acid (outside CR)
- start PW pump
- change pump speed

Gravity refill of RWST from other unit's RWST

- open four pump suction valves (outside CR)

Refill RWST from spent fuel pit

- open valve (1/1)
- open valve (1/1)
- open valve (1/1)
- close either valve
- close valve (1/1)

Maximize makeup to VCT

- close valves (2/2)
- open valves (2/2)
- open valves (2/2)
- adjust makeup controls for max flow
- perceive RWST low-low level alarm (level < 5 ft)
- stop RHR, SI, charging, and cntmnt spray pumps (8/8)

Establish normal charging flow

- close BIT outlet valves (2/2)
- open chg header isol. valves (2/2)
- open VCT valves (2/2)
- close emergency makeup valves (2/2)
- start chg pump (1/1)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

In general, operators should be well aware that they have a LOCA, especially in the case of a LB LOCA. However, the failure of the RHR pumps and ensuing need for RWST refill (ORT-d) is probably a low-likelihood event.

Help:

- a. initiating event symptoms/indications are very clear and lead to single conclusion
- b. operator workload is low when indications occur (ORT-p)

Hinder:

- a. event is perceived by operators to be a very low-likelihood event (ORT-d)
- b. operator workload is very high when indications occur (ORT-d)

2. Procedure Selection PSFs

The procedures may not help in the ORT-d case.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure (ORT-p)
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation (ORT-p)

Hinder:

- a. criterion for transition to correct procedure is not explicit in current procedure (ORT-d)
- b. criterion for transition to correct procedure requires judgment or interpretation (ORT-d)

3. Intention to Act PSFs - Specific Cues to Action

There may be high workload in the ORT-d case due to repeated attempts to get RHR pumps going.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear) (ORT-p)
- b. cues are highly salient (i.e., position, discriminability) (ORT-p)

Hinder:

- a. high operator workload when cues occur (ORT-d)

4. Intention to Act PSFs - Likelihood for Intentional Violation

Operators may be aware of some risks associated with extended use of the RWST.

Hinder:

- a. taking action may lead to reduced availability of safety systems, equipment, or instruments (may violate tech specs or design basis availability) (ORT-d and p)

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There may be difficulty in ORT-d with giving up on attempts to regain RHR pumps. There may also be difficulty associated with selecting and prioritizing sources of water (p + d)

Help:

- a. the action can be executed immediately; it does not rely on other actions (ORT-p)

Hinder:

- a. the action may not be executed immediately because operators are trying to achieve the goal through another (more preferred) action (ORT-d)
- b. the procedure is written to allow significant flexibility for sequencing of actions (operator must select subset of methods and prioritize)

6. Execution PSFs - Omission and Commission Errors

There are several potential difficulties here.

Help:

- a. procedure (for blender makeup only) is highly practiced or memorized

Hinder:

- a. execution requires a control action to be taken outside of control room
- b. specific information for some actions (e.g., valve control number) is not specified in procedure
- c. actions required for procedure are severely underspecified
- d. execution requires a long list of substeps

7. Execution PSFs - Detection of Errors

There are no significant problems here

8. Execution PSFs - Recovery from Error

There are no problems here

9. Local Stress Factor

Workload

ORT-p: workload is moderate

ORT-d: workload may be high because the operator may be trying to restore the RHR pumps before going to RWST refill.

Time available vs time required

time factors are somewhat unknown, so the operator may be concerned

Potential severe consequences of actions being taken

failure to establish another source of water when recirculation is not possible has serious potential consequences

Establish Normal Charging

Operator Action: ONC

Initiating Events: SGTR and SB LOCA

Action:

This is done subsequent to minimizing the ECCS flow (an extension of OIR). For an SGTR event, the operators are required to establish RCS inventory control via normal charging flow. This may occur subsequent to RCS depressurization, but depressurization may not be needed.

Set of Actions:

- verify chg pump miniflow valves open (2/2)
- close BIT inlet valves (2/2)
- close BIT outlet valves (2/2)
- open chg header isol valves (2/2)
- verify chg line stop valve open (1/1)
- open RCP seal return valve (1/1)
- check RCP LBRT deltaP 20-60 inches (1/1)
- control chg flow to maintain level between 20 and 80%

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

In general, operators should be well aware that they have an SGTR or a LOCA situation.

Help:

- a. initiating event symptoms/indications are very clear and lead to single conclusion
- b. initiating event symptoms/indications are frequently practiced in the simulator within the context of this event
- c. event is perceived by operators to be a high-likelihood event
- d. operator workload is low when indications occur
- e. other indicators are quiet (no other alarms) when indications occur

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)
- c. low operator workload when cues occur

4. Intention to Act PSFs - Likelihood for Intentional Violation

There seems to be no threat of this occurring

Help:

- a. the action's effect is clearly understood and fits well with the goals of the current procedure
- b. the operators are well trained on the goals of the action and of the larger procedure

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There seem to be no concerns here

Help:

- a. the action can be executed immediately; it does not rely on other actions

6. Execution PSFs - Omission and Commission Errors

No major problems have been identified in this area. All control room actions are executed with buttons or controls that only have two positions, and all controls are well marked and segregated.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There are no problems here.

9. Local Stress Factor

Workload

workload on key operator is probably low.

Time available vs time required

no significant time pressure. The action is expected to be completed within 52 minutes when RCS depressurization is used and; 45 minutes when depressurization is not required

Potential severe consequences of actions being taken

no severe consequences of failure

Steam Generator Depressurization for Primary Cooling

Operator Action: ODS2

Initiating Event: SGTR

Action:

For an SGTR event, the operators are required to initiate an RCS cooldown by dumping steam from the intact steam generators to attain subcooling in the RCS. The operator is expected to dump steam from the intact steam generators via the condenser cooldown valves (CCVs) or atmospheric relief valves (ARVs) at the maximum rate prior to the subsequent RCS depressurization step.

Set of Actions:

- turn off PRZR heaters (2/2)
- open 3 SG ARVs or 3 CCVs (3/3)
- verify RCS temp < 540 deg
- place switches in bypass interlock position momentarily (2/2)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

In general, operators should be well aware that they have an SGTR situation.

Help:

- a. initiating event symptoms/indications are very clear and lead to single conclusion
- b. initiating event symptoms/indications are frequently practiced in the simulator within the context of this event
- c. event is perceived by operators to be a high-likelihood event
- e. other indicators are quiet (no other alarms) when indications occur

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)

4. Intention to Act PSFs - Likelihood for Intentional Violation

There seems to be no threat of this occurring

Help:

- a. the action's effect is clearly understood and fits well with the goals of the current procedure

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There seem to be no concerns here

Help:

- a. the action can be executed immediately; it does not rely on other actions

6. Execution PSFs - Omission and Commission Errors

Although control room actions are executed with buttons or controls that only have two positions, and all controls are well marked and segregated, there may be some difficulties in execution.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

Hinder:

- a. execution requires a difficult coordination between operators
- b. execution requires following procedures with an unusual or difficult logic

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There are no problems here.

9. Local Stress Factor

Workload

workload on key operator is probably moderate.

Time available vs time required

some time pressure. The action is expected to be completed within 20 minutes

Potential severe consequences of actions being taken

failure leads to overfill of Steam Generators and possibility of an RCS rupture, which will lead to a LOCA

Reduce ECCS Injection

Operator Action: OIR
Initiating Event: SGTR

Action:

For an SGTR event, the operators are required to stop or minimize ECCS flow to the RCS by stopping all but one charging pump and then aligning the remaining charging pump for normal charging. This means that two RHR pumps, two SI pumps, and one charging pump must be stopped. Note that for the case in which charging pumps are not available, this represents the operator action to stop two RHR pumps and one SI pump.

Set of Actions:

- verify RCS press stable or increasing (1/1)
- verify PRZR level > 4% (1/1)
- verify RCS subcooling > 30 deg (1/1)
- verify total AFW flow > 340 GPM (combine 4 readings)
OR verify narrow range SG level > 4% (1/1)
- verify RVLIS > 60% (1/1)
- stop both RHR pumps (2/2)
- close both valves (2/2)
- stop both SI pumps (2/2)
- stop chg pumps (1/2)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

In general, operators should be well aware that they have an SGTR situation.

Help:

- a. initiating event symptoms/indications are very clear and lead to single conclusion
- b. initiating event symptoms/indications are frequently practiced in the simulator within the context of this event
- c. event is perceived by operators to be a high-likelihood event
- d. other indicators are quiet (no other alarms) when indications occur

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)
- c. low operator workload when cues occur

4. Intention to Act PSFs - Likelihood for Intentional Violation

There seems to be no threat of this occurring

Help:

- a. the action's effect is clearly understood and fits well with the goals of the current procedure

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There seem to be no concerns here

Help:

- a. the action can be executed immediately; it does not rely on other actions

6. Execution PSFs - Omission and Commission Errors

Although control room actions are executed with buttons or controls that only have two positions, and all controls are well marked and segregated, there may be some difficulties in execution.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

Hinder:

- a. execution requires a difficult coordination between operators
- b. execution requires following procedures with an unusual or difficult logic

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There are no problems here.

9. Local Stress Factor

Workload

workload on key operator is probably moderate.

Time available vs time required

no significant time pressure. The action is expected to be completed within 52 minutes when RCS depressurization is used and 45 minutes when depressurization is not required

Potential severe consequences of actions being taken

failure leads to overfill of steam generators and pressurizer and may create a small LOCA inside containment

Establish Normal RHR Cooling

Operator Action: ONR

Initiating Event: SB LOCA

Action:

This operator action, an alternative to low-pressure recirculation, is used to establish normal RHR cooling. This is used after a LOCA when the leak is stopped or minimized such that normal charging keeps up with coolant loss. It is also a component of normal shutdown.

Set of Actions:

- check RCS subcooling > 30 deg (1/1)
- turn PRZR heaters off (1/1)
- depressurize RCS PRZR with spray or with PORV until PRZR level > 20%
- stop chg pump (1/1)

Establish normal charging

- verify chg pump miniflow valves open (2/2)
- close BIT inlet valves (2/2)
- close BIT outlet valves (2/2)
- open RCP seal return valve (1/1)
- check RCP LBRTH deltaP 20-60 inches (1/1)
- stop SI pumps (2/2)
- isolate accumulators: close SI acc isol valves (4/4)

Depressurize RCS

- open PRZR spray valves (2/2)
OR open PORV (1/1)
- verify RCS temp < 350 deg
- verify RCS press < 400 psig

Initiate RHR system cooling

- close RWST to RHR suction valves (2/2)
- open RHR to loop A hot leg suction valves (2/2)
- close RHR HX bypass valve (1/1)
- close RHR HX flow control valves (2/2)
- open RHR to cold leg inject valves (2/2)
- start RHR pump (1/1)
- open valve (1/1)
- throttle valve to maintain cooldown rate < 50 deg/hr (1/1)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

In general, operators should be well aware that they have a LOCA situation.

Help:

- a. initiating event symptoms/indications are very clear and lead to single conclusion
- b. event is perceived by operators to be a high-likelihood event
- c. operator workload is low when indications occur
- d. other indicators are quiet (no other alarms) when indications occur

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)
- c. low operator workload when cues occur

Hinder:

- a. there is likely to be change in personnel (e.g., shift change) between initial event and time of action

4. Intention to Act PSFs - Likelihood for Intentional Violation

There seems to be no threat of this occurring

Help:

- a. the action's effect is clearly understood and fits well with the goals of the current procedure

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There seem to be no concerns here

Help:

- a. the action can be executed immediately; it does not rely on other actions

6. Execution PSFs - Omission and Commission Errors

No major problems have been identified in this area. All control room actions are executed with buttons or controls that only have two positions, and all controls are well marked and segregated.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

Hinder:

- a. execution requires a long list of substeps

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There are no problems here.

9. Local Stress Factor

Workload

workload on key operator is probably low.

Time available vs time required

no significant time pressure. This action is expected to be established within 2 hrs

Potential severe consequences of actions being taken

failure has no significant consequences. However, if the RWST level is getting close to low-level alarm, operators may feel pressure to get normal RHR to avoid going to recirculation

Bleed and Feed: Initiate Safety Injection and Open PORVs

Operator Actions: OSI and ODP

Initiating Event: General Plant Transient

Action:

Given a general plant transient with no auxiliary feedwater and no restoration of main feedwater, the operator is required to establish and maintain an emergency core coolant source for RCS bleed and feed cooling. Thus, there is a need to initiate SI manually. When the SG level reaches 36% of wide range indication, bleed and feed cooling must be established with the Pressurizer PORVs.

Set of Actions:

OSI

- verify wide range level in any 3 SGs < 24% (4/4)
- verify HPI flow path (1/1) OR
- check chg or SI pump running (1/2) OR
- align at least one chg or one SI pump OR
- turn SI switch to actuate SI (1/1)

ODP

- verify and open all PRZR PORV block valves (2/2)
- open at least one PRZR PORV (1/2)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

Operators may not understand why the plant tripped

Help:

- a. initiating event symptoms/indications are frequently practiced in the simulator in the context of this event

Hinder:

- a. there is no single initiating event
- b. event is perceived by operators to be a very low-likelihood event

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)

4. Intention to Act PSFs - Likelihood for Intentional Violation

Bleed and feed is not an action operators want to take

Hinder:

- a. taking the action will have negative consequences associated with cost (e.g., the plant will be shut down for cleanup)
- b. there is a significant uncertainty or unknown risk associated with taking the action (e.g., PORV after being opened may stick open)

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

Operators may delay taking the action

Hinder:

- a. the action may not be executed immediately because operators are trying to achieve the goal through another (more preferred) action

6. Execution PSFs - Omission and Commission Errors

No major problems have been identified in this area. All control room actions are executed with buttons or controls that only have two positions, and all controls are well marked and segregated. The exception to this rule is the PORV control, which is spring-loaded and must be held open.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

Hinder:

- a. controls go against standard operational stereotype (for PORV control)

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There are no problems here.

9. Local Stress Factor

Workload

workload on key operator is probably high.

Time available vs time required

there is time pressure to accomplish this action. This actions to initiate SI are expected to be completed within 5 minutes.

Potential severe consequences of actions being taken

failure has potentially significant consequences.

Restore Main Feedwater and/or Condensate Booster Pumps

Operator Action: ORF

Initiating Event: General Plant Transient

Action:

For a general plant transient (after reactor and turbine trip), the operator is required to establish an alternate feedwater source to the steam generators when auxiliary feedwater fails.

Set of Actions:

- recognize loss of heat sink and transfer to FR-H.1
- stop all RCP pumps (4/4)

Reset SI

- verify safeguards breaker targets matched
- depress SI reset pushbuttons (2/2)
- verify SI activated and auto SI blocked (1/1)
- locally (outside control room) reset main generator 86 relays (1/1)
- start AFW pump (1/1)
- open FW isolation and regulating by-pass valves (8/8)

PSF Summary:

1. Diagnosis/ Situation Assessment PSFs

Operators may not understand why the plant tripped

Help:

- a. initiating event symptoms/incubations are frequently practiced in the simulator in the context of this event

Hinder:

- a. there is no single initiating event

2. Procedure Selection PSFs

There should be no difficulties in making transition to correct procedure.

Help:

- a. criterion for transition to correct procedure is explicit step in current procedure or part of standard operating procedure
- b. criterion for transition to correct procedure requires simple reading of indications and requires no judgment or interpretation

3. Intention to Act PSFs - Specific Cues to Action

Cues to action are clear and salient.

Help:

- a. cues are identified in procedure in unambiguous way (i.e., objective, clear)
- b. cues are highly salient (i.e., position, discriminability)

4. Intention to Act PSFs - Likelihood for Intentional Violation

There seems to be no threat of this occurring

Help:

- a. the action's effect is clearly understood and fits well with the goals of the current procedure

5. Intention to Act PSFs - Scheduling/Prioritizing the Action

There seem to be no concerns here

Help:

- a. the action can be executed immediately; it does not rely on other actions

6. Execution PSFs - Omission and Commission Errors

One action is outside of control room.

Help:

- a. execution uses controls with only two settings; controls are clearly marked

Hinder:

- a. execution requires a control action to be taken outside of control room
- b. execution requires following procedures with an unusual or difficult logic

7. Execution PSFs - Detection of Errors

Procedures and indications support detection of errors

Help:

- a. procedure has explicit catch steps or verifications

8. Execution PSFs - Recovery from Error

There are no problems here.

9. Local Stress Factor

Workload

workload on key operator is probably moderate.

Time available vs time required

no significant time pressure. The action is expected to be completed within approximately 2 hours

Potential severe consequences of actions being taken

failure leads to a need for bleed and feed cooling.