

Brunswick Nuclear Project P. O. Box 10429 Southport, N.C. 28461-0429

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U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555

> BRUNSWICK STEAM ELECTRIC PLANT UNITS 1 AND 2 DOCKET NOS. 50-325 AND 50-324 LICENSE NOS. DPR-71 AND DPR-62 SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

Gentlemen:

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Enclosed is the Semiannual Radioactive Effluent Release Report for Brunswick Steam Electric Plant, covering the period from January 1, 1990, through June 30, 1990.

This report is submitted for the Brunswick Steam Electric Plant in accordance with Technical Specification 6.9.1.8.

Very truly yours,

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J. L. Harness, General Manager Brunswick Nuclear Project

JD/th

PDR ADOCK

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Enclosure

cc: Mr. S. D. Ebneter Mr. N. B. Le BSEP NRC Resident Office

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# Brunswick Steam Electric Plant

# Semiannual Radioactive Effluent Report

January 1, to June 30, 1990

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# Supplemental Information

January 1, to June 30, 1990

#### EFFLUENT WASTE DISPOSAL SEMIANNUAL REPORT Supplemental Information

Facility: Brunswick Steam Electric Plant Licensee: Carolina Power and Light Company

1. Regulatory Limits

A. Fission and activation gases (Technical Spec. 3.11.2.2)

- \*(1) Calendar Quarter
  - (a) 10 mrad gamma
  - (b) 20 mrad beta
  - (2) Calendar Year
    - (a) 20 mrad gamma
    - (b) 40 mrad beta
- B. Jodine-131, iodine-133, tritium, and particulates with halflives greater than eight days (Technical Spec. 3.11.2.3)
  - \*(1) Calendar Quarter

     (a) 15 mrem to any organ
    - (2) Calendar Year
      - (a) 30 mrem to any organ
    - (3) Calendar Quarter for Burning Contaminated Oil
      - (a) 436 "Ci
    - (4) Calendar Year for Burning Contaminated Oil
      - (a) 872 'aCi
- C. Liquid effluents (Technical Specification 3.11.1.2)
  \*\*(1) Calendar Quarter
  - (a) 3 mrem to total body
  - (b) 10 mrem to any organ
  - (2) Calendar Year
    - (a) 5 mrem to total body
    - (b) 20 mrem to any organ

NOTE: Dose calculations are determined in accordance with the Off-Site Dose Calculation Manual (ODCM)

\*Used for percent of technical specification limit determinations in Table 1A.

\*\*Used for percent of technical specification limit determinations in Table 2A.

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- 2. Maximum permissible concentrations and dose rates which determine maximum instantaneous release rates.
  - A. Fission and activation gases (Technical Specification 3.11.2.1.a)
    - (1) 500 mrem/year to total body
    - (2) 3000 mrem/year to the skin
  - B. Iodine-131, iodine-133, tritium, and particulates with halflives greater than eight days (Technical Specification 3.11.2.1.b)
    - (1) 1500 mrem/year to any organ
  - C. Liquid effluents (Technical Specification 3.11.1.1) The concentration of radioactive material released in liquid effluents to unrestricted areas after dilution in the discharge canal shall be limited to the concentrations specified in 10CFR20, Appendix B, Table II, column 2, for radionuclides other than noble gases.

\*\*(1) Tritium: MPC = 3 E-03 uCi/ml and \*\*(2) Dissolved and entrained gases: MPC = 2 E-04 uCi/ml

- 3. Measurements and Approximations of Total Radioactivity
  - A. Fission and activation gases

Analysis for specific radionuclides in representative grab samples by gamma spectroscopy.

B. Iodines

Analysis for specific radionuclides collected on charcoal cartridges by gamma spectroscopy.

C. Particulates

Analysis for specific radionuclides collected on filter papers by gamma spectroscopy.

D. Particulates for Burning Oil

Analysis for specific radionuclides by grab samples of each batch of oil to be burned.

E. Liquids Effluents

Analysis for specific radionuclides of individual releases by gamma spectroscopy.

\*\* Used as applicable limits for Table 2A

Nuclear counting statistics are reported utilizing 1-sigma error. Total error where reported represents a best effort to approximate the total of all individual and sampling errors.

- 4. Batch Releases
  - A. Liquid

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(1) Number of batch re	leases:	8.60E+01
(2) Total time period	for batch releases:	9.00E+03 Minutes
(3) Maximum time perio	d for a batch release:	1.57E+02 Minutes
(4) Average time perio	d for a batch release:	1.05E+02 Minutes
(5) Minimum time perio	d for a batch release:	1.40E+01 Minutes
<ul> <li>(6) Average stream flow of release of effly stream :</li> <li>B. Gaseous</li> </ul>	w during periods uent into a flowing	6.56E+05 GPM
(1) Number of batch re	leases:	0.00E 00 Minutes
(2) Total time period	for a batch release:	0.00E 00 Minutes
(3) Maximum time period	d for a batch release:	0.00E 00 Minutes
(4) Average time period	d for a batch release:	0.00E 00 Minutes
(5) Minimum time period	d for a batch release:	0.00E 00 Minutes
. Abnormal releases		
A. Liquid		0 005 00
(1) Number of releases		0.00E 00 0.00E 00
(2) Total activity rele	easea:	Curies
B. Gaseous		~~
(1) Number of releases		0.00E 00
(2) Total activity rele		0.00E 00
		Curies

#### ATTA WY AT

Effluent und ... posal Data Brunswick steam Electric Plant January 1, to June 30, 1990

Encl \_ure 1

Table 1A: Gaseous Effluents - Summation of all Releases Table 1B: Gascous Effluents - Elevated Releases Table 1C: Gaseous Effluents - Ground Level Releases Table 2A: Liquid Effluents - Summation of all Releases Table 3B: Liquid Effluents Lower Limits of Detection

Table 3: Solid Waste and Irradiated Fuel Shipments Enclosure 2

Combustion of Waste Oil

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TABLE 1A Effluent and Waste Disposal Semiannual Report for Year 1990 Gaseous Effluents - Summation of all Releases

					Est. Tot.
		Unit	Otr 1	Otr 2	Error %
Α.	FISSION AND ACTIVATION				
	GASES 1. Total release	<b>C</b> i	-		
	1. Total release	Ci	2.456+02	2.89E+02	1.15E OC
	2. Average release rate for period	uCi/sec	3.15E+01	3.68E+01	
	3. Percent of technical	1			
	specification limit		7.50E-02	5.80E-02	
в.	IODINES				
	1. Total I-131	Ci	1.63E-03	2.07E-03	7.00E 01
	2. Average release rate for period	uCi/sec	2.09E-04	2.63E-04	
C.	PARTICULATES				
~.	1. Total release	Ci	1.25E-03	2.85E-03	7.00E 01
	<ol> <li>Average release rate for period</li> </ol>	uCi/sec	1.61E-04	3.63E-04	
	3. Gross alpha	ci	1 828-06	3.24E-06	
	or orong arpin	~.	1.025-00	5.246-00	
D.	Tritium				
	1. Total release	Ci	7.81E-01	5.83E+00	7.00E 01
	2. Average release rate for period	uCi/sec	1.00E-01	7.42E-01	
T	TODINE 121 YODINE 122				
E.	IODINE-131, IODINE-133, 1 1. Total Release	Ci		5.85E+00	
	1. IStal Kelease		7.932-01	5.852+00	
	2. Average release a rate for period	uCi/sec	1.02E-01	7.44E-01	
	3. Percent of technical				
		The second se	6.00E-02	8.00F-02	
				0.000 02	
F.	PARTICULATES VIA BURNING				
	1. Total Release	Ci	0.00E+00	0.00E+00	
	2. Average release				
		uCi/sec	0.00E+00	0.00E+00	
	3. Percent of technical				
	specification limit	*	0.00E+00	0.00E+00	

# Effluent and Waste Disposal Semiannual Report for Year 1990 Gaseous Effluents - Elevated Releases Continuous Release

Nuclides Released	Unit	Otr 1	Otr 2
1. <u>FISSON GASES</u> argon-41 krypton-85m krypton-87 xenon-133 xenon-135m xenon-135 xenon-137 <u>xenon-138</u> total for period	Ci Ci Ci Ci Ci Ci	4.01E+00 <lld 1.38E+00 3.32E+00 2.14E+01 1.91E+01 5.87E+01 4.08E+01 1.49E+02</lld 	3.91E+00 7.27E-01 7.81E-01 4.04E+00 1.79E+01 1.27E.01 8.62E+01 3.44E+01 1.61E+02
2. <u>IODINES</u> iodine-131 iodine-132 iodine-133 <u>iodine-135</u> total for period	Ci Ci Ci Ci	1.40E-03 3.86E-03 7.20E-03 8.96E-03 2.14E-02	1.72E-03 6.24E-93 1.08E-62 <u>1.25E-02</u> 3.13E-02
3. <u>PARTICULATES</u> chromium-51 manganese-54 cobalt-58 cobalt-60 strontium-89 strontium-90 cesium-137 barium-140 lanthanum-140 <u>hafnium-181</u> total for period	Ci Ci Ci Ci Ci Ci Ci Ci	<lld 2.33E-05 <lld 5.15E-05 1.52E-04 5.90E-07 1.54E-05 1.12E-04 7.41E-05 <u>1.13E-07</u> 4.29E-04</lld </lld 	4.26E-05 9.70E-06 4.47E-06 2.97E-05 2.45E-04 1.15E-06 1.10E-05 3.15E-04 2.28E-04 < <u>LLD</u> 8.87E-04

4. TRITIUM hydrogen-3

Ci 4.95E-01 3.56E+00

# TABLE 1CEffluent and Waste Disposal Semiannual Report for Year 1990Gaseous Effluents - Ground Level ReleasesContinuous Release

Nuclides Released	Unit	Otr 1	Otr 2
1. FISSION GASES			
xenon-133	Ci	7.46E+00	7.88E+01
xenon-135m	Ci	1.59E+01	3.14E+01
xenon-135	Ci Ci	7.23E+01	1.73E+01
total for period	Ci	9.57E+01	1.28E+02
2. IODINES			
iodine-131	Ci	2.27E-04	3.47E-04
iodine-132	Ci	9.79E-04	2.27E-03
iodine-133	Ci	1.88E-03	2.92E-03
iodine-134	Ci	<lld< td=""><td>3.20E-04</td></lld<>	3.20E-04
iodine-135	Ci Ci	7.00E-04	2.45E-03
total for period	Ci	3.79E-03	8.31E-03
3. PARTICULATES			
chromium-51	Ci	8.57E-05	1.57E-03
maganese-54	Ci	3.49E-04	1.14E-04
cobalt-58	Ci	1.06E-04	5.60E-05
cobalt-60	Ci	2.68E-04	2.06E-04
strontium-89	Ci	1.60E-06	3.75E-06
strontium-90	Ci	1.86E-07	3.06E-07
cesium-137	<u>Ci</u> Ci	8.18E-06	8.51E-06
total for period	Ci	8.19E-04	1.96E-03
4. TRITIUM			
hydrogen-3	Ci	2.86E-01	2.27E+00

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TABLE 2A Effluent and Waste Disposal Semiannual Report for Year 1990 Liquid Effluents - Summation of all Releases

	Unit	<u>Otr 1</u>	Otr 2	Est Tot % Error
A. FISSION AND ACTIVATION PRODUCTS				-
1.Total release (excluding tritium, gases, & alpha)	Ci	1.23E-01	2.45E-02	3.50E 01
2. Avg. diluted conc.	uCi/ml	7.94E-09	3.57E-09	
3. Percent limit	8	1.87E-01	1.98E-02	
B. TRITIUM				
1. Total release	Ci	1.95E+00	6.99E+00	4.00E 00
2. Avg. diluted conc.	uCi/ml	1.26E-07	1.02E-06	
3. Percent limit	8	4.202-03	3.40E-02	
C. DISSOLVED AND ENTRAINED	GASES			
1. Total release	ci	9.35E-02	4.02E-02	3.50E 01
2. Avg. diluted conc.	uCi/ml	6.036 9	5.86E-09	
3. Percent limit	\$	3. ~ E-03	2 ^3E-03	
D. GROSS ALPHA RADIOACTIVIT	¥			
1. Total release	Ci		C.J.D	4.00E 01
E. VOLUME OF WASTE	liters	4.53E+00	1.84E+0 3	1.25E 01
F. TOTAL OF DILUTION WATER (used during release				
for average dil. conc.)	liters	1.55E+10	6.86E+09	1.30E 01
G. VOLUME OF COOLING WATER DISCHARGED FROM PLANT	liters	3.10E+11	3.96E+11	

# TABLE 2B Effluent and Waste Disposal Semiannual Report for Year 1990 Liquid Effluents - Batch Mode

Nuclides Released	Unit	Otr 1	Otr 2
1. FISSION AND ACTIVATION	PRODUCTS		
sodium-24	Ci	1-26E-03	<ĩ.LD
chromium-51	Ci	1.74E-02	2.67E-03
manganese-54	ci	2.15E-02	1.028-03
iron-55	Ci	<lld< td=""><td>7.352-04</td></lld<>	7.352-04
cobalt-57	Ci	1.27E-05	<lld< td=""></lld<>
cobalt-58	Ci	6.29E-03	1.56E-04
iron-59	Ci	6.19E-05	<lld< td=""></lld<>
cobalt-60	Ci	4.10E-02	7.00E-03
arsenic-76	Ci	5.00E-05	<lld< td=""></lld<>
strontium-90	Ci	3.44E-05	<lld< td=""></lld<>
yttrium-91m	Ci	3.18E-04	6.34E-04
yttrium-92	Ci	5.01E-03	8.26E-03
yttrium-93	Ci	<lld< td=""><td>2.59E-04</td></lld<>	2.59E-04
strontium-92	Ci	2.94E-05	1.14E-05
technetium-99m	Ci	1.83E-04	4.13E-04
technetium-101	Ci	<lld< td=""><td>3.48E-05</td></lld<>	3.48E-05
technetium-104	Ci	9.12E-05	2.57E-05
ruthenium-103	Ci	8.70E-06	<lld< td=""></lld<>
ruthenium-105	Ci	2.35E-04	9.31E-04
antimony-125	Ci	1.15E-04	<lld< td=""></lld<>
iodine-131	Ci	5.60E-04	<lld< td=""></lld<>
tellurium-132	Ci	3.01E-04	<lld< td=""></lld<>
iodine-132	Ci	2.61E-05	<lld< td=""></lld<>
iodine-133	Ci	2.31E-03	1.63E-05
iodine-134	Ci	<lld< td=""><td>3.29E-05</td></lld<>	3.29E-05
cesium-134	Ci	8.37E-03	1.03E-04
iodine-135	Ci	4.74E-04	<lld< td=""></lld<>
cesium-137	Ci	1.43E-02	4.83E-04
lanthanum-140	Ci	1.69E-04	2.27E-04
lanthanium-141	Ci	<lld< td=""><td>1.01E-03</td></lld<>	1.01E-03
lanthanum-142	Ci	1.12E-04	4.65E-04
neptunium-239	Ci	2.33E-03	<lld< td=""></lld<>
total for period	Ci	1.23E-01	2.45E-02
2. CASES			
xenon-133	Ci	1.38E-02	4.14E-03
xenon-133m	Ci	1.23E-04	<lld< td=""></lld<>
xenon-135m	Ci	4.32E-04	4.67E-05
xenon-135		7.91E-02	3.60E-02
total for period	<u>Ci</u> Ci	9.35E-02	4.02E-02
total for period		5.552 02	1.025-02

## Lower Limits of Detection

#### January through June 1990

#### uCi/ml

# 1. Liquid Releases

2. Gaseous Releases

Kr-85m

2.19E-08
1.11E-08
3.01E-09
1.81E-08
4.43E-08
1.88E-07
2.89E-08
5.48E-08
1.51E-08
4.08E-08
2.58E-08
1.27E-08
3.01E-08
1.39E-07
1.38E-08
7.36E-08
8.91E-08
8.90E-07
1.00E-07

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7.24E-09

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3. Iodines and Particulates

Cr-51		3.08E-13
Co-58		2.38E-14
Hf-181	10	3.95E-14
I-134	T <sup>1/2</sup>	Too short

#### NOTES

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- 1: The above values represent typical "a priori" LLDs for isotopes where values of "<LLD" are indicated in Tables 1A, 1B, 1C, 2A, and 2B.
- 2: Where activity for any nuclide is reported as " Less than LLD", that nuclide is considered not present and the LLD activity listed is not considered in summary data.

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# Table 3A Effluent and Waste Disposal Semiannual Report for Year 1990 Solid Waste and Irradiated Fuel Shipments

Wast	te Class A		January thi	rough June
1.	Total volume shipped (cubic met	ers)	3.17 1	52
	Total Curie quantity (estimated	1)	7.34 1	52
2.	Type of Waste	<u>Units</u>	Six-month Period	Est. Total <u>% Error</u>
	a. Spent resins, filter sludges		s <sup>3</sup> 6.88 E1 s, 7.05 E2	1.00E1
	b. Dry active waste, compacted noncompacted	meter	s <sup>3</sup> 2.49 E2 s <sub>3</sub> 2.88 E1	1.00E1
	c. Irradiated components	meter	5 0.00 EO	N/A
	d. Others (oil)		s 0.00 E0 s 0.00 E0 s 0.00 E0	N/A
3.	Estimate of major radionuclide	the second s		.,

a.	Ni-63 Mn-54	1.84 E0% 7.57 E0%
	Fe-55	5.84 E1%
	Co-58	7.78 E0%
	Co-60	2.01 E1%
	Cs-134	1.06 E0%
	Cs-137	2.19 E0%
b.	Mn-54	6.09 E0%
	Fe-55	8.24 E1%
	Co-60	1.07 E1%
с.	N/A	N/A
d.	N/A	N/A

# Effluent and Waste Disposal Semiannual Report for Year 1990 Solid Waste and Irradiated Fuel Shipments

# 4. Cross reference table, waste stream, form, and container type.

	Stream	Form	Container type	No. of shipments
a.	Resin	Dewatered & Solidified*	Туре А/Туре В	13/1
b.	Dry active waste	Compacted/non compacted waste	- STP	18
с.	Irradiated components		N/A	0
d.	Other		N/A	0
			ion agent or abs nt, urea formald	

# 5. Shipment Disposition

a. Solid Waste

Number of Shipments	Mode of Transportation	Destination
32	Sole Use	CNSI/Barnwell SC

b. Irradiated Components

Number of Shipments	Mode of Transportation	Destination
0	N/A	N/A

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### TABLE 3B

Effluent and Waste Disposal Semiannual Report for Year 1990 Solid Waste and Irradiated Fuel Shipments

# Waste Class B

# January through June

1.	Total volume shipped		(cubic meters)	0.00 E0	
	Total	Curie quantity	(estimated)	0.00 E0	

2.	TY	pe of Waste	Si	x-month E	st.Total
	a.	Spent resins, filter sludges	Units meters <sup>3</sup>	Period 0.00 E0	<u>% Error</u> N/A
			Curies.	0.00 E0	
	b.	Dry active waste, compacted,	meters	0.00 E0	N/A
		and noncompacted	Curies.	0.00 E0	
	c.	Irradiated components	meters	0.00 E0	N/A
			Curies,	0.00 E0	
	d.	Others (describe)	meters	0.00 E0	N/A
			Curies	0.00 E0	

# 3. Estimate of major radionuclide composition

a.	N/A	N/A

b.	A/R	N/A
с.	N/A	N/A
d.	N/A	N/A

4. Cross reference table, waste stream, form and container type

	Stream	Form	Contain	er type	No. of shipments
a.	Resin	Dewatered & Solidified	Туре	A/Type B	0 / 0
b.	Dry active	Compacted/nor compacted waste	n <b>-</b>	N/A	0
c.	Irradiated components			N/A	0
d.	Other			N/A	0
		lidification a .g., cement, u			

(e.g., cement, urea formaldehyde)

# Table 3B (cont.) Effluent and Waste Disposal Semiannual Report for Year 1990 Solid Waste and Irradiated Fuel Shipments

# 5. Shipment Disposition

a. Solid Waste

Number of Shipments	Mode of Transportation	Destination
0	N/A	N/A
b. Irradiated Fuel		
Number of Shipments	Mode of Transportation	Destination

0	N/A	N/A

#### TABLE 3C

Effluent and Waste Disposal Semiannual Report for Year 1990 Solid Waste and Irradiated Fuel Shipments

Waste Class C	Januar	y through .	June
1. Total volume shipped (cubic mete	rs)	0.00 E0	
Total Curie quantity (estimated)		0.00 E0	
2. Type of Waste		x-month E	
a. Spent resins, filter sludges	Units meters <sup>3</sup>	Period 0.00 E0	<u>% Error</u> N/A
b. Dry active waste, compacted and noncompacted	meters	0.00 E0 0.00 E0 0.00 E0	N/A
c. Irradiated components	meters <sup>3</sup>	0.00 E0	N/A
d. Others (describe)	Curies, meters <sup>3</sup> Curies	0.00 E0 0.00 E0 0.00 E0	N/A
3. Estimate of major radionuclide c	omposition		
а.	N/A	N/A	
b.	N/A	N/A	
3.	N/A	N/A	
d.	N/A	N/A	
4. Cross reference table, waste stre	eam, form	and contain	ner type
Stream Form Conta	iner Type	No. of sl	nipments
a. Resin Dewatered & Solidified *	N/A	0/0	D
b. Dry active Compacted/non- waste compacted	N/A	0	
c. Irradiated components	Type B	0	
d. Others	N/A	0	
* Solidification agent (	or absorbe	nt	

(e.g., cement, urea formaldehyde) NONE

# Table 3C (cont.) Effluent and Waste Disposal Semiannual Report for Year 1990 Solid Waste and Irradiated Fuel Shipments

5. Shipment Disposition

a. Solid Waste

Number of Shipments	Mode of Transportation	Destination
0	N/A	N/A
b. Irradiated Fuel (r	on-burial)	
Number of Shipments	Mode of Transportation	Destination
3	sole use/Rail	CP&L/SHNPP

# ENCLOSURE 2

Combustion of Waste Oil January 1, to June 30, 1990

There was no incineration of contaminated waste oil during this reporting period.

# Off-Site Dose Calculation Manual (ODCM) and Process Control Program (PCP) Revisions January 1, to June 30, 1990

Brunswick Steam Electric Plant

There were no revisions made to the Process Control Program during this reporting period.

There were no revisions made to the Off-Site Dose Calculational Manual during this reporting period.

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Environmental Monitoring Program January 1, to June 30, 1990

Enclosure 1: Milk and Vegetable Sample Locations Enclosure 2: Land Use Census

#### ENCLOSURE 1

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# Milk and Vegetable Sample Locations January 1, to June 30 1990

No milk sample locations were available during this time period. Vegetable sample locations remained unchanged.

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### ENCLOSURE 2

Land Use Census

# January 1, to June 30, 1990

The results of the 1990 Land Use Consus will be reported in the July 1, to December 31, 1990 Semianrual Report.

# Effluent Instrumentation

January 1, to June 30, 1990

Enclosure	1:	Radioactive Liquid Effluent Monitoring Instrumentation.
Enclosure	2:	Radioactive Gaseous Effluent Monitoring
Enclosure	3:	Liquid Hold-Up Tank

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#### ENCLOSURE 1

January 1, to June 30, 1990

Radioactive Liquid Effluent Monitoring Instrumentation

The liquid radwaste effluent flow measurement device (2-G16-FIT-N057) was inoperable for greater than 30 days during this period. The instrument was not returned to service within a 30 day period due to the inability to calibrate the instrument within its design accuracy.

#### ENCLOSURE 2

#### January 1, to June 30, 1990

#### Radioactive Gaseous Effluent Monitoring Instrumentation

Unit 1 & 2 Main Condenser Off-Gas treatment system explosive gas monitors 1(2)-OG-AIT-4284 (SJAE.A.H<sub>2</sub> Analyzer), 1(2)-OG-AIT-4285(SJAE.A.H<sub>2</sub> Analyzer), 1(2)-OG-AIT-4324 (SJAE.B.H<sub>2</sub> Analyzer), and 1(2)-OG-AIT-4325 (SJAE.B.H<sub>2</sub> Analyzer) were inoperable for greater than a 30 day period during January 1 to June 30, 1990. Due to design problems, these monitors were not returned to service within 30 days. Liquid Hold-Up Tank January 1, to June 30, 1990

No liquid hold-up tank exceeded the 10 Ci limit during this reporting period.

Major Modifications to the Radioactive Waste Treatment System January 1, to June 30, 1990

As per footnote 7 to Technical Specification 6.15, a discussion of any major modifications to the radioactive waste treatment systems will be submitted with the Final Safety Analysis Report update.

Meteorological Data January 1, to June 30, 1990

As per Technical Specification 6.9.1.10.a footnote 6, the annual summary of meteorological data collected over the calendar year will be submitted to a file and will be available for NRC review upon request.

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Annual Dose Assessment January 1, to June 30, 1990

As per Technical Specification 6.9.1.10.b, an assessment of radiation doses due to the radioactive liquid and gaseous effluents released during the calendar year will be reported within 90 days after January 1 of each year. The annual dose assessment is not included with this report.

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#### SUPPLEMENT TO PREVIOUS SEMIANNUAL REPORT

### BRUNSWICK STEAM ELECTRIC PLANT

January 1, to June 30, 1990

The attached sheet should be added to the July 1, to December 31, 1989 Semiannual Report (1989 second report). This information, concerning inoperable radioactive gasecus effluent monitoring instrumentation, was inadvertently ommitted.

Please replace page 5/3 of the July 1, to December 31, 1989 Semiannual Environmental and Radioactive Effluent Release Report with the attached page 5/3.

#### ENCLOSURE 2

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#### July 1, to December 31, 1989

Radioactive Gaseous Effluent Monitoring Instrumentation

Unit 1 & 2 Main Condenser Off-Gas treatment system explosive gas monitors 1(2)-OG-AIT-4284 (SJAE.A.H<sub>2</sub> Analyzer), 1(2)-OG-AIT-4285 (SJAE.A.H<sub>2</sub> Analyzer), 1(2)-OG-AIT-4324 (SJAE.B.H<sub>2</sub> Analyzer), and 1(2)-OG-AIT-4325 (SJAE.B.H<sub>2</sub> Analyzer) were inoperable for greater than a 30 day period during July 1 to December 31, 1989. Due to design problems, these monitors were not returned to service within 30 days.