ATTACHMENT A

Revised July 1990 Operating Status sheet SONGS Units 2 and 3 MOR



## NRC MONTHLY OPERATING REPORT

DOCKET NO:	50-361	1
UNIT NAME:	SONGS - 2	
DATE:		
OMPLETED BY:	T. M. Sarette	
TELEPHONE:	(714) 368-9335	

## OPERATING STATUS

Reporting Period: June 1990	ing Station. U	<u>III 2</u>	
Licensed Thermal Power (MWt):	3390	the state of the second second	
Nameplate Rating (Gross Mwe):	1127		
Design Electrical Rating (Net MWe):	1070		
Maximum Dependable Capacity (Gross MWe	e): <u>1127</u>		
Maximum Dependable Capacity (Net MWe):	1070		
If Changes Occur In Capacity Ratings (	Items Number 3	Through 7)	
Since Last Report, Give Reasons:		NA	
Power Level To Which Restricted, If An	y (Net MWe):	NA	
Reasons For Restrictions, If Any:		NA	
Number Of Hours Reactor Was Critical	720.00	4.308.17	43,375.01
	Ints Morth Ti	to-Date (	umulative
Number Of Hours Deactor Was Critical	720.00	4.343.00	43 375 01
Reactor Reserve Shutdown Hours	0.00	0.00	0.00
Hours Generator On-Line	720.00	4.301.90	42.444.93
Unit Reserve Shutdown Hours	0.00	0.00	0.00
Gross Thermal Energy Generated (MWH)	2.397.614.87	4.445.737.39	138,380,766,71
Gross Electrical Energy Generated (Mw	H) 815,935.00	4.966.640.00	46.941.347.00
Net Electrical Energy Generated (MWH)	*778,790.00	4.741.002.00	*44,483,846.24
Unit Service Factor	100.00%	99.05%	70.49
Unit Availability Factor	100.00%	99.05%	70.49%
Unit Capacity Factor (Using MDC Net)	*101.09%	*102.02%	69.04%
Unit Capacity Factor (Using DER Net)	*101.09%	*102.02%	69.04%
Unit Forced Outage Rate	0.00%	0.00%	6.06%
Shutdowns Scheduled Over Next 6 Month	is (Type, Date,	and Duration	of Each):
A Steam Generator Inspection outage i	s scheduled t	commence on o	r before
July 31, 1990. Outage duration is fo	recast for 34 v	ays.	and the state of the state
If Shutdown At End Of Report Period,	Estimated Date	of Startup: _	NA
Units In Test Status (Prior To Commer	cial Operation)	: Forecast	Achieved
INITIAL CRITICALITY		NA	NA
INITIAL ELECTRICITY		NA	<u>NA</u>
COMMERCIAL OPERATION		NA	NA

Data corrected since previous submittal

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ATTACHMENT B

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Challenge to Unit 3 Safety Valve SONGS Units 2 and 3 MOR

## DESCRIPTION OF A CHALLENGE TO UNIT 3 SAFETY VALVE

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On July 15, 1990 at 2257, with Unit 3 in Mode 3 following the Cycle 5 refueling outage, Main Steam Safety Valve (MSSV) 3PSV8412 lifted for approximately two minutes at approximately 1025 psia. Since Technical Specification (TS) 3.7.1.1 requires a minimum lift setpoint of 1103 psia for this safety valve, it was declared inoperable and the Power Level-High reactor trip set point was reduced within 4 hours as required per TS Action Statement 3.7.1.1.a.

On July 16, 1990, testing of MSSV 3PSV8412, which had been refurbished by the Crosby Valve and Gage Company during the Unit 3, Cycle 5 refueling outage, confirmed that it's lift setpoint was approximately 80 psia below the TS minimum lift set point. Since four additional Unit 3 MSSVs had been refurbished by the Crosby Valve and Gage Company during the Cycle 5 refueling outage, these four valves were tested to confirm their lift set points. Each of these valves were found to have lifted between 20 to 30 psia below their TS minimum set points. Following each valve test, the lift set point was promptly adjusted to the required TS setting.

On July 17, 1990, as a result of SCE's investigation unto the cause of the low lift set points, it was discovered that the lift setting adjustment and testing performed at Crosby was conducted with the valves in a configuration different than the plant configuration. Specifically, the MSSV's lift set points were established without lagging on the valves. Thus, the temperature profile of the valves differed from their normally lagged condition.

Also on July 17, 1990, a review of Unit 2 MSSVs identified five additional valves which had been refurbished by Crosby. These five Unit 2 valves had been installed with lagging during the Unit 2 Cycle 5 refueling outage and had been considered operable since December 2, 1989, when the unit first entered Mode 3. Further review identified that three of these valves had their lagging removed on January 25, 1990, for an unrelated reason, and that these valves remained unlagged (i.e. operable). At 1401, the remaining two Unit 2 valves were declared inoperable and TS action statement 3.7.1.1.a was entered. Lagging on these two valves was immediately removed and at 1700 the valves were declared operable.

The reporting of a challenge to a safety valve is required by Section 6.9.1.10 of Appendix A, Technical Specification to Facility Operating License NPF-15 for San Onofre Unit 3. Additional information on this event is provided in Licensee Event Report 90-008, Docket Nos. 50-361 and 50-362.