Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 10, 1990

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for July, 1990.

Very truly yours,

W. R. Smith

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/JWT/msh

Enclosure

xc: A. B. Davis, NRC, Region III
NRC Resident Inspector LaSalle
Ill. Dept. of Nuclear Safety
R. Pulsifer, NRR Project Manager
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
D. R. Eggett, NED
J. E. Kuskey, GE Resident
T. J. Kovach, Manager of Nuclear Licensing
W. F. Naughton, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle
T. R. Novotney, INPO Coordinator, LaSalle Station
T. A. Hammerich, Regulatory Assurance Supervisor
J. W. Gieseker, Technical Staff Supervisor
Central File

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JULY 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761, extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, generator off-line
	1635	Generator on-line
4	0030	Unit at 1000 MWE
5	0200	Reduced load to 690 MWE for rod-set
	1400	Increased load to 1075 MWE
6	0100	Reduced load to 850 MWE for System load
	1400	Increased load to 1093 MWE
7	0300	Reduced load to 850 MWE for System load
	1500	Increased load to 1100 MWE
8	0100	Reduced load to 750 to repair steam leak
	1500	Increased load to 1090 MWE
10	0300	Reduced load to 1000 MWE for CRD exercising
	1400	Increased load to 1090 MWE
12	0100	Reduced load to 850 MWE for System load
	1400	Increased load to 1100 MWE
13	0100	Reduced load to 750 MWE for Heater Bay work
	2000	Increased load to 1100 MWE
14	0130	Reduced load to 750 MWE for Heater-bay work
15	0200	Increased load to 1100 MWE
17	0130	Reduced load to 750 MWE for CRD exercising and Heater-bay work
	2200	Increased load to 1119 MWE
18	0400	Reduced load to 850 MWE for System load
	1400	Increased load to 1120 MWE

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1) (Continued)

Day	Time	Event
20	0100	Reduced load to 750 MWE for System load
	1500	Increased load to 1110 MWE
22	0100	Reduced load to 850 MWE for System load
	1100	Increased load to 1062 MWE
23	0100	Reduced load to 900 MWE for System load
	1100	Increased load to 1060 MWE
24	0100	Reduced load to 850 MWE for System load
	1400	Increased load to 1100 MWE
25	0030	Reduced load to 850 MWE for System load
	0900	Increased load to 1053 MWE
26	0030	Reduced load to 740 MWE for Heater-bay work.
	0645	Increased load to 960 MWE, holding for high Hydrogen temperature in generator.
29	2100	Reduced load to 800 MWE for System load.
30	0245	Reduced load to 700 MWE for generator hydrogen cooler repairs.
	1325	Increased load to 920 MWE
31	2140	Reduced load to 850 MWE for System load and scheduled surveillances
	2400	Reactor critical, generator on-line at 850 MWE

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.(Unit 1)

1. Amendments to the Facility License or Technical Specification.

   Amend the load profile currently listed in the battery Tech. Specs. to account for the Alternate Rod Insertion Modification.

2. Changes to procedures which are described in the Safety Analysis Report.
   (None.)

3. Tests and Experiments not described in the Safety Analysis Report.
   (None)

4. Major corrective maintenance to Safety-Related Equipment, including any SOR switch failure reports.
   (None.)

5. Completed Safety-Related Modifications.
   (See Table 2)

C. LICENSEE EVENT REPORTS (Unit 1)

LER Number	Date	Description

(None)

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 3)

2. Average Daily Unit Power Level (See Table 4)

3. Unit Shutdowns and Significant Power Reductions (See Table 5)

B.5 TABLE 2 (Unit 1)

COMPLETED SAFETY-RELATED MODIFICATIONS

NUMBER	DESCRIPTION
M01-1-87-031	Replaced the control room lighting "eggcrates" and support runner with an off-white shade. The new ceiling will replace the existing diffusers in both the "at controls" and back-panel areas, in conjunction with the new fixtures and reflectors.

TABLE 3
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE August 10, 1990
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: JULY 1990 — GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (None)

4. REASONS FOR RESTRICTION (IF ANY): (N/A)

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	4,802.3	36,936.4
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	727.4	4,657.5	36,075.8
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWHt)	2,116,488	14,618,097.4	102,451,665
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	698,752	4,969,029	34,037,570
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	673,186	4,805,560	32,538,262
12. REACTOR SERVICE FACTOR (%)	100.0	94.4	64.0
13. REACTOR AVAILABILITY FACTOR (%)	100.0	94.4	66.8
14. UNIT SERVICE FACTOR (%)	97.8	91.6	62.5
15. UNIT AVAILIBILITY FACTOR (%)	97.8	91.6	62.5
16. UNIT CAPACITY FACTOR (USING MDC) (%)	87.3	91.2	54.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	83.9	87.6	52.3
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	2.9	9.3

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
(None)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
(N/A)

TABLE 4
B.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE August 10, 1990
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

REPORT PERIOD: JULY 1990

DAY	POWER	DAY	POWER
1	30	17	943
2	121	18	1,037
3	669	19	1,064
4	951	20	969
5	979	21	1,073
6	999	22	957
7	994	23	993
8	967	24	994
9	1,064	25	922
10	1,040	26	882
11	1,958	27	915
12	999	28	911
13	932	29	904
14	861	30	800
15	1,072	31	882
16	1,068		

TABLE 5

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
(None)						

SUMMARY OF OPERATION:

The Unit returned to service following main condenser tube plugging. Most load reductions were due to System load conditions. Fouling of the main generator hydrogen cooler has reduced output due to excessive hydrogen temperatures; on-line chemical cleaning is in-progress.

E. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

(None)

2. ECCS System Outages
(See Table 6)

3. Changes to the Off-Site Dose Calculation Manual
(None).

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

(Unit 1)
Table 6

ECCS System Outages

Note: The year and unit data has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
(U-0)		
194	0DG01P (Cooling water pump)	Lubricate coupling
(U-1)		
326	1A D/G	Inspect fuel oil pressure sensing line
351	1E12-C003 (RH)	Lubricate coupling

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JULY, 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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(Unit 2)

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761 extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, generator on-line at 1110 MWE.
4	0300	Reduced load to 1000 MWE for scheduled surveillances.
	0800	Increased load to 1110 MWE.
11	0200	Reduced load to 900 MWE for CRD exercising
	1300	Increased load to 1110 MWE.
13	0200	Reduced load to 720 MWE for scram-time testing CRD 26-55.
14	0200	Reduced load to 120 MWE to adjust MSIV limit switches.
15	2200	Increased load to 850 MWE.
16	0700	Increased load to 985 MWE.
	2000	Reduced load to 720 MWE for Heater-bay work.
17	1700	Increased load to 1000 MWE.
19	0200	Reduced load to 625 MWE for rod-set.
	1800	Increased load to 1110 MWE.
20	0200	Reduced load to 850 MWE for System load
	1300	Increased load to 1110 MWE.
22	0200	Reduced load to 850 MWE for System load
	1200	Increased load to 1110 MWE.
26	0300	Reduced load to 850 MWE for System load.
	0900	Increased load to 1110 MWE.
31	2400	Reactor critical, generator on-line at 1110 MWE.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.(Unit 2)

1. Amendments to the Facility License or Technical Specification.

   Amend the load profile currently listed in the battery Tech. Specs. to account for the Alternate Rod Insertion Modification.

2. Changes to procedures which are described in the Safety Analysis Report.
   a. LaSalle Special Procedure, LLP-90-045, "MSIV Scram Functional Test." The purpose of this test was to outline the steps necessary to verify proper operation of the inboard/outboard MSIV RPS limit switches with some partial channel trips present. This procedure was required to prevent a unit scram due to MSIV combination trip logic. The procedure was maintained within the bounds of the existing Technical Specifications.

3. Tests and Experiments not described in the Safety Analysis Report.
   (None)

4. Major corrective maintenance to Safety-Related Equipment, including any SOR switch failures.
   (None)

5. Completed Safety-Related Modifications.
   (See Table 2)

C. LICENSEE EVENT REPORTS (Unit 2)

LER Number	Date	Description

(None)

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
   (See Table 3)

2. Average Daily Unit Power Level.
   (See Table 4)

3. Unit Shutdowns and Significant Power Reductions.
   (See Table 5)

B.5 TABLE 2 (Unit 2)

COMPLETED SAFETY-RELATED MODIFICATIONS

NUMBER	DESCRIPTION
M01-2-87-002	Installed an annunciator pushbutton station on the 2PM03J Panel in the control room per Detailed Control Room Design Review (DCRDR) program.
M01-2-87-026	Replaced the key operated control switches for the MSIV Leakage Control System with non-key operated type switches. Replaced the scale range indicator for the RCIC turbine exhaust pressure indicator and RCIC pump suction pressure indicator per Detailed Control Room Design Review (DCRDR) program.
M01-2-87-028	Rearranged the recorders on the P602 control room panel to allow consistency in right to left and top to bottom arrangement per Detailed Control Room Review (DCRDR) program.
M01-2-87-031	Rearranged the Vent and Purge damper controls located on the control room panel 2PM06J in a logical configuration per Detailed Control Room Review (DCRDR).
M01-2-87-033	Installed a PA override capability for control room inputs into the PA system, installed permanent telephones on the N62-P600 and P601 control room panels, and deleted the tripping hazards presented by overly long handset cords on the radiophones per Detailed Control Room Review (DCRDR).
M01-2-88-004	Replaced the existing 125 Volt, Division 1 battery and rack and revised the current monitoring instrumentation for the battery and it's associated charger.
M01-2-89-001	Replaced the double disc configuration for the Reactor Recirc 2B33-F067A/B valves with a single, flexible wedge disc. Installed new packing assembly consisting of a single set of packing, eliminating the need for a stem leak-off line.

TABLE 3
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE August 10, 1990
COMPLETED BY J.W THUNSTEDT
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: JULY 1990 — GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (None)

4. REASONS FOR RESTRICTION (IF ANY): (N/A)

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	2,963.1	33,110.2
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	2,845.2	32,528.3
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWHt)	2,262,984	8,563,207.2	94,883,255.2
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	748,726	2,877,306	31,328,232
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	722,104	2,749,610	29,984,915
12. REACTOR SERVICE FACTOR (%)	100.0	58.2	65.3
13. REACTOR AVAILABILITY FACTOR (%)	100.0	58.2	68.7
14. UNIT SERVICE FACTOR (%)	100.0	55.9	64.1
15. UNIT AVAILIBILITY FACTOR (%)	100.0	55.9	64.1
16. UNIT CAPACITY FACTOR (USING MDC) (%)	93.7	52.2	57.1
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	90.0	50.1	54.9
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	4.3	15.4

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
(None)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
(N/A)

TABLE 4

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE August 10, 1990
COMPLETED BY J.W. YOUNSTEDT
TELEPHONE (815)-357-6761

REPORT PERIOD: JULY 1990

DAY	POWER	DAY	POWER
1	1,001	17	802
2	1,069	18	938
3	1,067	19	946
4	1,059	20	1,006
5	1,064	21	1,066
6	1,066	22	977
7	1,069	23	1,059
8	1,071	24	1,065
9	1,068	25	1,060
10	1,062	26	1,021
11	1,028	27	1,059
12	1,048	28	1,059
13	777	29	1,061
14	190	30	1,060
15	367	31	1,063
16	854		

TABLE 5

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
10	900713	S	0.0	B	4	Reduction for scram-time testing CRD 26-55
11	900714	S	0.0	B	4	Reduction to adjust MSIV limit switches

SUMMARY OF OPERATION:

The Unit remained on-line with most reductions due to scheduled testing and adjustments. A deep load reduction was required to adjust MSIV limit-switches.

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E. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

(None)

2. ECCS System Outages
(See Table 6)

3. Changes to the Off-Site Dose Calculation Manual.
(None).

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

(Unit 2)
Table 6

ECCS System Outages

Note: The year and unit data has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
1527	2B D/G	Replace oil filters
1528	2B D/G	Remove and measure fuel pressure sensing line
1556	2A D/G	Troubleshoot and repair Cylinder #10 thermocouple
1583	2A D/G	Calibrate Cylinder #10 thermocouple

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