AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse #1
DATE 5/07/82
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000
ext. #384

MONTH April, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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R PDR

OPERATING DATA REPORT

DOCKET NO. 50-346
DATE 5/7/82
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000 ext. #384

OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
2. Reporting Period: April, 1982
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719	2,879	32,880
12. Number Of Hours Reactor Was Critical	0	1,708	17,938
13. Reactor Reserve Shutdown Hours	0	0	2,334.7
14. Hours Generator On-Line	0	1,707.4	16,957.6
15. Unit Reserve Shutdown Hours	0	0	1,731.4
16. Gross Thermal Energy Generated (MWH)	0	3,641,078	38,762,603
17. Gross Electrical Energy Generated (MWH)	0	1,202,294	12,884,545
18. Net Electrical Energy Generated (MWH)	0	1,124,093	12,021,378
19. Unit Service Factor	0	59.3	51.6
20. Unit Availability Factor	0	59.3	56.8
21. Unit Capacity Factor (Using MDC Net)	0	44.7	41.8
22. Unit Capacity Factor (Using DER Net)	0	43.1	40.4
23. Unit Forced Outage Rate	0	0	23.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: August 1, 1982
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346
UNIT NAME Davis-Besse Unit 1
DATE 5/7/82
COMPLETED BY Bilal Sarsour
TELEPHONE 419-259-5000, Ext. 384

REPORT MONTH April, 1982

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	82 03 13	S	719	C	4	NA	NA	NA	Unit outage which began on March 13, 1982 was still in progress through the end of April, 1982. See Operational Summary for further details.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation from Previous Month
5-Load Reduction
6-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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OPERATIONAL SUMMARY
April, 1982

04/01/82 - 04/30/82: The unit outage which began on March 13, 1982 was still in progress through the end of April, 1982.

The following are the more significant outage activities performed during this month:

(1) An Automatic Reactor Inspection System (ARIS) inspection of the reactor vessel hot leg nozzle welds was conducted.

(2) Eddy Current testing in steam generators. During that test it was discovered that some of the steam generator tubes located adjacent to the auxiliary feedwater header showed potential interaction with the header support system. A secondary side manway from steam generator 1-1 was removed and it was determined by direct visual observation and fiberoptic inspection that the auxiliary feedwater header was not securely fastened and had experienced damage. Inspection of the other steam generator yielded similar results. The root cause of this event has not yet been determined. Toledo Edison is working with other owners to evaluate possible corrective action.

(3) The turbine, associated valves, and the #2 main feed pump were disassembled and inspected. Problems were found in the 'A' low pressure turbine rotor (1-2) 11th stage buckets. The problem was cracks appearing on the steam admission side of the dovetail. The failure mechanism appears to be frequency related, high cycle fatique. Similar problems were found to exist on the 'B' low pressure turbine rotor (1-1) 11th stage buckets. All 11th stage buckets on both low pressure turbine rotors have been sent to the factory for modification to allow the installation of tie wires which will dampen out any vibration. The problems discovered do not have a nuclear safety significance.

(4) The condenser inspection revealed the ruptured 10" expansion joint in the high pressure condenser was one that was replaced in November. The probable cause of failure was pipe misalignment. Corrective action will be to add increased supports and braces to prevent piping disalignment in the future.

(5) The visual inspection of thermal shield bolts were completed satisfactorily with no abnormalities noted and no corrective action required.

(6) Fuel shuffle was successfully completed, one broken hold-down spring was found on a fuel assembly scheduled for removal this cycle. This spring was removed for the inspection.

(7) The Bailey 855 station computer was removed and replaced by the new MODCOMP classic dual central processing unit computer system.

(8) #2 diesel generator inspection was completed with only one problem found. The main oil pump assembly was worn due to a factory assembly problem and the whole pump was replaced.

(9) Atmospheric vent valves were disassembled and inspected.

(10) Thirteen of the eighteen main steam safety valves were removed and sent to Mark Controls for overhaul. Problems were found in the stems and the defective stems are being replaced.

(11) FCR work continued this month. A detailed list of the major FCRs completed will be provided at a later date.

BMS/lmr

REFUELING INFORMATION

DATE: April, 1982

1. Name of facility: Davis-Besse Unit 1

2. Scheduled date for next refueling shutdown: March 12, 1982

3. Scheduled date for restart following refueling: August 1, 1982

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   The final reload analysis for Cycle 3 has been completed and submitted to the NRC (See Serial No. 787 dated March 5, 1982). This analysis identifies several technical specification changes relating to core operational limits and reactor protection system setpoints. An option to provide flexibility in the overall cycle length is also provided therein.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. See response to No. 4 above

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   None identified to date

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   (a) 177 (b) 92 - Spent Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   Present 735 Increase size by 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   Date 1988 - assuming ability to unload the entire core into the spent fuel pool is maintained.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 79-194

SYSTEM: N/A

COMPONENT: Setpoint Index M-620S

CHANGE, TEST, OR EXPERIMENT: Facility Change Request 79-194 was written to revise setpoint index M-620S to include as-built setpoints.

REASON FOR CHANGE: To update setpoint index to reflect as-built setpoints.

SAFETY EVALUATION: All changes to the setpoint index have been reviewed by Bechtel Engineering and Toledo Edison Engineering. All changes in this revision accurately reflect as-built conditions and are consistent with the Davis-Besse Unit 1 systems designs as described in the FSAR. Exceptions include where changes to Davis-Besse Unit 1 have been made in accordance with 10 CFR 50.59 and documented by FCRs as required. Therefore, this change is not an unreviewed safety question.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 77-056

SYSTEM: Boric Acid Addition

COMPONENT: Boric Acid Pumps 1 and 2

CHANGE, TEST, OR EXPERIMENT: Facility Change Request 77-056 was implemented to replace the vent plug for Boric Acid Pumps 1 and 2 with a valved line to act as a casing vent.

REASON FOR CHANGE: Previously, the only vent path or means to vent the Boric Acid Pumps was the removal of the pipe plug. This modification allows the pump casing to be periodically vented by operation of a small manual valve instead of requiring tools to remove a vent plug.

SAFETY EVALUATION: The function of the Boric Acid Pumps will not be affected by the addition of casing vent. Therefore, an unreviewed safety question does not exist.