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July 2, 1982 NRC/TMI-82-041

MEMORANDUM FOR: Harold R. Denton, Director Office of Nuclear Reactor Regulation

> Bernard J. Snyder, Program Director TMI Program Office

FROM: Lake H. Barrett, Deputy Program Director TMI Program Office

SUBJECT: NRC TMI PROGRAM OFFICE WFTKLY STATUS REPORT

Enclosed is the status report for the period of June 27, 1982 to July 3, 1982. Major items included in this report are:

- -- Liquid Effluents
- -- EPA and NRC Environmental Data
- -- Radioactive Material and Radwaste Shipments
- -- Submerged Demineralizer System Status
- -- EPICOR II
- -- Reactor Coolant System Feed and Bleed
- -- Reactor Building Entry
- -- Groundwater Monitoring
- -- Public Meeting

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Lake H. Barrett Deputy Program Director TMI Program Office

Enclosure: As stated

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Harold R. Denton Bernard J. Snyder

cc w/encl: ED0 OGC Office Directors Commissioner's Technical Assistants NRR Division Directors NRR A/D's Regional Administrators IE Division Directors TAS EIS TMI Program Office Staff (15) PHS EPA DOE Projects Br. #2 Chief, DPRP, RI DPRP Chief, RI Public Affairs, RI State Liaison, RI

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NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT

June 27, 1982 - July 3, 1982

Plant Status

Core Cooling Mode: Heat transfer from the reactor coolant system (RCS) loops to reactor building ambient.

Available Core Cooling Modes: Decay heat removal (DHR) systems, Mini DHR (MDHR) system.

RCS Pressure Control Mode: Standby pressure control (SPC) system. NOTE: During Reactor Coolant System feed and bleed, pressure will be maintained with a Reactor Coolant Bleed Tank Pump. Automatic back up pressure control will be provided by the SPC system.

Backup Pressure Control Modes: MDHR and DHR system.

Major Parameters (as of 0600, July 2, 1982) (approximate values) Average Incore Thermocouples: 101°F Maximum Incore Thermoccuple: 125°F

RCS Loop Temperatures:

Hot Leg	A 94°F	97°F
Cold Leg (1)	85°F	81°F
(2)	88°F	81°F

Pressure: 72 psig

NOTE: During reactor coolant system feed and bleed, pressure is maintained at approximately 70 psig.

Reactor	Building:	Temperature: Pressure: Airborne Radio	75°F -0.6 psig nuclide Concentrations:
			<pre>3.1 E-6 uCi/cc H³ (sample taken 6/30/82) 1.3 E-6 uCi/cc Kr⁸⁵ (sample taken 6/30/82)</pre>
			5.8 E-8 uCi/cc particulates (sample taken 7/1/82)

1. Effluent and Environmental (Radiological) Information

Liquid effluents from the TMI site released to the Susquehanna River after processing, were made within the regulatory limits and in accordance with NRC requirements and City of Lancaster Agreement dated February 27, 1980.

During the period June 25, 1982, through July 1, 1982, the effluents contained no detectable radioactivity at the discharge point and individual effluent sources, which originated within Unit 2, contained no detectable radioactivity.

2. Environmental Protection Agency (EPA) Environmental Data

The EPA measured Kr-85 concentrations at several environmental monitoring stations and reported the following results:

Location	May 21, 1982 through June 11, 1982
	(pCi/m ³)
Goldsboro	21
Middletown	21
Yorkhaven	25
TMI Observation Center	32

3. NRC Environmental Data

Results from NRC monitoring of the environment around the TMI site were as follows:

-- The following are the NRC air sample analytical results for the onsite continuous air sampler:

Sample	Period	I-131 Cs-137 (uCi/cc) (uCi/cc)
HP-325	June 24, 1982 - June 30, 1982	<7.4 E-14 <7.4 E-14

4. Licensee Radioactive Material and Radwaste Shipment

No shipments of radioactive material or radwaste were made during this reporting period.

Major Activities

 Submerged Demineralizer System (SDS). The processing of the third batch of reactor coolant system (RCS) water was completed on June 25, 1982. Processing parameters for this 50,000 gallon batch are shown in Attachment I. Subsequent to processing SDS Batch No. 30, there was a zeolite vessel changeout: the vessel in position 1A was removed and placed in storage; the vessel that was in position 1B was placed in position 1A; and a new zeolite vessel was placed in position 1B. Processing of SDS Batch No. 31 (the fourth batch of RCS water), consisting of approximately 50,000 gallons, began on July 1, 1982; the anticipated completion date is July 9, 1982.

- <u>EPICOR II</u>. The EPICOR II system is currently shutdown on a standby status.
- 3. <u>Reactor Coolant System (RCS) Feed and Bleed</u>. The fourth feed and bleed cycle of the RCS water began on June 28 and was completed on June 30, 1982. Approximately 50,000 gallons were cycled. The cycling included the water in the pressurizer, to clean the stagnant water in the pressurizer. One more RCS feed and bleed cycle is scheduled before beginning the "Quick Look" experiment: this experiment will involve inserting a small television camera into the reactor to try to determine the condition of the core. (Additional descriptions of this experiment will appear in future Weekly Status Reports.)
- 4. <u>Reactor Building Entry</u>. The following tasks were accomplished during the reactor building entry on Thursday, July 1, 1982:
 - -- Two work platforms and the trolley/hoist rig were installed in preparation for the lead screw removal for the "Quick Look" experiment.
 - -- The interior of the "B" D-ring was decontaminated using a low-pressure water flush. Subsequent to this operation, a radiological survey team attempted to evaluate its effectiveness. The beta monitoring instrument failed during the survey, delaying the evaluation.

The next entry is scheduled for Thursday, July 8, 1982.

5. <u>Groundwater Monitoring</u>. The results from the test boring water samples that were taken on June 8, 1982, and sent to an off-site laboratory for analysis have not been received. Water samples from test borings 2, 3, 4, 10, 16, and 17, taken on June 15, 1982, were analyzed on-site and compared with the June 8, 1982 samples that were also analyzed on-site. These results show that the tritium concentrations have remained in the same range (order of magnitude) as previously reported. 632

Past Meetings

On Wednesday, June 30, 1982, Lake Barrett, with Carol Ramsey and Pat Rathbun (who are both with the NRC Office of Research) discussed various sociological aspects of emergency planning with a group of Middletown residents.

ATTACHMENT I

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SDS PERFORMANCE FOR BATCH NUMBER 30 (Reactor Coolant System Water)

Radionuclide	Average Influent (uc/ml)	Average Effluent (uc/ml)	Average DF
Cesium 137	3.7	5.3×10^{-4}	7.0×10^3
Strontium 90	9.7	2.7×10^{-2}	3.6×10^2

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