OPERATING DATA REPORT

DOCKET NO.	50-289
DATE	July 15, 1982
COMPLETED BY	C. W. Smyth
TELEPHONE	717 948-8551

OPERATING STATUS

1. Unit Name: _____ Three Mile Island Nuclear Station, Unit I

2. Reporting Period: _____June, 1982_____

3. Licensed Thermal Power (MWt): ____2535_____

4. Nameplate Rating (Gross MWe): _____871

5. Design Electrical Rating (Net MWe): 819

6. Maximum Dependable Capacity (Gross MWe): 840 776

7. Maximum Dependable Capacity (Net MWe): _____

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr -to-Date	Cumulative
11. Hours In Reporting Period	720.	4343.	68616.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14 Hours Generator On-Line	0.0	0.0	31180.9
15 Unit Reserve Shutdown Hours	0.0	0.0	0.0
6 Cross Thermal Energy Concrated (MWH)	0.0	0.0	76531071.
17 Gross Flactrical Energy Generated (MWH)	0.	0.	25484330.
Not Electrical Energy Generated (MWM)	0.	0.	23840053.
18. Net Electrical Energy Generated (MME)	0.0	0.0	45.4
19. Unit Service Factor	0.0	0.0	45.4
20. Unit Availability Factor	0.0	0.0	44.3
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	42.4
22. Unit Capacity Factor (Using DER Net)	100.0	100.0	49.1
23. Unit Forced Outage Rate	100.0	100.0	47.1
24 Shutdowns Scheduled Over Next 6 Months (Tv	ne Dair and Duration	of Each I:	

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _		
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	8. <u></u>	
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-289
UNIT	TMI-I
	July 15, 1982
COMPLETED BY	C. W. Smyth
TELEPHONE	717-948-8551

MONT	TH June, 1982	
DAY	AVERAGE DAILY POWER LEVEL (Mwe-Net)	DAY
1	0	17
2	0	18
3	0	19
4	0	20
5	0	21
6	0	22
7	0	23
8	0	24
9	0	25
10	0	26
11	0	20
12	0	25
13	0	20
14	0	29
14	0	30
16	0	31
10		

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
27	0
23	0
24	0
24	0
25	0
26	
27	
28	0
29	0
30	0
31	0

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June, 1982

DOCKET NO. 50-289 UNIT NAME TMI-I DATE July 15, 1982 COMPLETED BY C. W. Smyth TELEPHONE 717-948-8551

-

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor 3	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	6/1/82	F	720	D	1				Regulatory Restraint Order
T F: Fo S: Sch	rced heduled	Reaso A-Eq B-Ma C-Ref D-Re E-Op F-Ad G-Op H-Ot	on uipment Fa intenance o lueling gulatory Re erator Train ministrative erational E her (Explain	ilure (E: r Test striction ing & L ror (Ex	xplain) 1 icense Exa plain)	mination	Method 1-Mana 2-Mana 3-Auto 4-Other	l: ial scram. matic Scram. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I - Same Source

OPERATING SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System.

Major Safety Related Maintenance

During the month of June, Unit No. 1 operating in the cold shutdown condition while OTSG repair activities progressed and restart modifications continued. The following major maintenance items were performed.

- 1. The Once Through Steam Generator (OTSG) Program continued with 141 tube stabilizers installed in the "A" Generator and 24 installed in the "B" Generator. There were 4 tapered plugs installed in the "A" Generator and 6 installed in the "B" Generator. The tapered plugs were installed in the tubesheet where tubes were removed. There were 145 explosive plugs installed in the bottom of the "A" Generator and 30 installed in the "B" Generator. Preparations are being made to continue with the Eddy Current Testing Program.
- The Reactor Vessel Head Inspection Program concluded this month with the following work performed:
 - . torqued APSR closures
 - . torqued CRDM closures
 - . installed missile shields
- Nuclear Services cooler (NS-C-1D) work continued with 10 tubes being plugged and a stabilizer installed in 1 tube.
- Reactor Building Purge Valve (AH-V-1 A/D) work commenced with adjusting the segment bolts on the "C" and "D" valves and shimming the seats on the "A" and "B" valves.

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

 Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

 The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(ъ) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

100

N/A

OPERATING DATA REPORT

DOCKET NO.	_50-289
DATE	July 15, 1982
COMPLETED BY	C. W. Smyth
TELEPHONE	/1/ 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I

2. Reporting Period: ____June, 1982____

3. Licensed Thermal Power (MWt): _2535

4. Nameplate Rating (Gross MWe): 871

5. Design Electrical Rating (Net MWe): 819

6. Maximum Dependable Capacity (Gross MWe): 840

7. Maximum Dependable Capacity (Net MWe): 776

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr -to-Date	Cumulative
11 Hours In Reporting Period	720.	4343.	68616.
12 Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13 Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14 Hours Conceptor On-Line	0.0	0.0	31180.9
15 Linit Parama Shutdown Hours	0.0	0.0	0.0
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10. Gross Thermal Energy Generated (MWH)	0.	0.	25484330.
Gross Electrical Energy Generated (MWH)	0.	0.	23840053.
18. Net Electrical Energy Generated (MWH)	0.0	0.0	45.4
19. Unit Service Factor	0.0	0.0	45.4
20. Unit Availability Factor	0.0	0.0	44.3
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	42.4
22. Unit Capacity Factor (Using DER Net)	0.0	100.0	40.1
23. Unit Forced Outage Rate	100.0	100.0	
24 Shutdowns Scheduled Over Next 6 Months (Tv)	pe Date and Duration	of Each 1:	

25. If Shut Down At End Of Report Period, Estimated Date of Startup		
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-289			
UNIT	TMI-I			
DATE	July 15, 1982			
COMPLETED BY	C. W. Smyth			
TELEPHONE	717-948-8551			

MONTH	June, 1982
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3 .	0
4 .	0
5	0
6.	0
7 .	0
8.	0
9 .	0
10 .	0
11	0
12	0
13	0
14	0
15	0
16	0
	and a state of the

DAY	AVER AGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	Q
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

F Fo	-	No.
reduled	6/1/82	Date
D-Reas	শ্য	Type ¹
un mon mon menance fueling gulatory R gulatory R erational I	720	Duration (Hours)
anlure (F or Test e cstriction anng & I	D	Reason ²
n xplain)	-	Method of Shutting Down Reactor ³
noiten la		Licensee Event Report #
3 Metho 2-Man 3 Anta 4 Othe		System Code ⁴
d: nal Scram, r (Explain)		Component Code ⁵
4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUR 0161) 5 Exhibit L Same Source	Regulatory Restraint Order	Cause & Corrective Action to Prevent Recurrence

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289 UNIT NAME THI-I

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OPERATING SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System.

Major Safety Related Maintenance

During the month of June, Unit No. 1 operating in the cold shutdown condition while OTSG repair activities progressed and restart modifications continued. The following major maintenance items were performed.

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 - . installed missile shields
- 3. Nuclear Services cooler (NS-C-1D) work continued with 10 tubes being plugged and a stabilizer installed in 1 tube.
- 4. Reactor Building Purge Valve (AH-V-1 A/D) work commenced with adjusting the segment bolts on the "C" and "D" valves and shimming the seats on the "A" and "B" valves.

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Conmittee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

 Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

 The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(3) 208

 The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

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1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).