OPERATING DATA REPORT

DOCKET NO. 50-313
DATE 5/15/82
COMPLETED BY A.J.Gertsch
TELEPHONE (501)964-3155

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 1
2. Reporting Period: May 1 - 31, 1982
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 902.74
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 886

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	3,623.0	65,298.0
12. Number Of Hours Reactor Was Critical	388.9	2,435.9	44,683.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	5,044.0
14. Hours Generator On-Line	359.7	2,399.0	43,782.8
15. Unit Reserve Shutdown Hours	0.0	0.0	817.5
16. Gross Thermal Energy Generated (MWH)	486,646.	4,816,699.	104,346,905.
17. Gross Electrical Energy Generated (MWH)	145,644.	1,605,220.	34,441,976.
18. Net Electrical Energy Generated (MWH)	133,056.	1,518,386.	32,834,787.
19. Unit Service Factor	48.3	66.2	67.1
20. Unit Availability Factor	48.3	66.2	68.3
21. Unit Capacity Factor (Using MDC Net)	21.4	50.1	60.1
22. Unit Capacity Factor (Using DER Net)	21.0	49.3	59.2
23. Unit Forced Outage Rate	0.0	0.0	15.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-313

UNIT 1

DATE 5/15/82

COMPLETED BY A. J. Gertsch

TELEPHONE (501) 964-3155

MONTH May 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	305
2	0	18	316
3	0	19	315
4	0	20	297
5	0	21	297
6	0	22	346
7	0	23	718
8	0	24	792
9	0	25	197
10	108	26	0
11	372	27	0
12	296	28	0
13	297	29	0
14	298	30	0
15	297	31	0
16	296		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

WP-0331

NRC MONTHLY OPERATING REPORT
OPERATING SUMMARY - MAY,1982
UNIT I

The spring outage for replacement of the "A" OTSG feedwater nozzles was extended into the first 10 days of May when a radiography inspection of the HPI nozzles revealed potential problems in all but the "C" nozzle. "A" and "B" nozzles were replaced and the "D" nozzle was resized, a leaking seal on the "A" Decay Heat Pump required replacement of the pump seals and bearings, and Generator Hydrogen Seal Oil leaks required replacement of the hydrogen seal rings.

Unit One began heat up May 3, and the reactor was brought critical at 0720 hours May 9. The generator was subsequently brought on line at 0715 hours May 10. High level indications and the possibility of tube leaks in the E-1A, E-3A, and E-4A feedwater heaters delayed power escalation as the cause of the high levels was investigated. Failure of the E-5A, E-6A, and E-8A feedwater bypass valve in the bypass position had caused the high levels in E-3A and E-4A. Following repair of the bypass valve and tube leaks in the E-1A feedwater heater, Unit One resumed power escalation on May 22. The Unit achieved 97.17 percent full power May 24. Power was limited due to high "A" OTSG level and low feedwater pump suction pressure. At 0540 hours May 25, high radiation alarms on the condenser off-gas and the "A" OTSG N-16 monitors alerted operators of a possible steam generator tube leak. At 1213 hours May 25, Unit One was brought to cold shutdown for repair of a tube leak in the "A" OTSG. Reactor Coolant System leak rate was determined to be less than 1 gpm. Unit One remained in a cold shutdown through the end of May for repair of the "A" OTSG tube leak.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May

DOCKET NO. 50-313
UNIT NAME ANO-Unit 1
DATE June 2, 1982
COMPLETED BY A.J. Gertsch
TELEPHONE 501-964-3155

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-02	820326	S	223.3	B	4	N/A	CC	HTEXCH	Continuation of the outage to replace the feedwater nozzles in the "A" OTSG.
82-03	820525	F	161.0	A	1	012	CC	HTEXCH	A tube leak developed in the "A" OTSG. Unit was brought to cold shutdown. One tube was leaking. Eddy current testing revealed 9 additional tubes which required plugging. Eddy current was used to verify the integrity of other tubes.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely[1]. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following critieria:

A. If a component failed, use the component directly involved.

B. If not a component failure, use the related component, e.g., wrong valve operated through error: list valve as component.

C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

---

[1]Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

DATE: May 1982

REFUELING INFORMATION

1. Name of facility. Arkansas Nuclear One - Unit 1

2. Scheduled date for next refueling shutdown. 12/1/82

3. Scheduled date for restart following refueling. 2/15/83

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
If answer is yes, what, in general, will these be?
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Yes. Reload report and associated proposed Specification changes.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. 9/1/82

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Will reload 72 fresh fuel assemblies and operate for approximately 16 months. Four of which will be high burn-up test assemblies.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 244

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

present 589 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1986