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Summary and Bibliography of Safety-Related Events at Boiling-Water Nuclear Power Plants as Reported in 1980

K. E. McCormack
R. B. Gallaher

Prepared for the U.S. Nuclear Regulatory Commission
Office of Nuclear Regulatory Research
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SUMMARY AND BIBLIOGRAPHY OF SAFETY-RELATED EVENTS AT BOILING-WATER NUCLEAR POWER PLANTS AS REPORTED IN 1980

K. E. McCormack
R. B. Gallaher

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FOREWORD

The Nuclear Safety Information Center (NSIC), which was established in March 1963 at Oak Ridge National Laboratory, is sponsored by the U.S. Nuclear Regulatory Commission's Office of Analysis and Evaluation of Operational Data. Support for the technical progress review *Nuclear Safety* (see last page of this report) is provided by both the Breeder Reactor and Light-Water-Reactor Safety Programs of the Department of Energy. NSIC is a focal point for the collection, storage, evaluation, and dissemination of operational safety information to aid those concerned with the analysis, design, and operation of nuclear facilities. The Center prepares reports and bibliographies as listed on the inside covers of this document. NSIC has developed a system of keywords to index the information it catalogs. The title, author, installation, abstract, and keywords for each document reviewed are recorded at the central computing facility in Oak Ridge.

Computer programs have been developed that enable NSIC to (1) prepare monthly reports with indexed summaries of Licensee Event Reports, (2) make retrospective searches of the stored references, and (3) produce topical indexed bibliographies. In addition, the Center Staff is available for consultation, and the document literature at NSIC is available for examination. NSIC reports (i.e., those with the ORNL/NSIC and ORNL/NUREG/NSIC numbers) may be purchased from the National Technical Information Service (see inside front cover). All of the above services are available free of charge to U.S. Government organizations as well as their direct contractors. Persons interested in any of the services offered by NSIC should address inquiries to:

J. R. Buchanan, Director
Nuclear Safety Information Center
P. O. Box Y
Oak Ridge National Laboratory
Oak Ridge, Tennessee 37830

Telephone 615-574-0391
FTS 624-0391

PREVIOUS REPORTS IN THIS SERIES

1. W. R. Casto and E. N. Cramer, *Safety-Related Occurrences in Nuclear Facilities as Reported in 1967 and 1968*, ORNL/NSIC-69 (July 1970).

2. R. L. Scott and W. R. Casto, *Safety-Related Occurrences in Nuclear Facilities as Reported in 1969*, ORNL/NSIC-87 (August 1971).

3. R. L. Scott, *Safety-Related Occurrences in Nuclear Facilities as Reported in 1970*, ORNL/NSIC-91 (December 1971).

4. R. L. Scott and R. B. Gallaher, *Safety-Related Occurrences in Nuclear Facilities as Reported in 1971*, ORNL/NSIC-106 (September 1972).

5. R. L. Scott and R. B. Gallaher, *Safety-Related Occurrences in Nuclear Facilities as Reported in 1972*, ORNL/NSIC-109 (December 1973).

6. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Occurrences in Nuclear Power Plants as Reported in 1973*, ORNL/NSIC-144 (November 1974).

7. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Occurrences in Nuclear Power Plants as Reported in 1974*, ORNL/NSIC-122 (May 1975).

8. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Occurrences in Boiling-Water Nuclear Power Plants as Reported in 1976*, ORNL/NUREG/NSIC-126 (July 1975).

9. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Occurrences in Pressurized-Water Nuclear Power Plants as Reported in 1975*, ORNL/NUREG/NSIC-127 (July 1976).

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11. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Occurrences in Pressurized-Water Nuclear Power Plants as Reported in 1976*, ORNL/NUREG/NSIC-138 (July 1977).

12. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Events in Boiling-Water Nuclear Power Plants as Reported in 1977*, ORNL/NUREG/NSIC-149 (November 1978).

13. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Events in Pressurized-Water Nuclear Power Plants as Reported in 1977*, ORNL/NUREG/NSIC-150 (November 1978).

14. R. L. Scott and R. B. Gallaher, *Annotated Bibliography of Safety-Related Events at Boiling-Water Nuclear Power Plants as Reported in 1978*, ORNL/NUREG/NSIC-164 (September 1979).

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16 R. L. Scott and R. B. Gallaher, *Summary and Bibliography of Safety-Related Events at Boiling-Water Nuclear Power Plants as Reported in 1979*, ORNL/NUREG/NSIC-178 (June 1981).

17. R. L. Scott and R. B. Gallaher, *Summary and Bibliography of Safety-Related Events at Pressurized-Water Nuclear Power Plants as Reported in 1979*, ORNL/NUREG/NSIC-179 (July 1981).

SUMMARY AND BIBLIOGRAPHY OF SAFETY-RELATED EVENTS AT BOILING-WATER NUCLEAR POWER PLANTS AS REPORTED IN 1980

K. E. McCormack
R. B. Gallaher

ABSTRACT

This document presents a bibliography that contains 100-word abstracts of event reports submitted to the U.S. Nuclear Regulatory Commission concerning operational events that occurred at boiling-water-reactor nuclear power plants in 1980. The 1547 abstracts included on microfiche in this bibliography describe incidents, failures, and design or construction deficiencies that were experienced at the facilities. These abstracts are arranged alphabetically by reactor name and then chronologically for each reactor. Full-size keyword and permuted-title indexes to facilitate location of individual abstracts are provided following the text. Tables that summarize the information contained in the bibliography are also provided. The information in the tables includes a listing of the equipment items involved in the reported events and the associated number of reports for each item. Similar information is given for the various kinds of instrumentation and systems, causes of failures, deficiencies noted, and the time of occurrence (i.e., during refueling, operation, testing, or construction). Some of the more interesting events that occurred during the year are reviewed in detail.

INTRODUCTION

The Nuclear Safety Information Center (NSIC) has collected, reviewed, and/or published reports of safety-related reactor operational events since its inception in March 1963. During this same year, the Atomic Energy Commission (AEC) Division of Operational Safety started disseminating information on unusual reactor operating experiences in memorandum form. The first collection of this type of reactor operating experiences [disseminated by the AEC and later by the U.S. Nuclear Regulatory Commission (NRC)] was published by NSIC[1] covering the years 1963, 1964, and 1965, with subsequent publications appearing every three years.[2-5]

In 1968, a second series of reports presenting this type of information was started with the publication of a journal article on safety-related events obtained from the NSIC computerized file of events reported in 1966 by the licensees of nuclear facilities.[6] These events are now also known as reportable occurrences or licensee events. This series has continued with publications appearing each year.[7-23] However, the number

of event reports has increased tremendously each year; in the 1966 compilation,[6] 76 events were reported for all types of reactors, including research and development reactors. However, for the 1973 events, the compilation had to be restricted to events that occurred at nuclear power plants only, because the number of events had increased to 1053 (Ref. 12). For the 1975 events, this series was further restricted to events at boiling-water reactor (BWR) and pressurized-water reactor (PWR) nuclear power plants. At this point, two publications were required because 1169 events occurred for the BWRs and 1097 events for the PWRs.[14,15] Two publications have been issued each year since 1975 to report the BWR and PWR events: the 1980 compilation contains 1547 events for the BWRs (this report) and 2166 for the PWRs.[24]

Of special note is a report prepared by NSIC for the President's Commission on the Accident at Three Mile Island. This compilation and summary of the more serious reactor accidents from 1952 to 1979 deals with power reactors as well as experimental research and production reactors.[25]

This report [along with ORNL/NSIC-196 (Ref. 24)] is the 13th in a series issued annually by the NSIC presenting abstracts of reports of safety-related events submitted during the year to the NRC by the licensees of light-water reactors in the United States. In particular, the report contains abstracts of 1547 events reported in 1980. The abstracts are presented on microfiche, which are filed in an envelope attached to the back cover of the report.

Statistical data obtained from a review of the event reports are discussed and summarized in the tables in the following sections. The summarized information includes a listing of equipment items and the number of reports (events) in which each item was involved. Similar information is given for instrumentation, safety-related systems, causes of failures, types of deficiencies, type of personnel involved, and the time of occurrence (i.e., construction, refueling, operation, or testing). Selected events are also reviewed in detail because of their relevance to the safe operation and performance of nuclear plants.

The reports of safety-related events abstracted in the bibliography were submitted by power plant licensees to the NRC in accordance with federal regulations. The reporting requirements for nuclear facility licensees are included in Title 10, *Code of Federal Regulations*, Parts 20, 40, 50, 70, and 73 (Ref. 26). The *NRC Regulatory Guide 10.1*, Revision 3, May 1977 (Ref. 27), presents a compilation of the reporting requirements for persons subject to NRC regulations. Reporting of operating information is described in detail in *NRC Regulatory Guide No. 1.16* (Ref. 28). The requirements for reporting design or construction deficiencies in nuclear facilities that have been granted construction permits are given in Title 10, *Code of Federal Regulations*, Part 50, Section 55, Paragraph e (1 through 3) (Ref. 29).

The information for this report was obtained from the NSIC computer files in the form of 100-word abstracts of the reports submitted by the reactor licensees. The abstracts, with appropriate keywords used for computer storage and retrieval, were prepared by technical specialists at NSIC. Input to the computer is a continuing process; therefore, persons desiring an update of the information on operating experiences at nuclear power plants may obtain a literature search by contacting the NSIC.

The NSIC computer also provides a bimonthly printout of those events that resulted in reactor shutdown and their causes; this information is included in each issue of the bimonthly journal, *Nuclear Safety*.

The abstracts in this report are arranged alphabetically by reactor name and then chronologically for each reactor. In addition to the abstracts describing each event, full-size keyword and permuted-title indexes are provided for quick location of the abstracts in which a particular item is discussed. For example, persons interested in the problems experienced with diesel generators can find the relevant abstracts listed under the keyword *generator, diesel;* or using the permuted-title index, they can find the abstracts with the word *diesel* or the word *generator* in the title of the article. In the keyword index, some keywords appear twice, once with an asterisk and once without an asterisk. Asterisked keywords indicate that the reports (indicated by the accession number and page) discuss the keyword item in more detail than the reports listed under a nonasterisked keyword. The asterisk is also used to indicate that the keyword item actually failed, whereas a keyword item without an asterisk indicates that the keyword item was involved in an event but did not necessarily fail.

Before reviewing the bibliography, it may be helpful to review "Method of Indexing Licensee Event Reports" (p. 153), which shows a typical abstract with its component parts identified. Note that the list of keywords gives a quick indication of the content of the abstract. The availability of the original material is indicated (i.e., the NRC Public Document Room, 1717 H Street, Washington, DC 20555).

SUMMARY OF SAFETY-RELATED EVENT DATA

The 1547 abstracts in the bibliography were reviewed, and tabulations were made of significant items to indicate the total number of reports concerned with those items. These tabulations indicate items that should receive more attention by reactor operators, designers, or other interested parties.

Table 1 lists the number of reports concerned with the various systems. As in 1976, 1977, and 1978, the system most frequently reported on was the main cooling system. Major or safety-related systems were reported on more often than the less important systems; however, this occurrence only reflects the attention given to these major systems by the utilities in the form of surveillance testing as well as the emphasis that the NRC has placed on these systems in its reporting requirements. The total number of reports shown in Table 1 exceeds the total number of 1980 reports submitted by the licensees because, in some reports, more than one system may have been involved in the event. (Systems involved in <1% of the total number of reports are not listed.)

Table 2 lists the number of reports concerned with various items of equipment. This year, as in previous years, valves, piping, and pumps were the equipment items for which the most licensee event reports (LERs) were submitted. These three items alone were involved in 45% of the events reported. The total number of reports shown in this table exceeds the total number of 1980 reports submitted by the licensees because, in some reports, more than one item of equipment may have been involved in

Table 1. Reports concerned with listed systems

System	Number of reports	Percent of total
Main cooling	146	9
Steam	133	8
Electric power	127	8
High-pressure coolant injection	125	8
Reactor protection	111	7
Shut-down cooling	105	7
Containment isolation	101	6
Containment	96	6
Reactor core isolation cooling	96	6
Radiation monitoring	63	4
Fire protection	59	4
Emergency electric power	59	4
Service water	56	4
Core spray	53	4
Low-pressure coolant injection	42	3
Ventilation	40	3
Engineered safety features	39	3
Coolant purification	28	2
Standby gas treatment	22	1
Feedwater	16	1
Pneumatic	16	1

Table 2. Reports concerned with listed equipment

System	Number of reports	Percent of total
Valves	398	26
Pumps	160	10
Pipes and fittings	135	9
Valve operators	128	8
Torus	89	6
Seals	78	5
Cables and connectors	73	5
Pressure vessels	72	5
Turbines	71	5
Breakers	61	4
Internal combustion engines	60	4
Control rod drives	56	4
Diesel generators	54	3
Control rods	46	3
Fasteners	38	2
Storage container	37	2
Shock absorbers	35	2
Motors	34	2
Solenoids	31	2
Buildings	28	2
Containment vacuum breakers	25	1
Check valves	24	1
Heat exchangers	23	1
Filters	17	1

the event. (Equipment items involved in <1% of the total number of reports are not listed).

Table 3 is a list of various kinds of instrumentation that presented problems during the year and the number of reports concerned with each item. Since 1971 when similar tables were first prepared, switches, accounting for more malfunctions than any other instrument, have been at the top of the list. Apparently, the reason for this is the large number of switches in safety-related systems as well as their delicacy and sensitivity. Various monitors and sensors account for most of the remaining reports concerned with instrumentation problems. (Items involved in <1% of the events were not listed.)

Table 3. Reports concerned with the listed instrumentation

Instrumentation	Number of reports	Percent of total
Switch	272	18
Pressure sensors	134	9
Level sensors	90	6
Radiation monitors	63	4
Relays	63	4
Position instrument	40	3
Flow sensors	36	2
Electronic function units	33	2
Recorders	31	2
Temperature sensors	29	2
Alarms	20	1

Table 4 lists the identified causes of the safety-related events reported and the number of reports concerned with each cause. Fifty-eight percent of the events involved were inherent failures; these failures were caused by a natural means or essential character of the equipment item (e.g., unexplained set-point drift of an instrument, spurious trips, aging, and wear). A few of these characteristics are given in Table 5, which lists deficiencies and the associated number of reports submitted. Set-point drift and leakage were the most frequently reported items, followed by procedural deficiencies. Procedural deficiencies most frequently involved inadequate procedures, but failure of operators to follow procedures is also included. Deficiency in communication covers those events involving a misunderstanding between personnel; it also includes misinterpretation of procedures or technical specifications.

Many reports noted a deficiency but did not give the cause. For example, many reports discussed leaks and their repair, but did not discuss causes of the leaks. The deficiency listed as "Testing" in Table 5

Table 4. Reports concerned with the listed cause of safety-related events

Cause	Number of reports	Percent of total
Inherent failure	901	58
Operator error	107	7
Design error	106	7
Maintenance error	92	6
Administrative control error	90	6
Installation error	66	4
Construction error	51	3
Fabrication error	21	1

Table 5. Reports concerned with the listed deficiency

Deficiency	Number of reports	Percent of total
Set-point drift	236	15
Leak	140	9
Procedures	99	6
Test interval	64	4
Crud	61	4
Testing	55	4
Abnormal instrument indication	54	3
Instrument calibration	52	3
Wear	45	3
Crack	43	3
Lubrication	30	2
Age effect	29	2
Welds	27	2
Corrosion	24	2
Vibration	18	1
Stress corrosion	9	<1
Erosion	9	<1
Obstruction	7	<1
Airborne release	1	<1

generally denotes that a failure or other event occurred because of testing (e.g., the wrong switch operated during a test resulted in a failure or reactor trip). "Test interval" indicates the failure of personnel to perform a test within the required time interval between surveillance testing; it does *not* indicate an equipment failure.

Table 6 lists the various time periods in which the off-normal events took place and the associated number of reports. The 611 events that were discovered (or occurring) during testing could be remedied with little or no effect on operation, and most of these did occur during operation. The number listed for "operation" was determined by subtracting the verified numbers for testing, refueling, and construction.

Table 7 lists the types of personnel involved in the events and the number of reports that identified the type of personnel. Many reports did not make this identification.

Table 8 lists alphabetically the nuclear reactor units from which reports were received and the associated number of reports per unit. Only those units that were in commercial operation all year are listed. The total number of units listed is 25. For those 25 units that were in commercial operation all year, there are 1541 reports — an average of 62 reports per unit. The total number of reports listed in Table 8 is 1541, whereas the bibliography contains abstracts of 1547 reports. The reason for this discrepancy is that Humboldt Bay is not included in the table as an operating reactor.

Table 6. Reports for the listed time of occurrence of off-normal events

Time of occurrence	Number of reports	Percent of total
Testing	744	48
Operation	569	37
Refueling	233	15
Construction	1	<1

Table 7. Reports received vs types of personnel involved in events

Personnel	Number of reports	Percent of total
Licensed operators	107	7
Designers	106	7
Maintenance	95	6
Construction	51	3
Fabricators	21	1
Nonlicensed operators	20	1
Contractors	12	<1
Radiation protection	8	<1

Table 8. Reports involving the alphabetically listed unit in commercial operation all year

Name	Number of reports	Percent of total	Age (years)	Design electrical rating [net MW(e)]
Arnold	73	5	6.8	538
Big Rock Point	45	3	18.3	72
Browns Ferry 1	90	6	7.4	1065
Browns Ferry 2	57	4	6.5	1065
Browns Ferry 3	61	4	4.5	1065
Brunswick 1	104	7	4.3	821
Brunswick 2	115	7	5.9	821
Cooper	53	3	6.8	778
Dresden 2	46	3	10.9	809
Dresden 3	44	3	9.6	809
FitzPatrick	103	7	6.1	821
Hatch 1	131	8	6.3	786
Hatch 2	171	11	2.0	784
LaCrosse	17	1	12.9	50
Millstone 1	21	1	10.3	650
Monticello	31	2	10.0	545
Nine Mile Point 1	27	2	11.4	610
Oyster Creek	66	4	11.5	650
Peach Bottom 2	36	2	7.1	1065
Peach Bottom 3	33	2	6.5	1065
Pilgrim 1	96	6	8.7	655
Quad Cities 1	36	2	8.9	789
Quad Cities 2	37	2	8.8	789
Vermont Yankee	45	3	8.5	514

Tables 9 and 10 list the number of reports for the units that were commercially operable all year. Table 9 tabulates them by age, and Table 10 uses power − DER in megawatts (electrical). These tables were prepared to determine if age or power level was a factor in the number of reportable events occurring at a nuclear unit. Figures 1 and 2 are plots of the number of reports vs age and plant size, respectively.

Figure 1 indicates both visually and with a least-squares fit that some correlation exists between the number of reports and the age of the plant (i.e., as the plants get older fewer LERs are submitted). Figure 2 indicates both visually and with a least-squares fit that very little correlation exists between size and number of reports submitted. What correlation exists may be due to a correlation between size and age rather than size and number of reports. A final review of the bibliography shows that 31 of the 1547 reports indicated that a reactor shutdown occurred or was required because of equipment failure or malfunction. See Table 11 for a synopsis of these shutdowns.*

*References for Table 11 are numbers 30–36.

Table 9. Reports for listed units that were commercially operable all year (by age since first electrical generation)[a]

Name	Age[a] (years)	Number of reports	Percent of total
Dresden 1	20.9	3	<1
Big Rock Point	18.3	45	3
LaCrosse	12.9	1	1
Oyster Creek	11.5	66	4
Nine Mile Point 1	11.4	27	2
Dresden 2	10.9	46	3
Millstone 1	10.3	21	1
Monticello	10.0	31	2
Dresden 3	9.6	44	3
Quad Cities 1	8.9	36	2
Quad Cities 2	8.8	37	2
Pilgrim 1	8.7	96	6
Vermont Yankee	8.5	45	3
Browns Ferry 1	7.4	90	6
Peach Bottom 2	7.1	36	2
Arnold	6.8	73	5
Cooper	6.8	53	3
Browns Ferry 2	6.5	57	4
Peach Bottom 3	6.5	33	2
Hatch 1	6.3	131	8
FitzPatrick	6.1	103	7
Brunswick 2	5.9	115	7
Browns Ferry 3	4.5	61	4
Brunswick 1	4.3	104	7
Hatch 2	2.0	171	11

[a]Average age = 9.1; median age = 8.6.

Table 10. Reports for listed units that were commercially operable all year (by design electrical rating)[a]

Name	DER[a] [net MW(e)]	Number of reports	Percent of total
Browns Ferry 1	1065	90	6
Browns Ferry 2	1065	57	4
Browns Ferry 3	1065	61	4
Peach Bottom 2	1065	36	2
Peach Bottom 3	1065	33	2
Brunswick 1	821	104	7
Brunswick 2	821	115	7
FitzPatrick	821	103	3
Dresden 2	809	46	2
Dresden 3	809	44	3
Quad Cities 1	789	36	2
Quad Cities 2	789	37	2
Hatch 1	786	131	8
Hatch 2	784	171	11
Cooper	778	53	3
Pilgrim 1	655	96	6
Millstone 1	650	21	1
Oyster Creek	650	66	4
Nine Mile Point 1	610	27	2
Monticello	545	31	2
Arnold	538	73	5
Vermont Yankee	514	45	3
Dresden 1	200	3	<1
Big Rock Point	72	45	3
LaCrosse	50	17	1

[a]Average DER = 713; median DER = 785.

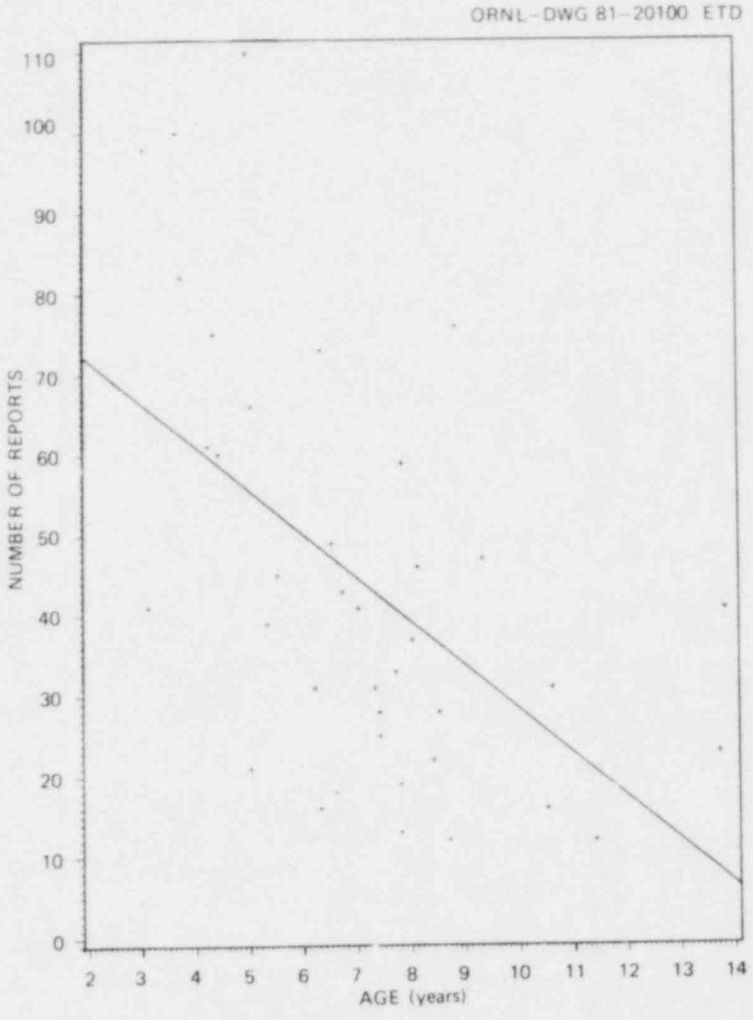

Fig. 1. Correlation between plant age and number of reports.

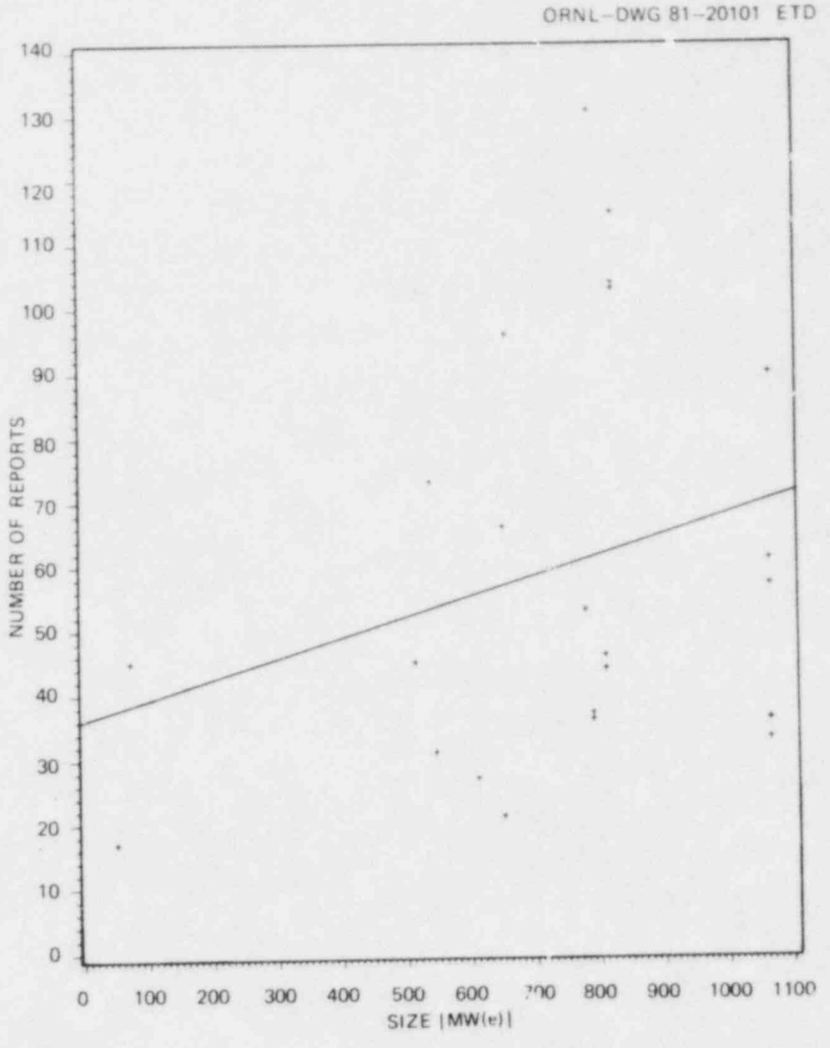

Fig. 2. Correlation between plant size and number of reports.

Table 11. Events which resulted in reactor shutdown

Report/event	Event	Cause	Component/system	Facility/docket
052780 043080	Reactor recirculation pump motor shorts	Unknown	Motors Coolant recirculation system and controls	Duane Arnold 50-331
062680 052880	Recirculation pump motor oil level is low	Unknown	Motors Coolant re irculation system and controls	Duane Arnold 50-331
112480 111080	Containment isolation valves fail to close	Wiring error during design change	Electrical conductors Containment isolation system and controls	Duane Arnold 50-331
031080 103178	Update report on control rod drive temperature	O-ring seal fabrication deficiency	Control rod drive mechanism Reactivity control system	Big Rock Point 50-135
052280 042480	Recirculation pump low oil level alarm shorts	Loss of insulation	Electrical conductors Coolant recirculation system and controls	Browns Ferry 1 50-259
071880 062480	Control valve fast-closure pressure switch miswired	Maintenance error	Instrumentation and controls Reactor trip system	Browns Ferry 1 50-259
010480 120879	Drywell to suppression chamber differential pressure out of limits	Leaking equipment hatch	Penetrations, primary containment Reactor containment system	Browns Ferry 3 50-296
101780 092080	Reactor scrams when breaker fails to close	Misalignment of breaker secondary	Circuit closers/ interrupters AC on-site power system and controls	Browns Ferry 3 50-296
121380 120280	RCIC steam isolation valve leaks	Maintenance error	Valves RCIC and controls	Brunswick 2 50-324
081880 080480	Turbine computer system spuriously scrams reactor	Voltage transients due to line fault	Instrumentation and controls Off-site power system and controls	Cooper 50-298
112580 111580	Reactor cooldown rate exceeded	Steam bypass valves stay open after scram	Instrumentation and controls Turbine bypass system and controls	Cooper 50-298
022880 020280	Jet pump fails	Not yet determined	Pumps Coolant recirculation system and controls	Dresden 3 50-249
050880 042580	Automatic depress valves fail to open	Valve operator failures	Valves ECCS and controls	Dresden 3 50-249
051980 051380	Spurious HPCI initiation occurs	Not yet determined	Component code not applicable ECCS and controls	Hatch 1 50-231
020580 012280	HPCI found inoperable	Booster pump discharge pressure line leak	Pipes, fitting RCIC and controls	Hatch 1 50-321
06080 052480	Operation within a single failure criteria region discovered	Defective procedures	Valves ECCS and controls	Hatch 1 50-321
071080 062680	HPCI fails to actuate	Isolation signal due to poor turbine-speed control	Circuit closers/ interrupters ECCS and controls	Hatch 1 50-321

Table 11 (continued)

Report/event	Event	Cause	Component/system	Facility/docket
080780 072080	Vacuum breaker failures force shutdown	Broken air lines and faulty air control valves	Valves Reactor containment system	Hatch 2 50-366
081380 071180	Core spray pumps actuate	High drywell pressure sensor setpoint drift	Instrumentation and controls ECCS and controls	Oyster Creek 50-219
012480 011080	ADS safety-relief valve air supply check valve leak	Design error	Valves ECCS and controls	Peach Bottom 2 50-277
090580 080680	LPCI injection valve failed to open	Cracks in drive sleeve of valve operator	Valve operators ECCS and controls	Peach Bottom 3 50-278
082880 080180	Reactor water conductivity exceeds limit	Demineralizer lateral backs out	Demineralizers Reactor coolant cleanup system and controls	Pilgrim 1 50-293
102080 100780	Reactor vessel relief valve opens spuriously	High nitrogen pressure opened relief valve	Valve operators ECCS and controls	Pilgrim 1 50-293
111480 103180	Reactor vessel relief valve opens spuriously	Failed nitrogen valve fed pressure to valve	Valves ECCS and controls	Pilgrim 1 50-293
112880 111380	Reactor vessel drain line leaks	Faulty weld	Pipes, fittings Reactor coolant cleanup system and controls	Pilgrim 1 50-293
120580 110680	Recirculation pump discharge valve leak	Valve packing leak	Valves Coolant recirculation system and controls	Pilgrim 1 50-293
092480 083180	Electromatic relief valve fails to close	Steam cutting off pilot valve	Valves Main steam system and controls	Quad Cities 1 50-254
052780 042880	Reactor coolant conductivity exceeds limit	Not determined	Demineralizers Reactor coolant cleanup system and controls	Quad Cities 1 50-265
062580 061180	Feedwater leak bypasses detection system	Design error	Other components Reactor coolant press bound leak Detection system	Vermont Yankee 50-271

REVIEW OF SELECTED SAFETY-RELATED EVENTS

A review of the reported events indicated that most of them were routine and inconsequential; however, a few were significant or unique. Three of those events are reviewed in detail in this report. The events reviewed were selected on the basis of their potential interest and/or applicability to safe operation and improved performance of nuclear power plants.

Partial Failure of Browns Ferry Scram System

On June 28, 1980, a partial failure of the scram system occurred at Browns Ferry 3 while the reactor was being shut down from 35% power to

allow repairs to the feedwater system. The malfunction had apparently been the result of water accumulating in the scram discharge volume (SDV) east header and blocking rods from fully inserting. Seventy-six out of 185 control rods failed to insert fully. The partially scrammed rods brought the reactor power level in the east side of the core to 2% after the first scram attempt. The west side of the core was subcritical. A second manual scram was initiated 6 min after the first. In all, four scram attempts over a period of 14 min were required to completely insert the control rods on the east side.

The unit remained shut down until July 13, 1980, while a series of tests were performed to determine the cause of the water accumulation in the SDV. Ultrasonic probes were installed on the SDVs to continuously monitor the water level.

No radioactivity was released to the environment during this event and no fuel damage was indicated; however, this type of occurrence could result in failure of additional control rods to insert fully in part or in all of the core on any given manual or automatic scram signal and thus has the potential to cause significant fuel damage. Core-coolant flow, temperature, and pressure remained normal for existing plant conditions.

Cause or causes

The problem has been determined to be hydraulic rather than electrical or mechanical. The control rod drives (CRDs), which insert and withdraw the attached control rods in a General Electic (GE) BWR are essentially water-driven hydraulic pistons. On a scram, a relatively high water pressure is applied to the bottom side of the piston by opening a scram inlet valve; a scram outlet valve opens to relieve water and pressure above the piston, and the rods are rapidly driven up into the reactor core. Water discharged from the 185 individual CRDs during scram insertion is collected in two separate headers, each consisting of a series of interconnected 6-in.-diam pipes (four on each side of the reactor) called the SDV. During normal operation, both SDVs are designed to remain empty by being continuously drained to a separate scram discharge instrument volume tank (SDIV). The SDVs are therefore normally ready to receive the scram discharge water when a scram occurs. This instrumented tank is monitored for water level and initiates an automatic scram on high level, in anticipation of a scram being prevented by too much water in the SDV. The CRDs at Browns Ferry 3 are grouped so that the east and west sides of the reactor core are connected to separate SDVs. Later tests, inspections, and analyses indicated that the east SDV was substantially full of water at the time of the event, leaving insufficient room for the discharge water. Accordingly, on scram actuation, the CRDs rapidly drove the control rods partially into the core, but rod motion prematurely ceased when pressure quickly equalized on each side of the pistons.

Following each scram actuation, the scram signal was reset by the operator's allowing some water to drain from the SDV, thus permitting the rods to insert further with each scram attempt. Sufficient water was finally drained from the SDV to allow the rods to insert fully on the fourth scram signal. The east SDV water-accumulation problem is believed to result from improper drainage into the SDIV from the SDV because of

(1) inadequate SDV venting, (2) an obstruction in the line between the SDV and SDIV, or (3) a combination of these problems.

The Office of Analysis and Evaluation of Operational Data (NRC) has since evaluated the procedure and equipment at Browns Ferry 1, 2, and 3 to determine their adequacy in providing assurance that the SDV will not fill with water and interfere with a successful scram. Some of the principal AEOD findings, recommendations, and conclusions regarding the procedures and equipment are presented in the following section.

Findings of AEOD report

The Browns Ferry 3 scram instrument volume (SIV) high-level-scram function did not and cannot provide protection against the undetected accumulation of water in the east SDV header with attendant loss of the east bank scram capability even during unobstructed venting and draining conditions.

A single blockage in the venting system or drain line of the west header SDV can result in an undetected accumulation of water in both the east and west headers, which could disable the scram capability of all control rods.

With the current SDV-SIV design, a blockage in the SDV drain or vent path can cause a partial loss of scram capability and can disable the protection system that was installed to ensure detection and corrective action.

The Browns Ferry 3 partial scram failure, together with recent events at other BWRs, shows that float-type instruments for monitoring water level have a significant degree of unreliability.

The Browns Ferry 3 emergency operating instructions did not include a procedure or guidance for the operator to follow in the event of a partial or complete scram failure.

AEOD recommendations

The SIV high-level-scram function should be independent of the SDV vent and drain systems. An acceptable configuration would be placement of the SIV tank directly under the low end of the 6-in. SDV header and connection of the top of the SIV tank to the bottom of the SDV header by a short, vertical 6-in. pipe rather than the long 2-in. horizontal pipe. This arrangement should ensure water spillage from the SDV directly into the tank containing level-monitoring instruments. Futhermore, it would not depend on venting or draining phenomena that are sensitive to blockages. All plants should also provide one SIV tank for each SDV header. Separate instrument volumes, in immediate proximity to their respective headers, should ensure proper water spillage into the SIVs and should provide adequate redundancy for protection against a total loss of scram capability.

Diversity should be added to SIV water-level monitoring instruments for the SIV high-level-scram function. Monitoring techniques such as differential-pressure cells, ultrasonic detection, or conductivity may be considered along with others for this purpose.

Emergency operating procedures and operator training covering both partial and complete scram failure should be provided.

AEOD conclusions

The AEOD analysis of the SDV-SIV design, together with its vent and drain characteristics, leads AEOD to conclude that several actual and postulated mechanisms exist that can cause the SDV to fill with water undetected and without protection against such filling. The analyses also show that certain scram events can result in an unisolated reactor coolant blowdown outside the primary containment following a single isolation-valve failure in the SDV-SIV systems.

In view of these design deficiencies, AEOD believes that modifications must be made to the SDV-SIV arrangement and isolation features. Specific recommendations for changes in the SDV-SIV design are provided in its report. The AEOD believes that the changes are necessary to adequately reduce the risks associated with unreliability of the BWR scram system caused by the undetected accumulation of water in the SDV.

Failure of HPCI to Inject

This event occurred at E. I. Hatch 1 on June 26, 1980, and involved a failure of the high-pressure coolant injection (HPCI) system to inject. Following a turbine trip, the reactor scrammed from a main turbine stop-valve fast closure. The HPCI system received an automatic initiation signal on low reactor water level, but failed to inject into the reactor pressure vessel (RPV) because the HPCI steam line isolated from a high-differential-pressure signal. This represented failure of a system to complete its required protection function, as required by the Technical Specifications.

The isolation was reset, but when the inboard isolation valve was opened, another steam line high-differential-pressure isolation signal was received. The isolation was again reset, and the isolation valves were opened. The HPCI then automatically started, injected into the RPV, and was used to control water level. The automatic depressurization system (ADS), core spray, and low-pressure coolant injection (LPCI) systems were operable. The reactor core isolation cooling (RCIC) system, however, was inoperable.

The determination was made that the isolation, due to steam line high differential pressure, was caused by poor turbine speed control resulting from the turbine control system's being out of dynamic calibration. System calibration had been affected over a period of time by the replacement of control system components that had been statically calibrated.

Following extensive testing, including manual starts and quick starts of the HPCI system and recalibration of the HPCI turbine control systems, the HPCI system demonstrated satisfactory performance during hot quick starts. Calibration, surveillance, and operability procedures are being revised to include quick-start tests for the HPCI and RCIC systems. Performance testing procedures to monitor these two systems are also being developed to assure satisfactory system dynamic response.

Condenser Leakage

Peach Bottom 2 was shut down on March 21, 1980, for a planned major outage. Earlier, on March 6, conductivity in the "A1" section of the condenser had indicated leakage. Conductivity returned to normal, and condenser vacuum was maintained after water box A1 was vented and valved out on March 14.

After the plant was shut down, the condenser vacuum was broken, and atmospheric conditions were established in the condenser steam space. Condenser hot-well level increased as a result of leakage from the condensate storage tank through a leaking makeup valve. Water level in the hot well remained above the leaking condenser tube from March 31 through April 7, when the hot well was drained per normal plant-outage procedures.

Again on April 21, however, hot-well level increased for a short time above the leaking tube, due to condensate system leakage through the condenser makeup valve. The water in the condenser at this time may have contained radioactivity levels similar to those contained in the water from the reactor head cavity, because the head cavity had just recently been drained. During the periods that leakage occurred from the hot well to the circulating water system, however, at least one circulating water pump was operating. The radioactive water leak was thus diluted by at least 250,000 gpm.

Following the event, tube sheets in the A1 water box were surveyed. Areas indicating high contamination levels were leak tested, and the one leaking tube was then measured at various differential pressures. Analysis of these data combined with the hot-well level indicated that the total release of coolant inventory between March 31 and April 7 was 11,088 gal. During the release on April 21, 244 gal was released. The radiation released throughout the period totals 0.01 curie; a dose calculation based on the isotopes involved indicates a minimal exposure well below radiation dose limits.

The event was caused by a failure to recognize a series of circumstances that would allow release of water from the hot well to the circulating water system during an outage. Instructions have been issued to shift personnel to limit hot-well level below the condenser tubes at all times during operation and shutdown, and corresponding procedural changes are being made.

Jet Pump Assembly Failure

On February 2, 1980, while Dresden 3 was operating at about 67% power in a coastdown to a refueling outage, changes observed in plant parameters indicated that jet pump failure had occurred. An orderly plant shutdown was begun to bring the unit to cold shutdown within 24 h, per plant Technical Specifications.

The following changes in plant parameters were readily observed by the control room operator:

1. Generator electrical output decreased from 539 to 511 MW(e).
2. Core thermal power decreased by 15% as indicated by decreased average power-range monitor readings and steam flows to the turbine.

3. Indicated total core flow increased from 97.6 x 10^6 to 104.7 x 10^6 lb/h.
4. Core-plate differential pressure decreased from 16.1 to 13.8 psid.
5. "B" recirculation loop flow increased from 49 x 10^3 to 54 x 10^3 gpm, while A recirculation loop flow remained at 49 x 10^3 gpm.

Individual jet pump readings and jet pump operability surveillance indicated that jet pump 13 on the B recirculation loop had apparently disassembled.

During cold shutdown following vessel head removal and refueling, remote television and visual inspections of the jet pumps and vessel annulus were performed. These showed that the hold-down beam assembly of jet pump 13 had broken across its ligament sections at the mean diameter of the bolt thread areas. This beam assembly failure resulted in pump decoupling of the diffuser connection. Subsequent *in-situ* ultrasonic examination of the other jet pump hold-down beams revealed indications of cracking at the same location in six beams, with estimates of crack depth ranging from 6 to 20 mils.

In situ ultrasonic examination was also done on all jet pump hold-down beam assemblies at Quad Cities 2 on March 15 and 16, 1980, while the unit was shut down for refueling. Of the 20 assemblies examined, a crack was indicated on one beam. The crack was estimated to be in excess of 100 mils and in the same location as found on the beams at Dresden. At Pilgrim 1 on March 28, 1980, ultrasonic examination showed crack indications in three hold-down beam assemblies.

The damaged beam assemblies were replaced during the refueling outage at the units mentioned previously, and Dresden 3 was restarted on May 3, 1980. General Electric (the nuclear steam system supplier contractor) performed a metallurgical analysis and determined that intergranular stress corrosion cracking under sustained loading was the cause of the failure. They also reported this potential problem to other licensees with operating BWR units with jet pumps.

On April 4, 1980, the NRC issued Inspection and Enforcement (IE) Bulletin No. 80-07, "BWR Jet Pump Assembly Failure." The bulletin addressed the concern that the hold-down beam assemblies and subsequent jet pump function may degrade significantly during operation. This potential problem could lead to jet pump disassembly and possible reduction of the safety margin during postulated accidents.

Bulletin 80-07 also listed several actions to be taken by licensees of GE-designed BWR-3 and BWR-4 plants with operating licenses. Prior to restart, licensees with plants in a refueling outage when the bulletin was received were to (1) visually inspect (assisted by television cameras) jet pump structures, hold-down beam assembly, hold downs, wedge, and restrainer assembly for evidence of distress that could indicate loss of beam assembly preload; (2) conduct ultrasonic examinations of the jet pump hold-down beams at the midlength ligament areas bounding the beam bolt; and (3) notify the appropriate NRC regional office of completion of (1) and (2) and submit a 14-d report. Licensees with plants that were operating or resuming operation were requested to provide, within 20 d of the receipt of the bulletin, written justification of continued operation until inspections (1) and (2) were made and any defects corrected. Procedures for instructing plant operators on identification and response to sudden jet pump failure were to be reviewed and revised as required. The

NRC is currently considering appropriate long-term measures to better assure reliable jet pump operability.[37,*]

Scram Discharge Volume Fails to Drain at Dresden 3

Following a manual scram at Dresden 3 on July 19, 1980, to gather data requested by IE Bulletin 80-17, an ultrasonic test of the west SDV revealed that the 4-in. pipe was 80% full after all level alarms cleared on the instrumented scram volume. The ultrasonic test of the east SDV revealed that they were, in fact, drained. Fifty-six minutes after the manual opening of the vent and drain valves, a ball check valve (installed as a vacuum breaker) was manually opened with some difficulty. Air inrush into the SDV through the valve was heard, and the ultrasonic test indicated that the SDV began draining at a faster rate. One hour and thirty-eight minutes after the vent and drain valves were opened, the west SDV 4-in. headers were drained.

The ball check valves on both the west and east SDV vent lines were removed, cleaned, and reinstalled. Also, an alternate, continuous, vent line path to the reactor building atmosphere was established on both units 2 and 3 on the vent piping between the SDV vent valves and drain for the reactor building equipment. To establish this positive vent path, the existing 1-in.-diam vent line was cut, opening the line to the reactor building atmosphere in the torus area. An extension was added to the cut vent line to extend it below the torus catwalk. The alternate vent path is open at all times and does not depend on any component operability other than the installed SDV vent valve.

On July 20, 1980, an automatic scram of Dresden 3 was initiated to complete the actions required by IE Bulletin 80-17. After the scam was reset and the vent and drain valves were manually opened, the east SDV 4-in. headers drained in 16 min and 36 s, and the west SDV 4-in. headers drained in 40 min and 36 s.

To ensure that the SDV remains drained and that sufficient volume is available to accommodate a scram, installation of a system to continuously monitor water level during operation is being investigated as required by IE Bulletin 80-17 Supplement 1.

SUMMARY

For 1980, 1547 event reports were submitted by licensees. Thirty-eight units were considered in this report: 25 were in commerical operation all year and submitted an average of 48 reports per unit; 1 unit in power ascension part of the year submitted 135 reports; 12 units under construction during the year submitted 21 reports. The system involved in

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*This report was taken primarily with modifications from Ref. 34 (pp. 226 through 229) with the figures from Ref. 37. For additional information see Refs. 37 through 39.

the greatest number of events was the main cooling system. The most frequently reported equipment item was valves. Switches were the most frequently reported instrument and, as expected, set-point drift was the most frequently reported deficiency. Approximately 45% of the events reported were discovered during performance testing. Designers were identified as having been at fault in the largest number of reports – 137. Licensed operators were only identified as being at fault in 80 reports (<6%). In fairness, one should note that the LERs were written by people responsible for plant operations.

In conclusion, note that this review has presented negative aspects of BWR nuclear-power plant performance not to degrade this type of plant but rather to provide some guidance as to where additional effort could be expended to minimize failures and malfunctions. This review is intended to make a contribution to the safe operation and improved performance of BWR nuclear-power plants.

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INDEXES FOR BIBLIOGRAPHY

KEYWORD INDEX

A COLLECTION OF KEYWORDS IS USED TO DENOTE THE MAIN SAFETY RELATED POINTS COVERED IN EACH ARTICLE. THE FOLLOWING INDEX IS AN ALPHABETICAL LISTING OF THE KEYWORDS GIVING REFERENCES TO EACH ARTICLE WHICH WAS KEYED TO IT.

ACCELERATION
157119 P360
*ACCIDENT ANALYSIS
155207 P 53 158286 P477 154694 P502 160490 P527
ACCIDENT ANALYSIS
153973 P 30 156954 P 36 159293 P 39 155192 P 51
156945 P 61 154946 P349 155385 P353 155196 P419
156339 P433 154932 P440 161798 P447 153946 P537
155469 P569 155497 P570
*ACCIDENT, LOSS OF COOLANT
159293 P 39
ACCIDENT, LOSS OF COOLANT
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*	#SET POINT	DRIFT IN RCIC REACTOR WATER LEVEL TRIP SWITCH AT BROWNS	159284 P113
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	#SET POINT	DRIFT IN REACTOR LOW WATER LEVEL SWITCHES AT BRUNSWICK 2*	161069 P182
	#SET POINT	DRIFT IN REACTOR PRESSURE SWITCH AT BROWNS FERRY 2*	156948 P 87
	#SET POINT	DRIFT IN REACTOR PRESSURE SWITCH AT HATCH 2*	160430 P397
	#SET POINT	DRIFT IN REACTOR PRESSURE SWITCHES AT BROWNS FERRY 3*	153456 P102
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*	#SET POINT	DRIFT IN SHUTDOWN COOLING PERMISSIVE SWITCHES AT	158634 P281
HATCH 2*	#SET POINT	DRIFT IN SHUTDOWN COOLING PRESSURE SWITCH AT PEACH BOTTOM 3	159410 P493
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1*	#SET POINT	DRIFT IN TURBINE FIRST STAGE PRESSURE PERMISSIVE SWITCHES	159418 P 68
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TOO LONG AT VERMONT YANKEE*
OPERATE AT BROWNS FERRY 3*
DRESDEN 2*
2*
*
1*
HATCH 1*
HATCH 1*
#SET POINT DRIFT IN RHR LOW PRESSURE PERMISSIVE SWITCH AT
#SET POINT DRIFT IN CONTAINMENT SPRAY PRESSURE SWITCHES AT
#BATTERY SURVEILLANCE DOCUMENTATION NOT COMPLETED AT
#REACTOR RECIRCULATION PUMP MOTOR SHORTS AT
#LOW DIESEL GENERATOR GOVERNOR HYDRAULIC OIL LEVEL FOUND AT
#EMERGENCY BUS RELAYS HAVE EXCESSIVE TIME DELAY AT
#HPCI TURBINE DOES NOT REACH RATED SPEED AT
#RECIRCULATION PUMP MOTOR OIL LEVEL IS LOW AT
#CONTAINMENT PRESSURE SWITCHES TRIP OUT OF SPECS AT
#RCIC INOPERABLE AT
#EMERGENCY BUS RELAYS HAVE EXCESSIVE TIME DELAY AT
#DRYWELL GASEOUS RADIATION RECORDERS FAIL AT
#SCRAM DISCHARGE VOLUME DRAIN VALVE LEAKS AT
#TORUS WATER LEVEL FALLS BELOW LIMIT AT
#LPCI LOOP SELECTION PRESSURE SWITCH FAILS TO RESET AT
#DIESEL GENERATOR FAILS TO START AT
#RCIC TURBINE TRIP AND THROTTLE VALVE FAILS TO OPEN AT
#RECIRCULATION PUMP TRIP SYSTEM NOT TESTED AT
#CORE SPRAY TC CORE PLATE DELTA P ALARM ACTIVATED AT
#CONTROL ROD BLOCK MONITOR INOPERABLE AT
#CONTROL ROOM AIR TREATMENT SYSTEM INOPERABLE AT
#DRYWELL PARTICULATE RADIATION RECORDERS FAIL AT
#SET POINT DRIFT IN CORE SPRAY PRESSURE SWITCH AT
#RIVER WATER SUPPLY LOOP OUT OF SERVICE AT
#CONTROL BUILDING ISOLATION DAMPER FAILS TO CLOSE AT
#SET POINT DRIFT IN CONTAINMENT SPRAY PRESSURE SWITCH AT
#RCIC INOPERABLE AT
#CONDENSATE STORAGE TANK LOW LEVEL SWITCH FAILS AT
#RCIC STEAM SUPPLY VALVE LIMIT SWITCH FAILS AT
#ONE OF THREE RCS SAMPLING POINTS ISOLATED AT
#RCIC STEAM SUPPLY LINE VALVE FAILS TO OPEN AT
#DIESEL GENERATOR FAILS TO RESPOND TO VOLTAGE CONTROL AT
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#TURBINE VALVE CLOSURE TEST NOT PERFORMED AT
#STEAM PRESSURE RELIEF VALVE STICKS OPEN AT
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#SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCH AT
#DRYWELL PRESSURE RECORDER INOPERABLE AT
#REACTOR COOLANT ISOTOPIC ANALYSIS MISSED AT
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#STACK FILTERS NOT ANALYZED FOR ALPHAS DURING APRIL AT
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#HPCI TURBINE REVERSING CHAMBERS CRACK AT
#RCIC TURBINE BEARINGS FAIL AT
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#BUS TIE BOARD DEENERGIZED
#RCIC TRIPPED ON OVERSPEED
#DIESEL GENERATOR INOPERABLE
#ROD BLOCK MONITOR DISABLED
#HPCI FAILS TO MEET FLOW REQUIREMENT
#RHR SUPPRESSION CHAMBER
#HPCI TRIPS
#STACK FILTERS NOT ANALYZED FOR ALPHAS
#CCW PUMPS INOPERABLE
#CAD STORAGE TANK LEVEL DROPS BELOW LIMIT
#RMOV BOARD BREAKER FAILS TO TRANSFER
#CONTAINMENT INTEGRITY VIOLATED
#CMPF EXCEEDS LIMIT
#ONE CONTROL ROD NOT PULLED TO TARGET POSITION
#CORE SPRAY VALVE FAILS TO REOPEN
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#HPCI TURBINE TRIPS
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#RCIC TURBINE EXHAUST ISOLATION VALVE LEAK RATE	EXCEEDS LIMIT AT HATCH 2*	161852 P404
#DRYWELL PNEUMATIC RETURN ISOLATION VALVE LEAK RATE	EXCEEDS LIMIT AT HATCH 2*	161880 P406
#CONTAINMENT ISOLATION VALVE LEAKAGE	EXCEEDS LIMIT AT HATCH 2*	162793 P407
#MAIN COOLANT ALPHA ACTIVITY	EXCEEDS LIMIT AT LA CROSSE*	160363 P414
#MAIN COOLANT ALPHA ACTIVITY	EXCEEDS LIMIT AT LA CROSSE*	160364 P415
#CONTAINMENT ISOLATION VALVE LEAK RATE	EXCEEDS LIMIT AT LACROSSE*	162016 P418
#LEAK RATE OF OUTBOARD MSIV	EXCEEDS LIMIT AT MONTICELLO*	156238 P428
#LEAK RATE OF HPCI TURBINE EXHAUST CHECK VALVE	EXCEEDS LIMIT AT MONTICELLO*	156239 P429
#LEAK RATE OF STANDBY LIQUID CONTROL VALVE	EXCEEDS LIMIT AT MONTICELLO*	156240 P429
#LEAK RATE OF TORUS VENT VALVE	EXCEEDS LIMIT AT MONTICELLO*	156241 P430
#LEAK RATE OF RCIC TURBINE EXHAUST CHECK VALVE	EXCEEDS LIMIT AT MONTICELLO*	156242 P431
#LEAK RATE OF TORUS INSTRUMENT AIR VALVE	EXCEEDS LIMIT AT MONTICELLO*	156243 P431
#LEAK RATE OF DRYWELL EQUIPMENT DRAIN SUMP VALVE	EXCEEDS LIMIT AT MONTICELLO*	156244 P432
#LEAK RATE OF INBOUND MSIV	EXCEEDS LIMIT AT MONTICELLO*	156245 P432
#AVERAGE PLANAR HEAT GENERATION RATE	EXCEEDS LIMIT AT NINE MILE POINT 1*	154448 P438
#INDICATED TEMPERATURE ACROSS CONDENSER	EXCEEDS LIMIT AT NINE MILE POINT 1*	154447 P439
#AVERAGE PLANAR LINEAR HEAT GENERATION RATE	EXCEEDS LIMIT AT NINE MILE POINT 1*	154445 P439
#LIQUID EFFLUENT DISCHARGE TEMPERATURE	EXCEEDS LIMIT AT NINE MILE POINT 1*	155298 P440
#CIRCULATING WATER DELTA TEMPERATURE	EXCEEDS LIMIT AT NINE MILE POINT 1*	156696 P442
#MILK SAMPLE IODINE ACTIVITY	EXCEEDS LIMIT AT NINE MILE POINT*	163383 P447
#PRESSURE DROP ACROSS HEPA FILTER	EXCEEDS LIMIT AT OYSTER CREEK 1*	160520 P466
#TORUS OXYGEN CONCENTRATION	EXCEEDS LIMIT AT OYSTER CREEK*	159327 P461
#RIVER WATER TEMPERATURE	EXCEEDS LIMIT AT OYSTER CREEK*	159137 P471
#DISCHARGE TEMPERATURE	EXCEEDS LIMIT AT OYSTER CREEK*	159520 P472
#WATER TEMPERATURE IN CREEK	EXCEEDS LIMIT AT OYSTER CREEK*	160357 P472
#ROD WITHDRAWAL BLOCK SETPOINT	EXCEEDS LIMIT AT PEACH BOTTOM 2*	155380 P476
#REACTOR COOLANT CONDUCTIVITY	EXCEEDS LIMIT AT PEACH BOTTOM 3*	155499 P490
#CONDENSER COOLING WATER TEMPERATURE RISE	EXCEEDS LIMIT AT PILGRIM 1*	155398 P502
#LEAKAGE OF TORUS TO RB VACUUM BREAKER	EXCEEDS LIMIT AT PILGRIM 1*	154689 P503
#REACTOR WATER CONDUCTIVITY	EXCEEDS LIMIT AT PILGRIM 1*	159277 P519
#CONDENSER TEMPERATURE GRADIENT	EXCEEDS LIMIT AT PILGRIM 1*	160523 P529
#CIRCULATING WATER DELTA T	EXCEEDS LIMIT AT PILGRIM 1*	161940 P534
#DRYWELL PURGE VALVE CLOSING TIME	EXCEEDS LIMIT AT QUAD CITIES 1*	153948 P538

#MSIV CLOSURE TIME	EXCEEDS LIMIT AT QUAD CITIES 1*	153491	P538
#SLC RELIEF VALVES UNTIL PRESSURE	EXCEEDS LIMIT AT QUAD CITIES 2*	154916	P551
#REACTOR COOLANT CONDUCTIVITY	EXCEEDS LIMIT AT QUAD CITIES 2*	158494	P553
#DISCHARGE BAY CHLORINE CONCENTRATION	EXCEEDS LIMIT AT QUAD CITIES*	159155	P550
#PRESSURE SWITCH SETPOINT	EXCEEDS LIMIT AT VERMONT YANKEE*	159525	P575
#CMPF	EXCEEDS LIMIT DURING STARTUP AT HATCH 2*	154741	P352
#DISCHARGE TEMPERATURE	EXCEEDS LIMIT A TIMES AT OYSTER CREEK*	160300	P472
#LEAK RATE OF OUTBOARD MSIV	EXCEEDS LIMITS AT MONTICELLO*	156237	P428
#CONTAINMENT OXYGEN CONCENTRATION	EXCEEDS LIMITS TWICE AT HATCH 1*	159627	P329
#FUEL EXPOSURE	EXCEEDS MAPLHGR LIMIT ANALYSIS EXPOSURES AT QUAD CITIES 1*	153946	P537
#RADIOACTIVITY RELEASE	EXCEEDS REPORT LIMIT AT FITZPATRICK*	158237	P275
#SR-90 CONCENTRATION IN MILK SAMPLE	EXCEEDS REPORT LIMIT AT PILGRIM 1*	154592	P504
#RUSSEL SAMPLE COBALT 60 CONCENTRATION	EXCEEDS REPORTABLE LIMIT AT PILGRIM 1*	158846	P514
#CS-137 LEVEL IN SEDIMENT SAMPLE	EXCEEDS REPORTING LEVEL AT FITZPATRICK*	154648	P263
#CORE SPRAY PUMP DISCHARGE SWITCH	EXCEEDS SETPOINT AT BROWNS FERRY 1*	158269	P 63
#OYSTER SAMPLE SILVER ACTIVITY	EXCEEDS TEN TIMES CONTROL ACTIVITY AT MILLSTONE*	161827	P426
*	# EXCESS FLOW CHECK VALVE MISSING INTERNALS AT BROWNS FERRY 2	161452	P 96
AT COOPER*	# EXCESSIVE EROSION NOTED IN SW PUMPS STRAINER BACKWASH LINE	153972	P240
#DIESEL GENERATOR MAKES	EXCESSIVE NOISE AT HATCH 2*	161853	P402
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	# EXCESSIVE PRIMARY CONTAINMENT LEAKAGE FOUND AT COOPER*	159542	P246
#EMERGENCY BUS RELAYS HAVE	EXCESSIVE TIME DELAY AT DUANE ARNOLD*	159645	P 10
#EMERGENCY BUS RELAYS HAVE	EXCESSIVE TIME DELAY AT DUANE ARNOLD*	160262	P 12
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#CONTAINMENT ISOLATION VALVE LEAKS	EXCESSIVELY AT LACROSSE*	162014	P417
#RESTRICTION IN COOLING WATER LINE FOR RHR PUMP SEAL HEAT	EXCHANGER AT BROWNS FERRY 3*	157437	P108
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#RHR HEAT	EXCHANGER INOPERABLE AT COOPER*	162523	P258
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#RHR HEAT	EXCHANGER LEAKS AT BROWNS FERRY 1*	158710	P 64
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#RHR HEAT	EXCHANGER LEAKS AT BROWNS FERRY 2*	160302	P 92
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#DG COOLING WATER HEAT	EXCHANGER LEAKS AT DRESDEN 2*	159532	P216
#LPCI HEAT	EXCHANGER LEAKS AT DRESDEN 3*	163345	P239
#LEAK IN REACTOR WATER CLEANUP HEAT	EXCHANGER LINE AT DRESDEN 3*	161882	P237
#RHR HEAT	EXCHANGER ON CROSSTIE LEAKS AT BROWNS FERRY 3*	159391	P115
#RHR HEAT	EXCHANGER RIB PLATE BUCKLES AT BRUNSWICK 2*	160296	P174
#LPCI HEAT	EXCHANGER SW OUTLET VALVE BREAKER FOUND OPEN AT DRESDEN 3*	153944	P224
#LPCI HEAT	EXCHANGER TUBES LEAK AT DRESDEN 2*	160387	P218
#LPCI HEAT	EXCHANGER TUBES LEAK AT DRESDEN 2*	158705	P233
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#LEAK RATE OF HPCI TURBINE	EXHAUST CHECK VALVE EXCEEDS LIMIT AT MONTICELLO*	156239	P429
#LEAK RATE OF RCIC TURBINE	EXHAUST CHECK VALVE EXCEEDS LIMIT AT MONTICELLO*	156242	P431
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#CONTROL ROD POSITION INDICATORS	FAIL AT BRUNSWICK 1*	160108	P149
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#SCRAM DISCHARGE VOLUME HIGH LEVEL SWITCHES	FAIL AT HATCH 2*		159584 P391
#AUTOMATIC DEPRESSURIZATION VALVES	FAIL AT HATCH 2*		161883 P407
#TWO MAIN STEAM LINE SUPPORT WELDS	FAIL AT MILLSTONE 1*		161470 P424
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#RADIATION MONITORS	FAIL AT VERMONT YANKEE*		156989 P573
#CONTAINMENT RADIOACTIVE GAS SAMPLER ISOLATION VALVES	FAIL CLOSED AT PEACH BOTTOM 2*		160073 P482
#CONTAINMENT ISOLATION VALVES	FAIL CLOSED AT PEACH BOTTOM 2*		160927 P483
#CONTAINMENT ATMOSPHERIC MONITORS	FAIL DOWNSCALE AT BRUNSWICK 1*		154922 P124
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#FOUR FIRE DOORS	FAIL TEST AT HATCH 1*		160840 P341
#MAIN STEAM RELIEF VALVES	FAIL TO ACTUATE PROPERLY AT BROWNS FERRY 2*		160153 P 94
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#HPCI AND RCIC FIRE DETECTORS	FAIL TO ALARM AT PILGRIM 1*		159268 P520
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#THREE RWCU ISOLATION VALVES	FAIL TO CLOSE AT BROWNS FERRY 1*		156878 P 59
#COOLING TOWER TRANSFORMER FEEDER BREAKERS	FAIL TO CLOSE AT BROWNS FERRY 1*		161794 [illegible]
#CONTAINMENT ISOLATION VALVES	FAIL TO CLOSE AT DUANE ARNOLD*		161715 P [illegible]
#TWO FIRE DOORS	FAIL TO CLOSE AT FITZPATRICK*		154651 P262
#CONTAINMENT ISOLATION VALVES	FAIL TO CLOSE AT HATCH 2*		156992 P362
#MAIN STEAM ISOLATION VALVES	FAIL TO CLOSE AT NINE MILE POINT 1*		160001 P441
#DIESEL GENERATOR PINION GEARS	FAIL TO DISENGAGE WHEN REQUIRED AT DRESDEN 2*		153494 P207
#SOURCE RANGE MONITORS	FAIL TO DRIVE INTO CORE AT BRUNSWICK 2*		159603 P179
#72 OF 185 CONTROL RODS	FAIL TO FULLY INSERT AT BROWNS FERRY 3*		158291 P111
#MAIN STEAM ISOLATION VALVES CLOSE TOO FAST AND	FAIL TO INDICATE CLOSED AT QUAD CITIES 2*		160835 P559
#CONTAINMENT VACUUM BREAKER LIGHTS	FAIL TO INDICATE OPEN AT HATCH 2.*		154819 P347
#CONTROL ROD DRIVE ACCUMULATORS	FAIL TO MAINTAIN NITROGEN PRESSURE AT HATCH 1*		160269 P321
#STATION BATTERY CHARGERS	FAIL TO MAINTAIN REQUIRED LOAD AT HATCH 2*		156121 P360
#EMERGENCY STATION BATTERIES	FAIL TO MAINTAIN THEIR LOADS AT HATCH 2*		156117 P359
#TWO MAIN STEAM ISOLATION VALVES	FAIL TO MEET LEAK RATE AT MILLSTONE 1*		160060 P423
#CONTROL ROD DRIVE HYDRAULIC PIPE SUPPORTS	FAIL TO MEET STRESS LIMITS AT MONTICELLO*		156339 P433
#SOURCE RANGE MONITORS	FAIL TO MOVE AND GIVE UNUSUAL INDICATION AT BRUNSWICK 2*		158655 P179
#CAC MONITOR ISOLATION VALVES	FAIL TO OPEN AT BRUNSWICK 2*		161595 P194
#TWO CAD SYSTEM VALVES	FAIL TO OPEN AT BRUNSWICK 2*		161653 P196
#TORUS VENT VALVES	FAIL TO OPEN AT DRESDEN 2*		158704 P215
#AUTOMATIC DEPRESSURIZATION VALVES	FAIL TO OPEN AT DRESDEN 3*		158231 P232
#MAIN STEAM ISOLATION SWITCHES	FAIL TO OPERATE AT BROWNS FERRY 3*		158235 P106
#CAD TANK FLOW CONTROL VALVES	FAIL TO OPERATE AT BRUNSWICK 2*		161655 P195
#SCRAM DISCHARGE VOLUME VENT AND DRAIN VALVES	FAIL TO OPERATE AT PEACH BOTTOM 2*		159644 P479
#LIQUID LEVEL SWITCHES	FAIL TO OPERATE PROPERLY AT BROWNS FERRY 2*		155204 P 81
#FOUR RIP VALVES	FAIL TO OPERATE PROPERLY AT BRUNSWICK 1*		154731 P123
#RHR SERVICE WATER PUMPS	FAIL TO PROVIDE SUFFICIENT FLOW AT FITZPATRICK*		160227 P286
#RHR SERVICE WATER PUMPS	FAIL TO START AT BRUNSWICK 2*		160466 P187
#CORE SPRAY PUMPS	FAIL TO START AT COOPER*		159565 P247
#REACTOR BLDG VENT RAD MONITOR SAMPLE PUMPS	FAIL TO START AT HATCH 1*		159248 F335
#MAIN STEAM LINE HIGH FLOW SENSORS	FAIL TO TRIP ON SETPOINT AT OYSTER CREEK*		160385 P470
#AIR TEMPERATURE RECORDER	FAILED AT BROWNS FERRY 1*		160936 P 74
#SECONDARY CONTAINMENT INTEGRITY CHECK	FAILED AT BRUNSWICK 1*		159164 P140
WATER TRAVELING SCREEN FAILS WHILE REDUNDANT SYSTEM IS	FAILED AT DUANE ARNOLD*	#RIVER	159051 P 14
#INDICATING VALVES ON THE FIRE PROTECTION SYSTEM	FAILED AT OYSER CREEK*		160263 P459
#REACTOR BUILDING VENT MONITOR	FAILED DOWNSCALE AT COOPER*		159299 P252
#TORUS VACUUM BREAKER VALVE	FAILED OPEN AT HATCH 2*		159629 P385
#LPCI INJECTION VALVE	FAILED TO OPEN AT PEACH BOTTOM 3*		160140 P497
#REACTOR DEPRESSURIZING SYSTEM BATTERY CELL	FAILS AT BIG ROCK POINT*		154904 P 32
#REACTOR DEPRESSURIZING SYSTEM CHANNEL BATTERY	FAILS AT BIG ROCK POINT*		159088 P 37
#PRESSURE INDICATOR SWITCH	FAILS AT BIG ROCK POINT*		160291 P 38
#CONTROL ROD DRIVE ACCUMULATOR LEVEL SWITCH	FAILS AT BROWNS FERRY 1*		155296 P 49
#VENTILATION SYSTEM ISOLATION CHANNEL RELAYS	FAILS AT BROWNS FERRY 1*		154919 P 49
#ONE OFFSITE POWER LINE	FAILS AT BROWNS FERRY 1*		156026 P 54
#RADIATION MONITOR	FAILS AT BROWNS FERRY 1*		156946 P 58
#HALOGEN DETECTOR	FAILS AT BROWNS FERRY 1*		157167 P 60
#DRYWELL AIR SAMPLING PUMP	FAILS AT BROWNS FERRY 1*		157738 P 64
#SCRAM DISCHARGE VOLUME HIGH LEVEL SWITCH	FAILS AT BROWNS FERRY 1*		158293 P 65
#TURBINE BUILDING ROOF VENT MONITOR	FAILS AT BROWNS FERRY 1*		159405 P 69
#WIND DIRECTION INDICATOR	FAILS AT BROWNS FERRY 1*		160225 P 70
#REACTOR BUILDING VENTILATION MONITOR	FAILS AT BROWNS FERRY 1*		160157 P 72
#ONE OFFSITE POWER LINE	FAILS AT BROWNS FERRY 2*		156031 P 84
#AIR PARTICULATE CHANNEL RECORDER	FAILS AT BROWNS FERRY 2*		158632 P 87
#HPCI TURBINE RUPTURE DISC	FAILS AT BROWNS FERRY 2*		159340 P 91
#REFUELING ZONE INBOARD ISOLATION TIME DELAY RELAY	FAILS AT BROWNS FERRY 2*		161065 P 96
#DIESEL GENERATOR	FAILS AT BROWNS FERRY 3*		157009 P105
#RCIC SPEED FEEDBACK MAGNETIC PICKUP CONNECTOR	FAILS AT BROWNS FERRY 3*		160115 P114
#RHR PUMP AREA COOLER FAN THERMAL OVERLOAD RELAY	FAILS AT BROWNS FERRY 3*		159389 P115
#RTGB CONTROL SWITCH FOR HPCI TORUS SUCTION VALVE	FAILS AT BRUNSWICK 1*		155397 P132
#CONTROL ROD POSITION INDICATION	FAILS AT BRUNSWICK 1*		156175 P135
#ROD BLOCK MONITOR	FAILS AT BRUNSWICK 1*		155308 P136
#CONTROL ROD POSITION INDICATION	FAILS AT BRUNSWICK 1*		160304 P141
#CONTROL POWER TO DIESEL GENERATOR	FAILS AT BRUNSWICK 1*		156922 P142
#RHR FLOW INDICATOR ON REMOTE SHUTDOWN PANEL	FAILS AT BRUNSWICK 1*		158605 P144
#REACTOR WATER LEVEL SWITCH	FAILS AT BRUNSWICK 1*		158545 P144
#CONTROL ROD POSITION INDICATOR	FAILS AT BRUNSWICK 1*		158600 P144

#RB VENTILATION RADIATION MONITORING SYSTEM	FAILS AT BRUNSWICK 1*		159166	P145
#DRYWELL FLOOR DRAIN FLOW INTEGRATOR	FAILS AT BRUNSWICK 1*		160211	P148
#OXYGEN CHANNEL OF CONTAINMENT ATMOSPHERE RECORDER	FAILS AT BRUNSWICK 1*		160084	P153
#REMOTE SHUTDOWN PANEL RHR FLOW INDICATOR	FAILS AT BRUNSWICK 1*		163394	P153
#CONTROL ROD POSITION INDICATION	FAILS AT BRUNSWICK 1*		161687	P155
#CONTAINMENT OXYGEN MONITOR	FAILS AT BRUNSWICK 1*		160527	P156
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS AT BRUNSWICK 1*		161811	P158
#SERVICE WATER HEADER ANTI-SIPHON PIPE SNUBBER	FAILS AT BRUNSWICK 1*		163391	P158
#SAFETY RELATED PIPE SNUBBER	FAILS AT BRUNSWICK 2*		159066	P177
#MAIN TURBINE CONTROL VALVE FAST CLOSURE RELAY	FAILS AT BRUNSWICK 2*		161070	P183
#RHR FLOW INSTRUMENTATION	FAILS AT BRUNSWICK 2*		160367	P185
#REACTOR WATER LEVEL INDICATION SWITCH	FAILS AT BRUNSWICK 2*		160045	P186
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS AT BRUNSWICK 2*		160464	P186
#RHR SERVICE WATER PRESSURE ALARM	FAILS AT BRUNSWICK 2*		161656	P192
#SERVICE WATER PIPE SNUBBER	FAILS AT BRUNSWICK 2*		162724	P198
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS AT BRUNSWICK 2*		161814	P199
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS AT BRUNSWICK 2*		161816	P200
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS AT BRUNSWICK 2*		162707	P202
#TRAVERSING INCORE PROBE POSITION INDICATION	FAILS AT BRUNSWICK 2*		162706	P202
#HPCI STOP VALVE STEM	FAILS AT COOPER*		155604	P242
#CONTROL POWER TO RHR VALVE	FAILS AT COOPER*		155580	P243
#DIESEL GENERATOR CYLINDER	FAILS AT COOPER*		159470	P252
#SUPPRESSION POOL COOLING INBOARD THROTTLE VALVE	FAILS AT COOPER*		159247	P253
#CONTAINMENT ISOLATION VALVE	FAILS AT COOPER*		160094	P254
#TORUS TO DRYWELL VACUUM BREAKER	FAILS AT DRESDEN 2*		158556	P211
#CONTROL ROD POSITION INDICATION	FAILS AT DRESDEN 2*		159447	P216
#SCRAM DISCHARGE VOLUME HIGH HIGH LEVEL ALARM	FAILS AT DRESDEN 2*		163371	P223
#ROD BLOCK MONITOR	FAILS AT DRESDEN 3*		153493	P223
#DRYWELL OXYGEN ANALYZER AUTOMATIC ADVANCE	FAILS AT DRESDEN 3*		155193	P225
#JET PUMP	FAILS AT DRESDEN 3*		155600	P226
#REACTOR RECIRCULATION PUMP SPEED CONTROLLER	FAILS AT DRESDEN 3*		160456	P236
#CONTROL ROD BLOCK MONITOR	FAILS AT DRESDEN 3*		160922	P237
#CONDENSATE STORAGE TANK LOW LEVEL SWITCH	FAILS AT DUANE ARNOLD*		160829	P 19
#RCIC STEAM SUPPLY VALVE LIMIT SWITCH	FAILS AT DUANE ARNOLD*		160856	P 19
#HPCI LEAK DETECTION TEMPERATURE DIFFERENTIAL SWITCH	FAILS AT DUANE ARNOLD*		160851	P 21
#STANDBY GAS TREATMENT SYSTEM	FAILS AT FITZPATRICK*		159581	P287
#RCIC TURBINE CONTROL	FAILS AT FITZPATRICK*		159382	P290
#OFF GAS RADIATION MONITOR	FAILS AT FITZPATRICK*		160104	P290
#DRYWELL SUMP PUMP	FAILS AT FITZPATRICK*		160880	P292
#HPCI STEAM SUPPLY VALVE MOTOR	FAILS AT FITZPATRICK*		162035	P295
#RCIC HIGH AMBIENT TEMPERATURE SWITCH	FAILS AT FITZPATRICK*		158636	P281
#REACTOR BUILDING VENT STACK RADIATION MONITOR	FAILS AT HATCH 1*		155588	P305
#CONTAINMENT HYDROGEN AND OXYGEN ANALYZER	FAILS AT HATCH 1*		156972	P311
#TORUS AIR AND WATER TEMPERATURE RECORDER	FAILS AT HATCH 1*		158597	P316
#DRYWELL/TORUS DIFFERENTIAL PRESSURE RECORDER	FAILS AT HATCH 1*		156995	P317
#DRYWELL VACUUM BREAKER POSITION INDICATOR	FAILS AT HATCH 1*		159486	P336
#ROD SEQUENCE CONTROL SYSTEM	FAILS AT HATCH 2*		155543	P351
#RCIC REMOTE SHUTDOWN FLOW CONTROL LOOP	FAILS AT HATCH 2*		154739	P352
#DRYWELL OXYGEN-HYDROGEN ANALYZER	FAILS AT HATCH 2*		156144	P355
#HYDRAULIC SNUBBER	FAILS AT HATCH 2*		158299	P364
#CONTAINMENT PURGE VACUUM BREAKER PRESSURE SWITCH	FAILS AT HATCH 2*		158726	P369
#RPS LOGIC FEEDER BREAKER	FAILS AT HATCH 2*		156936	P370
#TORUS VACUUM BREAKER	FAILS AT HATCH 2*		156933	P372
#TORUS TEMPERATURE RECORDER	FAILS AT HATCH 2*		156934	P373
#DRYWELL HYDROGEN-OXYGEN ANALYZER PUMP DIAPHRAGM	FAILS AT HATCH 2*		156858	P374
#DRYWELL HYDROGEN-OXYGEN ANALYZER	FAILS AT HATCH 2*		156857	P375
#REACTOR WATER LEVEL SWITCH	FAILS AT HATCH 2*		158674	P382
#REACTOR WATER LEVEL SWITCH	FAILS AT HATCH 2*		158675	P382
#MSL LEAKAGE CONTROL FLOW INDICATOR	FAILS AT HATCH 2*		159593	P385
#DIESEL GENERATOR	FAILS AT HATCH 2*		161881	P406
DISCHARGE VOLUME HIGH LEVEL ROD WITHDRAWAL BLOCK SWITCH	FAILS AT HATCH 2*	#SCRAM	159159	P390
#CONTAINMENT AIR LOCK DOOR INTERLOCK	FAILS AT LACROSSE*		161888	P415
#RECIR LOOP SAMPLE ISOLATION VALVE	FAILS AT MONTICELLO*		160374	P436
#HPCI STEAM SUPPLY ISOLATION VALVE	FAILS AT MONTICELLO*		159422	P436
#CONTAINMENT SPRAY ISOLATION VALVE	FAILS AT NINE MILE POINT 1*		158697	P441
#AVERAGE POWER RANGE MONITOR	FAILS AT NINE MILE POINT 1*		160862	P446
#REACTOR BUILDING TO SUPPRESSION CHAMBER VACUUM BREAKER	FAILS AT OYSTER CREEK*		153938	P449
#STANDBY GAS TREATMENT UNIT	FAILS AT OYSTER CREEK*		155994	P454
#REACTOR BUILDING AUTOMATIC ISOLATION VALVE	FAILS AT OYSTER CREEK*		157006	P455
#DIESEL DRIVEN FIRE PUMP	FAILS AT OYSTER CREEK*		162433	P473
#RHR PUMP MOTOR	FAILS AT PEACH BOTTOM 2*		158287	P487
#MAIN STEAM LINE RADIATION MONITOR	FAILS AT PEACH BOTTOM 3*		154729	P489
#RCIC AUTOMATIC FLOW CONTROLLER	FAILS AT PEACH BOTTOM 3*		155998	P491
#CONTAINMENT SAMPLE EXHAUST PUMP	FAILS AT PEACH BOTTOM 3*		160489	P498
#HPCI FLOW CONTROLLER	FAILS AT PEACH BOTTOM 3*		161772	P499
#SWITCHYARD 125 VOLT DC BATTERY CHARGER	FAILS AT PILGRIM 1*		153466	P500
#RHR SYSTEM SNUBBER	FAILS AT PILGRIM 1*		155210	P504
#DIESEL GENERATOR FIRE PROTECTION SYSTEM	FAILS AT PILGRIM 1*		157021	P500
#AUTOMATIC DEPRESSURIZATION SYSTEM RELIEF VALVE	FAILS AT PILGRIM 1*		158834	P514
#REACTOR PROTECTION SYSTEM FUNCTION	FAILS AT PILGRIM 1*		159383	P517
#125V DC CONTROL BATTERY CHARGER	FAILS AT PILGRIM 1*		160239	P523
#REACTOR COOLANT LEAK DETECTION AIR SAMPLING SYSTEM	FAILS AT PILGRIM 1*		160252	P526
#TORUS WATER LEVEL INDICATOR	FAILS AT PILGRIM 1*		161937	P535
#SUPPRESSION CHAMBER COOLING VALVE	FAILS AT QUAD-CITIES 1*		157008	P542
#RHR PUMP TIMER RELAY	FAILS AT QUAD-CITIES 2*		155985	P553
#SAFETY CLASS PIPE SNUBBER	FAILS AT VERMONT YANKEE*		155589	P568
#HPCI CONTROL POWER	FAILS AT VERMONT YANKEE*		160337	P575

#ENVIRONMENTAL AIR SAMPLER	FAILS AT VERMONT YANKEE*	160093 P580
#EMERGENCY EQUIPMENT COOLING WATER VALVE	FAILS CLOSED AT BROWNS FERRY 3*	761821 P120
#CONTAINMENT VACUUM BREAKER VALVE	FAILS CLOSED AT HATCH 2*	158445 P384
#REACTOR BUILDING VENTILATION SYSTEM ISOLATION VALVE	FAILS CLOSED AT QUAD CITIES 2*	155490 P552
#HPCI GOVERNOR VALVE	FAILS CLOSED AT VERMONT YANKEE*	155590 P568
#CONTAINMENT PRESSURE RECORDER	FAILS DOWNSCALE AT ARNOLD*	154507 P 1
#STEAM LINE RADIATION MONITOR	FAILS DOWNSCALE AT BROWNS FERRY 2*	155468 P 83
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS DOWNSCALE AT BRUNSWICK 1*	154516 P125
#CONTAINMENT ATMOSPHERE RECORDER	FAILS DOWNSCALE AT BRUNSWICK 1*	160387 P136
#HPCI HIGH STEAM FLOW INSTRUMENT	FAILS DOWNSCALE AT COOPER*	154051 P241
#DRYWELL OXYGEN CONCENTRATION INDICATOR	FAILS DOWNSCALE AT DRESDEN 2*	159531 P215
#MAIN STEAM LINE DRAIN VALVE	FAILS IN MID POSITION AT HATCH 1*	154667 P301
#VACUUM BREAKER VALVE INDICATING LIGHT	FAILS OFF AT HATCH 1*	160207 P322
#NEUTRALIZER TANK DISCHARGE VALVE	FAILS OPEN AT FITZPATRICK*	154669 P265
#GENERATOR LOAD REJECTION SENSOR PRESSURE SWITCH	FAILS OPEN AT OYSTER CREEK*	157102 P456
#STANDBY LIQUID CONTROL PUMP	FAILS OPERABILITY TEST AT HATCH 2*	156937 P368
#REACTOR LOW WATER LEVEL	FAILS TO ACTUATE AT BRUNSWICK 1*	160534 P155
#TORUS WATER LEVEL SWITCH	FAILS TO ACTUATE AT BRUNSWICK 2*	155572 P167
#AUTOMATIC DEPRESSURIZATION SYSTEM VALVE	FAILS TO ACTUATE AT BRUNSWICK 2*	160866 P189
#HYDROGEN MONITOR	FAILS TO CALIBRATE AT BROWNS FERRY 3*	160330 P116
#DRYWELL HYDROGEN SENSOR	FAILS TO CALIBRATE AT BROWNS FERRY 3*	161935 P121
#RHR ALARM	FAILS TO CLEAR AT DUANE ARNOLD*	160825 P 27
#MAIN STEAM ISOLATION VALVE	FAILS TO CLOSE AT BIG ROCK POINT*	161674 P 42
#SHUTDOWN BOARD ALTERNATE FEED BREAKER	FAILS TO CLOSE AT BROWNS FERRY 1*	155524 P 53
#RHR INJECTION VALVE	FAILS TO CLOSE AT BROWNS FERRY 1*	156014 P 56
#DRYWELL SAMPLE ISOLATION VALVE	FAILS TO CLOSE AT BROWNS FERRY 1*	158631 P 62
#MODULATING DAMPER	FAILS TO CLOSE AT BROWNS FERRY 1*	160214 P 71
#DRYWELL CONTROL AIR VALVE	FAILS TO CLOSE AT BROWNS FERRY 2*	160154 P 94
#REACTOR WATER CLEAN-UP ISOLATION VALVE	FAILS TO CLOSE AT BROWNS FERRY 3*	156965 P100
#RCIC MINIMUM FLOW BYPASS VALVE	FAILS TO CLOSE AT BROWNS FERRY 3*	159416 P112
#REACTOR SCRAMS WHEN BREAKER	FAILS TO CLOSE AT BROWNS FERRY 3*	160152 P117
#CONTAINMENT ATMOSPHERE MONITOR LINE VALVE	FAILS TO CLOSE AT BRUNSWICK 1*	156229 P129
#SUPPRESSION POOL EXHAUST VALVE	FAILS TO CLOSE AT BRUNSWICK 2*	160467 P185
#CONTAINMENT PURGE VALVE	FAILS TO CLOSE AT COOPER*	155582 P243
#DRYWELL EQUIPMENT DRAIN VALVE	FAILS TO CLOSE AT COOPER*	161594 P255
#RHR SERVICE WATER VALVE	FAILS TO CLOSE AT COOPER*	161844 P256
#4160 VOLT BREAKER	FAILS TO CLOSE AT COOPER*	161593 P257
#LPCI VALVE	FAILS TO CLOSE AT DRESDEN 3*	155294 P225
#CONTROL BUILDING ISOLATION DAMPER	FAILS TO CLOSE AT DUANE ARNOLD*	160457 P 18
#HPCI PUMP MINIMUM FLOW VALVE	FAILS TO CLOSE AT FITZPATRICK*	158301 P276
#RHR CONTAINMENT ISOLATION VALVE	FAILS TO CLOSE AT FITZPATRICK*	158610 P277
#CONTAINMENT ISOLATION VALVE	FAILS TO CLOSE AT FITZPATRICK*	160917 P293
#FIRE DOOR	FAILS TO CLOSE AT HATCH 1*	160486 P340
#HPCI MINIMUM FLOW VALVE	FAILS TO CLOSE AT HATCH 1*	160868 P340
#HPCI MINIMUM FLOW VALVE	FAILS TO CLOSE AT HATCH 2*	154744 P351
#PLANT SERVICE WATER ISOLATION VALVE	FAILS TO CLOSE AT HATCH 2*	156108 P358
#RCIC STEAM SUPPLY LINE ISOLATION VALVE	FAILS TO CLOSE AT HATCH 2*	158643 P370
#HPCI ISOLATION VALVE	FAILS TO CLOSE AT HATCH 2*	158656 P379
#MAIN STEAM ISOLATION VALVE	FAILS TO CLOSE AT HATCH 2*	158245 P380
#RCIC OUTBOARD ISOLATION VALVE	FAILS TO CLOSE AT HATCH 2*	159319 P394
#DRYWELL SUMP ISOLATION VALVE	FAILS TO CLOSE AT MONTICELLO*	161895 P438
#RCIC STEAM SUPPLY VALVE	FAILS TO CLOSE AT PEACH BOTTOM 2*	160122 P480
#MAIN STEAM SAMPLE LINE ISOLATION VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	154597 P488
#FEEDWATER FLUSH ISOLATION VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	153476 P489
#CONTAINMENT SPRAY OUTBOARD ISOLATION VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	159095 P494
#CONTAINMENT ISOLATION VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	160137 P495
#CONTAINMENT ISOLATION VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	159450 P496
#CONTAINMENT VENTILATION ISOLATION VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	159440 P497
#HPCI VALVE	FAILS TO CLOSE AT PEACH BOTTOM 3*	161759 P500
#RCIC ISOLATION VALVE	FAILS TO CLOSE AT PILGRIM 1*	159365 P516
#RCIC OUTBOARD ISOLATION VALVE	FAILS TO CLOSE AT PILGRIM 1*	159536 P519
#MAIN STEAM LINE DRAIN ISOLATION VALVE	FAILS TO CLOSE AT PILGRIM 1*	159280 P520
#NITROGEN PURGE SUPPLY ISOLATION VALVE	FAILS TO CLOSE AT PILGRIM 1*	160212 P526
#CONTAINMENT ISOLATION VALVE	FAILS TO CLOSE AT PILGRIM 1*	160553 P532
#MAIN STEAM ISOLATION VALVE	FAILS TO CLOSE AT QUAD CITIES 1*	158708 P544
#ELECTROMATIC RELIEF VALVE	FAILS TO CLOSE AT QUAD CITIES 1*	160452 P546
#CONTAINMENT OXYGEN ANALYZER VALVE	FAILS TO CLOSE AT QUAD CITIES 2*	153487 P550
#RHR SERVICE WATER VAULT BED PLATE DRAIN VALVE	FAILS TO CLOSE AT QUAD CITIES 2*	153951 P551
#SUPPRESSION POOL COOLING VALVE	FAILS TO CLOSE AT QUAD CITIES 2*	158495 P556
#MSIV	FAILS TO CLOSE AT QUAD-CITIES 1*	156024 P540
#CONTAINMENT ISOLATION NITROGEN VALVE	FAILS TO CLOSE AT QUAD-CITIES 2*	159688 P558
#CONTAINMENT MONITORING ISOLATION VALVE	FAILS TO CLOSE AT VERMONT YANKEE*	156889 P572
#CONTAINMENT SUMP ISOLATION VALVE	FAILS TO CLOSE AT VERMONT YANKEE*	158722 P573
#MAIN STEAM ISOLATION VALVE	FAILS TO CLOSE IN TIME AT COOPER*	163531 P260
#REACTOR WATER CLEANUP SYSTEM ISOLATION VALVE	FAILS TO CLOSE IN TIME AT DUANE ARNOLD*	160835 P 20
#NITROGEN PURGE ISOLATION VALVE	FAILS TO CLOSE IN TIME AT PILGRIM 1*	158840 P515
#PRIMARY VENT VALVE	FAILS TO CLOSE IN TIME AT QUAD-CITIES 2*	161930 P562
#MSIV	FAILS TO CLOSE WITHIN TIME LIMIT AT MONTICELLO*	160209 P436
#ROD SEQUENCE CONTROL SYSTEM	FAILS TO COME INTO SERVICE WHEN REQUIRED AT FITZPATRICK*	155925 P271
#RPS LOGIC RELAY	FAILS TO DEENERGIZE FOLLOWING MSIV CLOSURE AT BRUNSWICK 1*	160864 P154
#MSL LINIT SWITCH	FAILS TO DEENERGIZE SCRAM CHANNEL RELAY AT BROWNS FERRY 2*	159162 P 89
#SCRAM DISCHARGE VOLUME	FAILS TO DRAIN AND ALARM CLEAR AT DRESDEN 3*	158229 P235
#MSIV	FAILS TO FULLY OPEN AT VERMONT YANKEE*	153481 P565
#TORUS LEVEL INSTRUMENT	FAILS TO FUNCTION PROPERLY AT BRUNSWICK 1*	154516 P126
#ECCS LOGIC RELAY	FAILS TO FUNCTION PROPERLY AT PEACH BOTTOM 2*	160333 P481
#TORUS TO DRYWELL VACUUM BREAKER	FAILS TO GIVE OPEN INDICATION AT DRESDEN 2*	160266 P214
#APRM CHANNEL	FAILS TO GO INOPERABLE WITH LESS THAN 11 LPRM INPUTS AT HATCH 2*	155299 P353

#REACTOR DEPRESSURIZING BATTERY	FAILS TO HOLD CHARGE AT BIG ROCK POINT*	160972 P 40
#TORUS WATER LEVEL SENSOR	FAILS TO INDICATE ACTUAL LEVEL AT FITZPATRICK*	159104 P286
#CONTAINMENT ISOLATION VALVE	FAILS TO INDICATE POSITION AT BRUNSWICK 2*	161654 P196
#APRM	FAILS TO INITIATE ROD BLOCK AT DRESDEN 3*	159533 P235
#HPCI	FAILS TO INJECT FOLLOWING SCRAM AT HATCH 1*	163478 P325
#RCIC	FAILS TO INJECT INTO REACTOR VESSEL AT HATCH 1*	158685 P327
#CONTROL ROD	FAILS TO INSERT AT BROWNS FERRY 2*	154896 P103
#HPCI	FAILS TO ISOLATE AT HATCH 2*	159307 P391
#CONTROL ROD POSITION INDICATOR LIGHT	FAILS TO LIGHT AT BRUNSWICK 1*	154712 P125
#HYDRAULIC SNUBBER	FAILS TO LOCK AT OYSTER CREEK*	160235 P461
#SNUBBER	FAILS TO LOCK IN COMPRESSION DIRECTION AT OYSTER CREEK*	160295 P459
#CORE SPRAY SNUBBER	FAILS TO LOCK UP AT OYSTER CREEK*	160433 P464
#SHOCK SUPPRESSOR	FAILS TO LOCK UP AT VERMONT YANKEE*	160897 P577
1* #HPCI	FAILS TO MEET FLOW REQUIREMENT DUE TO TRIP AT BROWNS FERRY	158272 P 63
#STANDBY GAS TREATMENT FAN	FAILS TO MEET FLOW REQUIREMENTS AT HATCH 1*	153760 P390
#RHR SERVICE WATER PUMP	FAILS TO MEET REQUIREMENTS AT MONTICELLO*	156879 P434
1* #RHR SYSTEM	FAILS TO MEET SAFE SHUTDOWN EARTHQUAKE CRITERIA AT PILGRIM	160490 P527
#CORE PIPING RESTRAINT	FAILS TO MEET SEISMIC CRITERIA AT BROWNS FERRY 1*	155367 P 53
#STANDBY LIQUID CONTROL RELIEF VALVE	FAILS TO OPEN AT BRUNSWICK 1*	159139 P187
#RCIC INJECTION VALVE	FAILS TO OPEN AT BRUNSWICK 2*	160525 P187
#HPCI TURBINE CONTROL VALVE	FAILS TO OPEN AT BRUNSWICK 2*	160042 P188
#SERVICE WATER VALVE	FAILS TO OPEN AT COOPER*	162522 P258
#FIRE HYDRANT	FAILS TO OPEN AT COOPER*	159246 P253
#WASTE LINE VALVE OPERATOR	FAILS TO OPEN AT COOPER*	162519 P259
#HPCI STEAM SUPPLY VALVE	FAILS TO OPEN AT DRESDEN 2*	160586 P220
#LPCI INJECTION VALVE	FAILS TO OPEN AT DRESDEN 3*	156027 P228
#HPCI STEAM SUPPLY VALVE	FAILS TO OPEN AT DRESDEN 3*	160703 P238
#RCIC TURBINE TRIP AND THROTTLE VALVE	FAILS TO OPEN AT DUANE ARNOLD*	159059 P 15
#RCIC STEAM SUPPLY LINE VALVE	FAILS TO OPEN AT DUANE ARNOLD*	160501 P 21
#CORE SPRAY VALVE	FAILS TO OPEN AT FITZPATRICK*	153729 P266
#CORE SPRAY VALVE	FAILS TO OPEN AT FITZPATRICK*	154660 P267
#RHR SYSTEM DISCHARGE TO RADWASTE VALVE	FAILS TO OPEN AT FITZPATRICK*	158300 P276
#CONTAINMENT SPRAY VALVE	FAILS TO OPEN AT HATCH 1*	156991 P315
#SHUTDOWN COOLING SUCTION VALVE	FAILS TO OPEN AT HATCH 1*	158784 P321
#RHR INBOARD INJECTION VALVE	FAILS TO OPEN AT HATCH 1*	159314 P335
#TORUS-DRYWELL VACUUM BREAKER	FAILS TO OPEN AT HATCH 2*	156143 P355
#MSIV LEAKAGE CONTROL SYSTEM RELAY	FAILS TO OPEN AT HATCH 2*	156863 P378
#HPCI INJECTION VALVE	FAILS TO OPEN AT HATCH 2*	159157 P387
#HPCI SUPPRESSION CHAMBER SUCTION VALVE	FAILS TO OPEN AT HATCH 2*	159317 P390
#TORUS/DRYWELL VACUUM BREAKER	FAILS TO OPEN AT HATCH 2*	160476 P396
#ELECTROMAGNETIC RELIEF VALVE	FAILS TO OPEN AT NINE MILE POINT 1*	159511 P443
#PNEUMATIC RELIEF VALVE	FAILS TO OPEN AT OYSTER CREEK*	154450 P449
#STANDBY GAS TREATMENT FILTER ASSEMBLY ISOLATION VALVE	FAILS TO OPEN AT PEACH BOTTOM 2*	161071 P483
#LPCI INJECTION VALVE	FAILS TO OPEN AT PEACH BOTTOM 3*	155416 P490
#LPCI INJECTION VALVE	FAILS TO OPEN AT PEACH BOTTOM 3*	156969 P493
#CORE SPRAY PUMP FAILS TO START AND VALVE	FAILS TO OPEN AT PILGRIM 1*	158613 P508
#RELIEF VALVE	FAILS TO OPEN AT PILGRIM 1*	159499 P520
#CORE SPRAY VALVE	FAILS TO OPEN AT PILGRIM 1*	160251 P525
#TARGET ROCK SAFETY RELIEF VALVE	FAILS TO OPEN AT QUAD CITIES 1*	158751 P544
#CORE SPRAY VALVE	FAILS TO OPEN AT QUAD CITIES 2*	160067 P559
#RCIC PUMP DISCHARGE VALVE	FAILS TO OPEN AT QUAD CITIES 2*	161455 P560
#PRESSURE RELIEF VALVE	FAILS TO OPEN AT QUAD-CITIES 2*	156942 P555
#RHR CONTAINMENT COOLING VALVE	FAILS TO OPEN AT QUAD-CITIES 2*	160878 P560
#ELECTROMAGNETIC RELIEF VALVE	FAILS TO OPEN AT QUAD-CITIES 2*	163334 P564
#HPCI STOP VALVE	FAILS TO OPEN COMPLETELY AT BROWNS FERRY 1*	156211 P 56
#LPCI VALVE	FAILS TO OPEN FULLY AT QUAD-CITIES 2*	160567 P559
#RECIRCULATION PUMP DISCHARGE VALVE	FAILS TO OPEN FULLY AT VERMONT YANKEE*	157039 P571
#CRD SCRAM ACCUMULATOR LEVEL SWITCH	FAILS TO OPERATE AT BROWNS FERRY 1*	155208 P 52
#HPCI VALVE	FAILS TO OPERATE AT BROWNS FERRY 1*	160179 P 67
#RHR TORUS RETURN VALVE	FAILS TO OPERATE AT BROWNS FERRY 1*	160155 P 72
#DRYWELL TO TORUS DIFFERENTIAL PRESSURE INDICATOR	FAILS TO OPERATE AT BROWNS FERRY 2*	154439 P 79
#REACTOR WATER LEVEL SWITCH	FAILS TO OPERATE AT BROWNS FERRY 2*	154438 P 80
#RECIRC SYSTEM FLOW CONTROL VALVE	FAILS TO OPERATE AT BROWNS FERRY 2*	154437 P 80
#CORE SPRAY DISCHARGE SWITCH	FAILS TO OPERATE AT BROWNS FERRY 2*	160285 P 90
#LEVEL INDICATING SWITCH	FAILS TO OPERATE AT BROWNS FERRY 3*	157715 P107
#INBOARD THROTTLE ISOLATION VALVE	FAILS TO OPERATE AT COOPER*	159465 P250
#HPCI VALVE	FAILS TO OPERATE AT DRESDEN 2*	157698 P212
#HPCI VALVE	FAILS TO OPERATE AT DRESDEN 2*	157686 P213
#HYDROGEN RECOMBINER SYSTEM	FAILS TO OPERATE AT HATCH 2*	157161 P377
#ROD BLOCK MONITOR	FAILS TO OPERATE AT HATCH 2*	159594 P384
#REACTOR LO LO LO LEVEL SWITCH	FAILS TO OPERATE PROPERLY AT FITZPATRICK*	155293 P269
#REACTOR LOW-LOW WATER LEVEL SWITCH	FAILS TO OPERATE PROPERLY AT QUAD CITIES 1*	155369 P540
#CONTAINMENT TEMPERATURE RECORDER	FAILS TO PRINT AT HATCH 1*	158688 P328
#RCIC SYSTEM CONDENSATE PUMP	FAILS TO PUMP PROPERLY AT ARNOLD*	155310 P 4
#DIESEL GENERATOR	FAILS TO REACH OUTPUT VOLTAGE AT BIG ROCK POINT*	161980 P 43
#RCIC	FAILS TO REACH RATED FLOW AT BROWNS FERRY 3*	153459 P101
#RCIC	FAILS TO REACH RATED FLOW AT BROWNS FERRY 3*	153458 P101
#HPCI TURBINE	FAILS TO REACH RATED SPEED AT VERMONT YANKEE*	156151 P570
#DIESEL GENERATOR	FAILS TO REACH RATED VOLTAGE AT HATCH 2*	159313 P350
#CORE SPRAY VALVE	FAILS TO REOPEN DURING TEST AT FITZPATRICK*	154650 P262
#STANDBY GAS TREATMENT TRAIN	FAILS TO RESET AT BRUNSWICK 1*	156176 P136
#TURBINE CONTROL VALVE CLOSURE SCRAM	FAILS TO RESET AT DRESDEN 2*	154620 P207
#LPCI LOOP SELECTION PRESSURE SWITCH	FAILS TO RESET AT DUANE ARNOLD*	159057 P 14
#MAIN STEAM ISOLATION VALVE LIMIT SWITCH	FAILS TO RESET AT FITZPATRICK*	163403 P296
#RCIC TURBINE	FAILS TO RESET AT HATCH 2*	159311 P393
#RCIC	FAILS TO RESPOND TO SPEED CONTROL AT HATCH 1*	159502 P324
#DIESEL GENERATOR	FAILS TO RESPOND TO VOLTAGE CONTROL AT DUANE ARNOLD*	160801 P 21

#DIESEL GENERATOR	FAILS TO RUN AT BIG ROCK POINT*	161982 P 43
#HPCI AUXILIARY OIL PUMP	FAILS TO RUN AT BRUNSWICK 1*	155552 P131
#HPCI	FAILS TO RUN AT BRUNSWICK 1*	156183 P137
#CONTAINMENT ATMOSPHERIC MONITOR	FAILS TO RUN AT BRUNSWICK 1*	156184 P137
#CONTAINMENT ATMOSPHERE ANALYZER	FAILS TO RUN AT BRUNSWICK 1*	156926 P141
#CONTROL ROD	FAILS TO SCRAM AT NINE MILE POINT 1*	158780 P443
#RHR-SW CHECK VALVE	FAILS TO SEAT AT VERMONT YANKEE*	160297 P576
#CORE SPRAY PUMP	FAILS TO START AND VALVE FAILS TO OPEN AT PILGRIM 1*	158613 P508
#RHR SERVICE WATER PUMP	FAILS TO START AT BRUNSWICK 1*	155555 P129
#RHR SERVICE WATER PUMP	FAILS TO START AT BRUNSWICK 1*	155599 P134
#CONTROL BUILDING EMERGENCY FILTER FAN	FAILS TO START AT BRUNSWICK 2*	162727 P197
#DIESEL GENERATOR	FAILS TO START AT DRESDEN 3*	157050 P231
#DIESEL GENERATOR	FAILS TO START AT DUANE ARNOLD*	159058 P 15
#DIESEL GENERATOR	FAILS TO START AT HATCH 1*	156775 P311
#DIESEL GENERATOR	FAILS TO START AT HATCH 1*	158683 P325
#RB RADIATION MONITOR SAMPLE PUMP	FAILS TO START AT HATCH 1*	159567 P326
#LPCI PUMP	FAILS TO START AT HATCH 2*	156994 P370
#STANDBY GAS TREATMENT FILTER TRAIN	FAILS TO START AT HATCH 2*	156935 P372
#DIESEL GENERATOR	FAILS TO START AT HATCH 2*	160441 P396
#SERVICE WATER DIESEL GENERATOR	FAILS TO START AT LACROSSE*	160454 P415
#RHR PUMP ROOM COOLER	FAILS TO START AT PEACH BOTTOM 2*	160434 P481
#RHR PUMP ROOM COOLER	FAILS TO START AT PEACH BOTTOM 2*	162488 P487
#RHR PUMP	FAILS TO START AT QUAD-CITIES 2*	160504 P561
#RHR SERVICE WATER PUMPS	FAILS TO START AT VERMONT YANKEE*	155992 P569
FERRY 1* #DIESEL GENERATOR	FAILS TO START FOLLOWING LOSS OF SHUTDOWN BOARD AT BROWNS	156727 P 66
#DIESEL GENERATOR	FAILS TO START FROM THE CONTROL ROOM AT QUAD CITIES 1*	156006 P541
#DIESEL GENERATOR	FAILS TO START IN REQUIRED TIME AT HATCH 2*	154574 P348
#DIESEL FIRE/CORE SPRAY PUMP	FAILS TO START IN TIME AT BIG ROCK POINT*	163389 P 44
#DIESEL GENERATOR	FAILS TO START IN TIME AT BRUNSWICK 1*	161075 P156
#DIESEL GENERATOR	FAILS TO START WITHIN SEVEN SECONDS AT HATCH 1*	153506 P297
#HPCI STOP VALVE	FAILS TO STAY OPEN AT BROWNS FERRY 1*	153490 P 46
CREEK* #DIESEL GENERATOR	FAILS TO SYNCRONIZE WHILE REDUNDANT DIESEL DOWN AT OYSTER	155999 P455
#DIESEL GENERATOR	FAILS TO TIE INTO BUS AT HATCH 2*	159312 P392
#DIESEL GENERATOR	FAILS TO TIE TO BUS AT HATCH 2*	158298 P366
#RMOV BOARD BREAKER	FAILS TO TRANSFER DURING MANUAL TRANSFER AT BROWNS FERRY 2*	156028 P 83
#REACTOR LOW WATER LEVEL SWITCH	FAILS TO TRIP AT BRUNSWICK 1*	154880 P131
#RCIC TRIP AND THROTTLE VALVE	FAILS TO TRIP AT BRUNSWICK 2*	154678 P166
#DIESEL GENERATOR OUTPUT BREAKER	FAILS TO TRIP AT HATCH 2*	158296 P371
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#DRYWELL HYDROGEN OXYGEN ANALYZER GIVES FALSE READINGS AT HATCH 2# 159310 P393
#RECIRCULATION PUMP TRIPS ON LOW OIL PRESSURE AT HATCH 2# 159315 P394
#REACTOR HIGH PRESSURE SWITCH SET POINT INCORRECT AT HATCH 2# 159305 P394
#RCIC OUTBOARD ISOLATION VALVE FAILS TO CLOSE AT HATCH 2# 159319 P394
#INBOARD MSIV LEAKAGE CONTROL SYSTEM INOPERABLE AT HATCH 2# 159373 P395
#CONTAINMENT HYDROGEN OXYGEN ANALYZER INOPERABLE AT HATCH 2# 159376 P395
#DIESEL GENERATOR FAILS TO START AT HATCH 2# 160441 P396
#SET POINT DRIFT IN REACTOR HIGH PRESSURE SWITCHES AT HATCH 2# 160379 P395
#TORUS/DRYWELL VACUUM BREAKER FAILS TO OPEN AT HATCH 2# 160476 P396
#CONTAINMENT HYDROGEN OXYGEN ANALYZER INOPERABLE AT HATCH 2# 160446 P397
#SET POINT DRIFT IN REACTOR PRESSURE SWITCH AT HATCH 2# 160450 P397
#SET POINT DRIFTS IN CORE SPRAY PUMP PRESSURE SWITCH AT HATCH 2# 160455 P397
#REACTOR LEVEL SWITCHES TRIP TOO LATE AT HATCH 2# 160518 P398
#REACTOR COOLANT CONDUCTIVITY LIMITS EXCEEDED AT HATCH 2# 160509 P398
#SERVICE WATER LINE INADEQUATELY SUPPORTED AT HATCH 2# 160511 P398
#SET POINT DRIFT IN HPCI TURBINE PRESSURE SWITCH AT HATCH 2# 160519 P399
#TORUS PURGE ISOLATION VALVE LEAKS AT HATCH 2# 161712 P399
#RADWASTE PIPING IS NOT SEISMICALLY PROTECTED AT HATCH 2# 160522 P399
#DRYWELL-TORUS VACUUM BREAKERS NOT TESTED ON TIME AT HATCH 2# 160845 P400
#SET POINT DRIFT IN HPCI STEAM LINE DP SWITCH AT HATCH 2# 161872 P400
#FIRE WALL INTEGRITY BREACHED AT HATCH 2# 160841 P401
#SET POINT MAIN STEAM LINE PRESSURE SWITCH AT HATCH 2# 161873 P401
#REACTOR PRESSURE CALIBRATION PROCEDURE NOT REVIEWED AT HATCH 2# 161874 P401
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#DIESEL GENERATOR MAKES EXCESSIVE NOISE AT HATCH 2# 161833 P402
#TORUS WATER LEVEL BELOW LIMIT AT HATCH 2# 160832 P403
#RHR PUMP SUCTION VALVE LEAKS AT HATCH 2# 161628 P403
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#HPCI AND RHR VALVE LEAK RATES EXCEED LIMITS AT HATCH 2# 161914 P404
#FEEDWATER ISOLATION VALVE LEAK RATES EXCEED LIMIT AT HATCH 2# 161845 P404
#RHR AND CONDENSATE ISOLATION VALVES LEAK AT HATCH 2# 161759 P405
#CONTAINMENT ISOLATION BELLOWS APPEARS TO LEAK AT HATCH 2# 162417 P405
#MONORAIL HOIST OPERABILITY NOT VERIFIED BEFORE USE AT HATCH 2# 162423 P406
#DIESEL GENERATOR FAILS AT HATCH 2# 161881 P406
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#REACTOR	HIGH PRESSURE SWITCH SETPOINTS TOO HIGH AT HATCH 1*		159501 P323
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#SET POINT DRIFT IN REACTOR	HIGH PRESSURE SWITCHES AT HATCH 2*		160370 P396
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#HYDRANT HOSE	HOUSE LOST AT PILGRIM 1*		156034 P506
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HATCH 1*	#	HPCI	STEAM SUPPLY VALVES CLOSE ABOVE PRESSURE LIMIT AT	153503 P299
	#	HPCI	STEAM THROTTLE VALVE OSCILLATES AT PILGRIM 1*	160344 P524
	#	HPCI	STEAM THROTTLE VALVE OSCILLATES AT PILGRIM 1*	160240 P524
#SET POINT DRIFT IN		HPCI	STEAMLINE HIGH D/P SWITCH AT BRUNSWICK 2*	156107 P173
	#	HPCI	STEAMLINE LEAKS AT QUAD-CITIES 2*	159241 P557
#SET POINT DRIFT IN		HPCI	STEAMLINE LOW PRESSURE SWITCH AT HATCH 1*	161614 P342
	#	HPCI	STOP VALVE COVER LEAKS AT QUAD-CITIES 2*	161902 P562
	#	HPCI	STOP VALVE FAILS TO OPEN COMPLETELY AT BROWNS FERRY 1*	156211 P 56
	#	HPCI	STOP VALVE FAILS TO STAY OPEN AT BROWNS FERRY 1*	153490 P 46
	#	HPCI	STOP VALVE STEM FAILS AT COOPER*	155604 P242
	#	HPCI	STUCK ON HIGH SPEED AT QUAD CITIES 2*	159524 P558
	#	HPCI	SUCTION LINE ISOLATION VALVE LEAKS AT DRESDEN 3*	157051 P230
	#	HPCI	SUPPRESSION CHAMBER SUCTION VALVE FAILS TO OPEN AT	159317 P390
HATCH 2*				
BRUNSWICK 1*	#MANUAL VALVE ON	HPCI	SUPPRESSION POOL SUCTION LINE FOUND CLOSED AT	155515 P132
	#	HPCI	SYSTEM INOPERABLE AT COOPER*	160052 P255
	#	HPCI	SYSTEM INOPERABLE AT HATCH 1*	160493 P340
	#	HPCI	SYSTEM NOT OPERABLE WHEN REQUIRED AT HATCH 1*	156091 P309
#CABLE SEPARATION FOUND TO BE INADEQUATE FOR ADS AND		HPCI	SYSTEMS AT PILGRIM 1*	154691 P503
#RTGB CONTROL SWITCH FOR		HPCI	TORUS SUCTION VALVE FAILS AT BRUNSWICK 1*	155397 P132
	#	HPCI	TRIPS DURING A TEST AT BRUNSWICK 1*	154511 P122
#SET POINT DRIFT IN		HPCI	TURBINE AREA HIGH TEMPERATURE SWITCH AT QUAD CITIES 1*	160451 P547
	#	HPCI	TURBINE CONTROL VALVE FAILS TO OPEN AT BRUNSWICK 2*	160082 P188
FERRY 1*	#CRACK FOUND IN	HPCI	TURBINE COUPLING BEARING SUPPORT PEDESTAL AT BROWNS	155292 P 40
FERRY 2*	#CRACK DISCOVERED IN	HPCI	TURBINE COUPLING BEARING SUPPORT PEDESTAL AT BROWNS	155489 P 83
	#	HPCI	TURBINE DOES NOT REACH RATED SPEED AT DUANE ARNOLD*	160275 P 10
MONTICELLO*	#LEAK RATE OF	HPCI	TURBINE EXHAUST CHECK VALVE EXCEEDS LIMIT AT	158239 P429
	#MULTIPLE	HPCI	TURBINE EXHAUST LINE SNUBBER FAILURES AT PILGRIM 1*	160187 P522
	#	HPCI	TURBINE FAILS TO REACH RATED SPEED AT VERMONT YANKEE*	156151 P570
	#	HPCI	TURBINE FAILS TO TRIP ON OVERSPEED AT HATCH 1*	159434 P332
#SET POINT DRIFT IN		HPCI	TURBINE PRESSURE SWITCH AT HATCH 2*	160519 P399
	#	HPCI	TURBINE REVERSING CHAMBERS CRACK AT DUANE ARNOLD*	156929 P 28
	#	HPCI	TURBINE RUPTURE DISC FAILS AT BROWNS FERRY 2*	159340 P 91
	#	HPCI	TURBINE SPEED SWINGS AT HATCH 1*	158790 P319
	#	HPCI	TURBINE TRIPS AT BROWNS FERRY 1*	157033 P 57
	#	HPCI	TURBINE TRIPS DURING TESTING AT BROWNS FERRY 2*	156017 P 86
	#	HPCI	TURBINE TRIPS DURING TESTING AT VERMONT YANKEE*	154875 P566
	#	HPCI	VALVE FAILS TO CLOSE AT PEACH BOTTOM 3*	161755 P500
	#	HPCI	VALVE FAILS TO OPERATE AT BROWNS FERRY 1*	160179 P 67
	#	HPCI	VALVE FAILS TO OPERATE AT DRESDEN 2*	157698 P212
	#	HPCI	VALVE FAILS TO OPERATE AT DRESDEN 2*	157686 P213
	#	HPCI	VALVE INOPERABLE AT QUAD CITIES 2*	161891 P563
HATCH 2*	#RPS,	HPCI,	RCIS, AND CS PRESSURE SWITCHES SETPOINTS DRIFT AT	159591 P386
#SCREEN WASH PUMP PIPING LEAKS AT		HUMBOLDT	BAY 3*	163357 P411
#SCREEN WASH PUMP DISCHARGE VALVE LEAKS AT		HUMBOLDT	BAY 3*	160844 P411
#UNIT 1 TRIP TEST NOT PERFORMED WHEN REQUIRED AT		HUMBOLDT	BAY*	155480 P410
#SUPPRESSION POOL COOLER PUMP LOSES POWER AT		HUMBOLDT	BAY*	160292 P410
#SCREEN WASH PUMP MOTOR OVERHEATS AT		HUMBOLDT	BAY*	159437 P411
#DIESEL FIRE PUMP INOPERABLE AT		HUMBOLDT	BAY*	162008 P412
#REACTOR BUILDING		HVAC	ROOM SPRINKLER SYSTEM INOPERABLE AT HATCH 1*	162058 P345
#FIRE		HYDRANT	FAILS TO OPEN AT COOPER*	159246 P253
	#	HYDRANT	HOSE HOUSE LOST AT PILGRIM 1*	156034 P506
**	#	HYDRAULIC	ACCUMULATOR AIR VALVE BROKEN OFF AT BROWNS FERRY	161621 P119

FITZPATRICK*	#CRD	HYDRAULIC CONTROL UNIT INSTRUMENT READINGS DRIFT AT	159226 P285
#LOW DIESEL GENERATOR GOVERNOR	HYDRAULIC OIL LEVEL FOUND AT DUANE ARNOLD*		158616 P 9
MONTICELLO*	#CONTROL ROD DRIVE	HYDRAULIC PIPE SUPPORTS FAIL TO MEET STRESS LIMITS AT	156339 P433
#	HYDRAULIC SNUBBER FAILS AT HATCH 2*		158299 P364
#	HYDRAULIC SNUBBER FAILS TO LOCK AT OYSTER CREEK*		160235 P461
#	HYDRAULIC SNUBBER FOUND INOPERABLE AT FITZPATRICK*		154652 P268
#	HYDRAULIC SNUBBERS FAIL TEST AT FITZPATRICK*		160203 P286
#DRYWELL OXYGEN-	HYDROGEN ANALYSIS INDICATES LOW AT HATCH 2*		156116 P355
#DRYWELL OXYGEN-	HYDROGEN ANALYZER FAILS AT HATCH 2*		155144 P355
#OFF-GAS	HYDROGEN ANALYZERS NOT TESTED ON TIME AT MILLSTONE 1*		161679 P426
#CONTAINMENT	HYDROGEN AND OXYGEN ANALYZER FAILS AT HATCH 1*		156972 P311
#TORUS	HYDROGEN MONITOR ANNUNCIATES SPURIOUSLY AT BROWNS FERRY 1*		160305 P 71
#	HYDROGEN MONITOR FAILS TO CALIBRATE AT BROWNS FERRY 3*		160330 P116
#DRYWELL	HYDROGEN MONITORS FAIL TO CALIBRATE AT BROWNS FERRY 2*		160213 P 93
#DRYWELL	HYDROGEN OXYGEN ANALYZER GIVES FALSE READINGS AT HATCH 2*		159319 P393
#CONTAINMENT	HYDROGEN OXYGEN ANALYZER INOPERABLE AT HATCH 2*		159376 P395
#CONTAINMENT	HYDROGEN OXYGEN ANALYZER INOPERABLE AT HATCH 2*		160049 P397
#UPDATE ON CONTAINMENT	HYDROGEN OXYGEN ANALYZERS AT HATCH 2*		103426 P389
#DRYWELL AND TORUS	HYDROGEN OXYGEN RECORDER INOPERABLE AT HATCH 1*		159290 P533
#DRYWELL	HYDROGEN OXYGEN RECORDER READS ABNORMALLY AT HATCH 1*		159510 P37
HATCH 2*	#	HYDROGEN RECOMBINER RETURN VALVE LEAKAGE UNACCEPTABLE AT	155449 P3Lo
#	HYDROGEN RECOMBINE SHEATH TEMPERATURE LOW AT HATCH 2*		154944 P350
#	HYDROGEN RECOMBINER SYSTEM FAILS TO OPERATE AT HATCH 2*		157161 P377
#CONTAINMENT	HYDROGEN RECOMBINER TEST NOT PERFORMED PROPERLY AT HATCH 2*		156855 P375
#OFFGAS	HYDROGEN SAMPLE PUMP TRIPS AT HATCH 1*		159442 P334
#DRYWELL	HYDROGEN SENSOR FAILS TO CALIBRATE AT BROWNS FERRY 3*		161935 P121
#UPDATE ON	HYDROGEN SENSOR QUALIFICATIONS AT BROWNS FERRY 1, 2 AND 3*		163568 P 61
#DRYWELL	HYDROGEN-OXYGEN ANALYZER FAILS AT HATCH 2*		156857 P375
#DRYWELL	HYDROGEN-OXYGEN ANALYZER PUMP DIAPHRAGM FAILS AT HATCH 2*		156858 P374
#DRYWELL	HYDROGEN-OXYGEN ANALYZER READING IN ERROR AT HATCH 1*		155454 P359

#DG FAILS TO TRIP TO	IDLE SPEED FROM LOAD AT BROWNS FERRY 3*		160216 P116
#ROD BLOCK MONITOR PANEL	ILLUMINATES SPURIOUSLY AT BRUNSWICK 1*		159160 P138
#SECTION OF A SPLIT RING FOUND IN THE MAIN HPCI PUMP	IMPELLER AT ARNOLD*		156226 P 6
PIPING AT BROWNS FERRY 1*	#WATER	IMPINGEMENT FROM RECIRC LINE BREAK MIGHT DAMAGE PDCC*	155207 P 53
#SMOKE DETECTORS INSTALLED	IMPROPERLY AT HATCH 1 AND 2*		162087 P346
CREEK*	#	IMPROPERLY INSTALLED PIPE SNUBBERS DISCOVERED AT OYSTER	155601 P452
CREEK*	#	IMPROPERLY INSTALLED PIPE SUPPORTS DISCOVERED AT OYSTER	155474 P453
#THREE	INACCESSIBLE SNUBBERS FOUND INOPERABLE AT BRUNSWICK 1*		154513 P122
#CRD PIPING SUPPORTS	INADEQUATE AT BRUNSWICK 1*		160110 P149
#CRD PIPING SUPPORTS	INADEQUATE AT BRUNSWICK 2*		160112 P183
#INTEGRITY OF A FIRE STOP	INADEQUATE AT DRESDEN 2*		153940 P207
#DIESEL FIRE PUMP FLOW	INADEQUATE AT PILGRIM 1*		161907 P535
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HATCH 1*	#	INADEQUATE FIRE PROTECTION PIPE SUPPORTS INSTALLED AT	162742 P344
#CABLE SEPARATION FOUND TO BE	INADEQUATE FOR ADS AND HPCI SYSTEMS AT PILGRIM 1*		154691 P503
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#RHR MINIMUM FLOW VALVES CLOSED	INADVERTENTLY AT HATCH 1*		159500 P322
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#FEEDWATER	INBOARD CHECK VALVE LEAK RATE EXCEEDS LIMIT AT DRESDEN 3*		157518 P227
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#REFUELING ZONE	INBOARD ISOLATION TIME DELAY RELAY FAILS AT BROWNS FERRY 2*		161065 P 96
#LPCI	INBOARD ISOLATION VALVE LEAKS AT HATCH 1*		160289 P315
#	INBOARD MSIV LEAKAGE CONTROL SYSTEM INOPERABLE AT HATCH 2*		159373 P305
*	#	INBOARD THROTTLE ISOLATION VALVE FAILS TO OPERATE AT COOPER	159465 P250
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#TRAVERSING	INCORE PROBE FAILS TO WITHDRAW AT PEACH BOTTOM 3*		155585 P489
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#REACTOR LOW WATER LEVEL SWITCH SETPOINT IS	INCORRECT AT BRUNSWICK 2*		159356 P184
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#APR MONITOR SETPOINT	INCORRECT AT FITZPATRICK*		160228 P287
#HPCI AND RCIC STEAM PRESSURE SWITCH SETPOINTS	INCORRECT AT FITZPATRICK*		159576 P288
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#RHR SERVICE WATER BOOSTER PUMP	INOPERABLE AT COOPER*	159521 P248
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#HPCI	INOPERABLE AT COOPER*	159477 P249
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#RHR HEAT EXCHANGER	INOPERABLE AT COOPER*	162523 P258
#SHOCK SUPPRESSOR	INOPERABLE AT COOPER*	162520 P259
#CONTAINMENT VACUUM BREAKER	INOPERABLE AT DRESDEN 3*	160230 P234
#RCIC ISOLATION CONDENSER VALVE	INOPERABLE AT DRESDEN 3*	161457 P238
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#CONTROL ROOM AIR TREATMENT SYSTEM	INOPERABLE AT DUANE ARNOLD*	159558 P 16
#RCIC	INOPERABLE AT DUANE ARNOLD*	160462 P 19
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#RHR LOOP SUPPORT	INOPERABLE AT FITZPATRICK*	153728 P267
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#INTERMEDIATE RANGE MONITOR MADE	INOPERABLE AT FITZPATRICK*	155307 P272
#BOTH DRYWELL ATMOSPHERE MONITORS	INOPERABLE AT FITZPATRICK*	155306 P272
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#RCIC	INOPERABLE AT FITZPATRICK*	161685 P292
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#RCIC FIRE DETECTION SYSTEM	INOPERABLE AT FITZPATRICK*	161992 P295
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#CONTROL ROOM AIR INTAKE CHLORINE MONITOR	INOPERABLE AT HATCH 1*	159433 P333
#RHR	INOPERABLE AT HATCH 1*	159323 P334
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	#TORUS WATER	LEVEL BELOW LIMIT AT HATCH 2*	160802 P403
	#RCIC DRAIN POT	LEVEL CONTROL VALVE PACKING RUPTURES AT ARNOLD*	154711 P 1
	#DIESEL GENERATOR LUBE OIL	LEVEL DROPS BELOW LIMIT AT ARNOLD*	153730 P 2
	#TORUS WATER	LEVEL DROPS BELOW LIMIT AT FITZPATRICK*	154666 P265
	#FIRE WATER STORAGE TANK	LEVEL DROPS BELOW LIMIT AT HATCH 1*	157090 P313
	#CAD STORAGE TANK	LEVEL DROPS BELOW LIMIT DURING INERTING AT BRUNSWICK 2*	156110 P175
	#TORUS	LEVEL EXCEEDS HIGH LIMIT AT BRUNSWICK 2*	155513 P173
	#SUPPRESSION POOL	LEVEL EXCEEDS LIMIT AT BRUNSWICK 2*	160514 P190
	#TORUS	LEVEL EXCEEDS LIMIT AT BRUNSWICK 2*	161813 P200
	#DISCHARGE LIQUID EFFLUENT RADIATION	LEVEL EXCEEDS LIMIT AT DRESDEN 1*	156224 P205
	#TORUS WATER	LEVEL EXCEEDS LIMIT AT FITZPATRICK*	155550 P269
	#REACTOR LOW WATER	LEVEL FAILS TO ACTUATE AT BRUNSWICK 1*	160534 P155
	#TORUS WATER	LEVEL FALLS BELOW LIMIT AT DUANE ARNOLD*	159215 P 13
	#TORUS WATER	LEVEL FALLS BELOW LIMIT AT FITZPATRICK*	158604 P282
	#NITROGEN TANK	LEVEL FALLS BELOW LIMIT AT HATCH 1*	161950 P341
	#LOW DIESEL GENERATOR GOVERNOR HYDRAULIC OIL	LEVEL FOUND AT DUANE ARNOLD*	158616 P 9
BRUNSWICK 1*	#LOW	LEVEL FOUND IN CONTAINMENT ATMOSPHERE DILUTION TANK AT	156928 P139
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	#TORUS WATER	LEVEL INDICATOR FAILS AT PILGRIM 1*	161937 P535
	#SUPPRESSION POOL	LEVEL INDICATOR READS HIGH OFFSCALE AT BRUNSWICK 2*	155534 P168
	#TORUS	LEVEL INSTRUMENT FAILS TO FUNCTION PROPERLY AT BRUNSWICK 1*	154516 P126
	#TORUS WATER	LEVEL INSTRUMENT READS DOWNSCALE AT HATCH 2*	158297 P367
	#SCRAM DISCHARGE VOLUME HIGH	LEVEL INSTRUMENT TEST NOT PERFORMED AT DUANE ARNOLD*	161926 P 25
	#RECIRCULATION PUMP MOTOR OIL	LEVEL IS LOW AT DUANE ARNOLD*	160265 P 11
	#SUPPRESSION POOL	LEVEL OUT OF LIMITS THREE TIMES AT QUAD CITIES 1*	153947 P537
	#TORUS WATER	LEVEL PUMPED DOWN BELOW LIMIT AT DUANE ARNOLD*	163370 P 25
	#ABNORMAL INDICATION OF TORUS	LEVEL RECORDER AT BRUNSWICK 2*	154883 P169
	#TORUS WATER	LEVEL RECORDER INOPERABLE AT HATCH 1*	159291 P334
	#SCRAM DISCHARGE VOLUME HIGH	LEVEL ROD WITHDRAWAL BLOCK SWITCH FAILS AT HATCH 2*	159159 P390
	#SET POINT DRIFT IN LOW WATER	LEVEL SCRAM SWITCH AT MILLSTONE 1*	160923 P424
	#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SENSOR AT OYSTER CREEK*	160148 P465
	#TORUS WATER	LEVEL SENSOR FAILS TO INDICATE ACTUAL LEVEL AT FITZPATRICK*	159104 P286
POINT*	#REACTOR WATER	LEVEL SENSOR REMOVED FROM SERVICE FOR VENTING AT BIG ROCK	160071 P 40
	#UPDATE REPORT ON PRIMARY SYSTEM WATER	LEVEL SENSORS AT BIG ROCK POINT*	153973 P 30
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	#LO LO LO REACTOR WATER	LEVEL SETPOINT RAISED AT NINE MILE POINT 1*	154932 P440
	#SET POINT DRIFT IN REACTOR WATER	LEVEL SWITCH AT BROWNS FERRY 2*	155205 P 81
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	#SET POINT DRIFT IN REACTOR VESSEL SHROUD	LEVEL SWITCH AT BRUNSWICK 1*	154554 P124
	#SET POINT DRIFT IN CONDENSATE STORAGE TANK	LEVEL SWITCH AT BRUNSWICK 1*	155165 P127
	#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SWITCH AT BRUNSWICK 1*	160341 P139
	#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SWITCH AT BRUNSWICK 1*	156932 P143
	#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SWITCH AT BRUNSWICK 2*	156938 P178
	#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SWITCH AT BRUNSWICK 2*	161652 P197
	#SET POINT DRIFT IN REACTOR WATER	LEVEL SWITCH AT DRESDEN 2*	155197 P208

#SET POINT DRIFT IN RPS	LEVEL SWITCH AT ORESDEN 3*	161460 P237
#SETPOINT DRIFT IN CONDENSATE STORAGE TANK	LEVEL SWITCH AT FITZPATRICK*	156197 P274
#INCORRECT SET POINT FOUND IN REACTOR	LEVEL SWITCH AT HATCH 1*	161951 P341
#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SWITCH AT QUAD CITIES 1*	156912 P543
#CONTROL ROD DRIVE ACCUMULATOR	LEVEL SWITCH FAILS AT BROWNS FERRY 1*	155296 P 49
#SCRAM DISCHARGE VOLUME HIGH	LEVEL SWITCH FAILS AT BROWNS FERRY 1*	158293 P 65
#REACTOR WATER	LEVEL SWITCH FAILS AT BRUNSWICK 1*	158545 P144
#CONDENSATE STORAGE TANK LOW	LEVEL SWITCH FAILS AT DUANE ARNOLD*	160829 P 19
#REACTOR WATER	LEVEL SWITCH FAILS AT HATCH 2*	158674 P382
#REACTOR WATER	LEVEL SWITCH FAILS AT HATCH 2*	158675 P382
#TORUS WATER	LEVEL SWITCH FAILS TO ACTUATE AT BRUNSWICK 2*	155572 P167
#CRD SCRAM ACCUMULATOR	LEVEL SWITCH FAILS TO OPERATE AT BROWNS FERRY 1*	155208 P 52
#REACTOR WATER	LEVEL SWITCH FAILS TO OPERATE AT BROWNS FERRY 2*	154438 P 80
#REACTOR LO LO LO	LEVEL SWITCH FAILS TO OPERATE PROPERLY AT FITZPATRICK*	155293 P269
#REACTOR LOW-LOW WATER	LEVEL SWITCH FAILS TO OPERATE PROPERLY AT QUAD CITIES 1*	155369 P540
#REACTOR LOW WATER	LEVEL SWITCH FAILS TO TRIP AT BRUNSWICK 1*	154880 P131
#SCRAM DISCHARGE VOLUME	LEVEL SWITCH INOPERABLE AT BROWNS FERRY 2*	161864 P 99
#DRIFT IN REACTOR WATER	LEVEL SWITCH SET POINT AT FITZPATRICK*	161717 P294
#REACTOR LOW WATER	LEVEL SWITCH SETPOINT INCORRECT AT BROWNS FERRY 2*	159685 P 90
#REACTOR LOW WATER	LEVEL SWITCH SETPOINT IS INCORRECT AT BRUNSWICK 2*	159356 P184
#REACTOR LOW WATER	LEVEL SWITCH STICKS CLOSED AT BRUNSWICK 2*	158732 P180
#REACTOR WATER	LEVEL SWITCH TRIPS BELOW SETPOINT AT ORESDEN 3*	158749 P232
#REACTOR WATER	LEVEL SWITCH TRIPS BELOW SETPOINT AT ORESDEN 3*	158750 P233
#DRIFT IN REACTOR LOW WATER	LEVEL SWITCHES AT BRUNSWICK 1*	163468 P157
#SET POINT DRIFT IN REACTOR LOW WATER	LEVEL SWITCHES AT BRUNSWICK 2*	161969 P182
#SET POINT DRIFT IN LOW REACTOR	LEVEL SWITCHES AT COOPER*	161622 P256
#SET POINT DRIFT IN REACTOR WATER	LEVEL SWITCHES AT FITZPATRICK*	163361 P295
#SET POINT DRIFT IN REACTOR LO-LO-LO WATER	LEVEL SWITCHES AT OYSTER CREEK 1*	161873 P468
#REACTOR WATER	LEVEL SWITCHES EXCEED SET POINT LIMIT AT PILGRIM 1*	160345 P521
#REACTOR WATER	LEVEL SWITCHES FAIL AT BROWNS FERRY 3*	158537 P107
#SCRAM DISCHARGE VOLUME HIGH	LEVEL SWITCHES FAIL AT HATCH 2*	159584 P391
#LIQUID	LEVEL SWITCHES FAIL TO OPERATE PROPERLY AT BROWNS FERRY 2*	155204 P 81
#17 SCRAM ACCUMULATOR	LEVEL SWITCHES INOPERABLE AT BROWNS FERRY 2*	160907 P 95
#REACTOR	LEVEL SWITCHES TRIP TOO LATE AT HATCH 2*	160518 P398
#NITROGEN STORAGE TANK	LEVEL TOO LOW AT HATCH 1*	159503 P324
#SET POINT DRIFT IN REACTOR	LEVEL TRANSMITTER AT BIG ROCK POINT*	157074 P 34
#SET POINT DRIFT IN REACTOR	LEVEL TRANSMITTER AT BIG ROCK POINT*	157076 P 35
#SET POINT DRIFT IN REACTOR WATER	LEVEL TRANSMITTER AT BIG ROCK POINT*	158777 P 37
#SET POINT DRIFT IN REACTOR	LEVEL TRANSMITTER AT BIG ROCK POINT*	160170 P 41
#FEEDWATER CONTROL	LEVEL TRANSMITTER FOUND OUT OF CALIBRATION AT FITZPATRICK*	156915 P277
#NARROW RANGE TORUS WATER	LEVEL TRANSMITTER OUT OF CALIBRATION AT HATCH 2*	154948 P348
#TORUS	LEVEL TRANSMITTER OUT OF TOLERANCE AT BRUNSWICK 1*	155559 P129
#HPCI REACTOR HIGH WATER	LEVEL TRIP INOPERABLE AT BROWNS FERRY 1*	156011 P 55
#SET POINT DRIFT IN RCIC REACTOR WATER	LEVEL TRIP SWITCH AT BROWNS FERRY 3*	159284 P113
#COBALT AND SILVER ACTIVITY	LEVELS EXCEED TEN TIMES CONTROL ACTIVITIES AT MILLSTONE*	161680 P425
#CONTROL ROD POSITION INDICATOR LIGHT FAILS TO	LIGHT AT BRUNSWICK 1*	154712 P125
#VACUUM BREAKER VALVE INDICATING	LIGHT FAILS OFF AT HATCH 1*	160267 P322
#CONTROL ROD POSITION INDICATOR	LIGHT FAILS TO LIGHT AT BRUNSWICK 1*	154712 P125
#CONTAINMENT VACUUM BREAKER	LIGHTS FAIL TO INDICATE OPEN AT HATCH 2.*	154619 P347
#CONTROL ROD POSITION	LIGHTS REMAIN ON WHEN RODS MOVED AT OYSTER CREEK*	158695 P456
#FUEL EXPOSURE EXCEEDS MAPLHGR	LIMIT ANALYSIS EXPOSURES AT QUAD CITIES 1*	153946 P537
#DIESEL GENERATER LUBE OIL LEVEL DROPS BELOW	LIMIT AT ARNOLD*	153730 P 2
#SUPPRESSION CHAMBER OXYGEN CONCENTRATION EXCEEDS	LIMIT AT ARNOLD*	155529 P 2
#LEAK RATE OF MSIV'S EXCEEDS	LIMIT AT ARNOLD*	155526 P 4
#CONTAINMENT ISOLATION VALVES' LEAK RATES EXCEED	LIMIT AT BIG ROCK POINT*	163388 P 45
#LEAK RATE OF THREE MAIN STEAM ISOLATION VALVES EXCEED	LIMIT AT BROWNS FERRY 1,*	154617 P 48
#TOTAL CONTAINMENT LEAK RATE EXCEEDS	LIMIT AT BROWNS FERRY 1*	155532 P 55
#CORE SPRAY PUMP AREA COOLER FAN THERMOSTAT SET ABOVE	LIMIT AT BROWNS FERRY 1*	159399 P 68
#RIVER WATER TEMPERATURE RISE EXCEEDS	LIMIT AT BROWNS FERRY 1*	160156 P 73
#DIFFERENTIAL RIVER TEMPERATURE EXCEEDS	LIMIT AT BROWNS FERRY 1*	161754 P 78
#HPCI CHECK VALVE LEAK RATE EXCEEDS	LIMIT AT BROWNS FERRY 2*	160092 P 94
#DISCHARGE CHLORINE CONCENTRATION EXCEEDS	LIMIT AT BROWNS FERRY 2*	163470 P 97
#CONTAINMENT LEAK RATE EXCEEDS	LIMIT AT BROWNS FERRY 2*	161792 P 98
#FLOW THROUGH RHR PUMP SEAL COOLERS FALLS BELOW	LIMIT AT BROWNS FERRY 3*	155503 P105
#COOLING WATER FLOW TO RHR PUMP SEAL FALLS BELOW	LIMIT AT BROWNS FERRY 3*	159393 P114
#RIVER TEMPERATURE EXCEEDS	LIMIT AT BROWNS FERRY*	158730 P 66
#REACTOR VESSEL CONDUCTIVITY EXCEEDS	LIMIT AT BRUNSWICK 1*	155311 P135
#CONTAINMENT ATMOSPHERE DILUTION TANK VOLUME FALLS BELOW	LIMIT AT BRUNSWICK 1*	158658 P145
#REACTOR VESSEL CONDUCTIVITY EXCEEDS	LIMIT AT BRUNSWICK 1*	163465 P146
#REACTOR COOLANT CONDUCTIVITY EXCEEDS	LIMIT AT BRUNSWICK 1*	160403 P150
#MILK SAMPLE IODINE ACTIVITY EXCEEDS	LIMIT AT BRUNSWICK 1*	162480 P159
#GROSS BETA CONCENTRATION EXCEEDS	LIMIT AT BRUNSWICK 1*	162396 P159
#REACTOR COOLANT CONDUCTIVITY EXCEEDS	LIMIT AT BRUNSWICK 2*	153737 P164
#CRITICAL POWER RATIO IS LESS THAN	LIMIT AT BRUNSWICK 2*	155575 P166
#TORUS LEVEL EXCEEDS HIGH	LIMIT AT BRUNSWICK 2*	155513 P173
#SUPPRESSION POOL LEVEL EXCEEDS	LIMIT AT BRUNSWICK 2*	160514 P190
#CONTROL ROD INSERTION TIME EXCEEDS	LIMIT AT BRUNSWICK 2*	160513 P190
#CONTAINMENT OXYGEN CONCENTRATION EXCEEDS	LIMIT AT BRUNSWICK 2*	160874 P191
#UPDATE ON CONTAINMENT OXYGEN CONCENTRATION EXCEEDING	LIMIT AT BRUNSWICK 2*	163503 P191
#CONTAINMENT OXYGEN CONCENTRATION EXCEEDS	LIMIT AT BRUNSWICK 2*	160820 P193
#CONTAINMENT OXYGEN CONCENTRATION EXCEEDS	LIMIT AT BRUNSWICK 2*	161817 P199
#TORUS LEVEL EXCEEDS	LIMIT AT BRUNSWICK 2*	161813 P200
#SLC PUMP FLOW FALLS BELOW	LIMIT AT COOPER*	156169 P244
#DISCHARGE LIQUID EFFLUENT RADIATION LEVEL EXCEEDS	LIMIT AT DRESDEN 1*	156224 P205
#RADIOACTIVITY IN WASTE SAMPLE TANK EXCEEDS	LIMIT AT DRESDEN 2*	153491 P206
#WASTE SAMPLE TANK ACTIVITY EXCEEDS	LIMIT AT DRESDEN 2*	154927 P209
#FEEDWATER INBOARD CHECK VALVE LEAK RATE EXCEEDS	LIMIT AT DRESDEN 3*	155518 P227

#CONTAINMENT VENT LINE ISOLATION VALVE LEAKAGE EXCEEDS	LIMIT AT DRESDEN 3*	155521 P227
#FEEDWATER CHECK VALVE LEAK RATE EXCEEDS	LIMIT AT DRESDEN 3*	156159 P229
#REACTOR PRESSURE AND COOLANT TEMPERATURE EXCEED	LIMIT AT DRESDEN 3*	163344 P240
#TORUS WATER LEVEL FALLS BELOW	LIMIT AT DUANE ARNOLD*	159215 P 13
#INSTRUMENT AC UNDERVOLTAGE RELAY SETPOINT FALLS BELOW	LIMIT AT DUANE ARNOLD*	163373 P 24
#TORUS WATER LEVEL PUMPED DOWN BELOW	LIMIT AT DUANE ARNOLD*	163370 P 25
#TORUS WATER LEVEL DROPS BELOW	LIMIT AT FITZPATRICK*	154606 P265
#TORUS WATER LEVEL EXCEEDS	LIMIT AT FITZPATRICK*	155350 P269
#DRYWELL TO TORUS DIFFERENTIAL PRESSURE FALLS BELOW	LIMIT AT FITZPATRICK*	155305 P270
#CORE SPRAY PIPE BREAK DETECTOR SET POINT EXCEEDS	LIMIT AT FITZPATRICK*	155303 P270
#DRIFTING LPRM CAUSES INDICATED HIGH THERMAL	LIMIT AT FITZPATRIC	155* P273
#DRYWELL TO TORUS DIFFERENTIAL PRESSURE FALLS BELOW	LIMIT AT FITZPATRIC	157084 P275
#RADIOACTIVITY RELEASE EXCEEDS REPORT	LIMIT AT FITZPATRICK*	158237 P275
#IODINE RELEASE RATE EXCEEDS	LIMIT AT FITZPATRICK*	158622 P279
#TORUS WATER LEVEL FALLS BELOW	LIMIT AT FITZPATRICK*	158604 P282
#COBALT ACTIVITY IN PERIPHYTON SAMPLE EXCEED	LIMIT AT FITZPATRICK*	160498 P291
#MILK SAMPLE IODINE ACTIVITY EXCEEDS	LIMIT AT FITZPATRICK*	163381 P296
#HPCI STEAM SUPPLY VALVES CLOSE ABOVE PRESSURE	LIMIT AT HATCH 1*	153503 P161
#HPCI STEAM SUPPLY VALVES CLOSE ABOVE PRESSURE	LIMIT AT HATCH 1*	153503 P299
#HEAD DEVELOPED BY PLANT SERVICE WATER PUMP EXCEEDS	LIMIT AT HATCH 1*	156100 P310
#CONTAINMENT OXYGEN CONCENTRATION EXCEEDS	LIMIT AT HATCH 1*	156137 P312
#FIRE WATER STORAGE TANK LEVEL DROPS BELOW	LIMIT AT HATCH 1*	157090 P313
#CONTAINMENT OXYGEN CONCENTRATION EXCEEDS	LIMIT AT HATCH 1*	158598 P316
#SERVICE WATER PUMP FLOW RATE FALLS BELOW	LIMIT AT HATCH 1*	158601 P318
#NITROGEN TANK LEVEL FALLS BELOW	LIMIT AT HATCH 1*	161950 P341
#NITROGEN STORAGE TANK CONTENTS FALL BELOW	LIMIT AT HATCH 1*	162741 P344
#LEAKAGE OF FIVE MSIV'S EXCEED	LIMIT AT HATCH 2*	156131 P356
#CONTAINMENT ISOLATION VALVE LEAK RATES EXCEED	LIMIT AT HATCH 2*	157132 P362
#REACTOR COOLANT CHLORIDE CONCENTRATION EXCEEDS	LIMIT AT HATCH 2*	156996 P362
#FEEDWATER ISOLATION VALVE LEAK RATE EXCEEDS	LIMIT AT HATCH 2*	156222 P364
#RADWASTE DISCHARGE ISOLATION VALVE LEAKAGE EXCEEDS	LIMIT AT HATCH 2*	158308 P368
#DRYWELL TEMPERATURE EXCEEDS	LIMIT AT HATCH 2*	158673 P381
#DRYWELL AVERAGE TEMPERATURE EXCEEDED THE 145 F	LIMIT AT HATCH 2*	159626 P384
#TORUS TEMPERATURE EXCEEDS	LIMIT AT HATCH 2*	159308 P392
#TORUS WATER LEVEL BELOW	LIMIT AT HATCH 2*	160802 P403
#FEEDWATER ISOLATION VALVE LEAK RATE EXCEEDS	LIMIT AT HATCH 2*	161854 P403
#RCIC TURBINE EXHAUST ISOLATION VALVE LEAK RATE EXCEEDS	LIMIT AT HATCH 2*	161852 P404
#FEEDWATER ISOLATION VALVE LEAK RATES EXCEED	LIMIT AT HATCH 2*	161845 P404
#DRYWELL PNEUMATIC RETURN ISOLATION VALVE LEAK RATE EXCEEDS	LIMIT AT HATCH 2*	161850 P406
#CONTAINMENT ISOLATION VALVE LEAKAGE EXCEEDS	LIMIT AT HATCH 2*	162703 P407
#MAIN COOLANT ALPHA ACTIVITY EXCEEDS	LIMIT AT LA CROSSE*	160363 P414
#MAIN COOLANT ALPHA ACTIVITY EXCEEDS	LIMIT AT LA CROSSE*	160364 P415
#ELECTRICAL PENETRATION LEAK RATES EXCEED	LIMIT AT LACROSSE*	156223 P412
#CONTAINMENT ISOLATION VALVE LEAK RATE EXCEEDS	LIMIT AT LACROSSE*	162916 P418
#LEAK RATE OF OUTBOARD MSIV EXCEEDS	LIMIT AT MONTICELLO*	156238 P428
#LEAK RATE OF HPCI TURBINE EXHAUST CHECK VALVE EXCEEDS	LIMIT AT MONTICELLO*	156239 P429
#LEAK RATE OF STANDBY LIQUID CONTROL VALVE EXCEEDS	LIMIT AT MONTICELLO*	156240 P429
#LEAK RATE OF TORUS VENT VALVE EXCEEDS	LIMIT AT MONTICELLO*	156241 P430
#LEAK RATE OF RCIC TURBINE EXHAUST CHECK VALVE EXCEEDS	LIMIT AT MONTICELLO*	156242 P431
#LEAK RATE OF TORUS INSTRUMENT AIR VALVE EXCEEDS	LIMIT AT MONTICELLO*	156243 P431
#LEAK RATE OF DRYWELL EQUIPMENT DRAIN SUMP VALVE EXCEEDS	LIMIT AT MONTICELLO*	156244 P432
#LEAK RATE OF INBOUND MSIV EXCEEDS	LIMIT AT MONTICELLO*	156245 P432
#MSIV FAILS TO CLOSE WITHIN TIME	LIMIT AT MONTICELLO*	160209 P436
#AVERAGE PLANAR HEAT GENERATION RATE EXCEEDS	LIMIT AT NINE MILE POINT 1*	154448 P438
#INDICATED TEMPERATURE ACROSS CONDENSER EXCEEDS	LIMIT AT NINE MILE POINT 1*	154447 P439
#AVERAGE PLANAR LINEAR HEAT GENERATION RATE EXCEEDS	LIMIT AT NINE MILE POINT 1*	154445 P439
#LIQUID EFFLUENT DISCHARGE TEMPERATURE EXCEEDS	LIMIT AT NINE MILE POINT 1*	155298 P440
#CIRCULATING WATER DELTA TEMPERATURE EXCEEDS	LIMIT AT NINE MILE POINT 1*	158696 P442
#INSTRUMENT PROCEDURE NOT APPROVED IN TIME	LIMIT AT NINE MILE POINT 1*	160058 P446
#MILK SAMPLE IODINE ACTIVITY EXCEEDS	LIMIT AT NINE MILE POINT*	163383 P447
#PRESSURE DROP ACROSS HEPA FILTER EXCEEDS	LIMIT AT OYSTER CREEK 1*	160520 P466
#STANDBY GAS TREATMENT SYSTEM FLOWRATE FALLS BELOW	LIMIT AT OYSTER CREEK*	155456 P452
#TORUS OXYGEN CONCENTRATION EXCEEDS	LIMIT AT OYSTER CREEK*	159327 P461
#RIVER WATER TEMPERATURE EXCEEDS	LIMIT AT OYSTER CREEK*	159137 P471
#DISCHARGE TEMPERATURE EXCEEDS	LIMIT AT OYSTER CREEK*	159520 P472
#WATER TEMPERATURE IN CREEK EXCEEDS	LIMIT AT OYSTER CREEK*	160357 P472
#RCD WITHDRAWAL BLOCK SETPOINT EXCEEDS	LIMIT AT PEACH BOTTOM 2*	155380 P476
#DIESEL DRIVEN FIRE PUMP FLOWRATE FALLS BELOW	LIMIT AT PEACH BOTTOM 2*	155382 P476
#REACTOR COOLANT CONDUCTIVITY EXCEEDS	LIMIT AT PEACH BOTTOM 3*	155499 P490
#CONDENSER COOLING WATER TEMPERATURE RISE EXCEEDS	LIMIT AT PILGRIM 1*	155398 P502
#LEAKAGE OF TORUS TO RB VACUUM BREAKER EXCEEDS	LIMIT AT PILGRIM 1*	154689 P503
#SR-90 CONCENTRATION IN MILK SAMPLE EXCEEDS REPORT	LIMIT AT PILGRIM 1*	154592 P504
#MUSSEL SAMPLE COBALT 60 CONCENTRATION EXCEEDS REPORTABLE	LIMIT AT PILGRIM 1*	158846 P514
#REACTOR WATER CONDUCTIVITY EXCEEDS	LIMIT AT PILGRIM 1*	159277 P515
#REACTOR WATER LEVEL SWITCHES EXCEED SET POINT	LIMIT AT PILGRIM 1*	160345 P521
#CONDENSER TEMPERATURE GRADIENT EXCEEDS	LIMIT AT PILGRIM 1*	160523 P529
#CIRCULATING WATER DELTA T EXCEEDS	LIMIT AT PILGRIM 1*	161940 P534
#DRYWELL PURGE VALVE CLOSING TIME EXCEEDS	LIMIT AT QUAD CITIES 1*	153948 P538
#MSIV CLOSURE TIME EXCEEDS	LIMIT AT QUAD CITIES 1*	153491 P538
#CONTAINMENT ISOLATION VALVE LEAK RATES EXCEED	LIMIT AT QUAD CITIES 1*	160815 P546
#SLC RELIEF VALVES UNTIL PRESSURE EXCEEDS	LIMIT AT QUAD CITIES 2*	154916 P551
#REACTOR COOLANT CONDUCTIVITY EXCEEDS	LIMIT AT QUAD CITIES 2*	158494 P555
#DISCHARGE BAY CHLORINE CONCENTRATION EXCEEDS	LIMIT AT QUAD CITIES*	159155 P550
#PRESSURE SWITCH SETPOINT EXCEEDS	LIMIT AT VERMONT YANKEE*	159525 P575
#TWO CONTAINMENT ISOLATION VALVES EXCEED LEAK RATE	LIMIT AT VERMONT YANKEE*	161955 P581
#CAD STORAGE TANK LEVEL DROPS BELOW	LIMIT DURING INERTING AT BRUNSWICK 2*	156110 P175
#CMPF EXCEEDS	LIMIT DURING STARTUP AT HATCH 2*	154741 P352

#CONDENSER COOLING WATER TEMPERATURE	LIMIT EXCEEDED AT COOPER*		159493 P249
#CONDENSER COOLING WATER TEMPERATURE	LIMIT EXCEEDED AT COOPER*		159469 P249
#CONDENSER COOLING WATER TEMPERATURE	LIMIT EXCEEDED AT COOPER*		159485 P251
#CONDENSER COOLING WATER TEMPERATURE	LIMIT EXCEEDED AT COOPER*		159460 P251
#CONDENSER COOLING WATER TEMPERATURE	LIMIT EXCEEDED AT COOPER*		159491 P251
#LINEAR HEAT GENERATION RATE	LIMIT EXCEEDED IN THREE NODES AT BROWNS FERRY 1*		159192 P 66
* #TECH SPEC	LIMIT FOR RPS DELAY TIME NON-CONSERVATIVE AT VERMONT YANKEE		155469 P569
#NRC NOT NOTIFIED WITHIN TIME	LIMIT OF REACTOR SCRAM AT PILGRIM 1*		159550 P512
#RCIC STEAM SUPPLY VALVE	LIMIT SWITCH FAILS AT DUANE ARNOLD*		160856 P 19
BROWNS FERRY 2* #MSIV	LIMIT SWITCH FAILS TO DEENERGIZE SCRAM CHANNEL RELAY AT		159162 P 89
#MAIN STEAM ISOLATION VALVE	LIMIT SWITCH FAILS TO RESET AT FITZPATRICK*		163403 P296
#NON-QUALIFIED MSIC	LIMIT SWITCH FOUND AT HATCH 2*		158941 P371
#	LIMIT SWITCHES ON RHR PUMP VALVES ARE BROKEN AT HATCH 2*		157702 P380
#SETPOINT OF MSIV SCRAM	LIMIT SWITCHES OUT OF LIMITS AT DRESDEN 3*		155599 P227
#DRYWELL TO TORUS DIFFERENTIAL PRESSURE FALLS BELOW	LIMIT TOO LONG AT VERMONT YANKEE*		159171 P574
#CARBOX TANK PRESSURE FALLS BELOW	LIMIT WITHOUT ANNUNCIATION AT NINE MILE POINT 1*		158593 P442
#DISCHARGE TEMPERATURE EXCEEDS	LIMIT 4 TIMES AT OYSTER CREEK*		160300 P472
#CORE MAXIMUM FRACTION OF	LIMITING POWER DENSITY IS TOO HIGH AT HATCH 2*		159618 P383
#CORE MAXIMUM FRACTION OF	LIMITING POWER DENSITY NOT CALCULATED AT HATCH 2*		157157 P377
#CORE MAXIMUM FRACTION OF	LIMITING POWER DENSITY TOO HIGH AT HATCH 2*		157172 P378
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	#REACTOR	LOW WATER LEVEL FAILS TO ACTUATE AT BRUNSWICK 1*	160534 P155
	#SET POINT DRIFT IN	LOW WATER LEVEL SCRAM SWITCH AT MILLSTONE 1*	160923 P424
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SENSOR AT OYSTER CREEK*	160148 P405
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SWITCH AT BRUNSWICK 1*	160301 P139
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SWITCH AT BRUNSWICK 1*	160932 P143
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SWITCH AT BRUNSWICK 2*	160938 P178
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SWITCH AT BRUNSWICK 2*	161092 P197
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SWITCH AT QUAD CITIES 1*	160912 P543
CITIES 1*	#REACTOR LOW-	LOW WATER LEVEL SWITCH FAILS TO OPERATE PROPERLY AT QUAD	155369 P540
	#REACTOR	LOW WATER LEVEL SWITCH FAILS TO TRIP AT BRUNSWICK 1*	154880 P131
*	#REACTOR	LOW WATER LEVEL SWITCH SETPOINT INCORRECT AT BROWNS FERRY 2	159685 P 90
*	#REACTOR	LOW WATER LEVEL SWITCH SETPOINT IS INCORRECT AT BRUNSWICK 2	159356 P184
	#REACTOR	LOW WATER LEVEL SWITCH STICKS CLOSED AT BRUNSWICK 2*	158732 P180
	#DRIFT IN REACTOR	LOW WATER LEVEL SWITCHES AT BRUNSWICK 1*	163468 P157
	#SET POINT DRIFT IN REACTOR	LOW WATER LEVEL SWITCHES AT BRUNSWICK 2*	161069 P182
QUAD CITIES 1*	#REACTOR	LOW-LOW WATER LEVEL SWITCH FAILS TO OPERATE PROPERLY AT	155369 P540
	#DIESEL GENERATOR	LOWER CRANKSHAFT MAIN BEARING WIPED AT ARNOLD*	155401 P 6
	#DIESEL GENERATOR	LOWER CRANKSHAFT THRUST BEARING WIPED AT ARNOLD*	155403 P 7
	#	LPCI CHECK VALVE UPSTREAM DRAIN LINE LEAKS AT DRESDEN 2*	158554 P212
	#SET POINT DRIFT IN	LPCI DRYWELL HIGH PRESSURE SWITCH AT DRESDEN 2*	163379 P222
	#	LPCI HEAT EXCHANGER LEAKS AT DRESDEN 3*	163345 P239
DRESDEN 3*	#	LPCI HEAT EXCHANGER SW OUTLET VALVE BREAKER FOUND OPEN AT	153944 P224
	#	LPCI HEAT EXCHANGER TUBES LEAK AT DRESDEN 2*	160087 P218
	#	LPCI HEAT EXCHANGER TUBES LEAK AT DRESDEN 3*	158705 P233
*	#	LPCI INBOARD ISOLATION VALVE LEAKS AT HATCH 1*	160289 P315
	#	LPCI INDEPENDENT POWER SUPPLY INVERTER TRIPS AT FITZPATRICK	163463 P278
	#	LPCI INJECTION VALVE FAILED TO OPEN AT PEACH BOTTOM 3*	160140 P497
	#	LPCI INJECTION VALVE FAILS TO OPEN AT DRESDEN 3*	158037 P228
	#	LPCI INJECTION VALVE FAILS TO OPEN AT PEACH BOTTOM 3*	155416 P490
	#	LPCI INJECTION VALVE FAILS TO OPEN AT PEACH BOTTOM 3*	156969 P493
	#	LPCI INOPERABLE AT BRUNSWICK 1*	160231 P148
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	#	LPCI PIPE RESTRAIN CLEVIS PIN FOUND MISSING AT HATCH 1*	154670 P300
	#SET POINT DRIFT IN	LPCI PRESSURE PERMISSIVE AT PILGRIM 1*	157044 P507
	#	LPCI PUMP FAILS TO START AT HATCH 2*	156994 P370
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	#	LPCI VALVE FAILS TO CLOSE AT DRESDEN 3*	155294 P225
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	#HPCI	MADE INOPERABLE FOR REPAIRS AT FITZPATRICK*	155974 P174
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#SET POINT DRIFT IN MAIN STEAM LINE RADIATION	MONITOR AT BRUNSWICK 2*	154559 P162
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#CONTAINMENT SPRAY ISOLATION VALVE FAILS AT	NINE MILE POINT 1*		158697 P441
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	# NITROGEN STORAGE TANK LEVEL TOO LOW AT HATCH 1*		159503 P324
	# NITROGEN TANK LEVEL FALLS BELOW LIMIT AT HATCH 1*		161930 P341
#CONTAINMENT ISOLATION	NITROGEN VALVE FAILS TO CLOSE AT QUAD-CITIES 2*		159688 P558
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#CONTROL ROD HAS	NO POSITION INDICATION FOR ONE NOTCH AT BRUNSWICK 1*		153731 P128
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#RPS TRIP SETTING	NONCONSERVATIVE AT PILGRIM 1*		160894 P529
#POWER RANGE MONITORS ADJUSTED	NONCONSERVATIVELY AT VERMONT YANKEE*		160064 P576
#4160V BUS TIE BOARD DEENERGIZED AND RESTORED TO	NORMAL FEEDER AT BROWNS FERRY 3*		155391 P194
#CONTROL ROD DRIVE SCRAM DISCHARGE PIPE SUPPORTS ARE	NOT ADEQUATE AT DRESDEN 2*		158692 P214
#RHR MINIMUM FLOW LINES	NOT ADEQUATELY SUPPORTED AT HATCH 2*		155385 P353

#STACK FILTERS	NOT ANALYZED FOR ALPHAS DURING APRIL AT DUANE ARNOLD*	159069 P 27
#EMERGENCY VENT SYSTEM CHARCOAL	NOT ANALYZED WHEN REQUIRED AT NINE MILE PEINT 1*	154446 P439
#INSTRUMENT PROCEDURE	NOT APPROVED IN TIME LIMIT AT NINE MILE PCINT 1*	160058 P446
#CORE MAXIMUM FRACTION OF LIMITING POWER DENSITY	NOT CALCULATED AT HATCH 2*	157157 P377
#ACCELEROMETER RECORDERS	NOT CALIBRATED ON TIME AT HATCH 2*	157119 P360
#RHR PUMP BREAKER FOUND	NOT CHARGED AT COOPER*	154052 P241
#LINEAR HEAT GENERATION RATE	NOT CHECKED AT BROWNS FERRY 2*	159337 P 91
#BATTERY SURVEILLANCE DOCUMENTATION	NOT COMPLETED AT DUANE ARNOLD*	158305 P 9
#HANGER MODIFICATIONS	NOT COMPLETED AT HATCH 1*	158446 P322
#MAIN STEAM LINE LOW PRESSURE TEST	NOT COMPLETED ON TIME AT DRESDEN 2*	154618 P208
#HPCI PUMP DISCHARGE PIPE CLAMP	NOT CONNECTED AT BRUNSWICK 1*	163393 P155
#DEGRADED FIRE BARRIERS FOUND AND FIRE WATCH	NOT ESTABLISHED AT OYSTER CREEK*	158250 P457
#FIRE WATCH	NOT ESTABLISHED IN TIME AT LACROSSE*	161890 P418
#PROPER FIRE WATCH	NOT ESTASBLISHED IN CABLE SPREADING ROOM AT DUANE ARNOLD*	158304 P 8
#RIP VALVE DOES	NOT FULLY CLOSE AT BRUNSWICK 1*	156920 P145
#CONTROL ROD BLOCK	NOT GENERATED BY REFUEL BRIDGE MOVEMENT AT FITZPATRICK*	160210 P283
#CONTAINMENT	NOT INERTED IN TIME AT COOPER*	163552 P259
#FIRE PROTECTION SYSTEMS	NOT INSPECTED AT BROWNS FERRY 1*	160218 P 71
#RB CLOSED COOLING WATER PIPING SEISMIC RESTRAINTS	NOT INSTALLED AT BROWNS FERRY 1*	154610 P 48
#PSW SYSTEM SEISMIC PIPING SUPPORT	NOT INSTALLED AT HATCH 1*	160124 P337
#HIGH RADIATION AREA DOORS	NOT LOCKED AT HATCH 1*	159320 P336
#CONTAINMENT DIFFERENTIAL PRESSURE	NOT MAINTAINED AT DRESDEN 3*	153492 P224
#STANDBY SERVICE WATER - UNIT DOES	NOT MEET OPERATING SPECS AT HATCH 1*	157721 P302
#CAD SYSTEM DOES	NOT MEET SINGLE FAILURE CRITERION AT BRUNSWICK 1 AND 2*	161702 P194
#RHR SERVICE WATER PUMP COOLING DOES	NOT MEET SINGLE FAILURE CRITERION AT HATCH 1*	157123 P314
#PRIMARY CONTAINMENT ISOLATION VALVE TIMING REQUIREMENTS	NOT MET AT COOPER*	159556 P247
#SURVEILLANCE REQUIREMENTS	NOT MET PRIOR TO MOVING FUEL AT PILGRIM 1*	156065 P505
#STACK RELEASES	NOT MONITORED AT OYSTER CREEK 1*	160207 P462
#RPS MOTOR GENERATORS ARE	NOT MONITORED AT PILGRIM 1*	160479 P527
1* #NRC	NOT NOTIFIED WITHIN TIME LIMIT OF REACTOR SCRAM AT PILGRIM	159550 P512
#ENVIRONMENTAL AIR SAMPLE	NOT OBTAINED AT VERMONT YANKEE*	154606 P566
#ELECTRONIC ANNUNCIATION	NOT ON DOOR TO CONTROL BUILDING AT HATCH 1*	156085 P308
#EMERGENCY FLOODING VENT VALVE DOES	NOT OPEN AT LA CROSSE*	157165 P413
#HPCI SYSTEM	NOT OPERABLE WHEN REQUIRED AT HATCH 1*	156091 P309
#EQUIPMENT AND PERSONNEL LOCK LEAKAGE TESTS	NOT PERFORMED AT BIG ROCK POINT*	156984 P 34
#TEST	NOT PERFORMED AT BROWNS FERRY 1*	159449 P 67
#TURBINE VALVE CLOSURE TEST	NOT PERFORMED AT DUANE ARNOLD*	161752 P 22
#SCRAM DISCHARGE VOLUME HIGH LEVEL INSTRUMENT TEST	NOT PERFORMED AT DUANE ARNOLD*	161926 P 25
#RB VENTILATION EXHAUST SURVEILLANCE	NOT PERFORMED AT FITZPATRICK*	160272 P283
#TURBINE CONTROL VALVE FAST CLOSURE TEST	NOT PERFORMED AT HATCH 1*	154605 P301
#SMOKE DETECTOR SURVEILLANCE	NOT PERFORMED AT HATCH 1*	159232 P329
#SCRAM DISCHARGE VOLUME RESIDUAL WATER CHECK	NOT PERFORMED AT HATCH 1*	160135 P337
#PIPE INSPECTION	NOT PERFORMED AT PILGRIM 1*	160530 P529
#RHR PUMP OPERABILITY TEST	NOT PERFORMED CORRECTLY AT HATCH 1*	158599 P317
#REACTOR BUILDING RADIATION MONITOR TEST	NOT PERFORMED ON TIME AT BROWN'S FERRY 1*	154918 P 50
#STANDBY GAS TREATMENT OPERABILITY TEST	NOT PERFORMED ON TIME AT BRUNSWICK 2*	158602 P178
#HEAT DETECTOR SURVEILLANCE	NOT PERFORMED ON TIME AT DRESDEN 2*	160823 P219
#POST LOCA MONITOR CALIBRATION	NOT PERFORMED ON TIME AT HATCH 1*	157089 P314
#HIGH DRYWELL PRESSURE SWITCH SURVEILLANCE	NOT PERFORMED ON TIME AT HATCH 1*	159064 P330
#FIRE PROTECTION SYSTEM VALVE TESTS	NOT PERFORMED ON TIME AT HATCH 2*	154738 P354
#DIESEL GENERATOR AND CORE SPRAY TESTS	NOT PERFORMED ON TIME AT HATCH 2*	156998 P364
#CONTAINMENT AIR LOCK DOORS INTERLOCK TEST	NOT PERFORMED ON TIME AT HATCH 2*	158618 P372
#CONTROL ROD FREE TRAVEL INTERLOCK SURVEILLANCE	NOT PERFORMED ON TIME AT OYSTER CREEK*	158270 P456
#MAIN STEAM LINE RADIATION MONITOR CALIBRATION	NOT PERFORMED ON TIME AT VERMONT YANKEE*	156882 P571
#INTEGRATED GAMMA EXPOSURE TEST	NOT PERFORMED ON TIME AT VERMONT YANKEE*	155190 P567
#CONTAINMENT HYDROGEN RECOMBINER TEST	NOT PERFORMED PROPERLY AT HATCH 2*	156855 P375
#UNIT 1 TRIP TEST	NOT PERFORMED WHEN REQUIRED AT HUMBOLDT BAY*	155490 P410
#SEISMIC INSTRUMENT	NOT PROPERLY CALIBRATED AT HATCH 1*	159378 P346
#RHR	NOT PROPERLY TESTED AT BROWNS FERRY 2*	155457 P 85
#LPRM'S	NOT PROPERLY WIRED AT FITZPATRICK 1*	161782 P294
#ONE CONTROL ROD	NOT PULLED TO TARGET POSITION DURING STARTUP AT HATCH 2*	156132 P357
#DRYWELL EXHAUST SOLENOID COIL	NOT QUALIFIED AT BROWNS FERRY 1*	161892 P 76
#DRYWELL PRESSURE TRANSMITTERS	NOT QUALIFIED AT BROWNS FERRY 1*	161893 P 76
#DRYWELL PRESSURE TRANSMITTER	NOT QUALIFIED AT BROWNS FERRY 2*	161640 P 98
#DRYWELL PRESSURE TRANSMITTER	NOT QUALIFIED AT BROWNS FERRY 3*	161599 P119
#CABLES SPLICES ARE	NOT QUALIFIED AT BRUNSWICK 2*	159158 P180
#HPCI TURBINE DOES	NOT REACH RATED SPEED AT DUANE ARNOLD*	160275 P 10
BROWNS FERRY 3* #FEEDER BREAKER	NOT RECLOSED FOLLOWING RESTORATION OF OFFSITE POWER AT	156072 P105
#FIRE DOOR CLOSERS	NOT REPAIRED IN TIME AT BRUNSWICK 1 AND 2*	162412 P160
#INOPERABLE PIPING SUPPORT	NOT REPAIRED PRIOR TO STARTUP AT BRUNSWICK 2*	153733 P169
#REACTOR BUILDING VENT SAMPLE SYSTEM	NOT RETURNED TO SERVICE AT DUANE ARNOLD*	155516 P 26
#FIRE DOORS	NOT RETURNED TO SERVICE IN TIME AT BRUNSWICK 2*	162429 P205
#INSTRUMENT MAINTENANCE PROCEDURE CHANGES	NOT REVIEWED AT FITZPATRICK*	160259 P283
#STANDBY GAS TREATMENT PROCEDURE CHANGES	NOT REVIEWED AT HATCH 1*	160277 P321
#REACTOR PRESSURE CALIBRATION PROCEDURE	NOT REVIEWED AT HATCH 2*	161874 P401
#CONTAINMENT ATMOSPHERIC CONTROL CABLE	NOT ROUTED PROPERLY AT HATCH 1*	158791 P320
#HOLDUP TANK	NOT SAMPLED AT DRESDEN 1*	159515 P206
#FUEL BUNDLE DISCOVERED ROTATED AND	NOT SEATED AT OYSTER CREEK*	154934 P450
#REACTOR VESSEL HEAD INSULATION	NOT SEISMIC QUALIFIED AT HATCH 2*	162146 P408
#RADWASTE PIPING IS	NOT SEISMICALLY PROTECTED AT HATCH 2*	160522 P399
#SAFETY/RELIEF VALVE AIR LINE SUPPORT	NOT SEISMICALLY QUALIFIED AT MONTICELLO*	156018 P433
#DRYWELL TO TEMPERATURE RECORDER WAS	NOT STAMPING AT HATCH 1*	159669 P327
#TEN PIPES	NOT SUPPORTED PROPERLY AT QUAD-CITIES 1*	160904 P548
#DIESEL FIRE PUMP DAY TANK SAMPLES	NOT TAKEN AT DRESDEN 2*	160361 P218
#REACTOR WATER SAMPLES	NOT TAKEN AT HATCH 1*	159065 P331
#ENVIRONMENTAL AIR SAMPLES	NOT TAKEN AT VERMONT YANKEE*	158703 P573
#TURBINE BUILDING VENTILATION SAMPLES	NOT TAKEN ON TIME AT BROWNS FERRY 1*	159404 P 70

#REACTOR COOLANT SAMPLES NOT TAKEN WHEN REQUIRED AT FITZPATRICK* 154661 P286
#SBGT SYSTEM NOT TESTED AS REQUIRED AT HATCH 1* 154663 P302
#RECIRCULATION PUMP TRIP SYSTEM NOT TESTED AT DUANE ARNOLD* 160202 P 15
#APRM INSTRUMENTS NOT TESTED AT FITZPATRICK* 159222 P285
#SAFETY RELIEF VALVE TAILPIPE PRESSURE SWITCH NOT TESTED AT HATCH 1* 161059 P339
#CABLE VAULT SMOKE DETECTION SYSTEM NOT TESTED AT MILLSTONE 1* 157402 P421
#FIRE PUMPS NOT TESTED AT VERMONT YANKEE* 161637 P578
#REACTOR PROTECTION SYSTEM RELAYS NOT TESTED IN TIME AT HATCH 2* 159092 P387
#FIRE PROTECTION ZONE NOT TESTED ON TIME AT BROWNS FERRY 3* 158596 P109
#DRYWELL-TORUS VACUUM BREAKERS NOT TESTED ON TIME AT HATCH 2* 160845 P400
#SHUTDOWN CONDENSER CONTROL VALVES NOT TESTED ON TIME AT LACROSSE* 161891 P416
#OFF-GAS HYDROGEN ANALYZERS NOT TESTED ON TIME AT MILLSTONE 1* 161679 P426
#CONTROL ROD DRIVE SCRAM PILOT SOLENOIDS NOT TESTED ON TIME AT NINE MILE POINT* 163384 P447
#ROD BLOCK MONITOR NOT TESTED WHEN REQUIRED AT HATCH 1* 155411 P310
#ELECTRICAL POWER SOURCES NOT TESTED WHEN REQUIRED AT HATCH 2* 154942 P350
#MAIN STATION AND DG BATTERIES NOT TESTED WHEN REQUIRED AT OYSTER CREEK* 155443 P454
#MONORAIL HOIST OPERABILITY NOT VERIFIED BEFORE USE AT HATCH 2* 162423 P406
#CONTROL ROD HAS NO POSITION INDICATION FOR ONE NOTCH AT BRUNSWICK 1* 153731 P128
#CONTROL ROD HAS NO POSITION INDICATION FOR ONE NOTCH AT BRUNSWICK 2* 153735 P165
#EXCESSIVE EROSION NOTED IN SW PUMPS STRAINER BACKWASH LINE AT COOPER* 153972 P240
#NRC NOT NOTIFIED WITHIN TIME LIMIT OF REACTOR SCRAM AT PILGRIM 1* 159550 P512
#STEAM SUPPLY PIPING NOZZLE WELD CRACKS AT MILLSTONE 1* 161471 P425
PILGRIM 1* # NRC NOT NOTIFIED WITHIN TIME LIMIT OF REACTOR SCRAM AT 159550 P512
#THREE UNAUTHORIZED TRANSFERS OF SPECIAL NUCLEAR MATERIAL AT HATCH 1* 161949 P343
#A NUMBER OF SRM PROBLEMS WERE ENCOUNTERED AT FITZPATRICK* 154505 P263
#REACTOR COOLANT LEAK DETECTION PARTICULATE CHAMBER COVER NUTS STRIPPED AT PILGRIM 1* 159463 P522

#ENVIRONMENTAL AIR SAMPLE NOT OBTAINED AT VERMONT YANKEE* 154606 P566
#MULTIPLE HPCI AND RCIC FAILURES OCCUR AT PILGRIM 1* 159600 P510
AT BIG ROCK POINT* #DAMAGE MIGHT OCCUR TO CONTAINMENT ISOLATION VENTILATION VALVE OPERATORS 156954 P 36
#FISH KILL OCCURS AFTER A SHUT DOWN AT OYSTER CREEK* 154622 P348
#FISH KILL OCCURS AFTER A SHUT DOWN AT OYSTER CREEK* 154622 P470
#MOMENTARY BREACH OF SECONDARY CONTAINMENT OCCURS AT BROWNS FERRY 2* 155206 P 81
#SPURIOUS HPCI INITIATION OCCURS AT HATCH 1* 156997 P318
#GROUND FAULT OCCURS IN CONTAINMENT ISOLATION VALVE AT FITZPATRICK* 155587 P270
#WATER HAMMER OCCURS IN ISOLATION CONDENSER PIPING AT MILLSTONE 1* 153942 P419
FERRY 2* #SET POINT DRIFT OCCURS ON TURBINE PRESSURE PERMISSIVE SWITCHES AT BROWNS 157743 P 88
#DRYWELL SUMP FLOW TRANSMITTER FAILS OFF AT BROWNS FERRY 3* 159492 P109
#HYDRAULIC ACCUMULATOR AIR VALVE BROKEN OFF AT BROWNS FERRY 3* 161621 P119
#RHR PUMP TRIPS OFF AT BRUNSWICK 1* 155507 P132
#VACUUM BREAKER VALVE INDICATING LIGHT FAILS OFF AT HATCH 1* 160267 P322
#CONTAINMENT FILTER ACCESS COVER COMES OFF AT NINE MILE POINT 1* 160871 P446
#FIRE ALARM INADVERTENTLY GOES OFF AT PILGRIM 1* 154899 P505
#FIRE CHECK AT AUXILIARY OFF GAS BUILDING MISSED AT PILGRIM 1* 159523 P516
OFF GAS MONITORING SYSTEM INOPERABLE AT NINE MILE POINT 1* 159513 P444
OFF GAS RADIATION MONITOR FAILS AT FITZPATRICK* 160104 P290
OFF GAS RADIATION MONITOR INOPERABLE AT PEACH BOTTOM 2* 161909 P486
#FLOW REDUCED TO CONDENSER OFF GAS RADIATION MONITORS AT PILGRIM 1* 159610 P513
OFF GAS RADIATION MONITORS DEGRADED AT MONTICELLO* 159204 P435
OFF GAS RADIATION MONITORS INOPERABLE AT MILLSTONE 1* 160233 P422
#BOTH OFF GAS RADIATION MONITORS INOPERABLE AT VERMONT YANKEE* 154600 P568
#BOTH MAIN CHIMNEY OFF GAS SAMPLE PUMPS FAIL AT QUAD CITIES 1* 159331 P545
ANALYSIS AT ARNOLD* # OFF GAS STACK FILTER AND CARTRIDGE LOST ON SITE BEFORE 154675 P 26
#OPENING AND CLOSING TIME OF RHR VALVE OFF LIMITS AT HATCH 1* 154668 P301
#COMPUTER HAS A SOFTWARE STALL AND GOES OFF LINE AT BRUNSWICK 2* 154677 P167
OFF SITE ELECTRIC POWER LINE TRIPS AT BROWNS FERRY 1* 159267 P 68
1* # OFF-GAS HYDROGEN ANALYZERS NOT TESTED ON TIME AT MILLSTONE 161679 P426
#REACTOR STARTED WITH OFF-GAS MONITORS INOPERABLE AT PEACH BOTTOM 3* 156736 P476
OFF-GAS TIMER MISADJUSTED AT FITZPATRICK* 158628 P280
#ONE OFF-SITE POWER SOURCE TRIPS AT PEACH BOTTOM 2* 156027 P477
OFFGAS HYDROGEN SAMPLE PUMP TRIPS AT HATCH 1* 159442 P334
#SUPPRESSION POOL LEVEL INDICATOR READS HIGH OFFSCALE AT BRUNSWICK 2* 155534 P168
#FEEDER BREAKER NOT RECLOSED FOLLOWING RESTORATION OF OFFSITE POWER AT BROWNS FERRY 3* 156072 P105
#ONE OFFSITE POWER LINE FAILS AT BROWNS FERRY 1* 156026 P 54
#ONE OFFSITE POWER LINE FAILS AT BROWNS FERRY 2* 156031 P 84
OFFSITE POWER LOST AT BROWNS FERRY 1* 155528 P 54
#RECIRCULATION PUMP LOW OIL LEVEL ALARM SHORTS AT BROWNS FERRY 1* 156860 P 59
#DIESEL GENERATOR LUBE OIL LEVEL DROPS BELOW LIMIT AT ARNOLD* 153730 P 2
#LOW DIESEL GENERATOR GOVERNOR HYDRAULIC OIL LEVEL FOUND AT DUANE ARNOLD* 158616 P 9
#RECIRCULATION PUMP MOTOR OIL LEVEL IS LOW AT DUANE ARNOLD* 160265 P 11
#DIESEL GENERATOR TRIPS ON LOW LUBE OIL PRESSURE AT FITZPATRICK* 154949 P268
#RECIRCULATION PUMP TRIPS ON LOW OIL PRESSURE AT HATCH 2* 159315 P394
#HPCI EMERGENCY OIL PUMP DISCHARGE LINE LEAKS AT QUAD CITIES 2* 158714 P556
#HPCI AUXILIARY OIL PUMP FAILS TO RUN AT BRUNSWICK 1* 155552 P131
#HPCI AUXILIARY OIL PUMP FEEDER BREAKER TRIPS AT HATCH 2* 154737 P354
#HPCI AUXILIARY OIL PUMP FOUND INOPERABLE AT COOPER* 153455 P242
#DIESEL GENERATOR LUBE OIL PUMP INOPERABLE AT BROWNS FERRY 3* 161683 P117
#DG AUXILIARY OIL PUMP MOTOR TRIPS OUT AT BROWNS FERRY 3* 159490 P110
#DIESEL OIL SPILLED INTO CANAL AT PEACH BOTTOM 2* 159483 P480
#WASTE OIL STORAGE AREA SOIL CONTAMINATED AT HATCH 1 AND 2* 162080 P345
POINT DRIFT AND FAILURE OF AVERAGE POWER RANGE MONITORS IN ONE CHANNEL AT FITZPATRICK* #SET 158627 P280
STARTUP AT HATCH 2* # ONE CONTROL ROD NOT PULLED TO TARGET POSITION DURING 156132 P357
#REACTOR STARTED UP WITH ONE CORE SPRAY LOOP INOPERABLE AT MONTICELLO* 158553 P434
OYSTER CREEK* #ONLY ONE DILUTION PUMP OPERATING FOR TWENTY MINUTE PERIODS AT 153497 P468

60 F AT OYSTER CREEK*	#ONLY	ONE DILUTION PUMP OPERATING WHEN INTAKE TEMPERATURE BELOW	159070 P471
CONTAINMENT AT BROWNS FERRY 1*	#ONLY	ONE ISOLATION VALVE FOUND ON TWO SPARE LINES PENETRATING	159209 P 52
#CONTROL ROD HAS NO POSITION INDICATION FOR		ONE NOTCH AT BRUNSWICK 1*	153731 P126
#CONTROL ROD HAS NO POSITION INDICATION FOR		ONE NOTCH AT BRUNSWICK 2*	153735 P165
		# ONE OF THREE RCS SAMPLING POINTS ISOLATED AT DUANE ARNOLD*	160827 P 29
		# ONE OFF-SITE POWER SOURCE TRIPS AT PEACH BOTTOM 2*	156027 P477
		# ONE OFFSITE POWER LINE FAILS AT BROWNS FERRY 1*	156026 P 54
		# ONE OFFSITE POWER LINE FAILS AT BROWNS FERRY 2*	156031 P 84
BROWNS FERRY 1*		# ONE PIPING RESTRAINT FAILS TO MEET SEISMIC CRITERIA AT	155367 P 53
DOWNSCALE AT BRUNSWICK 1*		# ONE POINT OF CHLORIDE INTRUSION MONITOR INDICATES	153732 P127
AT OYSTER CREEK*		# ONLY ONE DILUTION PUMP OPERATING FOR TWENTY MINUTE PERIODS	153497 P448
BELOW 60 F AT OYSTER CREEK*		# ONLY ONE DILUTION PUMP OPERATING WHEN INTAKE TEMPERATURE	159070 P471
PENETRATING CONTAINMENT AT BROWNS FERRY 1*		# ONLY ONE ISOLATION VALVE FOUND ON TWO SPARE LINES	155209 P 52
BROWNS FERRY 2*		# ONLY OPERABLE RECIRCULATION PUMP ACCIDENTALLY TRIPPED AT	155594 P 82
#HPCI STOP VALVE FAILS TO STAY		OPEN AT BROWNS FERRY 1*	153440 P 46
#CONTAINMENT AIR LOCK DOORS FOUND		OPEN AT BRUNSWICK 1*	159143 P146
#STANDBY LIQUID CONTROL RELIEF VALVE FAILS TO		OPEN AT BRUNSWICK 1*	159199 P147
#RCIC INJECTION VALVE FAILS TO		OPEN AT BRUNSWICK 2*	160525 P187
#HPCI TURBINE CONTROL VALVE FAILS TO		OPEN AT BRUNSWICK 2*	160082 P198
#CAC MONITOR ISOLATION VALVES FAIL TO		OPEN AT BRUNSWICK 2*	161595 P194
#TWO CAD SYSTEM VALVES FAIL TO		OPEN AT BRUNSWICK 2*	161653 P196
#SERVICE WATER VALVE FAILS TO		OPEN AT COOPER,*	162522 P258
#FIRE HYDRANT FAILS TO		OPEN AT COOPER*	159246 P253
#WASTE LINE VALVE OPERATOR FAILS TO		OPEN AT COOPER*	162519 P259
#TORUS VENT VALVES FAIL TO		OPEN AT DRESDEN 2*	158704 P215
#HPCI STEAM SUPPLY VALVE FAILS TO		OPEN AT DRESDEN 2*	160596 P220
#LPCI HEAT EXCHANGER SW OUTLET VALVE BREAKER FOUND		OPEN AT DRESDEN 3*	153944 P224
#LPCI INJECTION VALVE FAILS TO		OPEN AT DRESDEN 3*	156037 P228
#AUTOMATIC DEPRESSURIZATION VALVES FAIL TO		OPEN AT DRESDEN 3*	158231 P232
#HPCI STEAM SUPPLY VALVE FAILS TO		OPEN AT DRESDEN 3*	160943 P238
#RCIC TURBINE TRIP AND THROTTLE VALVE FAILS TO		OPEN AT DUANE ARNOLD*	159059 P 15
#RCIC STEAM SUPPLY LINE VALVE FAILS TO		OPEN AT DUANE ARNOLD*	160501 P 21
#STEAM PRESSURE RELIEF VALVE STICKS		OPEN AT DUANE ARNOLD*	161716 P 22
#NEUTRALIZER TANK DISCHARGE VALVE FAILS		OPEN AT FITZPATRICK*	154869 P265
#CORE SPRAY VALVE FAILS TO		OPEN AT FITZPATRICK*	153729 P266
#CORE SPRAY VALVE FAILS TO		OPEN AT FITZPATRICK*	154660 P267
#RHR SYSTEM DISCHARGE TO RADWASTE VALVE FAILS TO		OPEN AT FITZPATRICK*	158330 P276
#CONTAINMENT SPRAY VALVE FAILS TO		OPEN AT HATCH 1*	156991 P315
#SHUTDOWN COOLING SUCTION VALVE FAILS TO		OPEN AT HATCH 1*	158734 P321
#RHR INBOARD INJECTION VALVE FAILS TO		OPEN AT HATCH 1*	159314 P335
#CONTAINMENT VACUUM BREAKER LIGHTS FAIL TO INDICATE		OPEN AT HATCH 1,*	154619 P347
#TORUS-DRYWELL VACUUM BREAKER FAILS TO		OPEN AT HATCH 2*	156143 P355
#MSIV LEAKAGE CONTROL SYSTEM RELAY FAILS TO		OPEN AT HATCH 2*	156863 P378
#TORUS VACUUM BREAKER VALVE FAILED		OPEN AT HATCH 2*	159629 P385
#HPCI INJECTION VALVE FAILS TO		OPEN AT HATCH 2*	159157 P387
#HPCI SUPPRESSION CHAMBER SUCTION VALVE FAILS TO		OPEN AT HATCH 2*	159317 P390
#TORUS/DRYWELL VACUUM BREAKER FAILS TO		OPEN AT HATCH 2*	160476 P396
#EMERGENCY FLOODING VENT VALVE DOES NOT		OPEN AT LA CROSSE*	157165 P413
#ELECTROMAGNETIC RELIEF VALVE FAILS TO		OPEN AT NINE MILE POINT 1*	159511 P443
#ELECTROMATIC RELIEF VALVE FAILS TO		OPEN AT OYSTER CREEK*	154450 P449
#GENERATOR LOAD REJECTION SENSOR PRESSURE SWITCH FAILS		OPEN AT OYSTER CREEK*	157162 P456
#TWO DRYWELL VENT VALVES FOUND TO BE FULLY		OPEN AT PEACH BOTTOM 2*	154740 P474
GAS TREATMENT FILTER ASSEMBLY ISOLATION VALVE FAILS TO		OPEN AT PEACH BOTTOM 2* #STANDBY	161071 P483
#LPCI INJECTION VALVE FAILS TO		OPEN AT PEACH BOTTOM 3*	155416 P490
#LPCI INJECTION VALVE FAILS TO		OPEN AT PEACH BOTTOM 3*	156969 P493
#LPCI INJECTION VALVE FAILED TO		OPEN AT PEACH BOTTOM 3*	160140 P497
#CORE SPRAY PUMP FAILS TO START AND VALVE FAILS TO		OPEN AT PILGRIM 1*	158613 P508
#RELIEF VALVE FAILS TO		OPEN AT PILGRIM 1*	159409 P520
#CORE SPRAY VALVE FAILS TO		OPEN AT PILGRIM 1*	160251 P525
#TARGET ROCK SAFETY RELIEF VALVE FAILS TO		OPEN AT QUAD CITIES 1*	158751 P544
#CORE SPRAY VALVE FAILS TO		OPEN AT QUAD CITIES 2*	160067 P559
#RCIC PUMP DISCHARGE VALVE FAILS TO		OPEN AT QUAD CITIES 2*	161455 P560
#PRESSURE RELIEF VALVE FAILS TO		OPEN AT QUAD-CITIES 2*	156942 P555
#RHR CONTAINMENT COOLING VALVE FAILS TO		OPEN AT QUAD-CITIES 2*	160878 P560
#ELECTROMAGNETIC RELIEF VALVE FAILS TO		OPEN AT QUAD-CITIES 2	163334 P564
#MSIV FAILS TO FULLY		OPEN AT VERMONT YANKEE*	153481 P565
#HPCI STOP VALVE FAILS TO		OPEN COMPLETELY AT BROWNS FERRY 1*	156211 P 56
#LPCI VALVE FAILS TO		OPEN FULLY AT QUAD-CITIES 2*	160567 P559
#RECIRCULATION PUMP DISCHARGE VALVE FAILS TO		OPEN FULLY AT VERMONT YANKEE*	157039 P571
#TORUS TO DRYWELL VACUUM BREAKER FAILS TO GIVE		OPEN INDICATION AT DRESDEN 2*	160266 P214
#UPDATE ON TWO		OPEN VENTILATION ISOLATION VALVES AT PEACH BOTTOM 3*	162595 P475
#RCIC VALVE TRANSFERS		OPEN WHEN BREAKER CONTACT SHORTS AT COOPER*	157036 P244
#CONTAINMENT AIR LOCK DOORS		OPENED SIMULTANEOUSLY AT PILGRIM 1*	160258 P526
	#	OPENING AND CLOSING TIME OF RHR VALVE OFF LIMITS AT HATCH 1	154658 P301
#SCAFFOLDING PREVENTS VACUUM BREAKER FROM		OPENING FULLY AT OYSTER CREEK*	153939 P449
#RCIC CONTROL POWER FUSE		OPENS AT MONTICELLO*	158818 P434
#VACUUM BREAKER VALVE		OPENS AT QUAD CITIES 1*	159231 P545
#RECIRC PUMP MOTOR GENERATOR BREAKER		OPENS AT QUAD-CITIES 2*	159639 P558
#REACTOR VESSEL RELEIF VALVE		OPENS SPURIOUSLY AT PILGRIM 1*	160497 P528
#REACTOR VESSEL RELIEF VALVE		OPENS SPURIOUSLY AT PILGRIM 1*	160559 P532
#ADS TIMER RELAY		OPENS TOO SOON AT BROWNS FERRY 2*	154917 P 51
#MONORAIL HOIST		OPERABILITY NOT VERIFIED BEFORE USE AT HATCH 2*	162423 P406
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#RHR PUMP		OPERABILITY TEST NOT PERFORMED CORRECTLY AT HATCH 1*	158599 P317
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#MAIN STEAM ISOLATION SWITCHES FAIL TO	OPERATE AT BROWNS FERRY 3*		158235 P106
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FERRY 3* #SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCH AT BROWNS 161596 P118
* #SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCH AT DUANE ARNOLD 161990 P 23
CREEK* #SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCHES AT OYSTER 160817 P485
#SET POINT DRIFT IN DRYWELL PRESSURE SWITCH AT PEACH BOTTOM 3* 157746 P494
SWITCH AT QUAD-CITIES 1* #SET POINT DRIFT IN ELECTROHYDRAULIC CONTROL FLUID PRESSURE 160962 P542
#SET POINT DRIFT IN FISSION PRODUCT MONITOR AT HATCH 1* 162743 P343

CREEK*	#SET	POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCH AT FITZPATRICK*	160435 P291
CREEK*	#SET	POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT HATCH 2*	154940 P381
BRUNSWICK 1*	#SET	POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT HATCH 2*	158774 P387
BRUNSWICK 1*	#SET	POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT OYSTER	160326 P462
*	#SET	POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT OYSTER	161644 P466
*	#SET	POINT DRIFT IN HIGH STEAMLINE RADIATION CHANNEL AT	157057 P134
	#SET	POINT DRIFT IN HIGH STEAMLINE RADIATION CHANNEL AT	160317 P142
	#SET	POINT DRIFT IN HPCI DIFFERENTIAL PRESSURE SWITCH AT HATCH 1	154492 P161
	#SET	POINT DRIFT IN HPCI DIFFERENTIAL PRESSURE SWITCH AT HATCH 1	154492 P298
	#SET	POINT DRIFT IN HPCI EXHAUST PRESSURE SWITCHES AT HATCH 2*	156130 P356
	#SET	POINT DRIFT IN HPCI STEAM FLOW SENSOR AT PEACH BOTTOM 2*	160485 P481
	#SET	POINT DRIFT IN HPCI STEAM LINE DP SWITCH AT HATCH 2*	161872 P400
BROWNS FERRY 1*	#SET	POINT DRIFT IN HPCI STEAM LINE FLOW SWITCH AT HATCH 2*	158771 P386
2*	#SET	POINT DRIFT IN HPCI STEAM LINE SPACE TEMPERATURE SWITCH AT	161796 P 77
3*	#SET	POINT DRIFT IN HPCI STEAMLINE HIGH D/P SWITCH AT BRUNSWICK	156107 P173
AT QUAD CITIES 1*	#SET	POINT DRIFT IN HPCI STEAMLINE LOW PRESSURE SWITCH AT HATCH	161614 P342
	#SET	POINT DRIFT IN HPCI TURBINE AREA HIGH TEMPERATURE SWITCH	160451 P547
AT DRESDEN 2*	#SET	POINT DRIFT IN HPCI TURBINE PRESSURE SWITCH AT HATCH 2*	160519 P399
*	#SET	POINT DRIFT IN ISOLATION CONDENSER CONDENSATE FLOW SWITCH	159243 P215
	#SET	POINT DRIFT IN ISOLATION CONDENSER FLOW SWITCH AT DRESDEN 2	160008 P211
	#SET	POINT DRIFT IN LOW REACTOR LEVEL SWITCHES AT COOPER*	161622 P256
DRESDEN 2*	#SET	POINT DRIFT IN LOW WATER LEVEL SCRAM SWITCH AT MILLSTONE 1*	160923 P424
	#SET	POINT DRIFT IN LPCI DRYWELL HIGH PRESSURE SWITCH AT	163379 P222
AT MONTICELLO*	#SET	POINT DRIFT IN LPCI PRESSURE PERMISSIVE AT PILGRIM 1*	157044 P507
MILLSTONE 1*	#SET	POINT DRIFT IN MAIN STEAM LINE AREA TEMPERATURE SWITCHES	156044 P430
BRUNSWICK 1*	#SET	POINT DRIFT IN MAIN STEAM LINE HIGH FLOW SWITCH AT	155195 P420
MONT.CELLO*	#SET	POINT DRIFT IN MAIN STEAM LINE HIGH RADIATION MONITOR AT	154882 P130
PEACH BOTTOM 2*	#SET	POINT DRIFT IN MAIN STEAM LINE LOW PRESSURE SWITCH AT	155191 P428
CITIES 2*	#SET	POINT DRIFT IN MAIN STEAM LINE LOW PRESSURE SWITCH AT	162487 P486
*	#SET	POINT DRIFT IN MAIN STEAM LINE LOW PRESSURE SWITCH AT QUAD	159342 P557
BRUNSWICK 2*	#SET	POINT DRIFT IN MAIN STEAM LINE PRESSURE SWITCH AT DRESDEN 2	163515 P222
HATCH 2*	#SET	POINT DRIFT IN MAIN STEAM LINE RADIATION MONITOR AT	154559 P162
	#SET	POINT DRIFT IN MAIN STEAM LINE TEMPERATURE SWITCHES AT	157129 P361
FERRY 2*	#SET	POINT DRIFT IN MAIN STEAM RELIEF VALVES AT VERMONT YANKEE*	160930 P578
	#SET	POINT DRIFT IN MAINSTEAM LINE HIGH FLOW SWITCH AT BROWNS	153489 P 79
	#SET	POINT DRIFT IN MSIV LOW PRESSURE SENSORS AT OYSTER CREEK*	159675 P464
	#SET	POINT DRIFT IN PRESSURE RELIEF VALVE AT VERMONT YANKEE*	160535 P579
	#SET	POINT DRIFT IN PRESSURE SWITCH BROWNS FERRY 3*	161620 P419
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	#SET	POINT DRIFT IN RCIC EXHAUST PRESSURE SWITCH AT BRUNSWICK 2*	155317 P173
	#SET	POINT DRIFT IN RCIC FLOW INSTRUMENT AT HATCH 1*	154493 P298
	#SET	POINT DRIFT IN RCIC FLOW SWITCH AT HATCH 1*	156096 P309
BROWNS FERRY 3*	#SET	POINT DRIFT IN RCIC HIGH STEAM FLOW SWITCH AT FITZPATRICK*	154653 P261
FITZPATRICK*	#SET	POINT DRIFT IN RCIC REACTOR WATER LEVEL TRIP SWITCH AT	159284 P113
HATCH 1*	#SET	POINT DRIFT IN RCIC STEAM LINE LOW PRESSURE SWITCH AT	163404 P256
BOTTOM 3*	#SET	POINT DRIFT IN RCIC TURBINE EXHAUST PRESSURE SWITCHES AT	155326 P307
AT PEACH BOTTOM 2*	#SET	POINT DRIFT IN RCIC TURBINE STEAM FLOW SWITCH AT PEACH	155347 P491
OYSTER CREEK*	#SET	POINT DRIFT IN RCIC TURBINE STEAM LINE HIGH FLOW SWITCHES	161779 P484
PERRY 2*	#SET	POINT DRIFT IN REACTOR HIGH PRESSURE SCRAM SENSORS AT	159326 P460
	#SET	POINT DRIFT IN REACTOR HIGH PRESSURE SWITCHES AT BROWNS	159336 P 90
	#SET	POINT DRIFT IN REACTOR HIGH PRESSURE SWITCHES AT HATCH 2*	160370 P396
	#SET	POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG ROCK POINT*	157074 P 34
	#SET	POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG ROCK POINT*	157076 P 35
	#SET	POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG ROCK POINT*	160170 P 41
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1*	#SET	POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT BROWNS FERRY	157716 P109
	#SET	POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT BRUNSWICK 2*	159604 P178
	#SET	POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT BRUNSWICK 2*	161985 P193
YANKEE*	#SET	POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT VERMONT	154714 P566
*	#SET	POINT DRIFT IN REACTOR LOW PRESSURE SWITCHES AT BRUNSWICK 2	158755 P171
CREEK*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SENSOR AT OYSTER	160148 P465
1*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT BRUNSWICK	160301 P139
1*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT BRUNSWICK	156932 P143
2*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT BRUNSWICK	156938 P178
2*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT BRUNSWICK	161652 P197
CITIES 1*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT QUAD	156912 P543
BRUNSWICK 2*	#SET	POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCHES AT	161069 P182
	#SET	POINT DRIFT IN REACTOR PRESSURE SWITCH AT BROWNS FERRY 2*	156948 P 87
	#SET	POINT DRIFT IN REACTOR PRESSURE SWITCH AT HATCH 2*	160450 P397
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PEACH BOTTOM 2*	#SET	POINT DRIFT IN REACTOR VESSEL SHROUD LEVEL SWITCH AT	154554 P124
	#SET	POINT DRIFT IN REACTOR WATER CLEANUP TEMPERATURE SWITCH AT	160431 P498
*	#SET	POINT DRIFT IN REACTOR WATER LEVEL SENSORS AT OYSTER CREEK*	159672 P463
	#SET	POINT DRIFT IN REACTOR WATER LEVEL SWITCH AT BROWNS FERRY 2	155205 P 81
	#SET	POINT DRIFT IN REACTOR WATER LEVEL SWITCH AT DRESDEN 2*	155197 P208
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*	#SET	POINT DRIFT IN REACTOR WATER LEVEL TRANSMITTER AT BIG ROCK	158777 P 37
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ARNOLD*	#SET	POINT DRIFT IN REFUELING FLOOR RADIATION MONITOR AT	160524 P221
	#SET	POINT DRIFT IN RHR LOW PRESSURE PERMISSIVE SWITCH AT DUANE	155510 P 4
	#SET	POINT DRIFT IN RHR PUMP START RELAY AT HATCH 2*	156133 P358
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	#SET	POINT DRIFT IN SHUTDOWN COOLING SYSTEM PRESSURE SWITCHES	158602 P379
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	#SET	POINT DRIFT IN STANDBY LIQUID CONTROL RELIEF VALVE AT	154889 P303

DRESDEN 3*
*
MILE POINT 1*
HATCH 2*
SWITCHES AT BROWNS FERRY 1*
POINT 1*
AT BROWNS FERRY 2*
CITIES 1*
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#SET POINT DRIFT IN STEAM LINE PRESSURE SWITCH AT PEACH BOTTOM 2 161961 P485
#SET POINT DRIFT IN THREE RELIEF VALVES AT ARNOLD* 156225 P 6
#SET POINT DRIFT IN TURBINE ANTICIPATORY TRIP SWITCH AT NINE 159235 P442
#SET POINT DRIFT IN TURBINE CONTROL VALVE PRESSURE SWITCH AT 156118 P359
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#SET POINT DRIFT IN TURBINE REACTOR SCRAM SWITCHES AT NINE MILE 160313 P444
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#SECTION OF A SPLIT RING FOUND IN THE MAIN HPCI PUMP IMPELLER AT ARNOLD* 156226 P 6
#DIESEL GENERATOR LUBE OIL PUMP INOPERABLE AT BROWNS FERRY 3* 161683 P117
#RHR PUMP INOPERABLE AT COOPER 1* 160051 P255
#RHR SERVICE WATER BOOSTER PUMP INOPERABLE AT COOPER* 159521 P248
#RHR SW PUMP INOPERABLE AT HATCH 1* 160145 P339
#HPCI PUMP INOPERABLE AT HATCH 2* 159592 P386
#STANDBY SERVICE WATER PUMP INOPERABLE AT HATCH 2* 161770 P402
#DIESEL FIRE PUMP INOPERABLE AT HUMBOLDT BAY* 162008 P412
#DIESEL FIRE PUMP INOPERABLE AT PILGRIM 1* 160083 P527
#RHR PUMP INOPERABLE AT QUAD CITIES 2* 162032 P564
#DEMINERALIZER SYSTEM PUMP LEAKS AT BIG ROCK POINT* 163500 P 42
#CAPACITY OF FIRE PUMP LESS THAN THAT REQUIRED BY NEW DEMAND AT DRESDEN 1* 155476 P295
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#HPCI	PUMP MINIMUM FLOW VALVE FAILS TO CLOSE AT FITZPATRICK*	158301 P276
#RHR	PUMP MOTOR FAILS AT PEACH BOTTOM 2*	158287 P487
#RECIRC	PUMP MOTOR GENERATOR BREAKER OPENS AT QUAD-CITIES 2*	159689 P558
#RECIRCULATION	PUMP MOTOR GENERATOR SET TRIPS AT QUAD CITIES 1*	158629 P543
#RECIRCULATION	PUMP MOTOR GENERATOR TRIPS AT BROWNS FERRY 1*	156862 P 60
#REACTOR RECIRCULATION	PUMP MOTOR GENERATOR TRIPS AT BRUNSWICK 2*	162709 P202
#RECIRCULATION	PUMP MOTOR GENERATOR TRIPS AT HATCH 1*	161795 P342
#DIESEL GENERATOR COOLING WATER	PUMP MOTOR INOPERABLE AT QUAD-CITIES 1*	161898 P549
#RECIRCULATION	PUMP MOTOR OIL LEVEL IS LOW AT DUANE ARNOLD*	160265 P 11
#SCREEN WASH	PUMP MOTOR OVERHEATS AT HUMBOLDT BAY*	159437 P411
#REACTOR RECIRCULATION	PUMP MOTOR SHORTS AT DUANE ARNOLD*	158609 P 9
#RECIRCULATION	PUMP MOTOR SNUBBER LEAKS AT VERMONT YANKEE*	159172 P574
#RHR	PUMP MOTOR TRIPS AT BROWNS FERRY 3*	159659 P113
#DG AUXILIARY OIL	PUMP MOTOR TRIPS OUT AT BROWNS FERRY 3*	159490 P110
#RHR	PUMP OPERABILITY TEST NOT PERFORMED CORRECTLY AT HATCH 1*	158599 P317
#JET	PUMP OPERABILITY TEST PROCEDURE REQUIRES MODIFICATION AT	159094 P389
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#ONLY ONE DILUTION	PUMP OPERATING FOR TWENTY MINUTE PERIODS AT OYSTER CREEK*	153497 P448
#ONLY ONE DILUTION	PUMP OPERATING WHEN INTAKE TEMPERATURE BELOW 60 F AT	159070 P471
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#DIESEL DRIVEN FIRE	PUMP OUT OF SERVICE TOO LONG AT DRESDEN 2*	160362 P217
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#FIRE PROTECTION	PUMP REMOVED FROM SERVICE AT FITZPATRICK*	158302 P276
#FIRE	PUMP REMOVED FROM SERVICE FOR REPAIR AT DUANE ARNOLD*	158271 P 27
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#RHR	PUMP ROOM COOLER FAILS TO START AT PEACH BOTTOM 2*	162488 P487
#RECIRC	PUMP RUNBACK RESULTS IN PUMP SPEED MISMATCH AT MILLSTONE 1*	154444 P419
#FLOW THROUGH RHR	PUMP SEAL COOLERS FALLS BELOW LIMIT AT BROWNS FERRY 3*	155503 P105
#COOLING WATER FLOW TO RHR	PUMP SEAL FALLS BELOW LIMIT AT BROWNS FERRY 3*	159393 P114
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#CORE SPRAY	PUMP STARTS TOO SOON AT PEACH BOTTOM 3*	160332 P498
#LEAK FOUND IN RECIRC	PUMP SUCTION TEST SECTION AT BRUNSWICK 2*	154732 P162
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#MAXIMUM COOLDOWN	RATE EXCEEDED AT LA CROSSE*	161831 P415
#TOTAL CONTAINMENT LEAK	RATE EXCEEDS LIMIT AT BROWNS FERRY 1*	155532 P 55
#HPCI CHECK VALVE LEAK	RATE EXCEEDS LIMIT AT BROWNS FERRY 2*	160092 P 94

	#CONTAINMENT LEAK	RATE EXCEEDS LIMIT AT BROWNS FERRY 2*	161792 P 98
	#FEEDWATER INBOARD CHECK VALVE LEAK	RATE EXCEEDS LIMIT AT DRESDEN 3*	155518 P227
	#FEEDWATER CHECK VALVE LEAK	RATE EXCEEDS LIMIT AT DRESDEN 3*	156159 P229
	#IODINE RELEASE	RATE EXCEEDS LIMIT AT FITZPATRICK*	158622 P279
	#FEEDWATER ISOLATION VALVE LEAK	RATE EXCEEDS LIMIT AT HATCH 2*	156232 P364
	#FEEDWATER ISOLATION VALVE LEAK	RATE EXCEEDS LIMIT AT HATCH 2*	161854 P402
	#RCIC TURBINE EXHAUST ISOLATION VALVE LEAK	RATE EXCEEDS LIMIT AT HATCH 2*	161852 P404
	#DRYWELL PNEUMATIC RETURN ISOLATION VALVE LEAK	RATE EXCEEDS L MIT AT HATCH 2*	161880 P406
	#CONTAINMENT ISOLATION VALVE LEAK	RATE EXCEEDS LIMIT AT LACROSSE*	162016 P418
	#AVERAGE PLANAR HEAT GENERATION	RATE EXCEEDS LIMIT AT NINE MILE POINT 1*	154448 P436
	#AVERAGE PLANAR LINEAR HEAT GENERATION	RATE EXCEEDS LIMIT AT NINE MILE POINT 1*	154445 P439
	#SERVICE WATER PUMP FLOW	RATE FALLS BELOW LIMIT AT HATCH 1*	156601 P318
	#TWO CONTAINMENT ISOLATION VALVES EXCEED LEAK	RATE LIMIT AT VERMONT YANKEE*	161955 P581
	#LINEAR HEAT GENERATION	RATE LIMIT EXCEEDED IN THREE NODES AT BROWNS FERRY 1*	159192 P 66
AT MONTICELLO*	#LINEAR HEAT GENERATION	RATE NOT CHECKED AT BROWNS FERRY 2*	159337 P 91
MONTICELLO*	#LEAK	RATE OF DRYWELL EQUIPMENT DRAIN SUMP VALVE EXCEEDS LIMIT	156244 P432
	#LEAK	RATE OF HPCI TURBINE EXHAUST CHECK VALVE EXCEEDS LIMIT AT	156239 P429
	#LEAK	RATE OF INBOUND MSIV EXCEEDS LIMIT AT MONTICELLO*	156245 P432
	#LEAK	RATE OF MSIV'S EXCEEDS LIMIT AT ARNOLD*	155526 P 4
	#LEAK	RATE OF OUTBOARD MSIV EXCEEDS LIMIT AT MONTICELLO*	156238 P428
MONTICELLO*	#LEAK	RATE OF OUTBOARD MSIV EXCEEDS LIMITS AT MONTICELLO*	156237 P428
MONTICELLO*	#LEAK	RATE OF RCIC TURBINE EXHAUST CHECK VALVE EXCEEDS LIMIT AT	156242 P431
HATCH 2*	#LEAK	RATE OF STANDBY LIQUID CONTROL VALVE EXCEEDS LIMIT AT	156240 P429
BROWNS FERRY 1,*	#LEAKAGE	RATE OF THREE CONTAINMENT ISOLATION VALVES UNACCEPTABLE AT	155432 P356
MONTICELLO*	#LEAK	RATE OF THREE MAIN STEAM ISOLATION VALVES EXCEED LIMIT AT	154617 P 48
	#LEAK	RATE OF TORUS INSTRUMENT AIR VALVE EXCEEDS LIMIT AT	156243 P431
	#LEAK	RATE OF TORUS VENT VALVE EXCEEDS LIMIT AT MONTICELLO*	156241 P430
	#CORE HEAT GENERATION	RATE TESTS MISSED AT OYSTER CREEK 1*	161877 P488
	#RCIC FAILS TO REACH	RATED FLOW AT BROWNS FERRY 3*	153459 P101
	#RCIC FAILS TO REACH	RATED FLOW AT BROWNS FERRY 3*	153458 P101
	#HPCI TURBINE DOES NOT REACH	RATED SPEED AT DUANE ARNOLD*	160275 P 10
	#HPCI TURBINE FAILS TO REACH	RATED SPEED AT VERMONT YANKEE*	156151 P570
	#DIESEL GENERATOR FAILS TO REACH	RATED VOLTAGE AT HATCH 2*	155313 P350
	#CONTAINMENT ISOLATION VALVES* LEAK	RATES EXCEED LIMIT AT BIG ROCK POINT*	163388 P 65
	#CONTAINMENT ISOLATION VALVE LEAK	RATES EXCEED LIMIT AT HATCH 2*	157132 P362
	#FEEDWATER ISOLATION VALVE LEAK	RATES EXCEED LIMIT AT HATCH 2*	161845 P404
	#ELECTRICAL PENETRATION LEAK	RATES EXCEED LIMIT AT LACROSSE*	156223 P412
	#CONTAINMENT ISOLATION VALVE LEAK	RATES EXCEED LIMIT AT QUAD CITIES 1*	160815 P546
	#HPCI AND RHR VALVE LEAK	RATES EXCEED LIMITS AT HATCH 2*	161914 P404
	#SEVERAL CONTAINMENT ISOLATION VALVES* LEAK	RATES EXCEED LIMITS AT PEACH BOTTOM 2*	159578 P479
INSTALLED AT BROWNS FERRY 1*	#CRITICAL POWER	RATIO IS LESS THAN LIMIT AT BRUNSWICK 2*	155575 P166
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	#WATER IMPINGEMENT FROM RECIRC LINE BREAK MIGHT DAMAGE	RBCCW PIPING AT BROWNS FERRY 1*	159267 P 53
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		# RCIC FAILS TO INJECT INTO REACTOR VESSEL AT HATCH 1*	158685 P327
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	#RCIC PIPE SUPPORT	REQUIRES MODIFICATION AT HATCH 1*	159090 P332
	#JET PUMP OPERABILITY TEST PROCEDURE	REQUIRES MODIFICATION AT HATCH 2*	159094 P389
	#STANDBY GAS TREATMENT TRAIN FAILS TO	RESET AT BRUNSWICK 1*	156176 P136
	#FIRE PROTECTION ALARM ANNUNCIATES AND WON'T	RESET AT BRUNSWICK 2*	158303 P204
	#TURBINE CONTROL VALVE CLOSURE SCRAM FAILS TO	RESET AT DRESDEN 2*	154620 P207
	#LPCI LOOP SELECTION PRESSURE SWITCH FAILS TO	RESET AT DUANE ARNOLD*	159067 P 14
	#MAIN STEAM ISOLATION VALVE LIMIT SWITCH FAILS TO	RESET AT FITZPATRICK*	163403 P296
	#RCIC TURBINE FAILS TO	RESET AT HATCH 2*	159311 P393
POSITION AT BROWNS FERRY 3*	#	RESIDUAL HEAT REMOVAL SYSTEM CONTROL VALVE TRIPS IN MID-	154707 P103
	#SCRAM DISCHARGE VOLUME	RESIDUAL WATER CHECK NOT PERFORMED AT HATCH 1*	160135 P337
	#ANION	RESIN TANK LEAKS AT BIG ROCK POINT*	161469 P 42
	#RCIC FAILS TO	RESPOND TO SPEED CONTROL AT HATCH 1*	159502 P324
	#DIESEL GENERATOR FAILS TO	RESPOND TO VOLTAGE CONTROL AT DUANE ARNOLD*	160801 P 21
	#MAIN TURBINE CONTROL VALVE CLOSURE SWITCH	RESPONSE TIME IS SLOW AT BRUNSWICK 2*	156930 P176
	#FEEDER BREAKER NOT RECLOSED FOLLOWING	RESTORATION OF OFFSITE POWER AT BROWNS FERRY 3*	156072 P105
	#4160V BUS TIE BOARD DEENERGIZED AND	RESTORED TO NORMAL FEEDER AT BROWNS FERRY 3*	155391 P104
	#LPCI PIPE	RESTRAIN CLEVIS PIN FOUND MISSING AT HATCH 1*	154670 P300
	#ONE PIPING	RESTRAINT FAILS TO MEET SEISMIC CRITERIA AT BROWNS FERRY 1*	155367 P 53
	#RB CLOSED COOLING WATER PIPING SEISMIC	RESTRAINTS NOT INSTALLED AT BROWNS FERRY 1*	154610 P 48
HEAT EXCHANGER AT BROWNS FERRY 3*	#	RESTRICTION IN COOLING WATER LINE FOR DIESEL GENERATOR	157438 P108
EXCHANGER AT BROWNS FERRY 3*	#	RESTRICTION IN COOLING WATER LINE FOR RHR PUMP SEAL HEAT	157437 P108
	#RECIRC PUMP RUNBACK	RESULTS IN PUMP SPEED MISMATCH AT MILLSTONE 1*	154444 P419
1*	#	RETENTION BUILDING FIRE DETECTORS FAIL TO ALARM AT PILGRIM	159083 P516
	#DRYWELL PNEUMATIC	RETURN ISOLATION VALVE LEAK RATE EXCEEDS LIMIT AT HATCH 2*	161880 P406
	#CONTAINMENT CONDENSATE	RETURN ISOLATION VALVE-LEAKS AT LA CROSSE*	158309 P413
	#LEAK IN CONTROL ROD DRIVE SYSTEM	RETURN LINE AT BIG ROCK POINT*	161911 P 45
	#RHR TORUS	RETURN VALVE FAILS TO OPERATE AT BROWNS FERRY 1*	160155 P 72
	#HYDROGEN RECOMBINER	RETURN VALVE LEAKAGE UNACCEPTABLE AT HATCH 2*	155449 P358
	#REACTOR BUILDING VENT SAMPLE SYSTEM NOT	RETURNED TO SERVICE AT DUANE ARNOLD*	155516 P 26

	#APRM REMAINS INOPERABLE #HEN		RETURNED TO SERVICE AT FITZPATRICK*	156220 P273
	#FIRE DOORS NOT		RETURNED TO SERVICE IN TIME AT BRUNSWICK 2*	162429 P208
	#HPCI TURBINE		REVERSING CHAMBERS CRACK AT DUANE ARNOLD*	156929 P 26
#INSTRUMENT MAINTENANCE PROCEDURE CHANGES NOT			REVIEWED AT FITZPATRICK*	160259 P283
#STANDBY GAS TREATMENT PROCEDURE CHANGES NOT			REVIEWED AT HATCH 1*	160277 P321
#REACTOR PRESSURE CALIBRATION PROCEDURE NOT			REVIEWED AT HATCH 2*	161874 P401
	#CONTAINMENT DESIGN		REVIEWED FOR STEAM LINE B "AKS AT BIG ROCK POINT*	159293 P 39
		# RHR	ALARM FAILS TO CLEAR .f DUANE ARNOLD*	160825 P 22
1*		# RHR	AND CONDENSATE ISOLATION VALVES LEAK AT HATCH 2*	161759 P405
		# RHR	AREA COOLER FAN THERMAL OVERLOAD TRIPS AT BROWNS FERRY	159407 P 69
2*		# RHR	CHECK VALVE LEAKS AT BROWNS FERRY 2*	159395 P 92
FITZPATRICK*		# RHR	CONTAINMENT COOLING VALVE FAILS TO OPEN AT QUAD-CITIES	160878 P560
QUAD CITIES 2*		# RHR	CONTAINMENT ISOLATION VALVE FAILS TC CLOSE AT	158610 P277
	#TWO	RHR	DELAY TIMERS FAIL TO ACTUATE THE TIMER CONTACTS AT	156019 P554
	#REMOTE SHUTDOWN PANEL	RHR	FLOW INDICATOR FAILS AT BRUNSWICK 1*	163[illegible] P153
BRUNSWICK 1*		# RHR	FLOW INDICATOR GIVES FALSE READINGS AT BRUNSWICK 1*	160159 P152
		# RHR	FLOW INDICATOR ON REMOTE SHUTDOWN PANEL FAILS AT	158605 P144
		# RHR	FLOW INSTRUMENTATION FAILS AT BRUNSWICK 2*	160367 P185
		# RHR	HEAT EXCHANGER INOPERABLE AT COOPER*	162523 P258
		# RHR	HEAT EXCHANGER LEAKS AT BROWNS FERRY 1*	157168 P 63
		# RHR	HEAT EXCHANGER LEAKS AT BROWNS FERRY 1*	158710 P 64
		# RHR	HEAT EXCHANGER LEAKS AT BROWNS FERRY 2*	159328 P 92
		# RHR	HEAT EXCHANGER LEAKS AT BROWNS FERRY 2*	160302 P 92
		# RHR	HEAT EXCHANGER ON CROSSTIE LEAKS AT BROWNS FERRY 3*	159391 P115
		# RHR	HEAT EXCHANGER RIB PLATE BUCKLES AT BRUNSWICK 2*	160296 P174
		# RHR	INBOARD INJECTION VALVE FAILS TO OPEN AT HATCH 1*	159314 P335
		# RHR	INJECTION VALVE FAILS TC CLOSE AT BROWNS FERRY 1*	156014 P 56
		# RHR	INOPERABLE AT HATCH 1*	159323 P334
		# RHR	LOGIC CIRCUIT INOPERABLE AT QUAD-CITIES 2*	163335 P563
		# RHR	LOOP INOPERABLE AT HATCH 1*	159691 P337
		# RHR	LOOP SUPPORT INOPERABLE AT FITZPATRICK*	153728 P267
	#SET POINT DRIFT IN	RHR	LOW PRESSURE PERMISSIVE SWITCH AT DUANE ARNOLD*	155510 P 4
		# RHR	MINIMUM FLOW LINES NOT ADEQUATELY SUPPORTED AT HATCH 2*	155385 P353
		# RHR	MINIMUM FLOW VALVES CLOSED INADVERTENTLY AT HATCH 1*	159500 P322
		# RHR	NOT PROPERLY TESTED AT BROWNS FERRY 2*	155457 P 85
BROWNS FERRY 3*		# RHR	PUMP AREA COOLER FAN THERMAL OVERLOAD RELAY FAILS AT	159389 P115
		# RHR	PUMP BREAKER FOUND NOT CHARGED AT COOPER*	154032 P241
		# RHR	PUMP DRAIN LINE PIPE CRACKS AT PEACH BOTTOM 3*	156971 P492
		# RHR	PUMP FAILS TO START AT QUAD-CITIES 2*	160564 P561
		* RHR	PUMP INOPERABLE AT COOPER 1*	160051 P255
		[illegible] RHR	PUMP INOPERABLE AT QUAD CITIES 2*	162032 P564
		[illegible] RHR	PUMP MOTOR FAILS AT PEACH BOTTOM 2*	158237 P487
1*		# RHR	PUMP MOTOR TRIPS AT BROWNS FERRY 3*	159650 P113
		# RHR	PUMP OPERABILITY TEST NOT PERFORMED CORRECTLY AT HATCH	158599 P317
		# RHR	PUMP ROOM COOLER FAILS TO START AT PEACH BOTTOM 2*	160434 P481
		# RHR	PUMP ROOM COOLER FAILS TO START AT PEACH BOTTOM 2*	162488 P487
	#FLOW THROUGH	RHR	PUMP SEAL COOLERS FALLS BELOW LIMIT AT BROWNS FERRY 3*	155503 P105
	#COOLING WATER FLOW TO	RHR	PUMP SEAL FALLS BELOW LIMIT AT BROWNS FERRY 3*	159393 P114
	#RESTRICTION IN COOLING WATER LINE FOR	RHR	PUMP SEAL HEAT EXCHANGER AT BROWNS FERRY 3*	157437 P108
	#SET POINT DRIFT IN	RHR	PUMP START RELAY AT HATCH 2*	159133 P358
		# RHR	PUMP SUCTION VALVE LEAKS AT HATCH 2*	161628 P403
		# RHR	PUMP TIMER RELAY FAILS AT QUAD-CITIES 2*	155985 P553
		[illegible] RHR	PUMP TRIPS OFF AT BRUNSWICK 1*	155507 P132
		# RHR	PUMP TRIPS ON OVERCURRENT AT BROWNS FERRY 3*	161959 P120
	#LIMIT SWITCHES ON	RHR	PUMP VALVES ARE BROKEN AT HATCH 2*	157702 P380
	#UPDATE ON	RHR	RADWASTE DISCHARGE VALVE FAILURE AT COOPER*	163481 P259
BRUNSWICK 2*	#BOTH	RHR	ROOM COOLERS INOPERABLE FOR FIFTEEN MINUTES AT	155539 P175
		# RHR	SERVICE WATER BOOSTER PUMP INOPERABLE AT COOPER*	159521 P248
	#UPDATE ON	RHR	SERVICE WATER LOOP INOPERABLE AT BRUNSWICK 2*	161984 P192
		# RHR	SERVICE WATER LOOP INOPERABLE AT HATCH 1*	159316 P331
		# RHR	SERVICE WATER PRESSURE ALARM FAILS AT BRUNSWICK 2*	161636 P192
CLOSED AT HATCH 1*		# RHR	SERVICE WATER PUMP BEARING COOLING WATER VALVES FOUND	158611 P316
FAILURE CRITERION AT HATCH 1*		# RHR	SERVICE WATER PUMP COOLING DOES NOT MEET SINGLE	157143 P314
MONTICELLO*		# RHR	SERVICE WATER PUMP FAILS TO MEET REQUIREMENTS AT	156879 P434
		# RHR	SERVICE WATER PUMP FAILS TO START AT BRUNSWICK 1*	155555 P129
		# RHR	SERVICE WATER PUMP FAILS TO START AT BRUNSWICK 1*	155399 P134
FERRY 1*		# RHR	SERVICE WATER PUMP FLOW DROPS SUDDENLY AT HATCH 2*	155433 P357
		# RHR	SERVICE WATER PUMP TIMER OUT OF TOLERANCE AT BROWNS	156212 P 56
		# RHR	SERVICE WATER PUMPS FAIL AT HATCH 2*	158671 P381
FITZPATRICK*		# RHR	SERVICE WATER PUMPS FAIL FLOW TEST AT BRUNSWICK 2*	154877 P170
		# RHR	SERVICE WATER PUMPS FAIL TO PROVIDE SUFFICIENT FLOW AT	160227 P286
		# RHR	SERVICE WATER PUMPS FAIL TO START AT BRUNSWICK 2*	160466 P187
		# RHR	SERVICE WATER PUMPS FAILS TO START AT VERMONT YANKEE*	155992 P569
CLOSE AT QUAD CITIES 2*		# RHR	SERVICE WATER VALVE FAILS TC CLOSE AT COOPER*	161844 P256
		# RHR	SERVICE WATER VAULT BED PLATE DRAIN VALVE FAILS TO	153951 P551
CITIES 1*		# RHR	SERVICE WATER VAULT PENETRATION LEAKS AT QUAD CITIES 2*	154915 P552
		# RHR	SUPPRESSION CHAMBER DUMP VALVE BREAKER TRIPS AT QUAD	155522 P539
		# RHR	SW PUMP INOPERABLE AT HATCH 1*	160145 P339
	#INADEQUATE PIPE SUPPORTS FOUND ON	RHR	SYSTEM AT HATCH 1*	159614 P328
FITZPATRICK*		# RHR	SYSTEM DISCHARGE TO RADWASTE VALVE FAILS TO OPEN AT	158300 P276
AT PILGRIM 1*		# RHR	SYSTEM FAILS TO MEET SAFE SHUTDOWN EARTHQUAKE CRITERIA	160490 P527
		# RHR	SYSTEM SNUBBER FAILS AT PILGRIM 1*	155210 P504
		# RHR	TORUS RETURN VALVE FAILS TO OPERATE AT BROWNS FERRY 1*	160155 P 72
	#CONTROL POWER TO	RHR	VALVE FAILS AT COOPER*	155580 P243
		# RHR	VALVE FOUND INOPERABLE AT BROWNS FERRY 3*	155341 P104
	#HPCI AND	RHR	VALVE LEAK RATES EXCEED LIMITS AT HATCH 2*	161914 P404
	#OPENING AND CLOSING TIME OF	RHR	VALVE OFF LIMITS AT HATCH 1*	154668 P301

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                                                  # RHR-S* CHECK VALVE FAILS TO SEAT AT VT .MONT YANKEE*                160297 P876
                                  #RHR HEAT EXCHANGER  RIB PLATE BUCKLES AT BRUNSWICK 2*                                160296 P 74
                                  #SECTION OF A SPLIT  RING FOUND IN THE MAIN HPCI PUMP IMPELLER AT ARNOLD*             156226 P  6
                                                  # RIP VALVE DOES NOT FULLY CLOSE AT BRUNSWICK 1*                     156920 P142
                                                #FOUR  RIP VALVES FAIL TO OPERATE PROPERLY AT BRUNSWICK 1*            154731 P123
                                                  # RIP VALVES FOR CONTAINMENT AIR MONITORS FOUND CLOSED AT            155531 P171
BRUNSWICK 2*
                                     #RIVER WATER TEMPERATURE  RISE EXCEEDS LIMIT AT BROWNS FERRY 1*                    160156 P 73
                                #CONDENSTR COOLING WATER TEMPERATURE  RISE EXCEEDS LIMIT AT PILGRIM 1*                  155398 P502
                                   #RADIOACTIVITY DISCHARGED TO  RIVER AT PEACH BOTTOM 2*                              159559 P478
                                                #DIFFERENTIAL  RIVER TEMPERATURE EXCEEDS LIMIT AT BROWNS FERRY 1*      161 .4 P 78
                                                  # RIVER TEMPERATURE EXCEEDS LIMIT AT BROWNS FERRY*                   15F.00 P 66
                                                  # RIVER WATER SUPPLY LOOP OUT OF SERVICE AT DUANE ARNOLD*            160458 P 18
                                                  # RIVER WATER TEMPERATURE EXCEEDS LIMIT AT OYSTER CREEK*             159137 P471
                                                  # RIVER WATER TEMPERATURE RISE EXCEEDS LIMIT AT BROWNS FERRY         160156 P 73
1*
                                                  # RIVER WATER TRAVELING SCREEN FAILS WHILE REDUNDANT SYSTEM           159051 P 14
IS FAILED AT DUANE ARNOLD*
                                                  # RMOV BOARD BREAKER FAILS TO TRANSFER DURING MANUAL                 156028 P 83
TRANSFER AT BROWNS FERRY 2*                                                                                          153935 P 29
                                  #UPDATE REPORT ON CONTROL ROD DRIVE HOUSING LEAK AT BIG  ROCK POINT*                153973 P 30
#UPDATE REPORT ON PRIMARY SYSTEM WATER LEVEL SENSORS AT BIG  ROCK POINT*                                              153498 P 31
                                  #BOTH PLANT VENT EXHAUST FANS OUT OF SERVICE AT BIG  ROCK POINT*                    153976 P 31
                                #UPDATE REPORT ON CONTROL ROD DRIVE TEMPERATURE AT BIG  ROCK POINT*                   154457 P 31
#EMERGENCY CONDENSER OUTLET VALVE LEAKS THROUGH SEAT AT BIG  ROCK POINT*                                              154904 P 32
                                #REACTOR DEPRESSURIZING SYSTEM BATTERY CELL FAILS AT BIG  ROCK POINT*                 157053 P 33
                                  #RDS SENSOR CHANNEL REMOVED FROM SERVICE AT BIG  ROCK POINT*                        155473 P 34
                                  #CONTROL ROD DRIVE PUMP VIBRATES EXCESSIVELY AT BIG  ROCK POINT*                    157074 P 34
                                  #SET POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG  ROCK POINT*                   157076 P 35
                                  #SET POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG  ROCK POINT*                   158777 P 37
                              #SET POINT DRIFT IN REACTOR WATER LEVEL TRANSMITTER AT BIG  ROCK POINT*                 159088 P 37
                             #REACTOR DEPRESSURIZING SYSTEM CHANNEL BATTERY FAILS AT BIG  ROCK POINT*                 160291 P 38
                                     #PRESSURE INDICATOR SWITCH FAILS AT BIG  ROCK POINT*                            159288 P 38
                                #REACTOR DEPRESSURIZATION SYSTEM INOPERABLE AT BIG  ROCK POINT*                       159256 P 39
                                     #RDS BATTERY HAS LOW SPECIFIC GRAVITY AT BIG  ROCK POINT*                        159293 P 39
                              #CONTAINMENT DESIGN REVIEWED FOR STEAM LINE BREAKS AT BIG  ROCK POINT*                  159257 P 39
                                     #RDS SENSOR CHANNEL INOPERABLE AT BIG  ROCK POINT*                               160072 P 40
                             #REACTOR DEPRESSURIZING BATTERY FAILS TO HOLD CHARGE AT BIG  ROCK POINT*                 160043 P 40
                                  #CONTAINMENT ISOLATION VALVE LEAKS ACROSS SEAT AT BIG  ROCK POINT*                  160170 P 41
                                  #SET POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG  ROCK POINT*                   163495 P 41
                                     #RDS CHANNEL REMOVED FROM SERVICE AT BIG  ROCK POINT*                            163500 P 42
                                  #DEMINERALIZER SYSTEM PUMP LEAKS AT BIG  ROCK POINT*                               161674 P 42
                                #MAIN STEAM ISOLATION VALVE FAILS TO CLOSE AT BIG  ROCK POINT*                       161469 P 42
                                     #ANION RESIN TANK LEAKS AT BIG  ROCK POINT*                                     161980 P 43
                                #DIESEL GENERATOR FAILS TO REACH OUTPUT VOLTAGE AT BIG  ROCK POINT*                  161982 P 43
                                     #DIESEL GENERATOR FAILS TO RUN AT BIG  ROCK POINT*                              161983 P 44
                                     #FEEDWATER CHECK VALVES LEAKS AT BIG  ROCK POINT*                               162017 P 44
                                  #CONTAINMENT ISOLATION VALVE LEAKS AT BIG  ROCK POINT*                              163389 P 44
                             #DIESEL FIRE/CORE SPRAY PUMP FAILS TO START IN TIME AT BIG  ROCK POINT*                  161911 P 45
                                #LEAK IN CONTROL ROD DRIVE SYSTEM RETURN LIN  AT BIG  ROCK POINT*                     161913 P 45
                                  #CRACK IN REACTOR CLEANUP SYSTEM LINE AT BIG  ROCK POINT*                           163386 P 46
                             #UNQUALIFIED SPLICES AND TERMINAL BLOCKS DISCOVERED AT BIG  ROCK POINT*        #TEST      155402 P 33
                               CIRCUIT ALARMS RECEIVED FROM SENSOR CHANNEL OF RDS AT BIG  ROCK POINT*      #UPDATE    153974 P 29
                             REPORT ON REACTOR CLEAN-UP SYSTEM PIPING INDICATIONS AT BIG  ROCK POINT*      #UPDATE    154458 P 30
                                REPORT ON EMERGENCY CONDENSER VALVE SEAT LEAKAGE AT BIG  ROCK POINT*      #CONTROL   154459 P 32
                                   AIR TUBING TO AUX SYSTEM COOLING VALVE BROKEN AT BIG  ROCK POINT*      #REACTOR   154905 P 32
                              DEPRESSURIZING SYSTEM CHANNEL REMOVED FROM SERVICE AT BIG  ROCK POINT*      #REACTOR   158574 P 35
                              DEPRESSURIZING SYSTEM CHANNEL REMOVED FROM SERVICE AT BIG  ROCK POINT*      #REACTOR   160071 P 40
                             WATER LEVEL SENSOR REMOVED FROM SERVICE FOR VENTING AT BIG  ROCK POINT*     #EQUIPMENT 156984 P 34
                                AND PERSONNEL LOCK LEAKAGE TESTS NOT PERFORMED AT BIG  ROCK POINT*      #CONTAINMENT 163388 P 45
                                   ISOLATION VALVES* LEAK RATES EXCEED LIMIT AT BIG  ROCK POINT*     #SETPOINT DRIFTS 158266 P 36
                            IN SENSOR CHANNEL OF REACTOR DEPRESSURIZING SYSTEM AT BIG  ROCK POINT*   #SETPOINT DRIFTS 158267 P 36
                            IN SENSORS CHANNEL OF REACTOR DEPRESSURIZING SYSTEM AT BIG  ROCK POINT*   #SETPOINT DRIFTS 158268 P 37
                            TO CONTAINMENT ISOLATION VENTILATON VALVE OPERATORS AT BIG  ROCK POINT*  #DAMAGE MIGHT OCCUR 156954 P 36
                                                #TARGET  ROCK SAFETY RELIEF VALVE FAILS TO OPEN AT QUAD CITIES 1*     158751 P544
NONCONSERVATIVE AT PILGRIM 1*                     # ROD BLOCK AND HIGH FLUX SCRAM CALIBRATIONS ARE                     159385 P517
                                  #APRM FAILS TO INITIATE  ROD BLOCK AT DRESDEN 3*                                     159533 P235
                             #CONTROL ROD POSITION SWITCH AND  ROD BLOCK INOPERABLE AT VERMONT YANKEE*                 160895 P576
                                                  # ROD BLOCK MONITOR BYPASSED TOO LONG AT BROWNS FERRY 2*             156015 P 85
                                                  # ROD BLOCK MONITOR DISABLED DUE TO LPRM FAILURES AT                 158633 P280
FITZPATRICK*
                                                  # ROD BLOCK MONITOR FAILS AT BRUNSWICK 1*                           155308 P136
                                                  # ROD BLOCK MONITOR FAILS AT DRESDEN 3*                             153493 P223
                                          #CONTROL  ROD BLOCK MONITOR FAILS AT DRESDEN 3*                             160922 P237
                                                  # ROD BLOCK MONITOR FAILS TO OPERATE AT HATCH 2*                     159594 P384
                                          #CONTROL  ROD BLOCK MONITOR INOPERABLE AT DUANE ARNOLD*                     159561 P 16
                                          #CONTROL  ROD BLOCK MONITOR INOPERABLE AT PILGRIM 1*                        159555 P511
                                                  # ROD BLOCK MONITOR NOT TESTED WHEN REQUIRED AT HATCH 1*             155411 P310
BRUNSWICK 1*
                                                  # ROD BLOCK MONITOR PANEL ILLUMINATES SPURIOUSLY AT                  159160 P138
                                          #CONTROL  ROD BLOCK MONITOR SETPOINT DRIFTS AT BRUNSWICK 1*                 160467 P151
                                          #CONTROL  ROD BLOCK NOT GENERATED BY REFUEL BRIDGE MOVEMENT AT              160260 P283
FITZPATRICK*
                                #ELECTRONIC DRIFT CAUSES INCORRECT  ROD BLOCK TRIP AT DRESDEN 2*                      15769o P232
                              #ROD WORTH MINIMIZER GIVES SPURIOUS  ROD BLOCKS AT BRUNSWICK 1*                         160206 P151
                                                #A  ROD DRIFT ALARMS ANNUNCIATES INTERMITTENTLY AT BRUNSWICK 1*       154557 P123
                                          #CONTROL  ROD DRIFTS OUT AT BROWNS FERRY 2*                                 158725 P 88
                                          #CONTROL  ROD DRIVE ACCUMULATOR LEVEL SWITCH FAILS AT BROWNS FERRY 1*       155296 P 49
                                #SET POINT DRIFT IN CONTROL  ROD DRIVE ACCUMULATOR PRESSURE INDICATORS AT HATCH 2*   158644 P378
                                          #CONTROL  ROD DRIVE ACCUMULATOR PRESSURE SWITCHES SET INCORRECTLY AT        157131 P361
HATCH 2*
                                          #CONTROL  ROD DRIVE ACCUMULATORS FAIL TO MAINTAIN NITROGEN PRESSURE         160269 P321
AT HATCH 1*
                                          #CONTROL  ROD DRIVE FAILS TO WITHDRAW AT BRUNSWICK 2*                       160047 P185
                                #UPDATE REPORT ON CONTROL  ROD DRIVE HOUSING LEAK AT BIG ROCK POINT*                  153935 P 29
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LIMITS AT MONTICELLO*	#CONTROL	ROD DRIVE HYDRAULIC PIPE SUPPORTS FAIL TO MEET STRESS	156339 P433
	#CONTROL	ROD DRIVE PUMP VIBRATES EXCESSIVELY AT BIG ROCK POINT*	155473 P 34
	#TWO CONTROL	ROD DRIVE PUMPS INOPERABLE AT OYSTER CREEK*	162046 P468
AT DRESDEN 2*	#CONTROL	ROD DRIVE SCRAM DISCHARGE PIPE SUPPORTS ARE NOT ADEQUATE	158692 P214
MODIFICATION AT DRESDEN 3*	#CONTROL	ROD DRIVE SCRAM DISCHARGE PIPE SUPPORTS REQUIRE	158706 P234
MILE POINT*	#CONTROL	ROD DRIVE SCRAM PILOT SOLENOIDS NOT TESTED ON TIME AT NINE	163364 P447
#LEAK IN CONTROL		ROD DRIVE SYSTEM RETURN LINE AT BIG ROCK POINT*	161911 P 45
#UPDATE REPORT ON CONTROL		ROD DRIVE TEMPERATURE AT BIG ROCK POINT*	153975 P 31
	#CONTROL	ROD FAILS TO INSERT AT BROWNS FERRY 3*	154896 P193
	#CONTROL	ROD FAILS TO SCRAM AT NINE MILE POINT 1*	159780 P443
	#CONTROL	ROD FAILS TO WITHDRAW AT MONTICELLO*	160370 P437
TIME AT OYSTER CREEK*	#CONTROL	ROD FREE TRAVEL INTERLOCK SURVEILLANCE NOT PERFORMED ON	158270 P458
*	#CONTROL	ROD HAS NO POSITION INDICATION FOR ONE NOTCH AT BRUNSWICK 1	153731 P128
*	#CONTROL	ROD HAS NO POSITION INDICATION FOR ONE NOTCH AT BRUNSWICK 2	153735 P165
	#CONTROL	ROD INCORRECTLY WITHDRAWN AT COOPER*	158686 P248
#FUEL ASSEMBLY LOADED INTO CELL WITHOUT CONTROL		ROD INSERTED AT ARNOLD*	156078 P 7
	#CONTROL	ROD INSERTION TIME EXCEEDS LIMIT AT BRUNSWICK 2*	160513 P190
	#CONTROL	ROD MOVED WITH SHORTING LINKS INSTALLED AT BRUNSWICK 2*	163471 P172
*	#ONE CONTROL	ROD NOT PULLED TO TARGET POSITION DURING STARTUP AT HATCH 2	156132 P357
	#CONTROL	ROD POSITION INDICATION FAILS AT BRUNSWICK 1*	156175 P135
	#CONTROL	ROD POSITION INDICATION FAILS AT BRUNSWICK 1*	160304 P141
	#CONTROL	ROD POSITION INDICATION FAILS AT BRUNSWICK 1*	161687 P155
	#CONTROL	ROD POSITION INDICATION FAILS AT DRESDEN 2*	159447 P216
	#CONTROL	ROD POSITION INDICATION INCORRECT AT BRUNSWICK 2*	161626 P198
	#CONTROL	ROD POSITION INDICATION LOST AT DRESDEN 2*	160056 P220
	#CONTROL	ROD POSITION INDICATION LOST AT DRESDEN 2*	160925 P222
	#ALL	ROD POSITION INDICATION LOST AT VERMONT YANKEE*	154604 P567
	#CONTROL	ROD POSITION INDICATIONS FAIL AT BRUNSWICK 2*	160146 P182
	#TWO CONTROL	ROD POSITION INDICATIONS LOST AT BRUNSWICK 1*	163506 P157
	#CONTROL	ROD POSITION INDICATOR FAILS AT BRUNSWICK 1*	158600 P144
	#CONTROL	ROD POSITION INDICATOR LIGHT FAILS TO LIGHT AT BRUNSWICK 1*	154712 P125
	#CONTROL	ROD POSITION INDICATORS FAIL AT BRUNSWICK 1*	159197 P146
	#CONTROL	ROD POSITION INDICATORS FAIL AT BRUNSWICK 1*	160108 P149
CREEK*	#CONTROL	ROD POSITION LIGHTS REMAIN ON WHEN RODS MOVED AT OYSTER	158695 P456
YANKEE*	#CONTROL	ROD POSITION SWITCH AND ROD BLOCK INOPERABLE AT VERMONT	160895 P576
	#TWO CONTROL	ROD SELECT SWITCHES INOPERABLE AT BRUNSWICK 2*	161651 P197
	#	ROD SEQUENCE CONTROL SYSTEM FAILS AT HATCH 2*	155543 P351
WHEN REQUIRED AT FITZPATRICK*	#	ROD SEQUENCE CONTROL SYSTEM FAILS TO COME INTO SERVICE	155525 P271
	#CONTROL	ROD UNCOUPLED FROM DRIVE AT BROWNS FERRY 1*	163339 P 78
	#CONTROL	ROD UNCOUPLED IN FULL WITHDRAWN POSITION AT DRESDEN 2*	155372 P210
	#RSCS BLOCKS	ROD WITHDRAWAL AT BRUNSWICK 2*	154662 P163
BOTTOM 2*	#	ROD WITHDRAWAL BLOCK SETPOINT EXCEEDS LIMIT AT PEACH	155380 P476
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	#UPDATE ON RHR	SERVICE WATER LOOP INOPERABLE AT BRUNSWICK 2*	161984 P192
	#RHR	SERVICE WATER LOOP INOPERABLE AT HATCH 1*	159316 P331
	#	SERVICE WATER PIPE SNUBBER FAILS AT BRUNSWICK 2*	162724 P198
	#	SERVICE WATER PIPING SUPPORTS DEGRADED AT BROWNS FERRY 1*	160515 P 75
	#RHR	SERVICE WATER PRESSURE ALARM FAILS AT BRUNSWICK 2*	161656 P192
CLOSED AT HATCH 1*	#RHR	SERVICE WATER PUMP BEARING COOLING WATER VALVES FOUND	158611 P316
CRITERION AT HATCH 1*	#RHR	SERVICE WATER PUMP COOLING DOES NOT MEET SINGLE FAILURE	157123 P314
*	#STANDBY	SERVICE WATER PUMP DOES NOT MEET OPERATING SPECS AT HATCH 1	157721 P302
	#HEAD DEVELOPED BY PLANT	SERVICE WATER PUMP EXCEEDS LIMIT AT HATCH 1*	156109 P310
*	#RHR	SERVICE WATER PUMP FAILS TO MEET REQUIREMENTS AT MONTICELLO	156879 P434
	#RHR	SERVICE WATER PUMP FAILS TO START AT BRUNSWICK 1*	155555 P129
	#RHR	SERVICE WATER PUMP FAILS TO START AT BRUNSWICK 1*	155399 P134
	#RHR	SERVICE WATER PUMP FLOW DROPS SUDDENLY AT HATCH 2*	159433 P357
	#	SERVICE WATER PUMP FLOW RATE FALLS BELOW LIMIT AT HATCH 1*	158601 P318
	#STANDBY	SERVICE WATER PUMP INOPERABLE AT HATCH 2*	161770 P402
	#PLANT	SERVICE WATER PUMP SEQUENCED TO BUS TOO SOON AT HATCH 2*	156122 P360
*	#RHR	SERVICE WATER PUMP TIMER OUT OF TOLERANCE AT BROWNS FERRY 1	156212 P 56
	#STANDBY DIESEL	SERVICE WATER PUMP TRIPS AT HATCH 1*	153505 P298
	#SALT	SERVICE WATER PUMP TRIPS AT PILGRIM 1*	159607 P516
	#	SERVICE WATER PUMP VIBRATES EXCESSIVELY AT COOPER*	162524 P257
	#TWO	SERVICE WATER PUMPS EACH TRIP TWICE AT PILGRIM 1*	160327 P521
	#RHR	SERVICE WATER PUMPS FAIL AT HATCH 2*	158671 P381
	#RHR	SERVICE WATER PUMPS FAIL FLOW TEST AT BRUNSWICK 2*	154877 P170
FITZPATRICK*	#RHR	SERVICE WATER PUMPS FAIL TO PROVIDE SUFFICIENT FLOW AT	160227 P286
	#RHR	SERVICE WATER PUMPS FAIL TO START AT BRUNSWICK 2*	160466 P187
	#RHR	SERVICE WATER PUMPS FAILS TO START AT VERMONT YANKEE*	155992 P569
	#ALL	SERVICE WATER PUMPS SECURED FOR MAINTENANCE BRUNSWICK 2*	163397 P203
	#DIESEL GENERATOR	SERVICE WATER SUPPLY VALVES FAIL AT BRUNSWICK 1*	156993 P141
	#SEISMIC DESIGN DEFICIENCY IN	SERVICE WATER SYSTEM AT PILGRIM 1*	160551 P531
	#TWO	SERVICE WATER SYSTEM SNUBBERS INOPERABLE AT BRUNSWICK 1*	154598 P126
	#BOTH	SERVICE WATER TRAINS INOPERABLE AT HATCH 1*	159408 P338
	#RHR	SERVICE WATER VALVE FAILS TO CLOSE AT COOPER*	161844 P256
	#	SERVICE WATER VALVE FAILS TO OPEN AT COOPER.*	162522 P258
AT QUAD CITIES 2*	#RHR	SERVICE WATER VAULT BED PLATE DRAIN VALVE FAILS TO CLOSE	153951 P551
	#RHR	SERVICE WATER VAULT PENETRATION LEAKS AT QUAD CITIES 2*	154915 P552
	#SCD SEQUENCE CONTROL SYSTEM FAILS TO COME INTO	SERVICE WHEN REQUIRED AT FITZPATRICK*	155525 P271
	#CORE SPRAY PUMP AREA COOLER FAN THERMOSTAT	SET ABOVE LIMIT AT BROWNS FERRY 1*	159399 P 68
	#INITIATION SWITCHES FOR ISOLATION CONDENSER INCORRECTLY	SET AT OYSTER CREEK*	161756 P458
	#MAIN STEAM LINE LEAK DETECTION TEMPERATURE SWITCH	SET HIGH AT DUANE ARNOLD*	160325 P 12
	#CONTROL ROD DRIVE ACCUMULATOR PRESSURE SWITCHES	SET INCORRECTLY AT HATCH 2*	157131 P361
	#DRIFT IN REACTOR WATER LEVEL SWITCH	SET POINT AT FITZPATRICK*	161717 P294
AT DUANE ARNOLD*	#	SET POINT DIRFT IN CORE SPRAY DIFFERENTIAL PRESSURE SWITCH	155514 P 5
MONITORS IN ONE CHANNEL AT FITZPATRICK*	#	SET POINT DRIFT AND FAILURE OF AVERAGE POWER RANGE	156627 P280
CITIES 2*	#	SET POINT DRIFT FOUND IN MSIV CLOSURE TIMING AT QUAD-	159569 P555
	#	SET POINT DRIFT IN APRM AT FITZPATRICK*	154672 P264
	#	SET POINT DRIFT IN APRM CHANNEL AT BROWNS FERRY 3*	153457 P102
	#	SET POINT DRIFT IN APRM CHANNEL AT FITZPATRICK*	154503 P264
CITIES 2*	#	SET POINT DRIFT IN AUTOMATIC BLOWDOWN TIMER RELAY AT QUAD	155491 P552
BRUNSWICK 1*	#	SET POINT DRIFT IN CONDENSATE STERAGE TANK LEVEL SWITCH AT	155565 P127
*	#	SET POINT DRIFT IN CONDENSER LOW VACUUM SWITCH AT PILGRIM 1	156158 P507
	#	SET POINT DRIFT IN CONDENSER PRESSURE SWITCH AT DRESDEN 2*	161871 P221
BRUNSWICK 1*	#	SET POINT DRIFT IN CONTAINMENT ATMOSPHERE MONITOR AT	163502 P159
	#	SET POINT DRIFT IN CONTAINMENT GAS ANALYZER AT BRUNSWICK 2*	160499 P186
MILLSTONE 1*	#	SET POINT DRIFT IN CONTAINMENT HIGH PRESSURE SWITCH AT	160459 P423
BRUNSWICK 2*	#	SET POINT DRIFT IN CONTAINMENT OXYGEN ANALYZER AT	161981 P201
CITIES 2*	#	SET POINT DRIFT IN CONTAINMENT PRESSURE SWITCH AT QUAD-	161458 P561
DUANE ARNOLD*	#	SET POINT DRIFT IN CONTAINMENT SPRAY PRESSURE SWITCH AT	160460 P 18
DUANE ARNOLD*	#	SET POINT DRIFT IN CONTAINMENT SPRAY PRESSURE SWITCHES AT	155512 P 5
OYSTER CREEK*	#	SET POINT DRIFT IN CONTAINMENT SPRAY PRESSURE SWITCHES AT	161763 P467
CREEK*	#	SET POINT DRIFT IN CONTAINMENT SPRAY SWITCHES AT OYSTER	162041 P470
AT DRESDEN 3*	#	SET POINT DRIFT IN CONTAINMENT VENT AND FUEL POOL MONITORS	156336 P229
INDICATORS AT HATCH 2*	#	SET POINT DRIFT IN CONTROL ROD DRIVE ACCUMULATOR PRESSURE	158644 P378

FITZPATRICK*
ARNOLD*
BOTTOM 2*
CREEK*
SWITCH AT PEACH BOTTOM 3*

PEACH BOTTOM 2*
FERRY 3*
ARNOLD*
OYSTER CREEK*
3*
SWITCH AT QUAD-CITIES 1*

FITZPATRICK*
OYSTER CREEK*
OYSTER CREEK*
2*
2*
BRUNSWICK 1*
BRUNSWICK 1*
HATCH 1*
HATCH 1*
2*
*

SWITCH AT BROWNS FERRY 1*
BRUNSWICK 2*
HATCH 1*
SWITCH AT QUAD CITIES 1*

SWITCH AT DRESDEN 2*
DRESDEN 2*

MILLSTONE 1*
DRESDEN 2*

SWITCHES AT MONTICELLO*
MILLSTONE 1*
AT BRUNSWICK 1*
MONTICELLO*
PEACH BOTTOM 2*
QUAD CITIES 2*
DRESDEN 2*
BRUNSWICK 2*
HATCH 2*
YANKEE*
BROWNS FERRY 2*
CREEK*

VERMONT YANKEE*
BRUNSWICK 2*

FITZPATRICK*
BROWNS FERRY 2*
FITZPATRICK*
AT HATCH 1*
BOTTOM 3*
SWITCHES AT PEACH BOTTOM 2*
OYSTER CREEK*
BROWNS FERRY 2*
2*
POINT*
POINT*
POINT*
AT OYSTER CREEK 1*
BRUNSWICK 2*
BRUNSWICK 2*
FERRY 3*
YANKEE*
BRUNSWICK 2*
OYSTER CREEK*
BRUNSWICK 1*
BRUNSWICK 1*
BRUNSWICK 2*
BRUNSWICK 2*
CITIES 1*
BRUNSWICK 2*
2*

FERRY 3*
BRUNSWICK 1*
SWITCH AT PEACH BOTTOM 3*
CREEK*
FERRY 2*

# SET POINT DRIFT IN CORE SPRAY PERMISSIVE SWITCH AT	154604	P265
# SET POINT DRIFT IN CORE SPRAY PRESSURE SWITCH AT DUANE	160353	P 17
# SET POINT DRIFT IN CORE SPRAY PRESSURE SWITCH AT PEACH	160550	P483
# SET POINT DRIFT IN CORE SPRAY PRESSURE SWITCHES AT OYSTER	162044	P469
# SET POINT DRIFT IN CORE SPRAY PUMP DISCHARGE PRESSURE	154017	P488
# SET POINT DRIFT IN CORE SPRAY SWITCH AT BROWNS FERRY 1*	163338	P 78
# SET POINT DRIFT IN DEMINERALIZER TEMPERATURE SWITCH AT	162047	P467
# SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCH AT BROWNS	161598	P118
# SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCH AT DUANE	161990	P 23
# SET POINT DRIFT IN DRYWELL HIGH PRESSURE SWITCHES AT	160817	P465
# SET POINT DRIFT IN DRYWELL PRESSURE SWITCH AT PEACH BOTTOM	157744	P424
# SET POINT DRIFT IN ELECTROHYDRAULIC CONTROL FLUID PRESSURE	156962	P542
# SET POINT DRIFT IN FISSION PRODUCT MONITOR AT HATCH 1*	162743	P343
# SET POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCH AT	160435	P297
# SET POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT	160326	P462
# SET POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT	161044	P466
# SET POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT HATCH	154960	P351
# SET POINT DRIFT IN HIGH DRYWELL PRESSURE SWITCHES AT HATCH	158774	P387
# SET POINT DRIFT IN HIGH STEAMLINE RADIATION CHANNEL AT	157057	P134
# SET POINT DRIFT IN HIGH STEAMLINE RADIATION CHANNEL AT	160317	P142
# SET POINT DRIFT IN HPCI DIFFERENTIAL PRESSURE SWITCH AT	154492	P161
# SET POINT DRIFT IN HPCI DIFFERENTIAL PRESSURE SWITCH AT	154492	P298
# SET POINT DRIFT IN HPCI EXHAUST PRESSURE SWITCHES AT HATCH	156130	P356
# SET POINT DRIFT IN HPCI STEAM FLOW SENSOR AT PEACH BOTTOM 2	160485	P481
# SET POINT DRIFT IN HPCI STEAM LINE DP SWITCH AT HATCH 2*	161872	P400
# SET POINT DRIFT IN HPCI STEAM LINE FLOW SWITCH AT HATCH 2*	158771	P386
# SET POINT DRIFT IN HPCI STEAM LINE SPACE TEMPERATURE	161796	P 77
# SET POINT DRIFT IN HPCI STEAMLINE HIGH D/P SWITCH AT	156107	P177
# SET POINT DRIFT IN HPCI STEAMLINE LOW PRESSURE SWITCH AT	161614	P342
# SET POINT DRIFT IN HPCI TURBINE AREA HIGH TEMPERATURE	160431	P547
# SET POINT DRIFT IN HPCI TURBINE PRESSURE SWITCH AT HATCH 2*	160519	P399
# SET POINT DRIFT IN ISOLATION CONDENSER CONDENSATE FLOW	159243	P215
# SET POINT DRIFT IN ISOLATION CONDENSER FLOW SWITCH AT	160002	P211
# SET POINT DRIFT IN LOW REACTOR LEVEL SWITCHES AT COOPER*	161622	P256
# SET POINT DRIFT IN LOW WATER LEVEL SCRAM SWITCH AT	160923	P424
# SET POINT DRIFT IN LPCI DRYWELL HIGH PRESSURE SWITCH AT	163379	P222
# SET POINT DRIFT IN LPCI PRESSURE PERMISSIVE AT PILGRIM 1*	157044	P507
# SET POINT DRIFT IN MAIN STEAM LINE AREA TEMPERATURE	156044	P430
# SET POINT DRIFT IN MAIN STEAM LINE HIGH FLOW SWITCH AT	155195	P420
# SET POINT DRIFT IN MAIN STEAM LINE HIGH RADIATION MONITOR	154882	P130
# SET POINT DRIFT IN MAIN STEAM LINE LOW PRESSURE SWITCH AT	155191	P428
# SET POINT DRIFT IN MAIN STEAM LINE LOW PRESSURE SWITCH AT	162487	P486
# SET POINT DRIFT IN MAIN STEAM LINE LOW PRESSURE SWITCH AT	159342	P557
# SET POINT DRIFT IN MAIN STEAM LINE PRESSURE SWITCH AT	163515	P223
# SET POINT DRIFT IN MAIN STEAM LINE RADIATION MONITOR AT	154359	P162
# SET POINT DRIFT IN MAIN STEAM LINE TEMPERATURE SWITCHES AT	157129	P361
# SET POINT DRIFT IN MAIN STEAM RELIEF VALVES AT VERMONT	160930	P578
# SET POINT DRIFT IN MAINSTEAM LINE HIGH FLOW SWITCH AT	153489	P 79
# SET POINT DRIFT IN MSIV LOW PRESSURE SENSORS AT OYSTER	159575	P464
# SET POINT DRIFT IN PRESSURE RELIEF VALVE AT VERMONT YANKEE*	160535	P579
# SET POINT DRIFT IN PRESSURE SWITCH AT BROWNS FERRY 2*	161620	P119
# SET POINT DRIFT IN RCIC DIFFERENTIAL PRESSURE SWITCH AT	153480	P565
# SET POINT DRIFT IN RCIC EXHAUST PRESSURE SWITCH AT	155317	P173
# SET POINT DRIFT IN RCIC FLOW INSTRUMENT AT HATCH 1*	154493	P298
# SET POINT DRIFT IN RCIC FLOW SWITCH AT HATCH 1*	156096	P309
# SET POINT DRIFT IN RCIC HIGH STEAM FLOW SWITCH AT	154693	P261
# SET POINT DRIFT IN RCIC REACTOR WATER LEVEL TRIP SWITCH AT	159284	P113
# SET POINT DRIFT IN RCIC STEAM LINE LOW PRESSURE SWITCH AT	163404	P296
# SET POINT DRIFT IN RCIC TURBINE EXHAUST PRESSURE SWITCHES	155326	P307
# SET POINT DRIFT IN RCIC TURBINE STEAM FLOW SWITCH AT PEACH	155347	P491
# SET POINT DRIFT IN RCIC TURBINE STEAM LINE HIGH FLOW	161779	P484
# SET POINT DRIFT IN REACTOR HIGH PRESSURE SCRAM SENSORS AT	159326	P460
# SET POINT DRIFT IN REACTOR HIGH PRESSURE SWITCHES AT	159336	P 90
# SET POINT DRIFT IN REACTOR HIGH PRESSURE SWITCHES AT HATCH	160370	P396
# SET POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG ROCK	157074	P 34
# SET POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG ROCK	157076	P 35
# SET POINT DRIFT IN REACTOR LEVEL TRANSMITTER AT BIG ROCK	161170	P 41
# SET POINT DRIFT IN REACTOR LO-LO-LC WATER LEVEL SWITCHES	161873	P468
# SET POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT	159634	P178
# SET POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT	161985	P193
# SET POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT BROWNS	157716	P109
# SET POINT DRIFT IN REACTOR LOW PRESSURE SWITCH AT VERMONT	154714	P566
# SET POINT DRIFT IN REACTOR LOW PRESSURE SWITCHES AT	158755	P171
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SENSOR AT	160148	P465
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT	160301	P139
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT	156932	P143
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT	156938	P178
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT	161652	P197
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCH AT QUAD	156912	P543
# SET POINT DRIFT IN REACTOR LOW WATER LEVEL SWITCHES AT	161069	P182
# SET POINT DRIFT IN REACTOR PRESSURE SWITCH AT BROWNS FERRY	156948	P 87
# SET POINT DRIFT IN REACTOR PRESSURE SWITCH AT HATCH 2*	160430	P397
# SET POINT DRIFT IN REACTOR PRESSURE SWITCHES AT BROWNS	153436	P102
# SET POINT DRIFT IN REACTOR VESSEL SHROUD LEVEL SWITCH AT	154554	P124
# SET POINT DRIFT IN REACTOR WATER CLEANUP TEMPERATURE	160431	P498
# SET POINT DRIFT IN REACTOR WATER LEVEL SENSORS AT OYSTER	159672	P463
# SET POINT DRIFT IN REACTOR WATER LEVEL SWITCH AT BROWNS	155205	P 81

	# SET POINT DRIFT IN REACTOR WATER LEVEL SWITCH AT DRESDEN 2*	155197	P298
FITZPATRICK*	# SET POINT DRIFT IN REACTOR WATER LEVEL SWITCHES AT	163361	P295
ROCK POINT*	# SET POINT DRIFT IN REACTOR WATER LEVEL TRANSMITTER AT BIG	158777	P 37
FERRY 3*	# SET POINT DRIFT IN RECIRCULATION VALVE SWITCH AT BROWNS	161934	P122
DRESDEN 2*	# SET POINT DRIFT IN REFUELING FLOOR RADIATION MONITOR AT	160524	P221
DUANE ARNOLD*	# SET POINT DRIFT IN RHR LOW PRESSURE PERMISSIVE SWITCH AT	155510	P 4
	# SET POINT DRIFT IN RHR PUMP START RELAY AT HATCH 2*	156133	P358
	# SET POINT DRIFT IN RPS LEVEL SWITCH AT DRESDEN 3*	161460	P237
FITZPATRICK*	# SET POINT DRIFT IN SHUTDOWN COOLING PERMISSIVE SWITCHES AT	158634	P281
PEACH BOTTOM 3*	# SET POINT DRIFT IN SHUTDOWN COOLING PRESSURE SWITCH AT	159410	P493
SWITCHES AT HATCH 2*	# SET POINT DRIFT IN SHUTDOWN COOLING SYSTEM PRESSURE	158662	P379
	# SET POINT DRIFT IN SOURCE RANGE MONITOR AT DRESDEN 3*	161456	P239
HATCH 1*	# SET POINT DRIFT IN STANDBY LIQUID CONTROL RELIEF VALVE AT	154889	P303
DRESDEN 3*	# SET POINT DRIFT IN STEAM LINE OVER TEMPERATURE SWITCH AT	155445	P229
BOTTOM 3*	# SET POINT DRIFT IN STEAM LINE PRESSURE SWITCH AT PEACH	161961	P485
	# SET POINT DRIFT IN THREE RELIEF VALVES AT ARNOLD*	156225	P 6
NINE MILE POINT 1*	# SET POINT DRIFT IN TURBINE ANTICIPATORY TRIP SWITCH AT	159235	P442
AT HATCH 2*	# SET POINT DRIFT IN TURBINE CONTROL VALVE PRESSURE SWITCH	156118	P359
SWITCHES AT BROWNS FERRY 1*	# SET POINT DRIFT IN TURBINE FIRST STAGE PRESSURE PERMISSIVE	159418	P 68
MILE POINT 1*	# SET POINT DRIFT IN TURBINE REACTOR SCRAM SWITCHES AT NINE	160313	P444
SWITCHES AT BROWNS FERRY 2*	# SET POINT DRIFT OCCURS ON TURBINE PRESSURE PERMISSIVE	157743	P 88
QUAD CITIES 1*	# SET POINT DRIFT OF HIGH DRYWELL PRESSURE SWITCH FOUND AT	156964	P543
HATCH 2*	# SET POINT DRIFTS IN CORE SPRAY PUMP PRESSURE SWITCH AT	160455	P397
#CORE SPRAY PIPE BREAK DETECTOR	SET POINT EXCEEDS LIMIT AT FITZPATRICK*	155303	P270
#INCORRECT	SET POINT FOUND IN REACTOR LEVEL SWITCH AT HATCH 1*	161951	P341
#INCORRECT	SET POINT IN REACTOR LEVEL SWITCH AT BROWNS FERRY 2*	161866	P 99
#REACTOR HIGH PRESSURE SWITCH	SET POINT INCORRECT AT HATCH 2*	159305	P394
#PRESSURE RELIEF	SET POINT INCORRECT FOR CORE SPRAY AT OYSTER CREEK*	160257	P457
#REACTOR WATER LEVEL SWITCHES EXCEED	SET POINT LIMIT AT PILGRIM 1*	160345	P521
#	SET POINT MAIN STEAM LINE PRESSURE SWITCH AT HATCH 2*	161473	P401
#TURBINE FIRST STAGE PRESSURE SWITCH	SET POINTS HIGH AT BROWNS FERRY 3*	159245	P114
#REACTOR RECIRCULATION MG	SET TRIPPED AT BROWNS FERRY 2*	157736	P 28
#REACTOR RECIRCULATION MOTOR-GENERATOR	SET TRIPS AT BROWNS FERRY 2*	156213	P 86
#RECIRCULATION PUMP MOTOR GENERATOR	SET TRIPS AT QUAD CITIES 1*	158629	P543
#CORE SPRAY PUMP DISCHARGE SWITCH EXCEEDS	SETPOINT AT BROWNS FERRY 1*	158269	P 63
#REACTOR WATER LEVEL SWITCH TRIPS BELOW	SETPOINT AT DRESDEN 3*	158749	P232
#REACTOR WATER LEVEL SWITCH TRIPS BELOW	SETPOINT AT DRESDEN 3*	158750	P233
#MAIN STEAM LINE HIGH FLOW SWITCH FAILS TO TRIP ON	SETPOINT AT DRESDEN 3*	158821	P233
#MAIN STEAM LINE HIGH FLOW SENSORS FAIL TO TRIP ON	SETPOINT AT OYSTER CREEK*	163385	P470
#TURBINE PRESSURE PERMISSIVE AND PRESSURE SWITCHES HAVE	SETPOINT DRIFT AT BROWNS FERRY 3*	159108	P110
FITZPATRICK*	# SETPOINT DRIFT IN CONDENSATE STORAGE TANK LEVEL SWITCH AT	156197	P274
AT FITZPATRICK*	# SETPOINT DRIFT IN CORE SPRAY PIPE BREAK DETECTION SWITCHES	159313	P248
SYSTEM AT MILLSTONE 1*	# SETPOINT DRIFT IN PRESSURE SWITCHES FOR REACTOR PROTECTION	158276	P422
AT OYSTER CREEK*	# SETPOINT DRIFT IN STANDBY GAS TREATMENT RADIATION MONITORS	155557	P451
VALVE AT HATCH 2*	# SETPOINT DRIFT IN STANDBY LIQUID CONTROL DISCHARGE RELIEF	157072	P367
DRESDEN 3*	# SETPOINT DRIFT IN STANDBY LIQUID CONTROL RELIEF VALVES AT	156010	P228
QUAD CITIES 2*	# SETPOINT DRIFT IN TORUS VACUUM BREAKER PRESSURE SWITCH AT	159344	P557
#CONTROL ROD BLOCK MONITOR	SETPOINT DRIFTS AT BRUNSWICK 1*	160467	P151
#REACTOR HIGH PRESSURE SWITCH	SETPOINT DRIFTS AT HATCH 2*	159588	P328
DEPRESSURIZING SYSTEM AT BIG ROCK POINT*	# SETPOINT DRIFTS IN SENSOR CHANNEL OF REACTOR	158266	P 36
DEPRESSURIZING SYSTEM AT BIG ROCK POINT*	# SETPOINT DRIFTS IN SENSOR CHANNEL OF REACTOR	158267	P 36
DEPRESSURIZING SYSTEM AT BIG ROCK POINT*	* SETPOINT DRIFTS IN SENSORS CHANNEL OF REACTOR	158268	P 37
#ROD WITHDRAWAL BLOCK	SETPOINT EXCEEDS LIMIT AT PEACH BOTTOM 2*	155380	P476
#PRESSURE SWITCH	SETPOINT EXCEEDS LIMIT AT VERMONT YANKEE*	159525	P575
#INSTRUMENT AC UNDERVOLTAGE RELAY	SETPOINT FALLS BELOW LIMIT AT DUANE ARNOLD*	163373	P 24
#REACTOR LOW WATER LEVEL SWITCH	SETPOINT INCORRECT AT BROWNS FERRY 2*	159685	P 90
#APR MONITOR	SETPOINT INCORRECT AT FITZPATRICK*	160228	P287
#TURBINE COMPARTMENT TEMPERATURE SWITCH	SETPOINT IS HIGH AT PILGRIM 1*	159242	P511
#REACTOR LOW WATER LEVEL SWITCH	SETPOINT IS INCORRECT AT BRUNSWICK 2*	159356	P184
#ADS TIMER	SETPOINT IS INCORRECT AT FITZPATRICK*	159573	P289
DRESDEN 3*	# SETPOINT OF MSIV SCRAM LIMIT SWITCHES OUT OF LIMITS AT	155599	P227
#LO LO LO REACTOR WATER LEVEL	SETPOINT RAISED AT NINE MILE POINT 1*	154932	P440
#RPS, HPCI, RCIC, AND CS PRESSURE SWITCHES	SETPOINTS DRIFT AT HATCH 2*	159591	P386
#APRM	SETPOINTS EXCEED LIMITS AT BRUNSWICK 1*	160204	P149
#HPCI AND RCIC STEAM PRESSURE SWITCH	SETPOINTS INCORRECT AT FITZPATRICK*	159576	P288
#REACTOR HIGH PRESSURE SWITCH	SETPOINTS TOO HIGH AT HATCH 1*	159501	P323
#ELECTROMATIC RELIEF VALVE SENSOR	SETPOINTS WERE TOO HIGH AT OYSTER CREEK*	160261	P458
#RPS TRIP	SETTING NONCONSERVATIVE AT PILGRIM 1*	160894	P529
BOTTOM 3*	# SEVEN CONTAINMENT ISOLATION VALVES FOUND CLOSED AT PEACH	160504	P499
#DIESEL GENERATOR FAILS TO START WITHIN	SEVEN SECONDS AT HATCH 1*	153506	P297
LIMITS AT PEACH BOTTOM 2*	# SEVERAL CONTAINMENT ISOLATION VALVES' LEAK RATES EXCEED	159578	P479
#HYDROGEN RECOMBINER	SHEATH TEMPERATURE LOW AT HATCH 2*	154944	P350
#CRYSTALLINE DEPOSITS FOUND ON DRYWELL SACRIFICIAL	SHIELD AT HATCH 2*	159325	P389
#CONTAINMENT GASEOUS MONITOR	SHIFTS DOWNSCALE AT BRUNSWICK 1*	156227	P133
	# SHOCK SUPPRESSOR FAILS TO LOCK UP AT VERMONT YANKEE*	160897	P577
	# SHOCK SUPPRESSOR INOPERABLE AT COOPER*	162520	P259
#CONTROL ROD MOVED WITH	SHORTING LINKS INSTALLED AT BRUNSWICK 2*	163471	P172
#RECIRCULATION PUMP LOW OIL LEVEL ALARM	SHORTS AT BROWNS FERRY 1*	156860	P 59
#RCIC VALVE TRANSFERS OPEN WHEN BREAKER CONTACT	SHORTS AT COOPER*	157036	P244
#REACTOR RECIRCULATION PUMP MOTOR	SHORTS AT DUANE ARNOLD*	158609	P 9
#HPCI STEAM LINE DELTA P SWITCH	SHORTS AT HATCH 2*	156883	P376
#SET POINT DRIFT IN REACTOR VESSEL	SHROUD LEVEL SWITCH AT BRUNSWICK 1*	154594	P124
#FUEL	SHUFFLING ERROR AT BROWNS FERRY 2*	160496	P 95
#FISH KILL OCCURS AFTER A	SHUT DOWN AT OYSTER CREEK*	154622	P348
#FISH KILL OCCURS AFTER A	SHUT DOWN AT OYSTER CREEK*	154622	P470
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#RHR SERVICE WATER PUMP FAILS TO START AT BRUNSWICK 1* 155739 P134
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HATCH 1*	#	STATION BATTERY CHARGERS FAIL TO MAINTAIN REQUIRED LOAD AT	156121	P360
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	#NO FAILURE -	STATION ORGANIZATION CHANGED AT PILGRIM 1*	160229	P523
	#HPCI STOP VALVE FAILS TO	STAY OPEN AT BROWNS FERRY 1*	153470	P 46
	#RCIC DRAIN LINE LEAKS	STEAM AT MONTICELLO*	160210	P437
	#MAIN	STEAM BYPASS VALVE INOPERABLE AT PEACH BOTTOM 2*	161618	P485
	#HPCI HIGH	STEAM FLOW INSTRUMENT FAILS DOWNSCALE AT COOPER*	154051	P241
	#SET POINT DRIFT IN HPCI	STEAM FLOW SENSOR AT PEACH BOTTOM 2*	160485	P481
	#SET POINT DRIFT IN HIGH	STEAM FLOW SWITCH AT FITZPATRICK*	154653	P261
	#SET POINT DRIFT IN RCIC TURBINE	STEAM FLOW SWITCH AT PEACH BOTTOM 3*	155347	P491
	#MAIN	STEAM ISOLATION SWITCHES FAIL TO OPERATE AT BROWNS FERRY 3*	158235	P106
	#HPCI	STEAM ISOLATION VALVE CLOSES AT FITZPATRICK*	155517	P273
	#MAIN	STEAM ISOLATION VALVE CLOSES TOO FAST AT BROWNS FERRY 3*	160080	P116
	#MAIN	STEAM ISOLATION VALVE FAILS TO CLOSE AT BIG ROCK POINT*	161674	P 42
	#MAIN	STEAM ISOLATION VALVE FAILS TO CLOSE AT HATCH 2*	158245	P380
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	#MAIN	STEAM ISOLATION VALVE FAILS TO CLOSE IN TIME AT COOPER*	163531	P260
	#RCIC	STEAM ISOLATION VALVE LEAKS AT BRUNSWICK 2*	162720	P209
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	#LEAK RATE OF THREE MAIN	STEAM ISOLATION VALVES EXCEED LIMIT AT BROWNS FERRY 1,*	154617	P 48
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1*	#TWO MAIN	STEAM ISOLATION VALVES FAIL TO MEET LEAK RATE AT MILLSTONE	160060	P423
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	#SET POINT DRIFT IN MAIN	STEAM LINE AREA TEMPERATURE SWITCHES AT MONTICELLO*	156044	P430
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1*	#MAIN	STEAM LINE DRAIN ISOLATION VALVE FAILS TO CLOSE AT PILGRIM	159280	P520
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	#SET POINT DRIFT IN MAIN	STEAM LINE LOW PRESSURE SWITCH AT MONTICELLO*	155191	P428
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*	#RCIC	STEAM SUPPLY LINE ISOLATION VALVE FAILS TO CLOSE AT HATCH 2	158643	P370
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#HPCI STEAM	SUPPLY VALVE FAILS TO OPEN AT DRESDEN 3*	160903	P238
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#SET POINT DRIFT IN REACTOR LOW WATER LEVEL	SWITCH AT BRUNSWICK 2*	156938 P178
#SET POINT DRIFT IN REACTOR LOW PRESSURE	SWITCH AT BRUNSWICK 2*	159604 P178
#SET POINT DRIFT IN REACTOR LOW PRESSURE	SWITCH AT BRUNSWICK 2*	161985 P193
#SET POINT DRIFT IN REACTOR LOW WATER LEVEL	SWITCH AT BRUNSWICK 2*	161652 P197
#SET POINT DRIFT IN REACTOR WATER LEVEL	SWITCH AT DRESDEN 2*	153197 P208
#SET POINT DRIFT IN ISOLATION CONDENSER FLO*	SWITCH AT DRESDEN 2*	160302 P211
#SET POINT DRIFT IN ISOLATION CONDENSER CONDENSATE FLO*	SWITCH AT DRESDEN 2*	159243 P215
#SET POINT DRIFT IN CONDENSER PRESSURE	SWITCH AT DRESDEN 2*	161871 P221
#SET POINT DRIFT IN LPCI DRYWELL HIGH PRESSURE	SWITCH AT DRESDEN 2*	163379 P222
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#SET POINT DRIFT IN RHR LOW PRESSURE PERMISSIVE	SWITCH AT DUANE ARNOLD*	155510 P 4
#SET POINT DRIFT IN CORE SPRAY DIFFERENTIAL PRESSURE	SWITCH AT DUANE ARNOLD*	155514 P 5
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#SET POINT DRIFT IN CONTAINMENT SPRAY PRESSURE	SWITCH AT DUANE ARNOLD*	160460 P 18
#SET POINT DRIFT IN DRYWELL HIGH PRESSURE	SWITCH AT DUANE ARNOLD*	161990 P 23
#SET POINT DRIFT IN RCIC HIGH STEAM FLO*	SWITCH AT FITZPATRICK*	154653 P261

#SET POINT DRIFT IN CORE SPRAY PERMISSIVE	SWITCH AT FITZPATRICK*	154664 P365
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#SET POINT DRIFT IN HPCI DIFFERENTIAL PRESSURE	SWITCH AT HATCH 1*	154492 P298
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#SET POINT DRIFT IN HPCI STEAMLINE LOW PRESSURE	SWITCH AT HATCH 1*	161614 P342
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#SET POINT DRIFT IN HPCI STEAM LINE FLOW	SWITCH AT HATCH 2*	158771 P386
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#RHR PUMP	TIMER RELAY FAILS AT QUAD-CITIES 2*		155985 P553
#ADS	TIMER RELAY OPENS TOO SOON AT BROWNS FERRY 1*		154917 P 51
#ADS	TIMER SETPOINT IS INCORRECT AT FITZPATRICK*		159573 P289
#TWO RHR DELAY	TIMERS FAIL TO ACTUATE THE TIMER CONTACTS AT QUAD CITIES 2*		156019 P554
DISCHARGE TEMPERATURE EXCEEDS LIMIT 4	TIMES AT OYSTER CREEK*		160300 P472
#SUPPRESSION POOL LEVEL OUT OF LIMITS THREE	TIMES AT QUAD CITIES 1*		153947 P537
#COBALT AND SILVER ACTIVITY LEVELS EXCEED TEN	TIMES CONTROL ACTIVITIES AT MILLSTONE*		161680 P425
#OYSTER SAMPLE SILVER ACTIVITY EXCEEDS TEN	TIMES CONTROL ACTIVITY AT MILLSTONE*		161827 P426
#DIESEL GENERATOR FAILS 3	TIMES DURING 24 HOUR LOAD TEST AT PILGRIM 1*		158646 P509
#MSIV CLOSURE	TIMES EXCEED LIMITS AT DRESDEN 3*		160176 P235
#SET POINT DRIFT FOUND IN MSIV CLOSURE	TIMING AT QUAD-CITIES 2*		159569 P555
#PRIMARY CONTAINMENT ISOLATION VALVE	TIMING REQUIREMENTS NOT MET AT COOPER*		159556 P247
FERRY 1* #SCRAM	TIMING TEST PERFORMED WITH WRONG RCS PRESSURE AT BROWNS		157046 P 57
ISOLATION TEST AT QUAD-CITIES 2*	# TIP FAILS TO WITHDRAW FROM THE CORE ON CONTAINMENT		156161 P554
#RHR SERVICE WATER PUMP TIMER OUT OF	TOLERANCE AT BROWNS FERRY 1*		156212 P 56
#TORUS LEVEL TRANSMITTER OUT OF	TOLERANCE AT BRUNSWICK 1*		155559 P129
#DRYWELL PRESSURE SWITCH IS OUT OF	TOLERANCE AT BRUNSWICK 2*		159621 P181
#DRYWELL DRAIN SUMP PUMP FLOW SENSORS OUT OF	TOLERANCE AT FITZPATRICK*		159575 P288
#MAIN STEAM ISOLATION VALVES CLOSE	TOO FAST AND FAIL TO INDICATE CLOSED AT QUAD CITIES 2*		160838 P559
#MAIN STEAM ISOLATION VALVE CLOSES	TOO FAST AT BROWNS FERRY 3*		160080 P116
#MSIV CLOSING TIME	TOO FAST AT COOPER*		159467 P250
#REACTOR HIGH PRESSURE SWITCH SETPOINTS	TOO HIGH AT HATCH 1*		159501 P323
#CORE MAXIMUM FRACTION OF LIMITING POWER DENSITY	TOO HIGH AT HATCH 2*		157172 P378
#CORE MAXIMUM FRACTION OF LIMITING POWER DENSITY IS	TOO HIGH AT HATCH 2*		159618 P383
#REACTOR HIGH PRESSURE SCRAM SENSOR TRIPS 9 PSIG	TOO HIGH AT OYSTER CREEK*		157699 P456
#ELECTROMATIC RELIEF VALVE SENSOR SETPOINTS WERE	TOO HIGH AT OYSTER CREEK*		160261 P458
#REACTOR LEVEL SWITCHES TRIP	TOO LATE AT HATCH 2*		160518 P398
#ROD BLOCK MONITOR BYPASSED	TOO LONG AT BROWNS FERRY 2*		156015 P 85
#DIESEL DRIVEN FIRE PUMP OUT OF SERVICE	TOO LONG AT DRESDEN 2*		160362 P217
#SMOKE DETECTORS OUT OF SERVICE	TOO LONG AT DRESDEN 2*		160824 P219
GENERATOR DAY TANK ROOM FIRE SPRINKLERS OUT OF SERVICE	TOO LONG AT DUANE ARNOLD*	#DIESEL	158665 P 28
#DRYWELL TO TORUS DIFFERENTIAL PRESSURE FALLS BELOW LIMIT	TOO LONG AT VERMONT YANKEE*		159171 P574
#CONTAINMENT PURGE OPERATED	TOO LONG WHEN INTEGRITY REQUIRED AT HATCH 2*		160652 P380
#DRYWELL/TORUS DIFFERENTIAL PRESSURE FALLS	TOO LOW AT HATCH 1*		156953 P315
#NITROGEN STORAGE TANK LEVEL	TOO LOW AT HATCH 1*		159503 P324
#ADS TIMER RELAY OPENS	TOO SOON AT BROWNS FERRY 1*		154917 P 51
#PLANT SERVICE WATER PUMP SEQUENCED TO BUS	TOO SOON AT HATCH 2*		156122 P360
#CORE SPRAY PUMP STARTS	TOO SOON AT PEACH BOTTOM 3*		160332 P498

	#STEAM LEAK DETECTION LOGIC	TOPAZ INVERTER TRIPS AT BRUNSWICK 2*	155[illegible] P165
	#UPDATE ON DEFECTIVE MSIV	TORQUE SWITCH AT HATCH 2*	163[illegible]62 P373
	#	TORUS AIR AND WATER TEMPERATURE RECORDER FAILS AT HATCH 1*	158597 P316
INDICATORS AT HATCH 2*	#	TORUS AND DRYWELL TEMPERATURE SENSORS CONNECTED TO WRONG	156983 P365
	#INTACT BLISTERS FOUND ON THE	TORUS COATING UNDER THE WATER LINE AT BRUNSWICK 2*	156076 P172
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	#LOW READING IN DRYWELL-	TORUS DIFFERENTIAL PRESSURE CHANNEL AT HATCH 1*	154881 P304
FITZPATRICK*	#DRYWELL TO	TORUS DIFFERENTIAL PRESSURE FALLS BELOW LIMIT AT	155305 P270
FITZPATRICK*	#DRYWELL TO	TORUS DIFFERENTIAL PRESSURE FALLS BELOW LIMIT AT	157084 P275
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	#DRYWELL TO	TORUS DP DROPPED BELOW LIMITS AT DRESDEN 2*	158247 P214
	#DESIGN DEFICIENCY FOUND IN	TORUS GRAVITY FILL VALVE AT HATCH 2*	154946 P349
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	#DRYWELL AND	TORUS HYDROGEN OXYGEN RECORDER INOPERABLE AT HATCH 1*	159290 P333
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	#	TORUS LEVEL EXCEEDS HIGH LIMIT AT BRUNSWICK 2*	159513 P173
	#	TORUS LEVEL EXCEEDS LIMIT AT BRUNSWICK 2*	161413 P209
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	#	TORUS OXYGEN CONCENTRATION EXCEEDS LIMIT AT OYSTER CREEK*	159327 P461
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	#RHR	TORUS RETURN VALVE FAILS TO OPERATE AT BROWNS FERRY 1*	160155 P 72
	#RTGB CONTROL SWITCH FOR HPCI	TORUS SUCTION VALVE FAILS AT BRUNSWICK 1*	155397 P132
	#	TORUS TEMPERATURE EXCEEDS LIMIT AT HATCH 2*	159308 P362
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	#	TORUS TEMPERATURE RECORDER FAILS AT HATCH 2*	156934 P373
	#DRYWELL AND	TORUS TEMPERATURE RECORDERS FAIL AT HATCH 1*	159271 P333
	#	TORUS TO DRYWELL VACUUM BREAKER FAILS AT DRESDEN 2*	158556 P211
INDICATION AT DRESDEN 2*	#	TORUS TO DRYWELL VACUUM BREAKER FAILS TO GIVE OPEN	160266 P214
	#LEAKAGE OF	TORUS TC RB VACUUM BREAKER EXCEEDS LIMIT AT PILGRIM 1*	154689 P503
	#	TORUS VACUUM BREAKER FAILS AT HATCH 2*	156933 P372
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	#	TORUS VACUUM BREAKER VALVE FAILED OPEN AT HATCH 2*	159629 P385
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	#LEAK RATE OF	TORUS VENT VALVE EXCEEDS LIMIT AT MONTICELLO*	156241 P430
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1*	#SOME	TORUS WATER DRAINED THROUGH CORE SPRAY SYSTEM AT BRUNSWICK	155314 P133
	#	TORUS WATER LEVEL BELOW LIMIT AT HATCH 2*	160802 P403
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	#	TORUS WATER LEVEL EXCEEDS LIMIT AT FITZPATRICK*	155550 P269
	#	TORUS WATER LEVEL FALLS BELOW LIMIT AT DUANE ARNOLD*	159215 P 13
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	#	TORUS WATER LEVEL INSTRUMENT READS DOWNSCALE AT HATCH 2*	158297 P367
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	#	TORUS WATER LEVEL RECORDER INOPERABLE AT HATCH 1*	159291 P334
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*	#	TORUS WATER LEVEL SWITCH FAILS TC ACTUATE AT BRUNSWICK 2*	155572 P167
	#NARROW RANGE	TORUS WATER LEVEL TRANSMITTER OUT OF CALIBRATION AT HATCH 2	154948 P348
	#	TORUS-DRYWELL VACUUM BREAKER FAILS TC OPEN AT HATCH 2*	156143 P355
	#	TORUS/DRYWELL VACUUM BREAKER FAILS TC OPEN AT HATCH 2*	160476 P396
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#STANDBY GAS TREATMENT		TRAIN FAILS TO RESET AT BRUNSWICK 1*	156176 P136
#STANDBY GAS TREATMENT FILTER		TRAIN FAILS TO START AT HATCH 2*	156935 P372
#A FIRE IN SOME ELECTRICAL RACEWAYS COULD EFFECT REDUNDANT		TRAINS AT PILGRIM 1*	154094 P502
#BOTH SERVICE WATER		TRAINS INOPERABLE AT HATCH 1*	159408 P338
#RMOV BOARD BREAKER FAILS TO TRANSFER DURING MANUAL		TRANSFER AT BROWNS FERRY 2*	156026 P 83
#CONTAINMENT ISOLATION VALVES LOSE POWER AND		TRANSFER CLOSED AT BROWNS FERRY 3*	157035 P106
#RMOV BOARD BREAKER FAILS TO		TRANSFER DURING MANUAL TRANSFER AT BROWNS FERRY 2*	156028 P 83
#RADIOACTIVE WATER LEAKS FROM CONDENSATE		TRANSFER PIPING AT OYSTER CREEK*	155566 P451
#THREE UNAUTHORIZED		TRANSFERS OF SPECIAL NUCLEAR MATERIAL AT HATCH 1*	161949 P343
#RCIC VALVE		TRANSFERS OPEN WHEN BREAKER CONTACT SHORTS AT COOPER*	157036 P244
* #COOLING TOWER		TRANSFORMER FEEDER BREAKERS FAIL TC CLOSE AT BROWNS FERRY 1	161794 P 77
#345KV SHUTDOWN		TRANSFORMER TRIPS AT PILGRIM 1*	155400 P502
#SET POINT DRIFT IN REACTOR LEVEL		TRANSMITTER AT BIG ROCK POINT*	157074 P 34
#SET POINT DRIFT IN REACTOR LEVEL		TRANSMITTER AT BIG ROCK POINT*	157076 P 35
#SET POINT DRIFT IN REACTOR WATER LEVEL		TRANSMITTER AT BIG ROCK POINT*	158777 P 37
#SET POINT DRIFT IN REACTOR LEVEL		TRANSMITTER AT BIG ROCK POINT*	160170 P 41
#DRYWELL SUMP FLOW		TRANSMITTER FAILS OFF AT BROWNS FERRY 3*	159492 P109
#FEEDWATER CONTROL LEVEL		TRANSMITTER FOUND OUT OF CALIBRATION AT FITZPATRICK*	156915 P277
#DRYWELL PRESSURE		TRANSMITTER NOT QUALIFIED AT BROWNS FERRY 2*	161640 P 98
#DRYWELL PRESSURE		TRANSMITTER NOT QUALIFIED AT BROWNS FERRY 2*	161599 P119
#NARROW RANGE TORUS WATER LEVEL		TRANSMITTER OUT OF CALIBRATION AT HATCH 2*	154946 P348
#TORUS LE[illegible]		TRANSMITTER OUT OF TOLERANCE AT BRUNSWICK 1*	155559 P129
#DIFFERENTIAL PRE[illegible]		TRANSMITTER VALVE FOUND CLOSED AT QUAD CITIES 1*	154443 P536
#DRYWELL PR[illegible]		TRANSMITTERS NOT QUALIFIED AT BROWNS FERRY 1*	161893 P 76
#LEAK FOUND IN RECOMBIN[illegible]		TRAP CONDENSATE LINE AT MONTICELLO*	153488 P427
OYSTER CREEK* #CONTR[illegible]		TRAVEL INTERLOCK SURVEILLANCE NOT PERFORMED ON TIME AT	158270 P458
DUANE ARNOLD* [illegible] WAT		TRAVELING SCREEN FAILS WHILE REDUNDANT SYSTEM IS FAILED AT	159051 P 14
*		TRAVERSIN[illegible] INCORE PROBE FAILS TC WITHDRAW AT PEACH BOTTOM 3	155585 P489

BRUNSWICK 2*		#	TRAVERSING INCORE PROBE POSITION INDICATION FAILS AT	162706 P202
	#STANDBY GAS		TREATMENT CHARCOAL BED LEAKS AT QUAD CITIES 1*	154920 P539
	#STANDBY GAS		TREATMENT FAN FAILS TO MEET FLOW REQUIREMENTS AT HATCH 1*	153740 P300
	#STANDBY GAS		TREATMENT FAN TRIPS ON OVERLOAD AT OYSTER CREEK*	159328 P462
PEACH BOTTOM 2*	#STANDBY GAS		TREATMENT FILTER ASSEMBLY ISOLATION VALVE FAILS TO OPEN AT	161071 P483
	#STANDBY GAS		TREATMENT FILTER TRAIN FAILS TO START AT HATCH 2*	156935 P372
BRUNSWICK 2*	#STANDBY GAS		TREATMENT OPERABILITY TEST NOT PERFORMED ON TIME AT	158602 P178
	#STANDBY GAS		TREATMENT POWER CABLES UNDERSIZED AT HATCH 2*	156881 P369
	#STANDBY GAS		TREATMENT PROCEDURE CHANGES NOT REVIEWED AT HATCH 1*	160277 P321
	#SETPOINT DRIFT IN STANDBY GAS		TREATMENT RADIATION MONITORS AT OYSTER CREEK*	155557 P451
	#STANDBY GAS		TREATMENT SYSTEM FAILS AT FITZPATRICK*	159501 P287
*	#STANDBY GAS		TREATMENT SYSTEM FLOWRATE FALLS BELOW LIMIT AT OYSTER CREEK	155456 P452
	#CONTROL ROOM AIR		TREATMENT SYSTEM INOPERABLE AT DUANE ARNOLD*	159558 P 16
	#STANDBY GAS		TREATMENT SYSTEM INOPERABLE AT PILGRIM 1*	159522 P517
	#STANDBY GAS		TREATMENT SYSTEM OPERATED IN DEGRADED MODE AT OYSTER CREEK*	160354 P463
	#FLOW LOST IN STANDBY GAS		TREATMENT SYSTEM TRAIN AT HATCH 2*	156990 P363
	#STANDBY GAS		TREATMENT SYSTEM TRIPS AT DRESDEN 3*	161062 P236
	*STANDBY GAS		TREATMENT TRAIN FAILS TO RESET AT BRUNSWICK 1*	156176 P136
	#STANDBY GAS		TREATMENT UNIT FAILS AT OYSTER CREEK*	155994 P454
		#	TRIAXIAL ACCELOGRAPH SYSTEMS ALARM AT BRUNSWICK 2*	155394 P204
CLOSES AT HATCH 2*	#RCIC		TRIP AND THROTTLE VALVE CLOSES AND HPCI ISOLATION VALVE	156957 P376
	#RCIC TURBINE		TRIP AND THROTTLE VALVE FAILS TO OPEN AT DUANE ARNOLD*	159059 P 15
	#RCIC		TRIP AND THROTTLE VALVE FAILS TO TRIP AT BRUNSWICK 2*	154678 P166
	#HPCI FAILS TO MEET FLOW REQUIREMENT DUE TO		TRIP AT BROWNS FERRY 1*	158272 P 63
	#SWITCHGEAR BREAKERS		TRIP AT BROWNS FERRY 3*	161682 P117
	#REACTOR LOW WATER LEVEL SWITCH FAILS TO		TRIP AT BRUNSWICK 1*	154880 P131
	#RCIC TRIP AND THROTTLE VALVE FAILS TO		TRIP AT BRUNSWICK 2*	154678 P166
	#ELECTRONIC DRIFT CAUSES INCORRECT ROD BLOCK		TRIP AT DRESDEN 2*	157696 P232
	#DIESEL GENERATOR OUTPUT BREAKER FAILS TO		TRIP AT HATCH 2*	158296 P371
	#RPS		TRIP CHANNEL INOPERABLE AT BRUNSWICK 2*	159451 P183
	#HPCI REACTOR HIGH WATER LEVEL		TRIP INOPERABLE AT BROWNS FERRY 1*	156011 P 55
	#HPCI TURBINE FAILS TO		TRIP ON OVERSPEED AT HATCH 1*	159434 P332
	#MAIN STEAM LINE HIGH FLOW SWITCH FAILS TO		TRIP ON SETPOINT AT DRESDEN 3*	158821 P233
	#MAIN STEAM LINE HIGH FLOW SENSORS FAIL TO		TRIP ON SETPOINT AT OYSTER CREEK*	163385 P470
	#CONTAINMENT PRESSURE SWITCHES		TRIP OUT OF SPECS AT DUANE ARNOLD*	160268 P 11
	#RPS		TRIP SETTING NONCONSERVATIVE AT PILGRIM 1*	160894 P529
	#SET POINT DRIFT IN RCIC REACTOR WATER LEVEL		TRIP SWITCH AT BROWNS FERRY 3*	159284 P113
	#SET POINT DRIFT IN TURBINE ANTICIPATORY		TRIP SWITCH AT NINE MILE POINT 1*	159235 P442
	#RECIRCULATION PUMP		TRIP SYSTEM NOT TESTED AT DUANE ARNOLD*	160202 P 15
	#UNIT 1		TRIP TEST NOT PERFORMED WHEN REQUIRED AT RUNSLOT BAY*	155480 P410
	#DG FAILS TO		TRIP TO IDLE SPEED FROM LOAD AT BROWNS FERRY 3*	160216 P116
	#REACTOR LEVEL SWITCHES		TRIP TOO LATE AT HATCH 2*	160518 P398
	#TWO SERVICE WATER PUMPS EACH		TRIP TWICE AT PILGRIM 1*	160327 P521
	#ONLY OPERABLE RECIRCULATION PUMP ACCIDENTALLY		TRIPPED AT BROWNS FERRY 2*	155594 P 82
	#REACTOR RECIRCULATION MG SET		TRIPPED AT BROWNS FERRY 2*	157736 P 88
	#MAIN STEAM LINE RADIATION MONITOR		TRIPPED AT DRESDEN 2*	160264 P213
BROWNS FERRY 3*	#RCIC		TRIPPED ON OVERSPEED DUE TO LOSS OF SPEED CONTROL AT	157436 P107
	#RCIC TURBINE		TRIPPED ON REACTOR STARTUP AT HATCH 1*	160250 P318
	#ATHENS 161 KV LINE		TRIPS AT BROWNS FERRY 1*	156013 P 55
	#HPCI TURBINE		TRIPS AT BROWNS FERRY 1*	157033 P 57
	#RECIRCULATION PUMP MOTOR GENERATOR		TRIPS AT BROWNS FERRY 1*	156862 P 60
	#RCIC TURBINE		TRIPS AT BROWNS FERRY 1*	156961 P 60
	#OFF SITE ELECTRIC POWER LINE		TRIPS AT BROWNS FERRY 1*	159267 P 68
	#RHR AREA COOLER FAN THERMAL OVERLOAD		TRIPS AT BROWNS FERRY 1*	159407 P 69
	#DIESEL GENERATOR BREAKER		TRIPS AT BROWNS FERRY 1*	163327 P 79
	#RECIRCULATION PUMP		TRIPS AT BROWNS FERRY 2*	155592 P 82
	#REACTOR RECIRCULATION MOTOR-GENERATOR SET		TRIPS AT BROWNS FERRY 2*	156213 P 86
	#BREAKER TO REACTOR MOV BOARD		TRIPS AT BROWNS FERRY 2*	158747 P 89
	#RCIC TURBINE		TRIPS AT BROWNS FERRY 3*	159488 P110
	#RHR PUMP MOTOR		TRIPS AT BROWNS FERRY 3*	159650 P113
	#REACTOR RECIRCULATION PUMP		TRIPS AT BROWNS FERRY 3*	160126 P115
	#RECIRCULATION PUMP		TRIPS AT BRUNSWICK 1*	158607 P140
	#FIRE PROTECTION DELUGE VALVE		TRIPS AT BRUNSWICK 1*	162414 P160
	#STEAM LEAK DETECTION LOGIC TOPAZ INVERTER		TRIPS AT BRUNSWICK 2*	155323 P185
	#CONTAINMENT ATMOSPHERIC MONITOR BREAKER		TRIPS AT BRUNSWICK 2*	155570 P167
	#REACTOR RECIRCULATION PUMP MOTOR GENERATOR		TRIPS AT BRUNSWICK 2*	162709 P202
	#PRESSURE SWITCH IN MAIN STEAM SYSTEM		TRIPS AT DRESDEN 3*	157058 P231
	#STANDBY GAS TREATMENT SYSTEM		TRIPS AT DRESDEN 3*	161062 P236
	#LPCI INVERTER		TRIPS AT FITZPATRICK 1*	160506 P291
	#LPCI INVERTER		TRIPS AT FITZPATRICK*	154753 P269
	#LPCI INDEPENDENT POWER SUPPLY INVERTER		TRIPS AT FITZPATRICK*	163463 P278
	#DIESEL GENERATOR		TRIPS AT FITZPATRICK*	158676 P284
	#MULTIPLE LPCI INVERTER		TRIPS AT FITZPATRICK*	160116 P290
	#STANDBY DIESEL SERVICE WATER PUMP		TRIPS AT HATCH 1*	153505 P298
	#REACTOR RECIRCULATION PUMP		TRIPS AT HATCH 1*	154887 P305
	#CONTROL ROOM FILTER FAN		TRIPS AT HATCH 1*	157121 P313
	#REACTOR PROTECTION SYSTEM MOTOR GENERATOR OUTPUT BREAKER		TRIPS AT HATCH 1*	157073 P314
	#LPCI INVERTER		TRIPS AT HATCH 1*	159189 P330
	#OFFGAS HYDROGEN SAMPLE PUMP		TRIPS AT HATCH 1*	159442 P334
	#RECIRCULATION PUMP MOTOR GENERATOR		TRIPS AT HATCH 1*	161795 P342
	#HPCI AUTOMATIC FLOW CONTROL POWER INVERTER		TRIPS AT HATCH 2*	153705 P349
	#HPCI AUXILIARY OIL PUMP FEEDER BREAKER		TRIPS AT HATCH 2*	154737 P384
	#DILUTION PUMP		TRIPS AT OYSTER CREEK*	161670 P473
	#ONE OFF-SITE POWER SOURCE		TRIPS AT PEACH BOTTOM 2*	156027 P477
	#REACTOR COOLANT LEAK DETECTION AIR SAMPLING SYSTEM		TRIPS AT PILGRIM 1*	153970 P501
	#FIRE ALARM INADVERTENTLY		TRIPS AT PILGRIM 1*	15 395 P501
	#24KV SHUTDOWN TRANSFORMER		TRIPS AT PILGRIM 1*	154400 P502

#STACK GAS GRAB SAMPLE PUMP	TRIPS AT PILGRIM 1*	159546	P512
#SALT SERVICE WATER PUMP	TRIPS AT PILGRIM 1*	159607	P516
#REACTOR RECIRCULATION PUMP	TRIPS AT PILGRIM 1*	159445	P524
#RHR SUPPRESSION CHAMBER DUMP VALVE BREAKER	TRIPS AT QUAD CITIES 1*	155522	P539
#SAFETY RELATED MOTOR CONTROL CENTER FEEDER BREAKER	TRIPS AT QUAD CITIES 1*	158624	P542
#RECIRCULATION PUMP MOTOR GENERATOR SET	TRIPS AT QUAD CITIES 1*	158629	P543
#POWER SUPPLY	TRIPS AT VERMONT YANKEE*	160103	P575
#REACTOR WATER LEVEL SWITCH	TRIPS BELOW SETPOINT AT DRESDEN 3*	158749	P232
#REACTOR WATER LEVEL SWITCH	TRIPS BELOW SETPOINT AT DRESDEN 2*	158750	P233
#HPCI	TRIPS DURING A TEST AT BRUNSWICK 1*	154511	P122
#HPCI TURBINE	TRIPS DURING TESTING AT BROWNS FERRY 2*	156017	P 86
#HPCI TURBINE	TRIPS DURING TESTING AT VERMONT YANKEE*	154875	P566
#RESIDUAL HEAT REMOVAL SYSTEM CONTROL VALVE	TRIPS IN MID-POSITION AT BROWNS FERRY 3*	154707	P103
#STANDBY LIQUID CONTROL PUMP BREAKER	TRIPS INADVERTENTLY AT BROWNS FERRY 3*	161736	P121
#RHR PUMP	TRIPS OFF AT BRUNSWICK 1*	155507	P132
#DIESEL GENERATOR	TRIPS ON HIGH ENGINE TEMPERATURE AT QUAD CITIES 1*	161063	P548
#DIESEL GENERATOR	TRIPS ON LOW LUBE OIL PRESSURE AT FITZPATRICK*	154949	P268
#RECIRCULATION PUMP	TRIPS ON LOW OIL PRESSURE AT HATCH 2*	159315	P394
#RCIC	TRIPS ON MANUAL STARTUP AT HATCH 2*	159127	P381
#RHR PUMP	TRIPS ON OVERCURRENT AT BROWNS FERRY 3*	161959	P120
#STANDBY GAS TREATMENT FAN	TRIPS ON OVERLOAD AT OYSTER CREEK*	159328	P462
#DG AUXILIARY OIL PUMP MOTOR	TRIPS OUT AT BROWNS FERRY 3*	159490	P110
#ERRONEOUS MANUAL OPERATION OF A RELAY	TRIPS RECIRC PUMP AT BROWNS FERRY 2*	155365	P 82
#FEEDER BUS FAULT	TRIPS RECIRCULATION PUMP AT BROWNS FERRY 1*	156960	P 58
#RECIRCULATION PUMP	TRIPS TWICE AT BRUNSWICK 2*	155567	P170
#REACTOR HIGH PRESSURE SCRAM SENSOR	TRIPS 9 PSIG TOO HIGH AT OYSTER CREEK*	157699	P456
#RCIC	TRUBINE WHEEL DAMAGED AT COOPER*	157032	P245
#LPCI HEAT EXCHANGER	TUBES LEAK AT DRESDEN 2*	160087	P218
#LPCI HEAT EXCHANGER	TUBES LEAK AT DRESDEN 3*	158705	P233
#CONTAINMENT SPRAY HEAT EXCHANGER	TUBES LEAK AT OYSTER CREEK*	155993	P453
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#	TURBINE BUILDING ROOF VENT MONITOR FAILS AT BROWNS FERRY 1* BROWNS FERRY 1*	159405	P 69
#	TURBINE BUILDING VENTILATION SAMPLES NOT TAKEN ON TIME AT PILGRIM 1*	159404	P 70
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#	TURBINE COMPUTER SYSTEM SPURIOUSLY SCRAMS REACTOR AT COOPER	159253	P253
#RCIC	TURBINE CONTROL FAILS AT FITZPATRICK*	159382	P290
#	TURBINE CONTROL VALVE CLOSURE SCRAM FAILS TO RESET AT DRESDEN 2*	154620	P207
#MAIN	TURBINE CONTROL VALVE CLOSURE SWITCH RESPONSE TIME IS SLOW AT BRUNSWICK 2*	156930	P176
#HPCI	TURBINE CONTROL VALVE FAILS TO OPEN AT BRUNSWICK 2*	160062	P188
#MAIN	TURBINE CONTROL VALVE FAST CLOSURE RELAY FAILS AT BRUNSWICK 2*	161070	P183
J	TURBINE CONTROL VALVE FAST CLOSURE TEST NOT PERFORMED AT HATCH 1*	154665	P301
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#LEAK RATE OF RCIC	TURBINE EXHAUST CHECK VALVE EXCEEDS LIMIT AT MONTICELLO*	156242	P431
#RCIC	TURBINE EXHAUST ISOLATION VALVE LEAK RATE EXCEEDS LIMIT AT HATCH 2*	161852	P434
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#RCIC OUTBOARD ISOLATION	VALVE FAILS TO CLOSE AT PILGRIM 1*	159536 P519
#MAIN STEAM LINE DRAIN ISOLATION	VALVE FAILS TO CLOSE AT PILGRIM 1*	159280 P520
#NITROGEN PURGE SUPPLY ISOLATION	VALVE FAILS TO CLOSE AT PILGRIM 1*	160212 P526
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#RHR SERVICE WATER VAULT BED PLATE DRAIN	VALVE FAILS TO CLOSE AT QUAD CITIES 2*	153951 P551
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#CONTAINMENT ISOLATION NITROGEN	VALVE FAILS TO CLOSE AT QUAD-CITIES 2*	159688 P558
#CONTAINMENT MONITORING ISOLATION	VALVE FAILS TO CLOSE AT VERMONT YANKEE*	156889 P572
#CONTAINMENT SUMP ISOLATION	VALVE FAILS TO CLOSE AT VERMONT YANKEE*	156722 P573
#MAIN STEAM ISOLATION	VALVE FAILS TO CLOSE IN TIME AT COOPER*	163531 P260
#REACTOR WATER CLEANUP SYSTEM ISOLATION	VALVE FAILS TO CLOSE IN TIME AT DUANE ARNOLD*	160855 P 20
#NITROGEN PURGE ISOLATION	VALVE FAILS TO CLOSE IN TIME AT PILGRIM 1*	158840 P515
#PRIMARY VENT	VALVE FAILS TO CLOSE IN TIME AT QUAD-CITIES 2*	161900 P562
#CONTAINMENT ISOLATION	VALVE FAILS TO INDICATE POSITION AT BRUNSWICK 2*	161654 P196
#STANDBY LIQUID CONTROL RELIEF	VALVE FAILS TO OPEN AT BRUNSWICK 1*	159199 P147
#RCIC INJECTION	VALVE FAILS TO OPEN AT BRUNSWICK 2*	160525 P187
#HPCI TURBINE CONTROL	VALVE FAILS TO OPEN AT BRUNSWICK 2*	160082 P188
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#LPCI INJECTION	VALVE FAILS TO OPEN AT DRESDEN 3*	156007 P228
#HPCI STEAM SUPPLY	VALVE FAILS TO OPEN AT DRESDEN 3*	160903 P238
#RCIC TURBINE TRIP AND THROTTLE	VALVE FAILS TO OPEN AT DUANE ARNOLD*	159059 P 15
#RCIC STEAM SUPPLY LINE	VALVE FAILS TO OPEN AT DUANE ARNOLD*	160501 P 21
#CORE SPRAY	VALVE FAILS TO OPEN AT FITZPATRICK*	153729 P266
#CORE SPRAY	VALVE FAILS TO OPEN AT FITZPATRICK*	154660 P267
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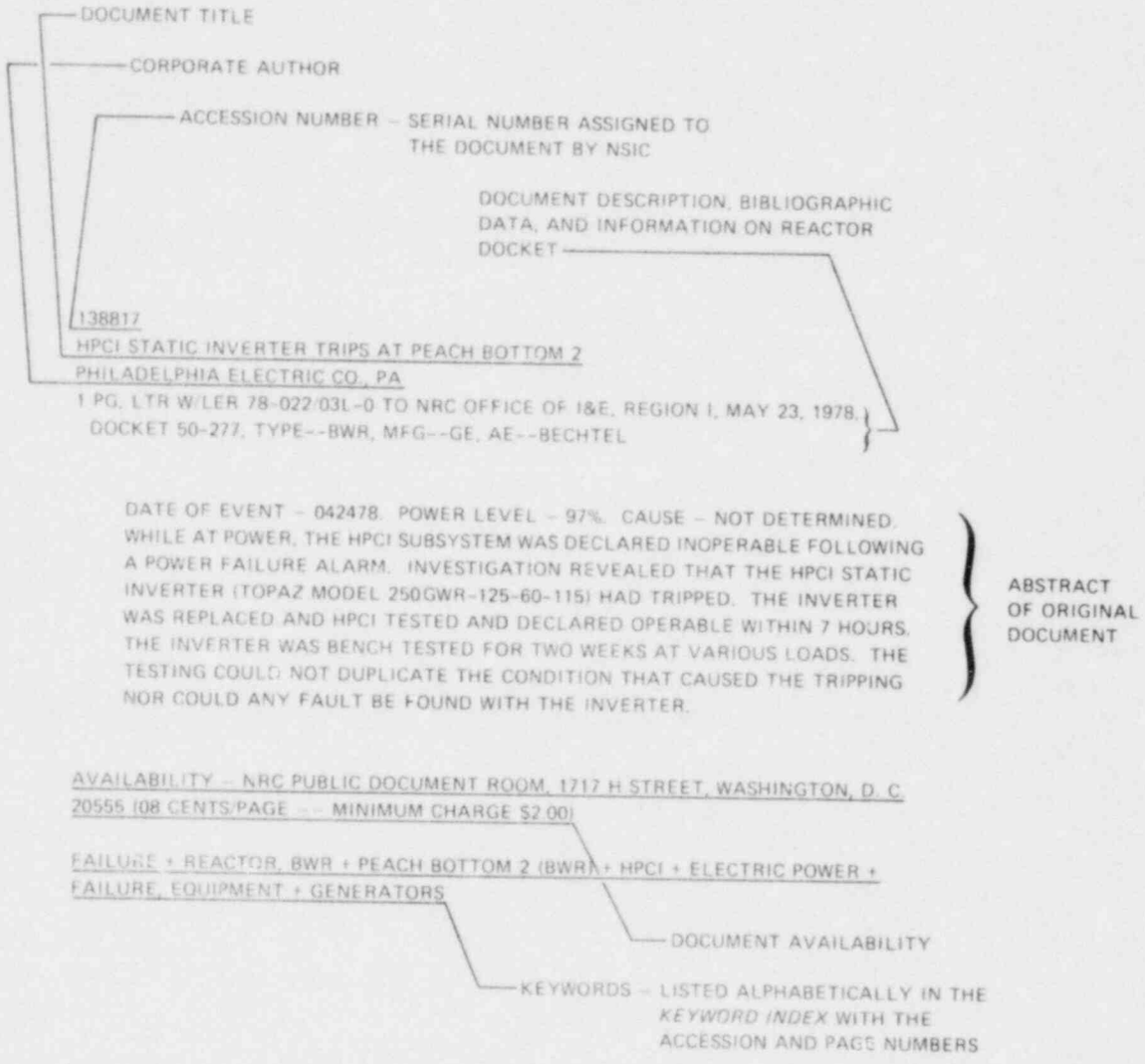

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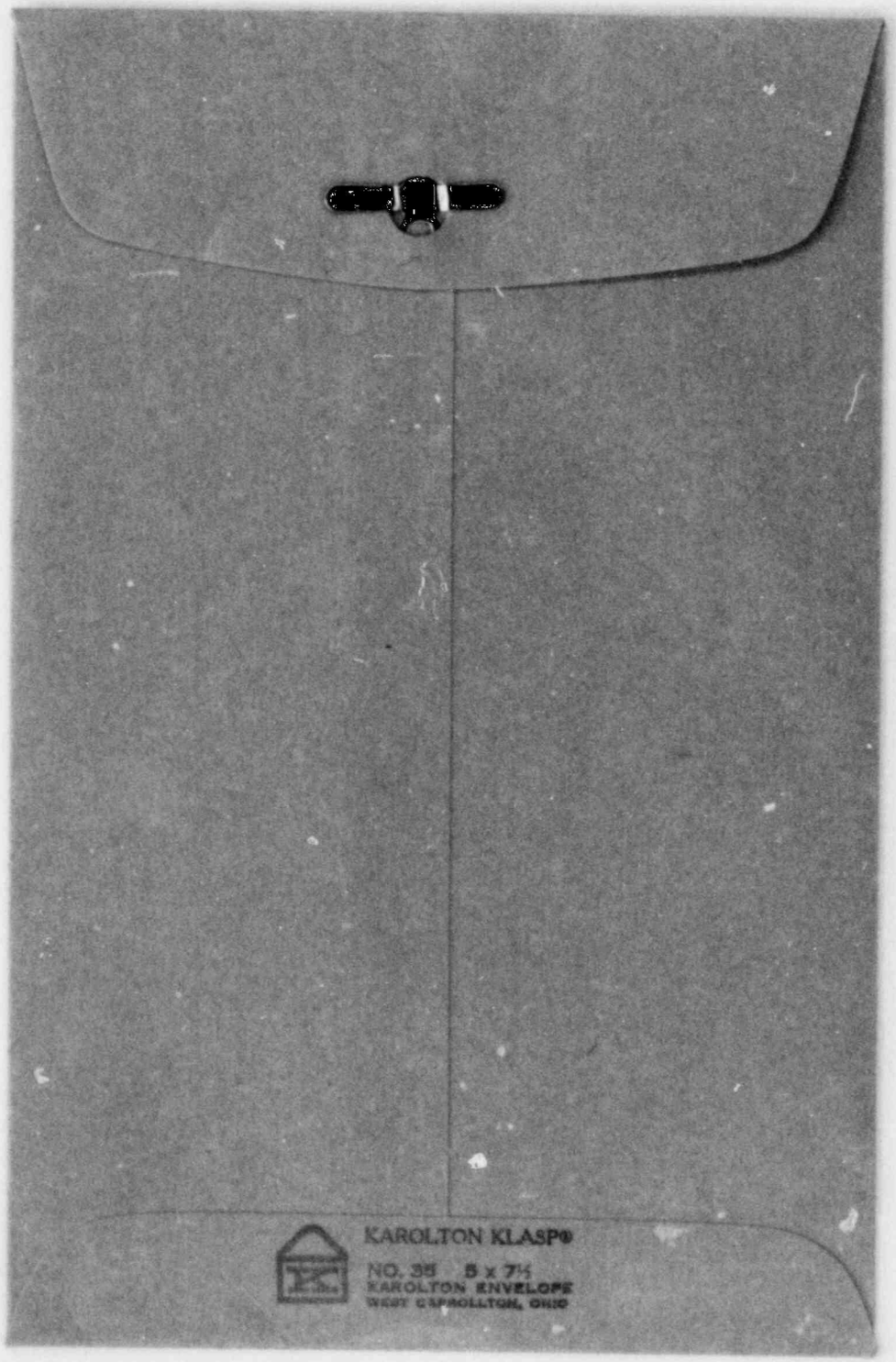

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