



**Wisconsin Electric** POWER COMPANY  
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201



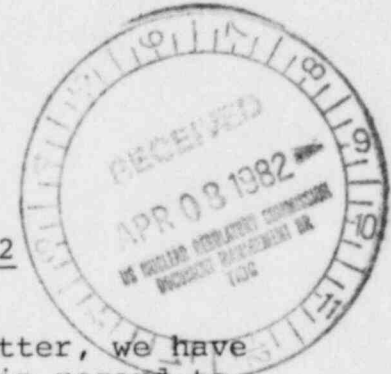
April 5, 1982

Mr. H. R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. NUCLEAR REGULATORY COMMISSION  
Washington, D. C. 20555

Attention: Mr. R. A. Clark, Chief  
Operating Reactors, Branch 3

Gentlemen:

DOCKET NOS. 50-266 AND 50-301  
INSERVICE INSPECTION RELIEF REQUESTS  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2



As requested in your February 26, 1982 letter, we have conducted a review of the submittals and responses in regard to inservice inspection relief requests. We believe the following two additional items should be included in your review program:

1. October 25, 1979 Schwencer (NRC) to Burstein (WE) letter with enclosure, "Granting of Relief from ASME Section XI Inservice Inspection (Testing) Requirements", Unit 2. (Attached.)
2. February 23, 1982 Fay (WE) to Denton (NRC) letter with enclosure, "Completion of First Ten-Year Inservice Inspection Interval", Unit 1. (This item has been received by your staff.)

In addition, we also believe that the item dated December 6, 1979, Burstein (WE) to Schwencer (NRC), found in your review document record should be dated February 6, 1979.

In your letter you indicated that the review of the third, Unit 2, 40-month inspection interval will be conducted against the 1977 Edition of Section XI of the ASME Code, up to and including the Summer 1978 Addenda. According to NRC guidelines, an inservice inspection plan is to comply with the requirements of the edition/addenda of ASME Section XI referenced in 10 CFR 50.55(a) no more than six months before the start of the period for an updated program. The inspection interval for Unit 2 commenced on June 1, 1979. The Unit 2 inservice inspection plan was developed

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Mr. H. R. Denton

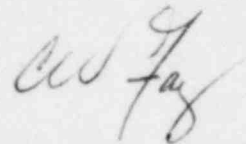
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April 5, 1982

according to the 1979 revision of 10 CFR 50.55(a), paragraph b(2), in which the 1974 Edition and addenda only through the Summer 1975 Addenda were in effect for Section XI of the ASME Code. Therefore, the review of our June 1979 through September 1982 Unit 2 inservice inspection program should be conducted against the 1974 Edition of Section XI of the ASME Code, up to and including the Summer 1975 Addenda.

In keeping with your request to expedite the review, Mr. Tim Colburn of your staff has been advised verbally of the contents of this letter.

Very truly yours,



Assistant Vice President

C. W. Fay

Attachment

Copy to NRC Resident Inspector



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 25, 1979

Docket No. 50-301

Mr. Sol Burstein  
Executive Vice President  
Wisconsin Electric Power Company  
231 West Michigan Street  
Milwaukee, Wisconsin 53201

Dear Mr. Burstein:

RE: POINT BEACH NUCLEAR PLANT UNIT NO. 2

By letter dated February 26, 1979, you submitted a proposed inservice inspection and testing program description and a request for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Although we have not completed our detailed review of your submittal, our preliminary review makes clear to us that your proposed program to implement those ASME Code requirements that you have found to be practical would increase the scope of inservice inspection and testing for your facility beyond that currently required by your Technical Specifications. We have concluded that this upgrading of your inservice inspection and testing program will further enhance safety.

Based on our preliminary review, we agree with your determination that it is impractical within the limitations of design, geometry and materials of construction of components, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level of quality or safety. Therefore, pursuant to 10 CFR 50.55a(g) (6)(f), we hereby grant relief, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that you have requested. Moreover, since the scope of the inservice inspection and testing will be increased by your proposed program, and the granting of this relief is based only on the impracticality of selected ASME Code requirements, we have determined that the relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins and that, therefore, it does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to implement your proposed program (except where your current Technical Specifications are more restrictive).

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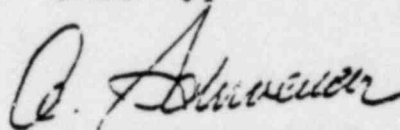
Mr. Sol Burstein  
Wisconsin Electric Power Company - 2 - October 25, 1979

During the period between now and the date we complete our detailed review of your submittal, you must comply with both your existing Technical Specifications and your proposed inservice inspection and testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter inspection intervals, increased number of parameters measured). In other words, the granting of this relief from ASME Code requirements should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications.

When our detailed review of your February 26, 1979 submittal is complete we will: (1) issue final approval of your program (which may contain modifications resulting from the staff's review), (2) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval and (3) issue appropriate changes to your Technical Specifications.

A copy of the Federal Register Notice related to this action is enclosed.

Sincerely,



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosure:  
Federal Register Notice

cc: w/enclosure  
See next page

Mr. Sol Burstein  
Wisconsin Electric Power Company

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October 25, 1979

cc: Mr. Bruce Churchill, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N.W.  
Washington, D. C. 20036

Document Department  
University of Wisconsin  
Stevens Point Library  
Stevens Point, Wisconsin 54481

Mr. Glenn A. Reed, Manager  
Nuclear Operations  
Wisconsin Electric Power Company  
Point Beach Nuclear Plant  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-301WISCONSIN ELECTRIC POWER COMPANYNOTICE OF GRANTING OF RELIEF FROM ASME SECTION XI  
INSERVICE INSPECTION (TESTING) REQUIREMENTS

The U. S. Nuclear Regulatory Commission (the Commission) has granted relief from certain requirements of the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components" to Wisconsin Electric Power Company. The relief relates to the inservice inspection (testing) program for the Point Beach Unit No. 2 (the facility) located in Two Creeks, Wisconsin. The ASME Code requirements are incorporated by reference into the Commission's rules and regulations in 10 CFR Part 50. The relief is effective as of its date of issuance.

The relief consists of exemption from the requirements for measuring certain parameters in the Pump and Valve Testing Program and from performing certain pressure vessel weld inspections in the Inservice Inspection Testing Program.

The request for relief complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the letter granting relief. Prior public notice of this action was not required since the granting of this relief from ASME Code requirements does not involve a significant hazards consideration.

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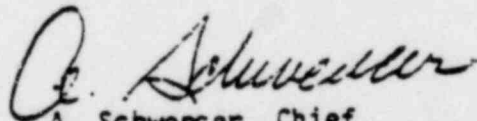
The Commission has determined that the granting of this relief will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

For further details with respect to this action, see (1) the request for relief dated February 26, 1979, (2) the Commission's letter to the licensee dated October 25, 1979.

The items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the University of Wisconsin, Stevens Point Library, Stevens Point, Wisconsin 54481. A copy of item (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 25th day of October, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors