

CO:I:RTC

June 26, 1968

J. P. O'Reilly, Chief, Reactor Inspection and Enforcement Branch, Division of Compliance, Headquarters

U. S. ARMY MATERIALS AND MECHANICS RESEARCH CENTER  
DOCKET NO. 50-47

The attached report of a special visit to the subject facility by our assigned inspector on April 24 and 25, 1968, is forwarded for action.

The principal purpose of the visit was to provide followup to a situation relating to the unauthorized modification to the facility. Specifically, during the previous visit, it was noted that certain modifications had been made without prior approval of DRL. The licensee was advised at that time that such approval would be necessary prior to the resumption of operation. Proper approval was subsequently requested and granted; the latter, in the form of Change No. 1, which authorizes operation at 2 MWt with certain specified facility modifications and with the additional requirement that one of the primary loops be isolated. During this visit, the inspector confirmed that the subject modifications had been completed in accordance with the authorization. Also, that the required isolation of a primary loop was in effect. It is noted that operation of the facility, following the outage for the subject modifications, was resumed on April 26, 1968.

In addition, during this visit, the inspector discussed the requirements of 10 CFR 50.36(c) and 50.59 with the licensee. Such action was suggested by DRL in the transmittal memorandum for Change No. 1. The results of this discussion are considered to be satisfactory.

The results of containment leak rate tests were reviewed by our inspector and observed to be within the requirements of the license. During his review, the inspector noted that the containment overpressurization protection device, a

water leg, had been inadvertently blown during one phase of the testing. This device is described in the application

D	OFFICE ▶			
J	SURNAME ▶	Carlson:hm		
	DATE ▶	6-19-68		18

for 5 Mwt operation\*, currently under review by DRL. Additional information relating to the device is contained in this report. It is recommended that DRL be requested to include in their review of the application for 5 Mwt operation the adequacy of the design of this device, including specifically the controls for maintaining proper water level and the lack of a feature for reset after relief.

It is noted that the facilities for the removal of a beam tube with the pool full of water were reviewed by the inspector. It was concluded that these facilities will permit adequate control of the beam tube during removal or installation. We do continue to have some reservation, however, about the desirability of performing this operation with the reactor core in place.

It is also noted that this visit includes a review of the adequacy of operator coverage and of the facility emergency plans as requested by CO:HQ directives. No deficiencies of significance were detected in either subject area.

R. T. Carlson  
Senior Reactor Inspector

Enclosure: (20 cys)  
CO Report No. 47/68-2  
by G. L. Madsen  
dated 6/17/68

\*Safety Analysis Report for the Army Materials Research Reactor, dated February 10, 1967 - Page 66 and Table II, page 68.