ANG TORIGADO (7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 2 2 0 0 - 0 0 0 - 0 0 0 3 4 1 1 1 1 16 UCENSE NUMBER 25 26 LICENSE TYPE NS LICENSE NUMBER LICENSEE CODE CON'T 6 0 5 0 0 0 3 3 6 0 0 3 1 0 8 2 8 0 4 0 8 8 2 9 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 REPORT 0 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) While performing Type B and C Local Leak Rate Test during refueling a combined leak- 1 0 2 age rate in excess of that allowed by Technical Specification 3.6.1.2.b was noted. 0 3 Valves 2-AC-6, 2-RB-28.2A, 3A, 2C, 3C and the Personnel Access Hatch were found to 0 4 have various mechanical deficiencies resulting in excessive leakage rates. The 051 plant was operated in accordance with Technical Specification Action Statement 0 6 3.6.1.2. Similar Events: LER 80-32. 0 7 0 8 8 9 VALVE COMP CAUSE CAUSE SYSTEM SURCODE COMPONENT CODE (16) IT R CA X (15 Z 1E 1(12 X (13) P EINIE XIX 0 9 19 13 REVISION OCCURRENCE REPORT SEQUENTIAL NO. CODE TYPE EVENTYEAR 0 3 L 0 0 6 0 REPORT 8 32 NUMBER 28 COMPONENT ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP ACTION FUTURE EFFECT ON PLANT SHUTDOWN METHOD MANUFACTURER (22) SUPPLIER HOURS [0]0]0] |Y | (23) X X 9 9 9 (26 Y (24) (25) Z](21) 01 Z (20) 18) Z 44 40 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Mechanical deficiencies of components were as follows: 2-AC-6, packing leak; 2-RB-1 0 28.2A, 3A, 2C, 3C, seat ring damage and misadjustment, Personnel Access Hatch, 1111 door operator shaft leaks. All deficiencies were corrected and the Local Leak Rate Test successfully completed. 1 3 1 4 80 9 METHOD OF (30)DISCOVERY DESCRIPTION (32) FACILITY OTHER STATUS N. POWER Surveillance Test B (31 NA 0 0 0 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35 RELEASED OF RELEASE NA NA (34) (33) 80 45 44 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE NA 0 0 37 Z 38 80 PERSONNEL INJURIES DESCRIPTION (41) UMBER NA 0 0 0 0 80 11 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION NA Z (42) 9 80 NRC USE ONLY 8204160589 820408 PDR ADOCK 05000336 80. 5 68 69 PDR 0 PHONE (203) 447-1791 0.0 J. L. Criscione NAME OF PREPARER _

ATTACHMENT TO LER 82-06/3L-0 NORTHEAST NUCLEAR ENERGY COMPANY MILLSTONE NUCLEAR POWER STATION - UNIT 2 PROVISIONAL LICENSE NUMBER DPR-65 DOCKET NUMBER 50-336

Event Description and Probable Consequences

While shut down for refueling and performing a Type B and C Local Leak Rate Test in accordance with Surveillance Procedure 2605C and 2605D, respectively on containment penetrations, a combined leakage rate in excess of that allowed by Technical Specifications 3.6.1.2.b was noted. Investigation revealed that valve 2-AC-6 had a stem packing leak, valves 2-RB-28.2A, 3A, 2C, and 3C were leaking by the T-ring seats. The personnel access hatch had packing gland leakage on the door operator shafts. The plant was operated in accordance with Technical Specification Action Statement 3.6.1.2. Similar Events: LER 80-32.

Cause Description and Corrective Actions

The packing was readjusted on valve 2-AC-6 to correct the stem leakage. Valves 2-RB-28.2A, 3A, 2C and 3C exhibited a combination of T-ring seat age hardening and overadjustment of the T-rings during initial installation preventing the T-ring from fully seating with the disc. New T-rings were installed and the seats/discs properly adjusted. The operator door shafts gland packing on the personnel access hatch was readjusted. The Local Leak Rate Test was successfully completed after the above repairs/adjustments were accomplished.

VALVE DATA

	6 inch butterfly, Fisher Type 9222 with a Bettis CB-525-SR-80 Robotarm Actuator	
	10 inch butterfly, Fisher Type 9222 with a Bettis CB-735-SR-80 Robotarm Actuator	