NRC FORM 366

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Field	Value
Licensee Code (2)	A L B R F 1
License Number (3)	0 0 - 0 0 0 0 0 - 0 0
License Type (4)	4 1 1 1 1
CAT (5)	
Report Source (6)	L
Docket Number (7)	0 5 0 0 0 2 5 9
Event Date (8)	0 3 1 0 8 2
Report Date (9)	0 4 0 8 8 2

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

During a normal reactor startup, the calculated R factor (FRP/CMFLPD) was less than the R factor used to set the APRM scram and rod block for 34 minutes in excess of the six hours allowed by tech specs (T.S. 3.5.L.2 & 3). There was no danger to the health or safety of the public in that the APRM scram value (calculated) was not exceeded. There is no redundant system.

Field	Value
System Code (11)	I A
Cause Code (12)	X
Cause Subcode (13)	Z
Component Code (14)	Z Z Z Z Z Z
Comp Subcode (15)	Z
Valve Subcode (16)	Z
LER/RO Report Number (17) – Event Year	8 2
Sequential Report No.	0 1 9
Occurrence Code	0 3
Report Type	L
Revision No.	0
Action Taken (18)	X
Future Action (19)	G
Effect on Plant (20)	Z
Shutdown Method (21)	Z
Hours (22)	0 0 0 0
Attachment Submitted (23)	Y
NPRD-4 Form Sub. (24)	N
Prime Comp. Supplier (25)	Z
Component Manufacturer (26)	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The RWM sequence in use was not optimum for current core loading and allowed excessive power peaking during startup. At the end of the initial 6-hour period, control rod insertion was initiated. The initial insertion brought "R" within required limits. The RWM sequence has been revised to reduce power peaking during startup.

Field	Value
Facility Status (28)	C
% Power (29)	0 7 2
Other Status (30)	NA
Method of Discovery (31)	A
Discovery Description (32)	Monitoring of reactor parameters
Activity Released (33)	Z
Content of Release (34)	Z
Amount of Activity (35)	NA
Location of Release (36)	NA
Personnel Exposures – Number (37)	0 0 0
Personnel Exposures – Type (38)	Z
Personnel Exposures – Description (39)	NA
Personnel Injuries – Number (40)	0 0 0
Personnel Injuries – Description (41)	NA
Loss of or Damage to Facility – Type (42)	Z
Loss of or Damage to Facility – Description (43)	NA
Publicity – Issued (44)	N
Publicity – Description (45)	

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PDR ADOCK 05000259
S PDR

NRC USE ONLY

NAME OF PREPARER L. L. Krause PHONE (205) 729-6846

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 82019 Technical Specification Involved 3.5.L.2 & 3

Reported Under Technical Specification 6.7.2.b.(2) * Date Due NRC 4/9/82

Event Narrative:

Unit 2 was operating at 99% power; unit 3 was in a refueling outage. These units were unaffected by this event. With unit 1 having reached 70% power during a normal reactor startup, the shift nuclear engineer noted, through normal core performance monitoring, that the calculated "R" (FRP/CMFLPD) of 0.837 was less that the R which had been used prior to startup to set the APRM rod blocks and scram setpoints (0.985) in anticipation of normal initial power peaking. Although core flux shaping procedures improved the calculated R to 0.927 within the allotted T.S. period of 6 hours.

At the end of the 6-hour period, control rod insertion was initiated. The initial insertion of shallow rods brought "R" within required limits with very little change in core thermal power. Reduction to less than 25% of rated thermal power was therefore not required. The RWM sequence has been revised to reduce power peaking during future startup.

* Previous Similar Events:

BFRO 50-260/8137; 296/8145, 8141, 8140

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP