U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)

LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

/0/1/ REPORT SOURCE /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/4/2/9/8/2/ (8) /0/5/2/4/8/2/ (9)

DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / With the Unit in Mode 1, the middle level corner support temperature (TI-AM-111A-/

/0/3/ / 2) for the "A" Steam Generator indicated less than 225°F three times on April 29,/

/0/4/ / 1982, two times on April 30, 1982 and one time on May 1, 1982. The support tem- /

/0/5/ / perature was restored above 225°F within 4 hours on each event except for May 1, /

/0/6/ / 1982. At this time, the 12 hour period to reduce plant pressure below 1000 /

/0/7/ / psig was entered. Thus the action of T.S. 3.4.10.2 was adhered to and the /

/0/8/ / health and safety of the general public were not affected. /

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/	/Z/Z/ (11)	/X/ (12)	/Z/ (13)	/X/X/X/X/X/X (14)	/Z/ (15)	/Z/ (16)

	LER/RO REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE	REVISION NO.
(17)		/8/2/	/-/ /0/2/8/	/ /	/0/3/	/L/	/-/ /0/

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
/X/ (18)	/F/ (19)	/Z/ (20)	/Z/ (21)	/0/0/0/0/ (22)	/Y/ (23)	/N/ (24)	/A/ (25)	/Z/9/9/9/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Condensation from the CRDM shroud cooler in cubical "A" caused by high secondary /

/1/1/ / steam leakage in the containment wetted the thermal blanket covering the "A" /

/1/2/ / Steam Generator support thereby lowering the support temperature. Support temper-/

/1/3/ / ture was increased by securing the cubical "A" dome fan, reducing cooling water /

/1/4/ / flow to the CRDM shroud coolers and verifying all support heaters energized. /

	FACILITY STATUS	%POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/	/E/ (28)	/1/0/0/ (29)	/ NA /	/A/ (31)	/Operator Observation/

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/	/Z/ (33)	/Z/ (34)	/ NA /	/ NA /

	PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/	/0/0/0/ (37)	/Z/ (38)	/ NA /

	PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/	/0/0/0/ (40)	/ NA /

	LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/	/Z/ (42)	/ NA /

	PUBLICITY ISSUED	DESCRIPTION (45)	
/2/0/	/N/ (44)	/ NA /	RC USE ONLY / / / / / / / / / / / /

8206070427 820524
PDR ADOCK 05000338
S PDR

NAME OF PREPARER W. R. CARTWRIGHT PHONE (703) 894-5151

Description of Event

With the unit in Mode 1, the middle level corner support temperature (TI-AM-111A-2) for the "A" Steam Generator indicated less than 225°F on the following occasions:

April 29, 1982 -	0830 to 0948 -	1 Hr.	18 Min.
	1028 to 1102 -		34 Min.
	1139 to 1301 -	1 Hr.	22 Min.
April 30, 1982 -	0815 to 0918 -	1 Hr.	3 Min.
	1330 to 1415 -		45 Min.
May 1, 1982 -	0516 to 1040 -	5 Hrs.	24 Min.

These events are reportable pursuant to T.S. 6.9.1.9.b.

Probable Consequences of Occurrence

For the A572 material, operation above 225°F provides a conservative temperature limit and thus toughness level in the steel. This assures the safety of the A572 material even under the worst postulated accident conditions. On May 1, 1982, at 0916 the 12 hour period was entered to reduce plant pressure below 1000 psig as required by the Action Statement. The temperature was restored above the 225°F limit 1 hour and 24 minutes later. On all other events where the support temperature dropped below 225°F, temperature was restored within 4 hours as required by T.S. 3.4.10.2. Thus the health and safety of the general public were not affected.

Cause of Event

Condensation from the CRDM shroud cooler in cubical "A" wetted the thermal blanket covering the "A" Steam Generator support thereby lowering the support temperature. A large amount of condensation was formed in the shroud coolers due to secondary steam leakage in the containment.

Immediate Corrective Action

To reduce condensation which wetted the "A" support thermal blanket the dome fan in cubical "A was secured, the cooling water flow to the CRDM shroud cooler was reduced and all support heaters were energized. This allowed the temperature of the support to increase above 225°F as required by T.S. 3.4.10.2.

To prevent containment temperature from increasing above T.S. limits the dome fan was re-energized and the cooling water flow to the shroud cooler was increased back to normal conditions which caused rewetting of the thermal blanket.

Scheduled Corrective Action

On May 1, 1982, a drainage trough complete with a hose to divert the condensation to the containment sump was installed below the cubical "A" shroud cooler. This modification prevented the thermal blanket from becoming wet and maintained the support above the required 225°F.

Actions Taken to Prevent Recurrence

On May 19, 1982, the unit was taken out of service to begin the 1982 refueling overhaul. The secondary leaks which were causing high moisture content in the containment atmosphere will be repaired at this time.

Generic Implications

Shroud cooler design for Unit 2 is identical to that of Unit 1. To prevent water from the shroud coolers wetting the thermal blankets in Unit 2, drip pans are being installed to divert the water to the containment sump.