Harry Tauber
Vice President
Engineering and Construction

Detroit Edison

2000 Second Avenue
Detroit, Michigan 48226
(313) 237-8000

May 26, 1982
EF2 - 57,563

Mr. B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Youngblood:

Reference:
1. Enrico Fermi Atomic Power Plant, Unit 2 NRC Docket No. 50-341
2. NRC Letter, "Completion of Fermi 2 Containment Analysis and Modifications", B. J. Youngblood to H. Tauber, March 23, 1982
3. Edison Letter, EF2-53,824, "Fatigue Evaluation of Safety Relief Valve (SRV) Discharge Piping in the Wetwell Air Space", W. F. Colbert to L. L. Kintner, June 22, 1981

Subject: Comments on Reference 2

We have reviewed your letter dated March 23, 1982 (Reference 2) and have the following comments:

- The fatigue analysis for the safety relief valve discharge piping in the wetwell airspace was evaluated using extended ASME Class 2 requirements. The method used to evaluate fatigue was presented to the NRR Technical Staff on June 15, 1981. A detailed report was provided to Mr. Kintner in our letter dated June 22, 1981 (Reference 3). The method presented in Reference 3 provides a sound engineering approach for evaluating fatigue, which, as demonstrated, gave results comparable to the results obtained using Class 1 rules for a typical line.

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- There is no requirement in NUREG-0661 to determine the local torus temperature by SRV testing. However, during plant startup tests, the loads caused by operation of the safety relief valves will be confirmed by testing as defined in NUREG-0661, Section 2.13.9.

If you have any questions regarding the above, please contact Mr. L. E. Schuerman, (313) 649-7562.

Sincerely,

C. M. Heidel for
H. Tauber

cc: L. L. Kintner
B. Little