

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
 /0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/5/0/1/8/2/ (8) /0/5/1/9/8/2/ (9)
 SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On May 1, 1982, following a reactor trip from 100% power, the containment average/
 /0/3/ / temperature exceeded the T.S. 3.6.1.5 limit of 105°F. On May 8, 1982 while ramp-/
 /0/4/ / ing down to repair steam leaks within the containment, the temperature again ex-/
 /0/5/ / ceeded 105°F. Since the temperature was restored within the requirements of T.S./
 /0/6/ / 3.6.1.5, the health and safety of the public were not affected. These events are/
 /0/7/ / reportable pursuant to T.S. 6.9.1.9.b. /
 /0/8/ /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE			
/0/9/ /S/B/ (11)	/X/ (12)	/Z/ (13)	/X/X/X/X/X/X/ (14)	/Z/ (15)	/Z/ (16)			
LER/RO	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.			
(17) REPORT NUMBER	/8/2/	/-/ /0/3/1/ / /	/0/3/	/L/	/-/ /0/			
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
/X/ (18)	/Z/ (19)	/Z/ (20)	/Z/ (21)	/0/0/0/0/ (22)	/Y/ (23)	/N/ (24)	/Z/ (25)	/Z/9/9/9/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The first event was due to low auxiliary steam pressure to the chilled water /
 /1/1/ / system equipment following a reactor trip. The second event was due to steam /
 /1/2/ / leakage inside of the containment. The unit was being shutdown to investigate /
 /1/3/ / and repair these leaks. Auxiliary steam pressure was restored and leakage /
 /1/4/ / repaired. /

FACILITY STATUS	ZPOWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /G/ (28)	/0/0/0/ (29)	/ NA / (30)	/A/ (31)	/ Operational Event /
ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)	
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /	
PERSONNEL EXPOSURES	NUMBER	TYPE	DESCRIPTION (39)	
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA		
PERSONNEL INJURIES	NUMBER	DESCRIPTION (41)		
/1/8/ /0/0/0/ (40)	/ NA			
LOSS OF OR DAMAGE TO FACILITY	TYPE	DESCRIPTION (43)		
/1/9/ /Z/ (42)	/ NA			
PUBLICITY	ISSUED	DESCRIPTION (45)		
/2/0/ /N/ (44)	/ NA			
NAME OF PREPARER			W. R. CARTWRIGHT	PHONE (703) 894-5151

NRC USE ONLY

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Virginia Electric and Power Company
North Anna Power Station, Unit No. 1
Docket No. 50-338
Report No. LER 82-031/03L-0

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Description of Event

On May 1, 1982, following a reactor trip, the containment average temperature exceeded 105°F due to the lowering of steam pressure to the chilled water system equipment. On May 8, 1982, the containment temperature again exceeded the T.S. 3.6.1.5 limit during a ramp-down to repair steam leaks within the containment. These events are reportable pursuant to T.S. 6.9.1.9.b.

Probable Consequences of Occurrence

The maximum and minimum containment temperatures are controlled to ensure that the design basis parameters assumed in the FSAR remain valid. Since the containment temperature was restored to within the limits as required by the Action Statement, the health and safety of the public were not affected.

Cause of Event

Auxiliary steam to the chilled water system is used to reduce the chilled water temperature which is used to remove containment heat. Due to the reactor trip on May 1, 1982, the auxiliary steam pressure decreased causing the chilled water system to function improperly.

On May 8, 1982 a ramp-down was initiated to cold-shutdown to repair steam leaks within the containment. As turbine load was decreased, steam pressure increased which aggravated the steam leakage. This increased the heat load in the containment and caused the average temperature to exceed 105°F.

Immediate Corrective Action

Auxiliary steam pressure was restored and the chilled water temperature decreased which reduced the average temperature. On May 8, 1982, the unit was placed in cold shutdown (Mode 5) and the steam leaks repaired.

Scheduled Corrective Action

No further action required.

Actions Taken to Prevent Recurrence

No further action required.

Generic Implications

These events are considered to be operational problems.