

0 1 | F | L | C | R | P | 3 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

CONT

0 1 | R | P | O | R | T | S | O | U | R | C | E | L | 6 | 0 | 5 | 0 | - | 0 | 3 | 0 | 2 | 7 | 0 | 5 | 0 | 3 | 8 | 2 | 0 | 5 | 2 | 1 | 8 | 2 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 | At 1100, during normal operation, Reactor Building pressure indicator(BS-16-PI) was discovered
0 3 | to be inoperable contrary to T.S. 3.3.3.6. Redundancy was provided by an alternate Reactor
0 4 | Building pressure channel. Maintenance was initiated and operability was restored on 5/3/82
0 5 | There was no effect upon the health or safety of the general public. This is the third oc-
0 6 | currence for Reactor Building pressure channel (BS-16-PI) and this is the eighth report
0 7 | under this Specification.

0 9 | SYSTEM CODE | CAUSE CODE | CAUSE SUBCODE | COMPONENT CODE | COMP. SUBCODE | VALVE SUBCODE
| I | D | 11 | X | 12 | Z | 13 | I | N | S | T | R | U | 14 | T | 15 | Z | 16
| 9 | 10 | 11 | 12 | 13 | 18 | 19 | 20
17 | REP. NO. | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | REVISION NO.
| 8 | 2 | 0 | 3 | 3 | / | 0 | 3 | L | 0
| 21 | 22 | 23 | 24 | 26 | 27 | 28 | 29 | 30 | 31 | 32
ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS | ATTACHMENT SUBMITTED | NFRD-4 FORMS SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER
| E | X | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | Y | 23 | N | 24 | A | 25 | L | 1 | 3 | 0 | 26
| 33 | 34 | 35 | 36 | 37 | 40 | 41 | 42 | 43 | 44 | 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

0 0 | The cause of this event is attributed to instrument drift. The pressure transmitter was
0 1 | calibrated per Surveillance Procedure (SP-162). An engineering evaluation will be con-
0 2 | ducted to determine if calibration frequency should be increased. No further corrective
0 3 | action is deemed necessary.

5 | FACILITY STATUS | % POWER | OTHER STATUS | METHOD OF DISCOVERY | DISCOVERY DESCRIPTION
| E | 28 | 0 | 9 | 9 | 29 | NA | A | 31 | Operator observation
6 | ACTIVITY CONTENT | AMOUNT OF ACTIVITY | LOCATION OF RELEASE
| Z | 33 | Z | 34 | NA | NA
7 | PERSONNEL EXPOSURES | DESCRIPTION
| 0 | 0 | 0 | 37 | Z | 38 | NA
8 | PERSONNEL INJURIES | DESCRIPTION
| 0 | 0 | 0 | 40 | NA
9 | LOSS OF OR DAMAGE TO FACILITY | DESCRIPTION
| Z | 42 | NA
0 | PUBLICITY | DESCRIPTION
| N | 44 | NA

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NRC USE ONLY

SUPPLEMENTARY INFORMATION

Report No.: 50-302/82-033/03L-0

Facility: Crystal River Unit 3

Report Date: May 21, 1982

Occurrence Date: May 3, 1982

Identification of Occurrence:

A Reactor Building pressure indicator inoperable contrary to Technical Specification 3.3.3.6.

Conditions Prior to Occurrence:

Mode 1 power operation (90%)

Description of Occurrence:

At 1100, during normal operation, Reactor Building pressure indicator (BS-16-PI) was not indicating correctly. Maintenance was initiated and operability was restored on May 3, 1982.

Designation of Apparent Cause:

The cause of this event is attributed to instrument drift.

Analysis of Occurrence:

Redundancy was provided by an alternate Reactor Building pressure channel. There was no effect upon the health or safety of the general public.

Corrective Action:

The pressure transmitter was recalibrated per Surveillance Procedure SP-162. An engineering evaluation will be conducted to determine if calibration frequency should be increased. No further corrective action is deemed necessary.

Failure Data:

This is the third occurrence for Reactor Building pressure channel (BS-16-PI), and this is the eighth report under this Specification.

/rc