U.S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7477) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: $\Box(0)$ 0 3 4 1 1 1 1 4 40 0 0 0 0 - 0 ALELI 1 (0 0 LICENSE NUMBER CONT 0 1 31 2 L (6) 0 5 | 0 | 01 01 3 | 2 -SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) while performing HNP-1-3653, Standby Gas Treatment (SBGT) System Oper-0 2 ability, on 3/22/82 and 4/5/82, and when investigating suspected air 0 3 leakage on 4/10/82, SBGT failed to maintain secondary containment integ-0 4 rity as required by T.S.3.7.C.1.a and T.S.4.7.C.1.b. Neither plant 0 5 operation nor the health and safety of the public was affected by this 0 6 non-repetitive event. 80 SYSTEM COMP CAUSE CAUSE VALVE SUBCODE Z (15 C (11 X 1(14) SI 1E 1(12 X (13) X X XI XL Z (16) IX. OCCURRENCE REVISION SEQUENTIAL REPORT EVENT YEAR REPORT NO CODE TYPE NO 8 2 0 3 0 2 5 1 0 NUMBER NPRD-4 FORM SUB COMPONENT PRIME COMP. EFFECT ON PLANT ATTACHA SUBMITTED ACTION FUTURE TAKEN ACTION SUPPLIER MANUFACTURER N (24) 0 0 0 0 Y Z 19 9 9 (26) B Z (20) Ζ 0 L (19 1(21 Z CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The failure to maintain secondary containment integrity was due to several leaking door seals, 8 open drain valves, and inaccurate c atrol room instruments. Maintenance requests (MR's) were written to have the seals Drain valves were closed. MR's were written to recalibrate repaired. the control room indicators. 4 80 METHOD OF DISCOVERY FACILITY OTHER STATUS. SCOVERY DESCRIPTION (32 0 9191 NA Testing SBGT Filter Train E (28) 80 CONTENT ACTIVITY AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) OF RELEASE HELEASED NA 34 80 PERSONNEL DESCRIPTIO NA 0 0 Z 80 PERSONNEL DESCRIPTION (41) 0 (40) NA 0 80 DSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION Z (42 NA PUBLICITY NRC USE ONLY DESCRIPTION (45) 1(44) NA 82060105 69 68 80 SBT 912-367-7781 NAME OF PREPARER . PHONE: _

LER #: 50-321/1982-25 Licensee: Georgia Power Company Facility Name: Edwin I. Hatch Docket #: 50-321

Narrative Report for LER 50-321/1982-25

On 3/22/82 and on 4/5/82 with the unit in run mode and while performing HNP-1-3653, Standby Gas Treatment (SBGT) System Operability, and on 4/10/82 while investigating suspected air leakage to Unit I secondary containment, SBGT failed to maintain secondary containment integrity as required by T.S.3.7.C.1.a and T.S.4.7.C.1.b (reference deviation reports 1-82-49, 1-82-46, and 1-82-50). Both T.S. sections require that secondary containment must be capable of maintaining a minimum of 1/4" of water vacuum with each filter train flow rate not more than 4000 CFM. This condition was violated since 1/4" water vacuum could not be maintained according to control room instrumentation. Neither plant operation nor the health and safety of the public was affected. This is a non-repetitive event.

The violations of secondary containment integrity on 3/22/82 and 4/5/82 (deviation reports 1-82-49 and 1-82-46) were due to several leaking door seals. Maintenance requests (MR's) 1-82-1810 through 1-82-1813 were written to have the seals repaired. The seals were repaired on 4/5/82. On 4/10/82, a violation of secondary containment integrity occurred when 8 drain valves (Master Parts List numbers T45-F013,-F014,-F015,-F016,-F017,-F018,-F019, and a new fuel storage vault drain valve) connecting the Unit I and Unit II refueling floor to the Unit II reactor building were left open (reference deviation report number 1-82-50). The valves were closed. HNP-2-1442, Reactor Building Leak Detection System, will be revised by 4/26/82 to include a valve lineup for the 8 drain valves referenced in deviation report 1-82-50. This valve lineup requires that these valves remain closed.

Plant personnel fee1 that the violations of secondary containment integrity for the aforementioned deviation reports were due to the inaccuracy of control room instrumentation. MR's 1-82-1818 through 1-82-1822 were written to recalibrate the differential pressure instrumentation. Following recalibration of the instrumentation, HNP-1-3653 was successfully performed. A revision to HNP-1-3653 was submitted and approved by the Plant Review Board on 4/15/82 to include the use of an incline manometer to verify the pressure difference in the reactor building. The manometer will be zeroed (with interior ventilation off) to outside air conditions at the time of the test, since control room instrumentation cannot compensate for outside conditions.