

# GPU Nuclear

**GPU Nuclear**  
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Writer's Direct Dial Number:  
May 18, 1982

Mr. Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Crutchfield:

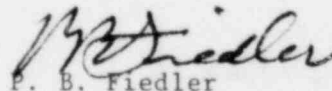
Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219, SEP Topic III-5B  
Pipe Break Outside Containment

By this letter, we transmit the enclosed report entitled "FRACTURE MECHANICS ANALYSIS OF THE OYSTER CREEK NUCLEAR GENERATING STATION EMERGENCY COOLING SYSTEM" dated April 20, 1982.

An evaluation by Jersey Central Power & Light Company (JCP&L) and the NRC site inspection in 1980 identified a potential damage to the emergency condenser isolation valves and controls following a pipe break on the 75' elevation of the reactor building.

JCP&L letter to the NRC dated September 30, 1981 stated that an analysis to demonstrate that the emergency condenser piping will leak before a significant break could occur was to be performed. Our analysis indicates that the 75' elevation piping satisfies "leak before break" criteria recently developed by the NRC. The results and the detailed description of the analysis are given in the enclosed report.

Very truly yours,



P. B. Fiedler  
Vice President and  
Director-Oyster Creek

lr  
cc: Ronald C. Haynes, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pa. 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, N.J. 08731

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