May 26, 1982

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, DC 20555

Subject: LaSalle County Station Units 1 and 2
Teledyne Open Item and Error/Deviation
Reports for the LaSalle Independent
Design Review, Supplemental Responses
NRC Docket Nos. 50-373 and 50-374

- Reference (a): C. W. Schroeder letter to H. R. Denton dated March 16, 1982, "Independent Design Review Initial Status Report for the Period of February 11 through March 12, 1982."
 - (b): C. W. Schroeder letter to H. R. Denton dated May 7, 1982, "Teledyne Open Item and Error/Deviation Reports for the LaSalle Independent Design Review."
 - (c): C. W. Schroeder letter to H. R. Denton dated May 13, 1982, "Teledyne Open Item and Error/Deviation Report for the LaSalle Independent Design Review Second Transmittal."
 - (d): C. W. Schroeder letter to H. R. Denton dated May 14, 1982, "Teledyne Open Item and Error/Deviation Reports for the LaSalle Independent Derign Review -Final Transmittal; and 1st Transmittal of Responses."
 - (e): C. W. Schroeder letter to M. R. Denton dated May 20, 1982, "Teledyne Open Item and Error/Deviation Reports for the LaSalle Independent Design Review, Responses to Remaining Items."

Dear Mr. Denton:

Reference (a) provided you with an initial status report of the Independent Design Review being conducted at LaSalle County Station. References (b), (c), and (d) provided you with three sets of Teledyne Open Item and Error/Deviation Reports. References (d)

Bool 5:11

May 26, 1982 - 2 -H. R. Denton and (e) also transmitted our responses to the Open Item and Error/ Deviation Reports. The purpose of this letter is to transmit to you a supplemental response to error/deviation No. 7 which provides additional information. Also enclosed are tables which were inadvertantly omitted from the response to error/deviation No. 12 which was transmitted to you in Reference (e). Under separate cover, this material is being provided to Mr. James G. Keppler. If there are any questions regarding this matter, please contact this office. Very truly yours, CW School 5/26/82 C. W. Schroeder Nuclear Licensing Administrator 1 m Attachment cc: NRC Resident Inspector - LSCS - 1/0 4216N

Anton

May 25, 1982

Mr. L.O. DelGeorge:

Subject: Supplemental Responses for the LaSalle Independent Design Review

Enclosed are two supplemental responses to items identified by Teledyne. The supplemental response to error/deviation No. 7 provides additional information. The remaining pages are tables which were inadvertently omitted from the response to error/deviation No. 12 which I transmitted to you on May 19th.

You should transmit this information to Mr. Denton and Mr. Keppler.

> B. R. Shelton 5/25 B.R. Shelton

BRS/bmb/1574L

cc: J. Flaherty (Teledyne)

R.H. Holyoak T.E. Watts

C. Reed

J.J. Maley

B.B. Stephenson

May 24, 1982 Project No. 4266-24

Commonwealth Edison Company
LaSalle County Station - Units 1 & 2

Third Party Independent Review

Mr. B. R. Shelton
Project Engineering Manager
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

Dear Mr. Shelton:

Enclosed are 12 copies of Sargent & Lundy's supplemental response to Teledynes Error/Deviation Report 7. Also enclosed are pages E/D 12-4, 5, 6, 7, 8 and 9 which were inadvertantly left out from my letter to you of May 18, 1982.

It is our understanding that Commonwealth Edison Company will distribute these simultaneously to Teledyne, the NRC and internally.

If you have any questions, please do not hesitate to call me.

Yours very truly,

A H. POLLOCH

R. H. Pollock Mechanical Project Engineer

RHP: jam Enclosures Copies:

W. A. Chittenden (1/1)(1/1)E. V. Abraham (1/1)G. C. Kuhlman (1/1)R. J. Mazza (1/1)E. B. Branch (1/1)D. C. Haan W. G. Schwartz (1/0)(1/0)E. R. Weaver (1/1)S. D. Killian File 85

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SUPPLEMENTAL RESPONSE FOR ERROR REPORT 7

The question of incorporating code revisions in PIPSYS on projects subject to earlier code editions has been raised before and Sargent & Lundy has undertaken review of these changes in the past.

In the majority of cases, the Code revisions and associated PIPSYS modifications made since 1974 represent an analytical approach which more accurately reflects the physical characteristics of the piping components involved. In our view, this leads to a more reliable design. All of the code revisions issued since 1974 represent a significant effort on the part of the ASME through research and review, to accomplish that same goal.

In the same spirit, it is our policy with the PIPSYS Program to apply Code paragraph NA-1140 that allows Code addenda and editions, to be used that are published after the piping fabrication contract date. As Code analysis revisions are published, Sargent & Lundy reviews them for positive or negative impact on current designs. A revision is considered to have positive impact if it will not cause design changes and if it provides better methods or tools for analysis. A revision is considered to have negative impact if it has potential to cause design changes. Revisions that have positive impact are immediately incorporated into the PIPSYS Program. Those that have a negative impact are not incorporated unless a definite safety concern is generated by not doing so. Any revision causing a negative impact is reviewed with the Project team prior to implementation.

Current design documents specify piping design based on the 1974 Code. It is important to note that nearly all Code piping analysis revisions published since the 1974 Code was issued have been in the direction of either reducing unnecessary conservatism or providing better analytical techniques. On balance, the industry has benefited greatly from these changes.

The following discussion details the specific modifications to the PIPSYS stress analysis that have been incorporated since the 1974 Code. These items are the sum total of the differences between the PIPSYS analysis and the 1974 Code.

1. Class 1 Stress Analysis

The PIPSYS Class 1 stress analysis is based on the 1977 Code, up to and including the Summer of 1979 addenda. The differences from the 1974 Code are minor and, with the exception of Item C in the following list, the overall effect of all revisions was to reduce conservatism. This means that any reanalysis based on the later requirements will not generate system design changes. The specific revisions are as follows.

a) For branch connections, stress index B was reduced from 1.0 to 0.5, index C was reduced from 2.0 to 1.5, and index K was increased from 1.7 to 2.2.

- b) For curved pipe or butt weld elbows, stress index B₁ was reduced from 1.0 to 0.5.
- c) For butt weld reducers, stress indices $C_1 = 1.5$, $K_1 = 2.0$, $C_2 = 1.3$, and $K_2 = 1.0$ were replaced by index equations based on reducer geometry.
- d) For butt weld tees (B16.9), index B_1 was reduced from 1.0 to 0.5. Indices B_{2b} and B_{2r} were revised as follows:
- e) The allowable stress for Service Level B (Upset Condition) for equation (9) was increased from 1.5 Sm to 1.8 Sm.
- f) The T₁ stress term was eliminated from equation (10) and a thermal stress ratchet check was added in NB-3653.7.

2. Class 2 and 3 Stress Analysis

The PIPSYS Class 2 and 3 stress analysis is based on the 1977 Code, up to and including the Winter 1978 addenda. The latter analysis is essentially the same as that required in the 1974 Code except as follows:

- a) A stress intensification factor (i) was added for brazed joints.
- b) Equation (10a) was added to provide a criteria for single non-repeated anchor motions such as building settlements.
- c) A Service Level D stress limit (Faulted Condition) was added where no criteria was previously given. The limit is 2.4 Sh.

None of these code changes were made as a result of changes in other sections of the Code (e.g. NB-4000) nor did they require that other sections be changed. Therefore, we believe the use of the 1977 Code with addenda identified as above is appropriate for the analysis of Class 1, 2 or 3 piping otherwise designed, fabricated and installed to the 1974 Code.

TABLE 1

RHR PUMPS		alculations d Allowable PSI	Cla : 1 Calculations Calculated Allowable PSI PSI			
Discharge Column	21989	26250	21989	30000		
Pump Casing	5878	8250	5878	26250		
Suction Column	5368	16550	5368	32550		
Discharge Head Hold-Down Bolt	σ=12623 τ= 2565	σ=37500 τ=15500	13124	61000		
Motor Hold-Down Bolts	σ=22981 τ= 4005	σ=37500 τ=15550	23659	61000		
Foundation Bolts	$\sigma = 24329^{\circ}$ $\tau = 6373$	σ=29000 τ=11950	25897	35000		
Shaft Stresses	8314	15000	6149	17500		
Junction Between Stuffing Box and Discharge Elbows	25763	28875	43600	45000		

TABLE 2

PCS Pump	Class 2 Ca	lculations	Class 1 Cal	Allowable
escription	Calculated PSI	PSI	Calculated PSI	PSI
Stresses on Shaft .	7095	17500	9780	17500
Discharge Head Bolting	12912	55000	13333	60000
Thread Engagement	4159	25000	4974	30000
Stuffing Box-Discharge Elbow Interface	10202	27390	22345	63900
Foundation Load on Bolts	21829 (°) 6701 (r)	29000 11950	23743	35000
Motor Stand	1244	27390	1024	30000
Suction Barrel/Shell Interface	28300	28875		-
Motor Mounting Bolting	1877	55000	1879	60000
Minimum Thickness Pump Shell	required .242 in.	supplied .75 in.	required .197 in.	supplied .75 in.
Discharge Nozzle	.81 in.	1.312 in. .688 in.	.604 in.	1.312 in.
Suction Nozzle Torispherical Head of Shell	.152 in. .159 in,	.75 in.	.137 in.	.75 in.

EVALUATION AT CRITICAL LOCATIONS

LPCS Pump

TABLE 3

Location .	Evaluated	Evaluated as Class					
	Calculated	Allowable	CalculatedAllowable				
Motor Stand	5221 psi	19250 psi	5211 psi	30000 psi			
Motor Bolting	5587 psi(2) 2281 psi(2)	37500 psi 15500 psi	7449 psi	60000 psi			
Suction Barrel Shell	-	-	3180 psi	37800 psi			
Suction Barrel Shell at Inlet Nozzle	8195 psi	16500 psi		-			
Suction Barrel Head/Pin Interface	5024 psi	16500 psi	5024 psi	21000 psi			
Pump First Stage Casing Minimum Section	2446 psi	8250 psi	2446 psi	15000 psi			
Pump Series Casing Minimum Section	3000 psi	8250 psi	3000 psi	15000 psi			
Pump Series Casing Bolts	Am= 7.73 in ² (Required)	Ab=11.02 in ² (Available)	5617 psi	30000 psi			
Pump Top Casing Bolts	Am= 7.65 in ² (Required)	Ab=11.02 in ² (Available)	5592 psi	30000 psi			
Stuffing Box/ Discharge Elbow Interface	24170 psi	31500 psi	52900 psi	63900 psi			
Discharge Head Bolts	Am= 19.60 in ² (Required)	Ab=19.84 in ² (Available)	21676 psi	60000 psi			
Foundation Bolts	12730 psi (2) 4129 psi (2)	29000 psi 11950 psi	12730 psi	35000 psi			
Discharge Column	6101 psi	16500 psi	6101 psi	30000 psi			
Discharge Column Bolts	Am= 8.73 in ² (Required)	Ab=11.02 in ² (Available)	11112 psi	60000 psi			
Pump Shaft at Minimum Section	5838 psi	15000 psi	5838 psi	17500 psi			

RHR Pump

Analysis of Discharge Elbows at Stuffing Box Junction

The junction between the discharge elbow and the stuffing box is not a cylindrical nozzle but an intersection with two radii of curvature. The radius of curvature is in the transverse plane.

To accurately compute the local stresses at this junction under imposed piping loads, a detailed analysis using Bijlaard's stress tables was completed. (See attached sheets) From these calculations the worst combined stresses for the emergency loads are found to be 25763 psi at points A.B. and 19926 psi at points C.D. These stresses are well below the allowable value of 1.8S = 1.8 X17,5000 = 31500 psi for emergency and 1.65S = 1.65 X 17500 - 28875 psi for upset condition.

E/D 12-8 Table 3-Computation Sheet for Local Stresses in Spherical Sheils (Hollow Attachment) . NOZZLE Du -DL 1. Applied Loods" Redial Lood. Bu -4.9071 - 18948 Shear Load, Shear Load, Overturning Homent, Oversuring Moment, Terrional Mament. 4. Stress Concentration Factors 2. Cometry - 47' - 479' Yessel Thickness, due to: membrene load, Kn - 1.0 HOLLOW ATTACHMENT Vessel Mean Radius, bending load, Kb = ______ NOTE: Enter all tarce values in Nosale Thickness, Nossle Hean Radius,

From	Read curves	Compute absolute values of	STRESS	STRESSES - if load is apposite that shown, severse signs shown							
Fig.	for	stress and enter result +	Au	AL	8.	5L	Cu	CL	Du	DL	
SP-1 re 10	MxT + . 5) 6	Kn (N+T) · P · 676	-	-	-	-	- 676	-676	- 675	-67	
1	* .	Kb (Mx) GP	-	+	-	+	-	+		+	
1M-1 to 10	Moty Rat .	Kn (M+T \ R+T) + M. T= \ R+T - /2			X		-7244	-924	+ 9244	+724	
	May Rat	Kb (Mr. N-T) - 6M.	X		X		-	+	+	-	
	Maty KnT #	$K_{m} \left(\frac{N_{n}T \sqrt{R_{m}T}}{M_{2}} \right) \xrightarrow{\mathbf{M}_{2}} \frac{M_{2}}{T^{2} \sqrt{R_{m}T}} = 0$		-	+	+				X	
¥	May Rot	Kb (Max Rm) - 6M2 TZV RmT	-	+	+	-				N	
Add elgen	raically for sum	motion of 0 x, =					-9910	-1470	1252	1250	
SF-1 +0 10	#yT =	Kn (N,T) .P = 760	-	-	-	-	-760	- 760	-760	- 76	
Y	# ·	KI (M) (60 T) .	-	+	-	+	-	+	-	+	
SM-1 re 10	MyTy Rot	$K_{n} \left(\frac{N_{n}T \sqrt{R_{n}T}}{M_{n}} \right) \cdot \frac{\beta_{-m} - S}{T^{2} \sqrt{R_{n}T}} =$		X	X		- 535	- 6355	+ 6355	+635	
	My\ RmT	Kb (My (RMT) - 6M. TEV RMT	1			1	-	+	+	-	
	HyTY BET	Kn (NyTV RNT) - 12 V RNT	7 -	-	+	+	X				
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pr. strem = 7741 PSi

Stresses in Shells

	Table	3-Computati	on Sheet for Lo	cal Stres	ses in Spi	nerical S	heils (Ho	llow Atta	chinient)	-	
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Nossi	e Hean Kadius, Outside Radio	rm			ce values in		PHR	Pum	105.	R	m
From	Read curves		lute values of		S = if load						
Fig.	for	stress and e		Au	AL	5v	BL	Cu	CL	Du	DL
SP-1 10 10	H+T = -037	$K_{P} = \left(\frac{N \times T}{P}\right) + \frac{P}{T^{2}}$	-1563	-1563	-1563	-1563	-1563	-	-	-	-
V	Mr *	Kb (+)- 45	• = +	-	+	-	+		+	-	+
SM-1 m 10	MaT \ R-T = A 2	Kn (NATY HAT) - 115.37.			X	X	-	-	+	+ '
	May RmT =	Kb (May Ray) + 6 m. T 2 \ RmT			X		-	+	+	-
	Maty KmT =	Kn (NYTY RAT	102703 T2VRMT	-10703	-10703	+10703	+10703				
· ·	May Rat	Kb (ME ANT)	* T2 V RmT *	-	+	+	-		X		
		mation of 0 x, =			12266		-	-	-		
SP-1 to 10	HyT = 05 9		-2492	-2452	-2492	-2492	-2452	-	-	<u> -</u>	-
V	# ·	Kb (#)	*	-	+	-	+	-	+	-	+
SM-1 to 10	MyTy Rat	Kn (N+T) RmT) · 164 5 =	\times	\geq			-	-	+	+
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	NyTy RmT	Kn (NyTy Fat) + 147m 30 +	-1473	-14750	+147%	+147/0				
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Max. A shows = 7741 Psi

Stresses in Shells 5 = 18022 PSI (may of ASE) 7