

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Station
 DATE May 7, 1982
 COMPLETED BY R. W. Short
 TELEPHONE (402)536-4543

MONTH April, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>485.5</u>
2	<u>316.3</u>
3	<u>225.5</u>
4	<u>470.2</u>
5	<u>482.2</u>
6	<u>484.2</u>
7	<u>485.4</u>
8	<u>485.6</u>
9	<u>485.9</u>
10	<u>485.7</u>
11	<u>485.0</u>
12	<u>484.9</u>
13	<u>484.6</u>
14	<u>484.1</u>
15	<u>483.7</u>
16	<u>483.8</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>484.3</u>
18	<u>484.0</u>
19	<u>484.2</u>
20	<u>484.6</u>
21	<u>484.9</u>
22	<u>485.0</u>
23	<u>484.8</u>
24	<u>484.1</u>
25	<u>463.4</u>
26	<u>483.1</u>
27	<u>482.9</u>
28	<u>483.6</u>
29	<u>483.1</u>
30	<u>483.0</u>
31	<u></u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8205180361

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE May 7, 1982
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: April, 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>719.0</u>	<u>2,879.0</u>	<u>75,360.0</u>
12. Number Of Hours Reactor Was Critical	<u>706.4</u>	<u>2,812.9</u>	<u>59,051.9</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>702.8</u>	<u>2,803.8</u>	<u>57,893.8</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,041,907.5</u>	<u>4,103,593.0</u>	<u>70,805,423.1</u>
17. Gross Electrical Energy Generated (MWH)	<u>354,008.0</u>	<u>1,395,159.9</u>	<u>23,469,105.5</u>
18. Net Electrical Energy Generated (MWH)	<u>337,620.6</u>	<u>1,330,583.8</u>	<u>22,178,452.3</u>
19. Unit Service Factor	<u>97.7</u>	<u>97.4</u>	<u>76.8</u>
20. Unit Availability Factor	<u>97.7</u>	<u>97.4</u>	<u>76.8</u>
21. Unit Capacity Factor (Using MDC Net)	<u>98.2</u>	<u>96.7</u>	<u>64.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>98.2</u>	<u>96.7</u>	<u>63.8</u>
23. Unit Forced Outage Rate	<u>2.3</u>	<u>2.6</u>	<u>3.9</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation): None

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
<u> </u>	<u> </u>
<u> </u>	<u> </u>
<u> </u>	<u> </u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April, 1982

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE May 7, 1982
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-03	820402	F	16.2	H	3	N/A	EA	ZZZZZZ	Generator tripped due to a transient on 345 KV power line. Probable cause was high winds from stormy weather. The District is continuing to investigate the event to determine if any corrective actions can prevent the incident from recurring.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending April 1982.

1. Scheduled date for next refueling shutdown. January 7, 1983
2. Scheduled date for restart following refueling. April 1, 1983
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?

A Technical Specification Change

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
 - c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. October 1, 1982
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>237</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

J K Gayer

Date

May 3, 1982

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

April, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power for the month of April. One forced outage occurred on April 2, 1982 at approximately 1600. The unit was returned to service at approximately 0800 on April 3, 1982. The cause of the forced outage involved protective relaying on the main transformer.

Two new employees reported to the operations department during April and are currently participating in the training program at Fort Calhoun Station.

The Institute of Nuclear Power Operations conducted their second evaluation and assistance audit at Fort Calhoun Station. The audit was two weeks in length and involved all major areas of interest at Fort Calhoun Station.

Normal surveillance tests and operational tests were completed.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
LER-006	On March 23, 1982 at approximately 0930, HCV-506A failed to close via the control room switch. This was observed during the performance of surveillance test ST-ISI-WD-1, F.1. Upon discovery, air was immediately isolated from the valve allowing it to fail closed. Emergency Procedure EP-25, "Loss of Containment Integrity" was initiated and carried out. During the incident, the redundant isolation valve, HCV-506B, remained operable and did close via control switch demand.
LER-007	During the performance of the monthly test (ST-ESF-6) of diesel generator #2, while the plant was at approximately 93% power, a leak was discovered in the copper tubing vent line from the thermal mixing valve to the coolant expansion tank. Diesel generator #2 was shutdown for repairs in accordance with Tech. Spec. 2.7(2)i. During this time, diesel generator #1 and its associated safeguards equipment were operable. In addition, both offsite power supplies (161 KV and 345 KV) were available.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operation Incidents and are not reported elsewhere in the report:

<u>Operation Incidents</u>	<u>Deficiency</u>
OI-1526 ST-ISI-NG-1	HCV-2604A valve stroke time was not reviewed as required by the procedure.
OI-1530 ST-ISI-CC-3	Review of the surveillance test was not completed within the specified time limit.
OI-1529 ST-FW-3	A procedure step was skipped which resulted in both auxiliary feedwater pumps inadvertently starting. The containment isolation valves remained closed, preventing the flow of auxiliary feedwater.
OI-1528 ST-ESF-1, F.2	During performance of ST-ESF-1, F.2, C/PIA-102Y upper alarm contact would not reset.
OI-1475 ST-ESF-1, F.4	During performance of ST-ESF-1, F.4 the 86A/CIAS lockout relay failed to trip.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1 4-6-82	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure

Description

SP-FAUD-1 4-23-82 Fuel Assembly Uplift Condition Detection.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF

NONE

G. TRAINING

Licensed operator requalification training continued as scheduled. P.T.S. has been included. Initial auxiliary operator training was conducted for three new operators. Fire brigade training and drills were conducted. Maintenance and Technical training was conducted as scheduled.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

NONE



S. C. Stevens
Manager
Fort Calhoun Station

Monthly Operations Report
April, 1982
Page Four

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
13868	2-4-82	Area Monitor RM-054A, Alarm setpoint coming in too early.	Cleaned contacts.
14363	3-24-82	DG-2 Governor needs repair.	Replaced governor.
14346	3-23-82	Sigma for SGLS initiation out of tolerance on setpoint.	Reset trip setpoint performed Surveillance Test.
14477	4-6-82	Sigma Meter C/PIA-102Y, Upper alarm will not clear.	Replaced upper & lower alarm units.
14355	3-29-82	DG-2 leak at flange by thermal mixing valve.	Installed new line.
14340	3-22-82	RM-050 belt burnt up.	Replaced pump & belt.
14368	4-5-82	Fire Pump FP-1B replace diesel cooling water copper tubing.	Complete.
14530	4-7-82	Replace DC3-1 Breaker.	Complete.
14410	4-2-82	HCV-2501 leak on RCS Loop 2.	Tightened packing.
14523	4-7-82	Component Cooling Water Heat Exchange Valves HCV-402C & 402D will not open.	Replaced solenoid valves.
14450	4-2-82	CEDM Rod #17 moves slowly during withdrawal.	Completed ST-CEA-1, F.9 for verification of Drop Time.
14633	4-19-82	Component Cooling Water Heat Exchanger Valve HCV-402D will not open.	Repaired ASCO solenoid valve.