



Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

May 12, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Byron Station Units 1 and 2
Braidwood Station Units 1 and 2
NSSS Support Materials
NRC Docket No. 50-454, 455, 456
and 457

Reference (a): January 11, 1982, letter from
T. R. Tramm to H. R. Denton.

Dear Mr. Denton:

This is to provide additional information regarding the materials used in the supports for reactor coolant pumps and steam generators at Byron and Braidwood Stations. Attachment A to this letter describes the specifications for these materials as they relate to fracture toughness. It supplements the information provided in reference (a) regarding Unresolved Safety Issue A-12.

Please address questions regarding this matter to this office.

One signed original and fifteen copies of this letter are provided for your use.

Very truly yours,

T. R. Tramm
Nuclear Licensing Administrator

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Attachment

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ATTACHMENT A

BYRON/BRAIDWOOD REACTOR COOLANT PUMP AND
STEAM GENERATOR SUPPORT MATERIALS

ASTM A588 Grade A or B, A618 Grade III, and SA533 Class 2 are the materials used in the steam generator and reactor coolant pump supports which are welded. Charpy V notch (C_v) testing was specified for these supports to insure adequate fracture toughness. The C_v tests were performed on the base metal and qualification welds in accordance with ASME Code, Section III, Div. 1, Subsection NF 2320 and NF 4335, Summer of 1975 addenda. These C_v tests were performed at 10⁰F, 30⁰F below the assumed lowest service metal temperature of 40⁰F and well below 65⁰F, the minimum operating temperature for the containment. NF 2331 contains the acceptance criteria; 15 mils lateral expansion for materials 5/8" to 1" and 25 mils lateral expansion for material 1" and over.

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