VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT NO. 82-03

MARCH, 1982

APPROVED BY: J. L. Wilson
STATION MANAGER

OPERATING DATA REPORT

DOCKET NO. 50-280
DATE 07 APR 82
COMPLETED BY Vivian H. Jones
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 1
2. REPORTING PERIOD	30182 TO33182
3. LICENSED THERMAL POWER (MWT)	2441
4. NAMEPLATE RATING (GROSS MWE)	847.5
5. DESIGN ELECTRICAL RATING (NET MWE)	788
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A
10. REASONS FOR RESTRICTIONS, IF ANY	N/A

NOTES

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	2160.0	81288.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	732.2	1768.5	47803.0
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3731.5
14. HOURS GENERATOR ON-LINE	728.6	1753.6	46828.4
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	3736.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1735017.5	4081076.5	108414342.9
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	556885.0	1285955.0	35105168.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	530421.0	1220522.0	33297758.0
19. UNIT SERVICE FACTOR	97.9 o/o	81.2 o/o	57.6 o/o
20. UNIT AVAILABILITY FACTOR	97.9 o/o	81.2 o/o	62.2 o/o
21. UNIT CAPACITY FACTOR (USING MDC NET)	92.0 o/o	72.9 o/o	52.9 o/o
22. UNIT CAPACITY FACTOR (USING DER NET)	90.5 o/o	71.7 o/o	52.0 o/o
23. UNIT FORCED OUTAGE RATE	3.4 o/o	4.7 o/o	24.8 o/o

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE,DATE,AND DURATION OF EACH) MAINTENANCE - 11/19/82 - DAYS-10

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

DOCKET NO. 50-281
DATE 07 APR 82
COMPLETED BY Vivian H. Jones
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 2	
2. REPORTING PERIOD	30182 TO 33182	
3. LICENSED THERMAL POWER (MWT)	2441	NOTES
4. NAMEPLATE RATING (GROSS MWE)	847.5	
5. DESIGN ELECTRICAL RATING (NET MWE)	788	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775	
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A	
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A	
10. REASONS FOR RESTRICTIONS, IF ANY	N/A	

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	2160.0	78168.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	611.8	1993.2	46853.9
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	589.8	1956.4	46068.1
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1285201.7	4218736.1	107515522.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	402635.0	1354950.0	35032129.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	377753.0	1273667.0	33202382.0
19. UNIT SERVICE FACTOR	79.3 %	90.6 %	58.9 %
20. UNIT AVAILABILITY FACTOR	79.3 %	90.6 %	58.9 %
21. UNIT CAPACITY FACTOR (USING MDC NET)	65.5 %	76.1 %	54.8 %
22. UNIT CAPACITY FACTOR (USING DER NET)	64.4 %	74.8 %	53.9 %
23. UNIT FORCED OUTAGE RATE	5.4 %	2.9 %	16.8 %
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)	SPRING MAINTANCE - 5/7/82 - 10DAYS		

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1982

DOCKET NO. 50-280
UNIT NAME Surry One
DATE April 7, 1982
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-6	03-20-82	S	0.0	H	4				Reduced power to allow stopping equipment to reduce load on "A" Reserve Station Service Transformer to <2000 amps during Unit 2 recovery.
82-7	03-25-82	F	15.4	H	3	50-280/ 82-040/03L-0			Instrument technicians performing a periodic test placed instrumentation in "trip" which in coincidence with a switch out adjustment caused the "A" reactor coolant pump to trip causing a low flow reactor trip. The switch was adjusted prior to unit startup.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-281
UNIT NAME Surry Two
DATE April 7, 1982
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

REPORT MONTH March, 1982

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-17	03-01-82	S	120.7	H	1				Continuation of shutdown for maintenance which began 02-27-82.
82-18	03-08-82	F	0.0	A	4				Power reduction to remove 2-SD-P-1A (high pressure drains pump) from service for repairs.
82-19	03-11-82	F	24.1	G	3	50-281/ 82-017/03L-0			The unit was shutdown IAW T.S. 3.3.B due to a loss of recirculation flow to the boron injection tank. The recirculation flow was reestablished prior to startup.
82-20	03-20-82	F	9.4	A	2				Loss of EHC pressure due to a relief valve lifting caused all turbine governor valves to drift closed. Operator manually tripped the turbine and reactor; problem was corrected prior to unit startup.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-281
UNIT NAME Surry Two
DATE April 7, 1982
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

REPORT MONTH March, 1982

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-21	03-30-82	F	0.0	A	4				Reduced power to allow 2-FW-P-1B (main feed pump) to be taken out of service for repairs.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 1 MONTH: March, 1982

DATE	TIME	HOURS	LOAD, MW	REDUCTIONS, MW	MWH	REASON
			None during this reporting period.			
				MONTHLY TOTAL		

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 2 MONTH: March, 1982

DATE	TIME	HOURS	LOAD, MW	REDUCTIONS, MW	MWH	REASON
			None during this reporting period.			

MONTHLY TOTAL	

 DOCKET NO 50-280
UNIT SURRY I
DATE 4-1-82
COMPLETED BY Vivian H. Jones

AVERAGE DAILY UNIT POWER LEVEL

MONTH: MARCH 82

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	751.7	17	747.7
2	752.4	18	745.5
3	748.8	19	744.6
4	747.8	20	654.2
5	748.5	21	744.8
6	747.8	22	745.8
7	749.0	23	743.6
8	749.2	24	744.9
9	751.9	25	329.5
10	748.5	26	280.0
11	748.3	27	678.3
12	744.4	28	744.1
13	746.6	29	739.0
14	741.9	30	745.4
15	744.5	31	745.2
16	746.4		

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 o/o LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

DOCKET NO 50-281
UNIT SURRY II
DATE 4-1-82
COMPLETED BY Vivian H. Jones

AVERAGE DAILY UNIT POWER LEVEL

MONTH: MARCH 82

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.0	17	697.0
2	0.0	18	696.3
3	0.0	19	694.3
4	0.0	20	333.9
5	0.0	21	602.4
6	381.8	22	622.0
7	428.7	23	626.5
8	538.3	24	624.8
9	712.1	25	694.5
10	722.9	26	694.8
11	490.0	27	696.9
12	86.2	28	696.5
13	614.9	29	696.0
14	669.5	30	682.4
15	693.8	31	649.4
16	696.6		

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 o/o LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

SUMMARY OF OPERATING EXPERIENCE

MARCH, 1982

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

March 1 — This reporting period begins with the unit at 100% power.

March 20 — 1000 - Started reducing power to allow stopping 1-FW-P-1A (main feed pump) and 1-CN-P-1A (main condensate pump) to reduce "A" Reserve Station Service Transformer load to < 2000 amps during Unit 2 recovery.

1155 - Stopped power decrease at 72% power.

1157 - Stopped 1-FW-P-1A and 1-CN-P-1A.

1548 - Started 1-FW-P-1A and 1-CN-P-1A and commenced increasing power at 3% per hour.

March 21 — 0330 - The unit is at 100% power.

March 25 — 1035 - During the performance of Monthly Periodic Test (safety injection and feedwater control isolation logic) the breaker to the "A" Reactor Coolant Pump ("A" RCP) opened. This resulted in a RCP "A" breaker open reactor trip. Reactor and generator power dropped off to zero very rapidly along with steam flow from "A" S/G. Steam flow from S/G's B & C didn't fall as fast as programmed flow, thus a 2 out of 3 high steam flow signal was generated. Tave dropped below 543°F giving a low Tave signal, which combined with the high steam flow signal to give a Safety Injection.

1912 - The reactor was critical.

1958 - The reactor tripped on a Safety Injection (SI) signal from steam header to steam line ΔP. The SI was spurious and caused by vibration of the header pressure transmitters mounted in the turbine building.

2312 - The reactor was critical.

March 26 — 0157 - The generator was placed on the line.

0219 - Power increase was stopped at 35% to verify Steam Generator chemistry in specification.

1102 - S/G chemistry was verified in specification and a power increase at 3% per hour commenced.

SUMMARY OF OPERATING EXPERIENCE
MARCH, 1982

March 27 1300 - The unit was at 100% power.

March 29 1230 - Leakage past the reactor coolant system (RCS) make-up valves during boration to the spent fuel pool introduced unwanted boron in the RCS. This caused a reduction in RCS temperature (Tave).

1245 - Started reducing power to recover Tave.

1305 - Stopped power decrease at 86%/710 MWe. Tave has been recovered.

1348 - Started power increase at 3% per hour.

1525 - The unit was at 100% power.

March 31 This reporting period ends with the unit at 100% power.

UNIT TWO

March 1 This reporting period begins with the unit at cold shutdown for environmental upgrade of various power supply cable splices and various maintenance items.

March 4 1715 - Commenced primary system heatup.

1810 - RCS temperature exceeded 200°F (Cold Shutdown condition).

2243 - Started 2-RC-P-1A (reactor coolant pump) after repairs on the "open" limit switch on the cold leg stop valve were completed. The failure of this LS prevented makeup of the logic circuit for starting 2-RC-P-1A.

March 5 0025 - RCS temperature and pressure exceeded 350°F and 450 PSIG respectively.

0800 - The unit reached hot shutdown condition.

1349 - The reactor was critical.

March 6 0045 - The generator was placed on the line. The delay between reactor criticality achievement and the generator going on line was due to problems encountered with the auxiliary overspeed governor solenoid valves. The solenoid valves were disassembled and repaired.

SUMMARY OF OPERATING EXPERIENCE
MARCH, 1982

March 6 (cont'd)

0150 - The unit was at 35% power and holding for steam generator (S/G) chemistry verification.

0319 - S/G chemistry verified in specification and power increase started.

1418 - Stopped power increase at 66% to maintain condensate polishing building $\Delta P \leq 50$ PSIG unitl the high pressure heater drain pump is returned to service.

March 7

0403 - Attempted to start the 2-FW-P-1A main feed pump. The attempt failed due to a ground on the inboard motor. The gound was a result of water in the motor. The water entered the motor when the relief valve on a feedwater heater lifted.

1553 - The high pressure heater drain pump was returned to service.

March 8

0825 - The "A" MFP was returned to service.

0840 - Started increasing power.

1030 - The unit was at 100% power.

1427 - Commenced power reduction to stop the high pressure heater drain pump.

1433 - Stopped power decrease at 94% power and secured the high pressure heater drain pump for shaft seal repairs.

1446 - Commenced power reduction to reduce ΔP across the condensate polishing demineralizers to $\leq$ 50 PSIG.

1645 - Stopped power decrease at 70% power and 540 MWe with six condensate polishing demineralizers in service.

2342 - Returned seventh condensate polishing demineralizer to service and commenced increasing power until ΔP is equal to 50 PSIG.

March 9

0146 - Stopped power increase at 86% power.

0226 - Started the high pressure heater drain pump.

0320 - Started increasing power.

0527 - The unit was at 100% power.

1720 - Numbers 2 and 3 Governor Valves started drifting closed causing loss of approximately 50 MWe.

SUMMARY OF OPERATING EXPERIENCE
MARCH, 1982

March 9 (cont'd)

1743 - Stabilized the unit at 690 MWe/85% power.

1753 - No. 2 and 3 GV's slowly opening.

1838 - The unit was at 100% power.

1942 - Isolated EHC fluid to no. 3 GV to clean the EHC strainer in the servo-valve.

March 10

1111 - Number 2 GV failed closed causing drop in load to approximately 700 MWe and in power to approximately 90%. The EHC fluid to no. 2 GV was isolated to allow cleaning the strainer in the servo-valve.

1345 - Valved in EHC fluid to no. 3 GV.

1400 - Valved in EHC fluid to no. 2 GV.

1548 - Commenced increasing power. The strainers for no. 1 and no. 4 GV's were also cleaned.

1620 - The unit was at 100% power.

March 11

1200 - The Chemist reported low boron concentration in the samples drawn on the boron injection tank (BIT). Investigation by the Operations Department revealed no recirculation flow to or from the BIT and no boric acid flow to the blender. Started two (2) hour "clock" for start of shutdown due to loss of BIT recirc.

1208 - A valve was discovered closed on the suction of the boric acid transfer pump supplying boric acid to the blender and boric acid recirc on the BIT. The valve was opened and boric acid flow to the blender was restored.

1355 - All attempts to restore recirculation flow on the BIT have proven unsuccessful and a shutdown has commenced.

1735 - The reactor tripped form 15% power due to a high level signal from the 6A feedwater heater. The high level was a result of leaking tube(s) in the feedwater heater. The 5A and 6A feedwater heaters were subsequently bypassed and isolated.

March 12

1500 - The reactor was critical after the plug in the inlet line to the BIT recirc was cleared and the heat tracing on the lines restored.

1742 - The generator was placed on the line. The upper limit for turbine power is 90% with the 5A and 6A feedwater heaters bypassed and isolated.

SUMMARY OF OPERATING EXPERIENCE
MARCH, 1982

March 12 (cont'd) 1903 - Stopped power increase at 35% to verify S/G chemistry.

1910 - S/G chemistry was in specification and a power increase commenced.

March 13 0005 - Stopped power increase at 90%.

0145 - Stopped the high pressure heater drains pump for shaft seal repair.

March 14 0527 - Started the high pressure heater drains pump.

0835 - Started increasing power to 740 MWe.

0915 - Stopped power increase at 740 Mwe/95% power.

March 20 0545 - All turbine governor valves started drifting closed causing a loss of turbine load. The control room operator manually tripped the turbine and the turbine trip caused a reactor trip. A relief valve lifting on the EHC system caused a loss of EHC pressure to the hydraulically operated GV's allowing them to close.

0642 - The reactor was critical.

1506 - The generator was placed on the line.

1601 - Stopped power increase at 35% to verify S/G chemistry.

1623 - S/G chemistry was in specification and a power increase has commenced.

2045 - Stopped increasing load and power at 87% on the turbine and 97% on the reactor.

2205 - Reduced turbine load to 85% and assumed all auxiliary steam loads.

March 30 2129 - Started reducing power to allow removing 2-FW-P-1B (main feed pump) from service for repairs.

2210 - Stopped B MFP.

2218 - Stopped power decrease at 560 MWe/72% power.

March 31 0545 - Started B MFP.

0554 - Started power increase.

0715 - Stopped power increase at 740 MWe/96% power.

This reporting period ends with the unit at 96% power and limited to 740 MWe as a result of 5A and 6A feedwater heaters being out of service.

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

MARCH, 1982

The Nuclear Regulatory Commission, on January 19, 1982, issued Amendment Nos. 73 and 74 to the Operating License for Surry Power Station Unit Nos. 1 and 2 respectively.

These amendments revise the Technical Specifications to allow an increase in enrichment for new and spent fuel from 3.7 to 4.1 weight percent of U-235.

Accordingly, the paragraph 3.B of the Operating License for Unit 1 and 2, respectively, is amended as follows:

(Unit 1) "B. Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

(Unit 2) "B. Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, are hereby incorporated in the license, The licensee shall operate the facility in accordance with the Technical Specifications."

The Nuclear Regulatory Commission, on February 2, 1982, issued Amendment Nos. 74 and 75 to the Operating License for Surry Power Station Unit Nos. 1 and 2 respectively.

These amendments revise the Technical Specifications to reduce the minimum number of thimbles required for incore flux mapping from 40 to 38.

Accordingly, the paragraph 3.B of the Operating License for Unit 1 and 2, respectively, is amended as follows:

(Unit 1) "B. Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

(Unit 2) "B. Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

FACILITY CHANGES REQUIRING NRC APPROVAL

March, 1982

None during this reporting period.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

March, 1982

		Unit
D/C 80-45	Add Gasket to Top and Bottom of RSHX	1 & 2

This design change isolates the diaphragm from potential excessive positive containment pressure by installing a gasket between the diaphragm and the coverplate. This is a more conservative measure taken to provide protection for the diaphragms during a LOCA.

SUMMARY OF SAFETY ANALYSIS

The modification will not affect the operation of any safety related equipment; it serves only as an added degree of protection for the system involved.

D/C 81-03	Liquid Storage Warehouse	1 & 2

To insure that proper fire and safety precautions are utilized, a storage facility was built for flammable, combustible and corrosive liquids. This warehouse was constructed in Storage Area "C" east of the Plant Warehouse.

SUMMARY OF SAFETY ANALYSIS

This facility allows for the proper storage of flammable and corrosive chemicals, thereby enhancing the safe and efficient operation of the plant.

D/C 81-08	Ballistic Protection and Modification of Masonry Block Walls SB-27-0-6, 7, and 4	1 & 2

To ensure that Masonry Walls SB-27-0-6, 7, and 4 meet ballistic requirements and seismic criteria, the following design change(s) were made in response to IE Bulletin 80-11:

a. Installed ballistic shielding (Level 4)
b. Relocated safety related equipment
c. Reinforced wall by the addition of steel members.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

March, 1982

		Unit
D/C 81-08	Ballistic Protection and Modification of Masonry Block Walls SB-27-0-6, 7, and 4 (continued)	1 & 2

SUMMARY OF SAFETY ANALYSIS

The modifications associated with this design change support the FSAR commitments. The structural steel added by this modification does not adversely affect the structural capability of the existing floor slabs.

D/C 81-19	Machine Shop Replacement Facility	1 & 2

D/C 81-19B Structural Steel Frame for Machine Shop Replacement Facility, was implemented.

SUMMARY OF SAFETY ANALYSIS

The addition of the Machine Shop Replacement Facility does not minimize the safety of operating units or effect the operation of safety related equipment.

D/C 81-31	Installing Indicating Light for Valve Position	1 & 2

This design change installed indicating lights on the vertical board of control room for AOV-1A-103 (Unit 1) and AOV-1A-203 (Unit 2) to verify if the valves are positioned open or closed. These valves open to supply suction to air compressors upon containment isolation.

SUMMARY OF SAFETY ANALYSIS

This modification does not effect the operation of any safety related equipment. It will improve the operational ability of containment Instrument Air System.

D/C 81-57	Fuel Pool Inspection Light Receptacles	1 & 2

This design change installed four receptacles on the pool side of the crane well for fuel inspection flood lights. This was accomplished by installing a 480/120 1ϕ transforemer and four water tight electrical outlets.

SUMMARY OF SAFETY ANALYSIS

This modification does not effect the operation of any safety related equipment.

D/C 81-101	Replace Impellers in Radiation Monitoring Sample Pumps	2

This design change replaces the existing impellers with larger impellers to provide the required TDH. The TDH was increased due to D/C 80-56.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

March, 1982

		Unit
D/C 81-101	Replace Impellers in Radiation Monitoring Sample Pumps (continued)	2

SUMMARY OF SAFETY ANALYSIS

This modification does not affect station operations or operations of the radiation monitoring sample pumps. It does provide the increased TDH requirements.

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

March, 1982

None during this reporting period.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

March, 1982

Special Test No.	Unit	Title	Date Completed
ST-52	2	Reactor Coolant System Flow Measurement Test	03-08-82
ST-36	2	Steam Generator Moisture Carryover Measurement	03-16-82

OTHER CHANGES, TESTS AND EXPERIMENTS

March, 1982

None during this reporting period.

SURRY POWER STATION

CHEMISTRY REPORT

March, 1982

T.S. 6.6.3.d

PRIMARY COOLANT ANALYSIS	UNIT NO. 1 (B)			UNIT NO. 2 (C)		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., µCi/ml	4.27 (A)	1.70 (A)	2.51 (A)	3.60E-1	5.41E-2	1.73E-1
Suspended Solids, ppm	0.1	0.1	0.1	0.1	0.1	0.1
Gross Tritium, µCi/ml	1.50E-1	6.65E-2	1.04E-1	1.23E-1	7.71E-2	9.94E-2
Iodine-131, µCi/ml	5.58 (A)	7.33E-2	7.33E-2	7.63E-2	9.58E-4	1.38E-2
I-131/I-133	1.2214	.3708	.8673	.8955	.4808	.7358
Hydrogen, cc/kg	38.1	21.5	29.1	49.8	27.6	36.8
Lithium, ppm	1.35	.86	1.10	1.38	.55	.99
Boron-10, ppm +	204	109	133	456	162	246.
Oxygen-16, ppm	.000	.000	.000	.005 (D)	.000	.000
Chloride, ppm	<.05	<.05	<.05	<.05	<.05	<.05
pH @ 25°C	6.88	6.43	6.66	6.66	5.66	6.26

+ Boron-10 = Total Boron x 0.196

NON-RADIOACTIVE CHEMICAL RELEASES, POUNDS (D)
T.S. 4.13.A.6

Phosphate	-	Boron	1055
Sulfate	-	Chromate	.15
50% NaOH	-	Chlorine	-

Remarks: (A) Indicates possible failed fuel elements (B) Unit trip @ 1035 3-25-82 on line @ 0300 3-26-82 (C) Unit shutdown condition 3-1-82 ~ 3-6-82; on line 2 0045 3-6-82; 3-11-82 @ 1355 commenced ramp-down due to low boron injection tank concentration - Rx trip 1735; Rx on line 3-12-82 @ 1742; Rx 3-20-82 @ 0454, on line @ 1540 3-20-82 (D) These levels of chemicals should create no adverse environmental impact.

DESCRIPTION OF ALL INSTANCES WHERE THERMAL DISCHARGE LIMITS WERE EXCEEDED

March, 1982

Due to the impairment of the circulating water system on the following days, the thermal discharge limits were exceeded as noted.

March 6, 1982	Exceeded 17.5°F ΔT across station*
March 7, 1982	Exceeded 17.5°F ΔT across station
March 8, 1982	Exceeded 15°F ΔT across station*
March 9, 1982	Exceeded 17.5°F ΔT across station*
March 10, 1982	Exceeded 17.5°F ΔT across station
March 11, 1982	Exceeded 17.5°F ΔT across station*
March 13, 1982	Exceeded 17.5°F ΔT across station
March 14, 1982	Exceeded 15°F ΔT across station
March 15, 1982	Exceeded 17.5°F ΔT across station
March 16, 1982	Exceeded 17.5°F ΔT across station
March 17, 1982	Exceeded 17.5°F ΔT across station
March 18, 1982	Exceeded 17.5°F ΔT across station
March 19, 1982	Exceeded 15°F ΔT across station
March 20, 1982	Exceeded 15°F ΔT across station*
March 21, 1982	Exceeded 15°F ΔT across station*
March 22, 1982	Exceeded 15°F ΔT across station
March 23, 1982	Exceeded 15°F ΔT across station*
March 24, 1982	Exceeded 15°F ΔT across station
March 25, 1982	Exceeded 15°F ΔT across station*
March 27, 1982	Exceeded 15°F ΔT across station*
March 28, 1982	Exceeded 15°F ΔT across station*
March 29, 1982	Exceeded 15°F ΔT across station*
March 30, 1982	Exceeded 15°F ΔT across station*
March 31, 1982	Exceeded 17.5°F ΔT across station*

*Indicates dates where station ΔT was less than or equal to 15.0°F across station for some time during the day.

The ΔT excursions were allowable under Technical Specification 4.14.B.2. There were no reported instances of adverse environmental impact.

The temperature change at the station discharge exceeded 3°F per hour on the following dates and for the noted reasons:

On March 20, 1982, due to a Unit 2 reactor trip;
On March 25, 1982, due to a Unit 1 reactor trip; and
On March 28, 1982, due to natural causes (2.5°F increase in station inlet water temperature).

These events were allowable in accordance with Technical Specification 4.14.B.1. There were no reported instances of adverse environmental impact.

The temperature change at the station discharge exceeded 3°F per hour on March 12, 1982, due to a rapid increase in circulating water flow through Unit 2 main condenser. This event was reported in accordance with Technical Specification 4.14.C.1.

FUEL HANDLING

March, 1982

UNIT ONE

Twenty four (24) new fuel assemblies and twenty four (24) new BPRA's were received in March, 1982 for Unit one refueling.

UNIT TWO

None during this reporting period.

FUEL HANDLING

March, 1982

UNIT ONE

DATE SHIPPED/RECEIVED	NO. OF ASSEMBLIES PER SHIPMENT	ANSI NO. INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
03-04-82	12	LM0AM3/3.6%	<2.5 MR/HR
		LM0ALF/3.6%	<2.5 MR/HR
		LM0AMN/3.6%	<2.5 MR/HR
		LM0AL5/3.6%	<2.5 MR/HR
		LM0ALG/3.6%	<2.5 MR/HR
		LM0AML/3.6%	<2.5 MR/HR
		LM0AMH/3.6%	<2.5 MR/HR
		LM0ALW/3.6%	<2.5 MR/HR
		LM0ALJ/3.6%	<2.5 MR/HR
		LM0AME/3.6%	<2.5 MR/HR
		LM0ALU/3.6%	<2.5 MR/HR
		LM0AMK/3.6%	<2.5 MR/HR
03-11-82	12	LM0AMP/3.6%	<2.5 MR/HR
		LM0ALP/3.6%	<2.5 MR/HR
		LM0ALL/3.6%	<2.5 MR/HR
		LM0AMQ/3.6%	<2.5 MR/HR
		LM0ALN/3.6%	<2.5 MR/HR
		LM0AM2/3.6%	<2.5 MR/HR
		LM0AM1/3.6%	<2.5 MR/HR
		LM0ALM/3.6%	<2.5 MR/HR
		LM0ALQ/3.6%	<2.5 MR/HR
		LM0AMJ/3.6%	<2.5 MR/HR
		LM0AM5/3.6%	<2.5 MR/HR
		LM0AM0/3.6%	<2.5 MR/HR

FUEL HANDLING

March, 1982

UNIT TWO

DATE SHIPPED/RECEIVED	NO. OF ASSEMBLIES PER SHIPMENT	ANSI NO. INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
	None during this reporting period.		

PROCEDURE REVISIONS THAT CHANGED THE OPERATING MODE DESCRIBED IN THE FSAR

March, 1982

None during this reporting period.

DESCRIPTION OF PERIODIC TESTS WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

March, 1982

None during this reporting period.

INSERVICE INSPECTION

MARCH, 1982

UNITS ONE AND TWO

No Inservice Inspection work was conducted.

REPORTABLE OCCURENCES PERTAINING TO ANY OUTAGE OR POWER REDUCTIONS

March, 1982

None during this reporting period.

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 1

MECHANICAL MAINTENANCE

MARCH, 1982

DEPT=MECH

UNIT1-
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RFTSERVIT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTMAN
03/27/82	CC	HT	1-CC-E-1B	TUBE LEAK INDICATED BY MOPS1.1B	INSP CLEANED TUBES FOUND NO LEAKS	1	203260745	25
DEPT TOTAL								25

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 2

MECHANICAL MAINTENANCE

MARCH, 1982

UNIT2-
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTIWNTM
03/01/82	MS	FLANGE	2-MS-7	FLANGE LEAKS	REPLACED BONNET GASKET AND REPACKED	2	201310135	698
03/02/82	MS	VALVE	NRV-MS-201C	REPACK NON-RETURNED VALVE	REPACKED	2	201201040	24
03/02/82	SI	VALVE	2-SI-232	BODY TO BONNET	TIGHTENED BONNET	2	202090504	24
03/02/82	FW	VALVE	2-FW-89	REPAIR FURMANITE REPAIR	INSTALLED NEW BONNET RING GASKET	2	203011630	696
03/03/82	MS	VALVE	2-MS-378	REPACK VALVE	REPACKED VALVE	2	202021046	24
03/03/82	MS	VALVE	2-MS-168	ROOT VALVE LEAKING	TIGHTEN GLAND TIGHTEN BONNET REPPACK	2	202141836	48
03/03/82	BD	VALVE	2-BD-212	VALVE LEAKS	INSTALLED CAP	2	202160210	18
03/03/82	RC	VALVE	2-RC-11	REPACK OR ADJ AS REQ A LOOP RM	ADJUSTED PACKING	2	203030830	3
03/03/82	RC	VALVE	2-RC-56	REPACK OR ADJ B LOOP ROOM	ADJUSTD PACKING	2	203030831	2
03/03/82	RC	VALVE	2-RC-82	REPACK OR ADJ C LOOP DRAIN TH DRAIN	ADJUSTED PACKING	2	203030833	2
03/03/82	RH	VALVE	MOV-RH-2700	REPACK	REPACKED WITH GARLOCK 98	2	203030839	2
03/03/82	BD	SNUBBER	2-WGCB-HSS-003	FILL SNUBBER TO 80 PERCENT	FILLED TO 80PERC WITH 1154 FLUID	2	203031413	6
03/03/82	MS	SNUBBER	2-SHP-HSS-4B	FILL SNUBBER TO 80 PERCENT	FILLED SNUBBER TO 80 PERC WITH 1154	2	203031414	6
03/04/82	MS	VALVE	TV-MS-201A	REPACK TRIP VALVE	REPACKED VALVE	2	201201050	72
03/04/82	MS	VALVE	TV-MS-201C	STUFFING BOX GASKET LEAK	TIGHTEN DOWN ON GASKET	2	201201054	315
03/04/82	MS	VALVE	2-MS-8	BODY TO BONNET LEAK	REPLACED VALVE	2	201310130	720
03/04/82	SI	VALVE	TV-2884B	LEAKAGE REDUCTION MAINT ITEM	ADJUSTED PACKING	2	202081931	189
03/04/82	MS	VALVE	2-MS-145	VALVE LEAKS BY SEAT	LAPPED VALVE DISC AND SEAT	2	202120200	72
03/04/82	MS	VALVAE	2-LS-176	INSPECT VALVE	INSPECTED VALVE INTERNALS	2	202241851	45
03/04/82	MS	VALVAE	2-MS-182	INSPECT VALVE	INSPECTED VALVE INTERNALS	2	202241852	45
03/04/82	MS	VALVE	2-MS-178	INSPECT VALVE	INSPECTED VALVE INTERNALS	2	202241853	45
03/04/82	CH	VALVE	HCV-2310	LEAK OUT OF FURMANITE HOLE AND LEAKA	REPLACED GASKETS AND ADJUSTED PACKIN	2	203020346	50
03/04/82	IA	PIPING	2-IA-516	REPAIR BROKEN PIPE TO TV-CC-205B	REPLACED APPROX 6' PIPE AND 2 90DEG	2	203020730	49
03/04/82	RC	VALVE	HCV-2556C	REPACK OR ADJ LOOP FAIL BET PUMP	ADJUSTED PACKING	2	203030836	18
03/04/82	CH	VALVE	HCV-2311	REPACK OR ADJ	ADJUSTED PACKING	2	203030840	19
03/04/82	SI	SNUBBER	2-SI-HSS-20	REPAIR TEST AND REINSTALL	REPLACED PACKING SEAL	2	203031412	18
03/04/82	HSS	SNUBBER		SIGHT GLASS BROKEN	REPAIRED SIGHT GLASS	2	203032055	2
03/04/82	CH	VALVE	2-CH-170	VALVE HAS LEAK	PT ON UPSTREAM SIDE OF 2-CH-170	2	203040301	7
03/08/82	MS	SOV	SOV-MS-202A	AIR LEAKING OUT OF TOP HALF	PERFORMED TEMP REPAIR TO DIAPH	2	202041435	119
03/11/82	FW	PUMP	2-FW-P-3B	CASING LEAK	TIGHTENED BOLTING	2	202090531	145
03/11/82	FW	PUMP	2-FW-P-3B	NO GUARD ON COUPLING	INSTALLED COUPLING GUARD	2	202090532	145
03/12/82	MS	PIPING		LINE BETWEEN 2-MS-145 FLANGE HAS A H	WELD REPAIR	2	212020158	269
03/13/82	CH	VALVE	RV-2382	BOLT MISSING	INSTALLED STUD +2 NUTS	2	203041003	192
03/13/82	SW	PUMP	2-SW-P-10A	INSPECT PUMP IMPELLERS	REMOVED PUMP AND CHECKED	2	203041157	8
03/14/82	SW	PUMP	2-SW-P10B	INSPECT IMPELLER NUT	IMPELLER NUT TIGHT	2	203041154	3
03/21/82	CH	PIPE	1-CH-356-152	LAG PIPE TO INLET OF FCV 2113A	LAGGED PIPE TO INLET ON 2113A	2	203171353	95
03/22/82	CC	HX	1-CC-E-1D	CLEAN HEAT EXCHANGER	CLEAN HEAT EXCHANGE	2	203182000	3
03/24/82	EE	VALVE	2-EG-40	CHECK VALVE ON DISCH	REPAIRED LEAK	2	203122043	216
03/31/82	SW	PUMP	2-SW-P-10B	PACKING LEAK ON PUMP	REPACKED PUMP	2	203220307	4
UNIT TOTAL								4368

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 1

ELECTRICAL MAINTENANCE

MARCH, 1982

UNIT=ELEC

UNIT1-
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RSTS&WUIT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTLWMN
03/20/82	FW	MOTOR	1-FW-P-3A	MOTOR WAS SPRAYED WITH STEAM	PERFORMED PI CURVE	1	203200519	5
03/25/82	SW	MOV	MOV-SW-105D	ZERO GROUND	MOTOR REPAIRED	1	203251000	8
								13

DEPT TOTAL

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 2

ELECTRICAL MAINTENANCE

MARCH, 1982

DEPT=ELEC

UNIT2-
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RSTSERVIT	SYS	COMP	MARKNO	SUMMARY	WKTERF	U	WR	TOTMNTN
03/01/82	SI	VALVE	IV-SI-288AB	BIT RECIRC OUTLET ISOLATION DOES NOT	CYCLED VALVE SAT	2	203011853	3
03/02/82	CH	VALVE	HCV-2303B	VALVE INDICATES INTERMEDIATE POSITIO	CLEANED OUT LIMIT CYCLE SAT	2	201281216	26
03/02/82	RC	RTD		ERRATIC READING	CHECKED TIGHTENED CONNECTIONS	2	202130840	26
03/02/82	RC	MOTOR	2-RC-P-1A	INSPECT COOLERS	INSPECTED COOLERS	2	202221105	24
03/02/82	RC	MOTOR	2-RC-P-1B	INSPECT COOLERS	INSPECTED COOLERS	2	202221106	26
03/02/82	RC	MOTOR	2-RC-P-1C	INSPECT COOLERS	INSPECTED COOLERS	2	202221107	24
03/02/82	EP	WIRE		ASSIST OPERATIONS	ASSISTED OPS ON DC B0-S96 SAT	2	203021145	5
03/03/82	RC	MOV	RC-2535	CHECK TORQUE SWITCHES	CHECKED TORQUE SWITCHES OPERATED SAT	2	202011100	57
03/04/82	RC	VALVE	HCV-2557B	VALVE SHOWS INTERM. INDICATION	ADJ LIMITS CYCLE	2	202142259	82
03/04/82	CH	HT	PNL11CKT4C	BAD HEAT TAPE	CIRCUIT REPLACED	2	203041030	3
03/05/82	CS	MOTOR	2-CS-P-1A	BRIDGE MEGGER + RUN PI CURVE	BRIDGE AND MEGGERED OK	2	203050930	1
03/05/82	CS	MOTOR	2-CS-P-1B	BRIDGE MEGGER	BRIDGERED + MEGGERED SAT	2	203050931	1
03/05/82	FW	MOTOR	2-FW-P-3B	BRIDGE MEGGER + RUN PI CURVE	BRIDGED + MEGGERED MOTOR	2	203050956	4
03/05/82	FW	MOTOR	2-FW-P-3A	BRIDGE MEGGER + RUN PI CURVE	BRIDGE + MEGGERED MOTOR	2	203050957	4
03/07/82	RS	ALARM	LS-RS-100B	ALARM WILL NOT CLEAR TR	REPAIRED CONTACTS IN FIELD ALARM CLP	2	202252220	166
03/11/82	SS	ELEC		INSTALL RAY CHEM	WORK HAS BEEN COMPLETED	2	202271342	249
03/11/82	CH	HT	PNL11CKT6B	REPAIR CIRCUIT	CIRCUIT REPAIRED	2	203110846	6
03/12/82	CH	HT	PANEL8	LOW AMPS	REPAIRED HEAT TAPE SAT	2	203120740	5
03/12/82	EP	CARDOX	ZONE 6	COIL NEEDS REPLACING	COIL REPLACED	2	203121130	1
03/13/82	SW	MOTOR	2-SW-P-10A	DISC AND RECON FOR MECH	DISCONNECTED MOTOR	2	203130722	16
03/14/82	RI	ANNC	E-G-10	DOES NOT FLASH	ALARM REPAIRED	2	203131902	19
03/14/82	RI	ANNC	E-H-10	DOES NOT FLASH WHEN TESTED	ALARM REPAIRED	2	203131903	19
03/14/82	SW	MOTOR	2-SW-P-10B	DISC AND RECON FOR MECH	DISCONNECTED RECONNECTED MOTOR	2	203140745	6
03/20/82	SS	MOTOR		LUBE OIL CIRC PUMP MOTOR VERY HOT	MOTOR CHECKED SAT	2	203200140	1
03/20/82	MS	MOV	MOV-MS-201C	VALVE FAILED TO SHUT	REPLACED TORQUE SWITCH	2	203200700	4
03/22/82	SI	VALVE	MOV-2890A	VALVE FAILED TO OPEN ELEC	CLEANED TORQUE SW CYCLED SAT	2	203161200	29
03/22/82	EPIC	CHARGER	2B1	INVESTIGATE CAUSE OF OVERHEATING	REPLACED DIODE COMPLETE	2	203192300	49
03/23/82	EP	HORN	HAZ 2	REPLACE HORN SAFETY ITEM	REPLACED HORN AND OPERATED SAT	2	203130836	126
03/25/82	VS	VALVE	2-VS-104B	VALVE INDICATES INTERMEDIATE	READJUSTED ARM ON LIMIT SWITCH	2	203221549	2
DEPT TOTAL								984

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 1

INSTRUMENT MAINTENANCE

MARCH, 1982

DEPT=INST

UNIT1-

(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RSTSERVIT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTWMTN
03/25/92	RM	MONITOR	RM-RMS-150	ALARM SETPOINTSTHAN 7DERC BAND	RESET ALARM SETPOINT	1	203230805	1
DEPT TOTAL								1

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 2

INSTRUMENT MAINTENANCE

MARCH, 1982

DEPT=INST

UNIT2-
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RPTSERVIT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTLWMN
03/02/82	CV	INSTR	FI-150	INDICATED FLOW	PROBLEM CLEARED	2	010230295	22
03/02/82	SI	INSTR	PI-931	TANK APPEARS TO HAVE MORE PRESS	SIMULATED INPUT ADJUSTED INDICATOR	2	202102325	456
03/02/82	SI	INSTR	PI-929	CHECK CALIB	VERIFIED INDICATOR WORKS SATISFACTOR	2	202102335	456
03/02/82	SS	SWITCH		ADJUST PRESSURE SWITCH	RESET SWITCH	2	202282359	3
03/04/82	SI	INSTR	PI-927	CHANNEL DRIFTED HIGH-CHECK CALIB	REPLACED TRANSMITTER	2	202200201	217
03/04/82	RM	VALVE	HCV-2758	VALVE WILL NOT FULLY CLOSE	REPLACED RELAY GASKET UNPLUGGED E/P	2	202281400	96
03/04/82	DA	VALVE	TV-DA-203B	AIR LEAK ON VALVE	REPAIRED AIR LEAK ON INSTRUMENT LINE	2	203011825	99
03/04/82	SI	TRANSM	2-FI-2934	FLOW TRANSMITTER INDICATES 30GPM	FIXED BROKEN POINTER	2	203012310	43
03/04/82	RM	VALVE	SOV-RM-200C	INSTALL TEMPORARY AIR LINE	TEMPORARY LINE INSTALLED	2	203030841	23
03/12/82	SI	HST	TIC-2934A/2934	ALL WIRING CONNECTIONS ARE BRITTLE	NO PROBLEM	2	203110253	29
03/21/82	CH	INSTR		GAGE NEEDS REPLACED	CALIBRATED TRANSMITTER	2	203061307	126
03/21/82	SW	INSTR	PS-SW-203B	CLEAN SENSING LINE	UNCLOGGED LINE	2	203150715	126
DEPT TOTAL								1506

HEALTH PHYSICS

MARCH, 1982

There was no single release of radioactivity or radiation exposure specifically associated with an outage that accounted for more than 10% of the allowable annual values in 10CFR20.

PROCEDURE DEVIATIONS REVIEWED BY STATION NUCLEAR SAFETY AND OPERATING COMMITTEE AFTER TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MARCH, 1982

Procedure No.	Unit	Title	Date Deviated	Date SNSOC Reviewed
Nuclear Assurance Corp. Document 720, Revision 1	1	Out-of-Core Sipping Procedure	01/30/82	03/11/82