NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

UPDATE REPORT
PREVIOUS REPORT DATE 4-7-81

CONTROL BLOCK: [| | | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	I A D A C 1 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)	(5)
	LICENSEE CODE	LICENSE NUMBER	LICENSE TYPE	CAT

CON'T

0 1	REPORT SOURCE: L (6)	0 5 0 0 0 3 3 1 (7)	0 3 2 4 8 1 (8)	0 4 2 3 8 2 (9)
		DOCKET NUMBER	EVENT DATE	REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During scheduled leak rate testing MSIVs 4412, 4413, 4416, 4419, 4420, a
0 3 nd 4421 were found to have seat leakage on the inboard and combined seat
0 4 and stem leakage on the outboard valves in excess of the 11.5 SCFH limit
0 5 specified in T.S. 4.7.A.2.c.3. In addition, the total allowable leakage
0 6 for Type B and C tests specified in T.S. 4.7.A.2.c.2 was exceeded. Detai
0 7 led leakage rate test results in Unique Report 81-04. Reactor was shutdo
0 8 wn for refueling. Two prev. similar occurrences. (R.O. 80-04 and 76-17).

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	C D (11)	E (12)	B (13)	V A L V E X (14)	E (15)	X (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 1	—	0 1 3	/	0 1	T	—	1

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
B (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	N (25)	R 3 4 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Leakage attributed to normal wear. All valves were repaired as necessary
1 1 . All were lapped and repacked. Four stems were replaced, three discs we
1 2 re replaced and two seats were machined. Valves are Rockwell Manufacturi
1 3 ng 20 inch pneumatically operated MSIVs rated at 1250 PSI and 575 degree
1 4 s. Valves were all retested and found satisfactory.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	4 (28)	0 0 0 (29)	NA	B (31)	Surveillance Test

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

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DUANE ARNOLD ENERGY CENTER

Iowa Electric Light and Power Company

Licensee Event Report - Supplemental Data

Docket No. 050-0331

Licensee Event Update Report Date: 4-23-82

Reportable Occurrence No: 81-013 Update

Event Description:

During the performance of scheduled leak rate testing, inboard MSIV's 4412 and 4420 were found to have seat leakage, and outboard MSIV's 4413, 4419, 4416 and 4421 were found to have combined seat and packing leakage in excess of 11.5 SCFH limit specified in Technical Specification 4.7.A.2.c.3. In addition, because of outboard MSIV packing leakage and other type C valve leakage, the total allowable leakage for type B and C leak rate tests specified in Technical Specifications Section 4.7.A.2.c.2 was exceeded. The results of the leakage testing were detailed in Unique Report 81-04 submitted in accordance with the requirements of Technical Specification 4.7.A.2.f. There have been two previous similar occurrences. (R.O. 80-04 and 76-17).

Cause Description:

The leakage was attributed to normal wear.

Corrective Action:

Valves are Rockwell Manufacturing 20 inch pneumatically operated MSIVs rated at 1250 PSI and 575 degrees. All valves were repaired as necessary. Seat faces were machined on MSIVs 4412 and 4413. Discs were replaced in MSIVs 4413, 4419, and 4420. Stems were replaced in MSIVs 4416, 4419, 4420 and 4421. All six valves were repacked and all seats were lapped. All six valves were retested and found satisfactory.