

MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

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April 9, 1982

NUCLEAR PRODUCTION DEPARTMENT

U. S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Washington, D. C. 20555

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station

Units 1 and 2

Docket Nos. 50-416 and 50-417

File: 0260/0840

RHR Containment Spray; SER Confirmatory

Issue, Item 1.10(15)

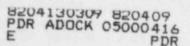
AECM-82/134

The Grand Gulf Safety Evaluation Report (SER), NUREG-0831, identified NRC concerns in the human factors area, related to the initiation logic for containment spray [SER Confirmatory Issue 1.10(15)]. Mississippi Power & Light (MP&L) advised the NRC in letter AECM-81/410, dated October 23, 1981, that the necessary design changes would be made to alleviate the need for sustained operator action to manually initiate spray train "B".

Due to subsequent evaluations associated with affecting the design change, MP&L has elected to revise the design as discussed in AECM-81/410; namely, the subject 90 second time delay between train "A" initiation and manual train "B" initiation will be deleted. This revised design configuration provides the operator greater availability of train "B" in the unlikely event of train "A" failure. The time delay will be retained for automatic initiation of train "B".

It is MP&L's position that the elimination of the time delay in the manual initiation mode is superior from human factor considerations in that the operator's action to initiate train "B" work can be immediately confirmed by observing the appropriate parameters.

A subsequent engineering evaluation was performed regarding rapid depressurization effects on the containment resulting from simultaneous spray train "A" and train "B" initiation and determined that the maximum containment depressurization event is within the allowable stress limits of the containment. This conclusion is consistent with the discussion provided in FSAR subsection 6.2.1.1.4.2.



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In conclusion, MP&L contends that the design modification, as described here, is the most appropriate approach to the NRC staff concerns from both human factor and containment/plant design standpoints. Implementation of the necessary design changes has been completed. In addition, FSAR subsection 7.3.1.1.4.4 and the response to NRC Question 31.58 will be incorporated into FSAR amendment 55 to reflect the revised design configuration.

If you have any questions or require further information, please contact this office.

Yours truly,

L. F. Dale

Manager of Nuclear Services

JTB/JGC/JDR:rg

cc: Mr. N. L. Stampley

Mr. R. B. McGehee

Mr. T. B. Conner

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