



Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402

April 8, 1982
LIC-82-154

Mr. Robert A. Clark, Chief
U. S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Division of Licensing
Operating Reactors Branch No. 3
Washington, D.C. 20555



Reference: Docket No. 50-285

Dear Mr. Clark:

Item II.K.2.17 of NUREG-0737 requires the Omaha Public Power District to analyze the potential for voiding in the reactor coolant system (RCS) during anticipated transients at the Fort Calhoun Station. The District participated in the Combustion Engineering Owners Group (CEOG) program for responding to this NUREG-0737 item and the attached CEOG report entitled "Effects of Vessel Head Voiding During Transients and Accidents in CE-NSSS's" provides the District's response. This report addresses the potential for void formation during various types of plant transients and concludes that any potential RCS void formation would not impair reactor coolant circulation or core coolability.

Please note that Section 2.4 of the subject CEOG report addresses Chapter 15 of the plant's Safety Analysis Report when discussing various transients due to plant accidents. Chapter 14 of the Fort Calhoun Station's Safety Analysis Report provides the applicable accident analyses. The discrepancy arises since this report is generic in nature and is applicable to many Combustion Engineering nuclear power plants.

Sincerely,

W. C. Jones
Division Manager
Production Operations

Attachment

cc: LeBoeuf, Lamb, Leiby & MacRae
1333 New Hampshire Avenue, N.W.
Washington, D.C. 20036

Handwritten initials/signature: A046 S1/1