TO:

Ronald C. Haynes
Administrator of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

FROM:

G P U Nuclear
Oyster Creek Nuclear Generating Station
Docket No. 50-219
Forked River, New Jersey 08731

SUBJECT:

Reportable Occurrence Report No.50-219/82-20/01P

The following is a preliminary report being submitted in compliance with the Technical Specifications, paragraph 6.9.2.a. (3).

REPORT DATE:

April 2, 1982

Preliminary Approval:

Peter B. Fiedler
Vice-President - Director Oyster Creek

Director (2)
Office of Management Information and
Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

NRC Resident Inspector (1)
Oyster Creek Nuclear Generating Station
Forked River, New Jersey 08731

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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Licensee Event Report
Reportable Occurrence No. 50-219/82-20/01P

Report Date

April 2, 1982

Occurrence Date

April 1, 1982

Identification of Occurrence

It was identified that an abnormal degradation of the primary containment existed based on the results of leak rate testing performed on the Main Steam Isolation Valves.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.a.3.

Conditions Prior to Occurrence

The reactor was in the cold shutdown condition at the time the occurrence was identified.

Description of Occurrence

On February 8, 1982, while performing local leak rate tests on Main Steam Isolation Valves, the leak rate for valve NS03A was found to be outside of the acceptable limit. On March 28, 1982, the other Main Steam Isolation Valve in the "A" line, NS04A, was leak tested and the results of this test were also unacceptable due to a packing leak in NS04A. In the "A" line piping configuration, a packing leak in the NS04A and excessive leakage through NS03A provides an abnormal flow path from the reactor vessel to the secondary containment.

Apparent Cause of Occurrence

The apparent cause of the occurrence is under investigation.

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Analysis of Occurrence

At the present time it is not known if this abnormal leakage path existed during power operation. A more complete analysis will be made as the investigation of this event continues.

Corrective Action

Valve NS03A was disassembled and repaired, and it passed the subsequent leak rate test. The valve packing on valve NS04A was tightened and the valve was re-tested, and it subsequently passed.