

# NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

#### CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68 License No. DPR-53

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated January 22, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

 Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:

#### (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief Operating Reactors Branch #3 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: March 9, 1982

#### ATTACHMENT TO LICENSE AMENDMENT NO. 68

#### FACILITY OPERATING LICENSE NO. DPR-53

#### DOCKET NO. 50-317

Replace the following pages of the Appendix A and B Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are provided to maintain document completness.

#### Appendix A

Page

6-2

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Appendix B

Figure 5:.2-1

#### 6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

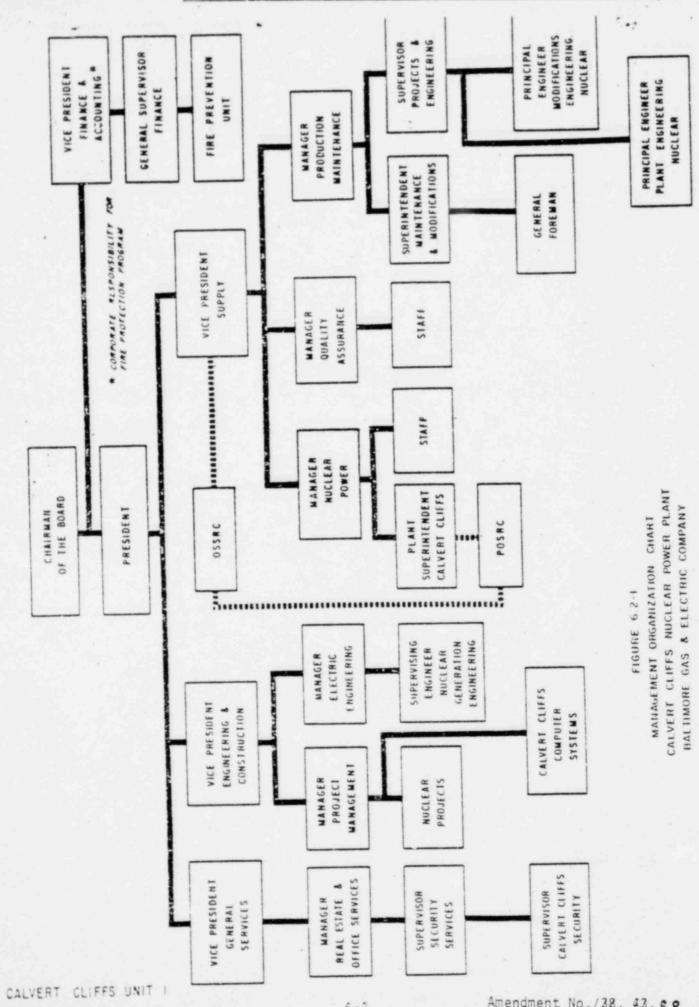
#### 6.2 ORGANIZATION

#### OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

#### FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:
  - a. Each on duty shif: shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
  - b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
  - c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
  - d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
  - e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
  - f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.



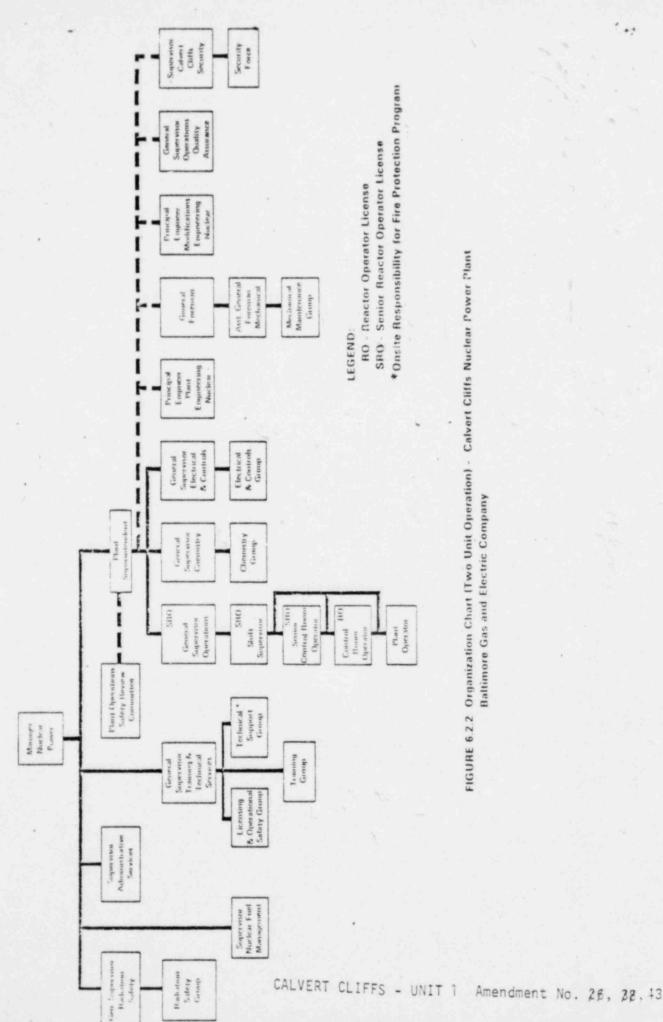


TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION #

### Condition of Unit 1 - Unit 2 in MODES 1, 2, 3 or 4

LICENSE	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

## Condition of Unit 1 - Unit 2 in MODES 5 or 6

LICENSE	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SDL**	2	11*
OL**	3	2
Non-Licensed	3	3 .
Shift Technical Advisor	1##	0

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

## ANNUAL REPORTS 1/

- 6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.
- 6.9.1.5 Reports required on an annual basis shall include:
  - a. A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
  - b. The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).
  - c. Documentation of all failures and challenges to the pressurizer PORVs or safety valves.

A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

<sup>2/</sup> This tabulation supplements the requirements of \$20.407 of 10 CFR Part 20.

#### MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Inspection and Enforcement, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, ATTN: Document Control Desk, with a copy to the Regional Administrator and to the NRC Resident Inspector, no later than the 15th of each month following the calendar month covered by the report.

#### REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9 below, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

#### PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

- 6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within 14 days. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.
  - Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.
  - b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
  - c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

FIGURE 5.2-1

MANAGEMENT ORGANIZATION CHART RELATING TO ENVIRONMENTAL MATTERS

CALVERT CLIFFS NUCLEAR POWER PLANT BALTIMORE GAS & ELECTRIC COMPANY



# \*UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### BALTIMORE GAS AND ELECTRIC COMPANY

#### DOCKET NO. 50-318

#### CALVERT CLIFFS NUCLEAR POWER PLANT UNIT NO. 2

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50 License No. DPR 69

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Baltimore Gas & Electric Company (the licensee) dated January 22, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-69 is hereby amended to read as follows:
  - 2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 50 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief Operating Reactors Branch #3

Division of Licensing

Attachment: Changes to the Technical Spesifications

Date of Issuance: March 9, 1982

#### ATTACHMENT TO LICENSE AMENDMENT NO. 50

#### FACILITY OPERATING LICENSE NO. DPR- 69

#### DOCKET NO. 50-318

Replace the following pages of the Appendix A and B Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are provided to maintain document completness.

#### Appendix A

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#### Appendix B

Figure 5.2-1

#### 6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

#### 6.2 ORGANIZATION

#### OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

#### FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:
  - a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
  - b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
  - c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
  - d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
  - e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
  - f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of both units (4 members) or any personnel required for other essential functions during a fire emergency.

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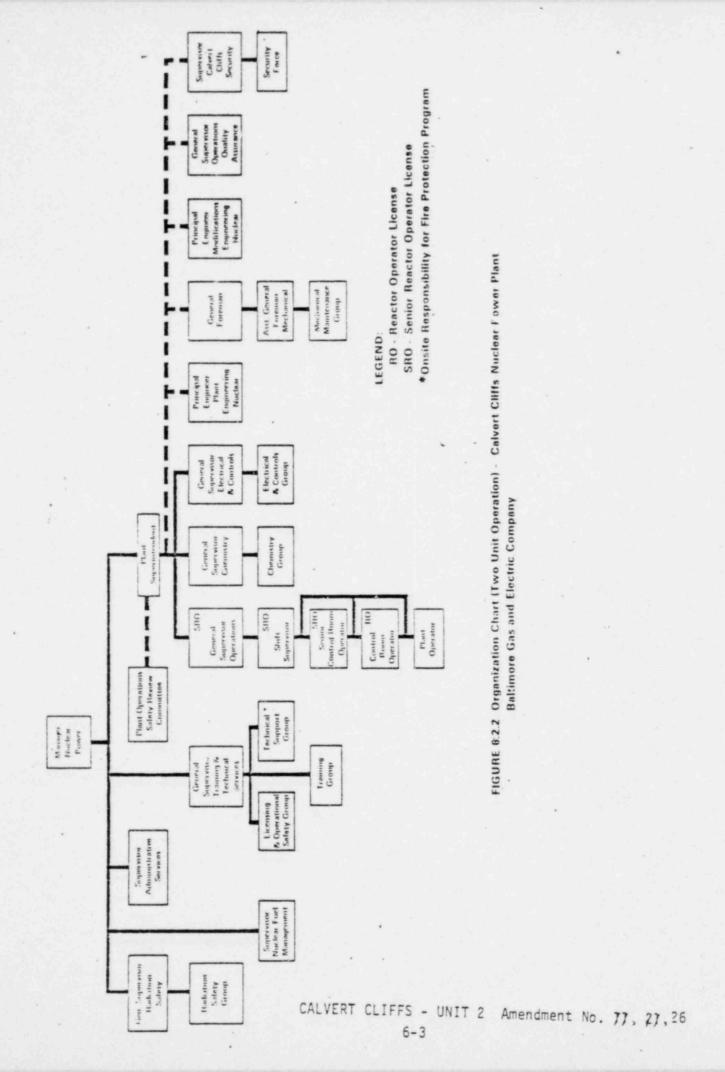


TABLE 6.2-1 MINIMUM SHIFT CREW COMPOSITION #

## Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

LICENSE	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL**	3	3
Non-Licensed	3	3
Shift Technical Advisor	1##	1##

# Condition of Unit 2 - Unit 1 in MODES 5 or 6

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL**	3	2
Non-Licensed	3	3
Shift Technical Advisor	1##	0

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

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  - a. A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, 27 e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
  - The complete results of steam generator tube inservice inspections performed during the report period (reference Specification 4.4.5.5.b).
  - Documentation of all failures and challenges to the pressurizer PORVs or safety valves.

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<sup>2/</sup> This tabulation supplements the requirements of \$20.407 of 10 CFR Part 20.

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- 6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within 14 days. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.
  - a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.
  - b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
  - c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

FIGURE 5.2 - 1 MANAGEMENT GREANIZATION CHART RELATING TO NVIRONMENTAL MATTERS CALVERT CLIFFS NUCLEAR POWER PLANT BALTIMORE GAS & ELECTRIC COMPANY