LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 81074 R1 Technical Specification Involve	2.2.B
Reported Under Technical Specification 6.7.2.a(1) *	Date Due NRC_N/A
Date of Occurrence 12/30/81 Time of Occurrence 0800	Unit 3

IDENTIFICATION AND DESCRIPTION OF OCCURRENCE: Through correspondence we have received data indicating that eight of eleven 2-stage Target Rock relief valves tested at Wyle Laboratories for as-found set pressure initially failed to lift within the allowable range. Three valves (Serial Nos. 1019, 1061 and 1065) were tested satisfactorily. Tabulated below are valve serial numbers, set pressures, as-found set pressures and percent deviation for the first lift.

Valve Serial Number	Set Pressure	As Found	Percent Deviation
1018	1115	1127	1.1
1030	1115	1143	2.5
1069	1105	1117	1.1
1024	1125	1140	1.3
1063	1105	1118	1.2
1061	1115	1112	0.269
1023	1105	*1325	19.909
1026	1125	1147	1.955
1059	1105	1118	1.176
1019	1125	1121	0.356
1065	1115	1113	0.179

*off chart - estimated value

The total average deviation is 2.82% for all MSRVs tested. Of the eight valves that . failed, six met the criteria on subsequent lifts. Previous report included the results of six valves that were tested.

CONDITIONS PRIOR TO OCCURRENCE: Unit 1 at 97%; Unit 2 at 98%; Unit 3 in refueling outage.

ACTION SPECIFIED IN THE TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENTS MET DUE TO INOPERABLE EQUIPMENT. DESCRIBE. None

APPARENT CAUSE OF OCCURENCE: As stated in previous LERs (BFRO-50-259/81025, 260/80040, 296/80054), the cause is being evaluated by the manufacturer.

OTHER RELATED EVENTS: Reference: BFRO-50-259/81025, 260/80040, 296/80054. The evaluations performed in the referenced LERs remain valid in that this problem does not result in an overpressure condition of reactor vessel piping nor does it result in any appreciable increase in MCPR operating limit. Previous calculations performed by the General Electric Company indicated that for the previous unit 3 operating cycle, a total average deviation in relief valve setpoint of 5.0% could have existed with no significant effect on nuclear safety.

ANALYSIS OF OCCURRENCE: There was no danger to the health or safety of the public, no damage to the plant or equipment, and no significant chain of events.

CORRECTIVE ACTION TAKEN OR PLANNED: The eight valves that failed will be repaired, reset, and retested prior to installation.

Retention: Period - Lifetime; Responsibility - Document Control Supervisor