NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | V A S P S 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T 01 | REPORT SOURCE L (6) | 0 5 0 0 0 2 8 0 (7) | 0 1 3 1 8 2 (8) | 0 3 0 1 8 2 (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With unit no. 1 at 100% power, PT-27 revealed that amp readings for heat tracing
03 panels 8 & 9, circuits 24A & C, were below the acceptance criteria stipulated.
04 This event is contrary to T.S.3.2.B.5 and is reportable per T.S.6.6.2.b.(2).
05 Flow was verified by PT-18.4 and adjacent circuits were operable. Therefore, the
06 health and safety of the public were not affected.
07
08

09	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	S H (11)	E (12)	A (13)	H E A T E R (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	0 1 7	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
C (18)	F (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	A (25)	T 1 8 5 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The loss of heat tracing was due to excessive heat. The defective heat tracing was
11 replaced and tested. A design change has been initiated to change the manner by
12 which these borated lines are heat traced.
13
14

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
15	E (28)	1 0 0 (29)	N/A	B (31)	Routine Test

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z (33)	Z (34)	N/A	N/A

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
17	0 0 0 (37)	Z (38)	N/A

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
18	0 0 0 (40)	N/A

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
19	Z (42)	N/A

PUBLICITY

	ISSUED	DESCRIPTION		NRC USE ONLY
20	N (44)		N/A	

8203180318 820301
PDR ADOCK 05000280
S PDR

NAME OF PREPARER J. L. PHONE (804) 357-3184

ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 1
DOCKET NO: 50-280
REPORT NO: 82-017/03L-0
EVENT DATE: 01-31-82

TITLE OF THE EVENT: HEAT TRACING FAILURE

1. DESCRIPTION OF EVENT:

With unit no. 1 at 100% power, performance of PT-27 revealed that the amp readings for heat tracing panels 8 and 9, circuit 24A (Unit #1 Filter 1-CH-FL-1) and Panel 9, circuit 24C (Flow Transmitter FT-1110) were below the acceptance criteria stipulated in the PT. This event is contrary to Technical Specification 3.2.B.5 and is reportable per Technical Specification 6.6.2.b(2).

2. PROBABLE CONSEQUENCES AND STATUS OF REDUNDANT EQUIPMENT:

The heat tracing circuits are intended to maintain a fluid temperature above that needed for flow. Performance of PT 18.4 verified flow through the unit 1 filter. The redundant heat tracing circuits on the pipes adjacent to the flow transmitter were operable. Therefore, the health and safety of the public were not affected.

3. CAUSE:

The loss of heat tracing was due to excessive heat.

4. IMMEDIATE CORRECTIVE ACTION:

The immediate corrective action was to verify flow by performance of PT-18.4 and to verify that adjacent circuits were operable.

5. SUBSEQUENT CORRECTIVE ACTION:

The defective heat tracing tape was replaced and tested within the time span specified by Technical Specifications.

6. ACTION TAKEN TO PREVENT RECURRENCE:

No additional actions were deemed necessary.

7. GENERIC IMPLICATIONS:

A task force has reviewed the total spectrum of the Heat Tracing System and a Design Change has been prepared as a result of the Task Force Study. Installation of this design change has commenced.