

NOTICE OF DEVIATION

Consumers Power Company
Palisades Nuclear Plant

Docket No. 50-255
License No. DPR-20

During an NRC inspection conducted on April 6 through May 4, 1993, a deviation of your written response to a safety evaluation report (SER) dated February 28, 1986, regarding the single failure issue for main steam isolation valves was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action", 10 CFR Part 2, Appendix C, the deviation is listed below:

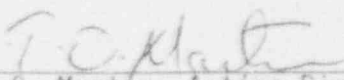
In an April 28, 1986 letter to the NRC, the licensee committed, in part, to perform certain actions for a main steam line break coincident with failure of the intact steam generator's main steam isolation valve to close. These specific actions include opening the power-operated relief valves, maximizing safety injection flow (feed and bleed), and maximizing auxiliary feed flow to one steam generator if the decision is made not to rely on any in-containment instrumentation.

In the basis document for emergency operating procedures (EOP) 9.0, "Functional Recovery Procedure," and EOP 6.0, "Excessive Steam Demand Event," the licensee made the conservative assumption that no in-containment instrumentation would remain operable.

Contrary to the above, as of May 4, 1993, the emergency operating procedures do not decisively instruct the operators to perform the above actions. Instead, EOP 9.0 ambiguously directs the operators to monitor in-containment parameters, which may be inoperable or unreliable, to determine the proper mitigating procedure.

Please provide to the U. S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D. C. 20555, with a copy to the Regional Administrator, Region III, in writing within 30 days of the date of this Notice, the reason for the deviation, the corrective steps which have been taken and the results achieved, the corrective steps which will be taken to avoid further deviations, and the date when your corrective action will be completed. Where good cause is shown, consideration will be given to extending the response time.

Dated at Glen Ellyn, Illinois
this 24th day of May, 1993


T. O. Martin, Acting Director
Division of Reactor Safety