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402/636-2000

May 14, 1993
LIC-93-0142

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

REFERENCE: Docket No. 50-285

Gentlemen:

SUBJECT: April 1993 Monthly Operating Report (MOR)

Enclosed is the April 1993 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification Section 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/mle

Enclosures

c: LeBoeuf, Lamb, Leiby & MacRae
J. L. Milhoan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

April 1993
Monthly Operating Report

1. OPERATIONS SUMMARY

Fort Calhoun Station (FCS) operated at approximately 77% power until the unit was taken off-line on April 24, 1993 at 0244 hours for a scheduled one-week maintenance outage. The purpose of the outage was to improve plant reliability and thermal performance for the peak summer season. During the outage, the plant was maintained in hot shutdown.

On April 24 and 25, work on 161KV Circuit 1587 (components located in the switchyard) was conducted. After work on Circuit 1587 was completed, 161KV power was supplied to FCS through Transformer T1A4 while Transformer T1A3 was out-of-service. The oil in Transformer T1A3 was changed and Transformer T1A3 and its fire protection deluge piping were tested. On April 29, Transformer T1A3 was returned to service and the 161KV system was restored to its normal lineup.

During the outage, the condensers were cleaned, six containment area radiation monitors were replaced, three safety injection tanks were drained and refilled (to increase the boron concentration), and a turbine electrohydraulic control system circuit card was replaced.

FCS returned to reactor critical status at 2025 hours on April 30, 1993 and the turbine-generator was synchronized to the grid at 0429 hours on May 1, 1993.

The following NRC inspection was completed during this reporting period:

IER No. Description

93-04 Residents' Routine Inspection

No LERs were submitted during this reporting period.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

None

3. RESULTS OF LEAK RATE TESTS

In early April, possibly due to the repacking of Charging Pump CH-1A, the reactor coolant system (RCS) leak rate increased to approximately 0.300 gpm over several days before returning to a nominal rate between 0.100 gpm and 0.200 gpm. The leak rate stabilized at the nominal rate through the remainder of April 1993.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
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None

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF APRIL 1993

- Adjusted lift on Raw Water Pumps AC-10A, AC-10B and AC-10C
- Replaced Raw Water Pump AC-10D
- Installed new packing and adjusting rings, plungers, front cap gaskets and steam chest "O" rings on Charging Pump CH-1A
- Replaced Schrader valve assembly on Suction Accumulator (H-26B for Charging Pump CH-1B)
- Repaired the fresh air supply damper alarm circuit (YCV-871G) for Diesel Generator No. 1
- Replaced the 52/HH switch for Breaker 1A4-12, (feeder for Raw Water Pump AC-10D)
- Repaired interlocks on containment Personnel Air Lock AE-2

Outage Activities

- Replaced the 52/HH switch for Breaker 1A3-1, the 161KV normal feed to Bus 1A3
- Repaired a loose terminal and calibrated reactor coolant Temperature Indicator D/TI-112H
- Replaced the control switch for Boric Acid Storage Pumps CH-4A and CH-4B
- Installed 28 new EGS Corporation power supplies (120 VAC to 52.5 VDC)
- Replaced 13 General Electric CR120A relays
- Performed circuit analysis testing on the control element drive mechanism (CEDM) clutch coils
- Checked the power supply output voltage and the AC ripple for the 48 trip units and 12 matrix power supplies on Reactor Protective System Power Supply Assembly AI-31A-AW8
- Sampled Containment Air Cool/Filter Unit "A" Carbon Filter to verify filter efficiency
- Repaired the control switch on outlet isolation Valve HCV-258 for Boric Acid Storage Tank CH-11B
- Repaired noise problem on wide range channel "D" (NT-004)

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

Monthly Operating Report
April 1993
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8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	MAY 04, 1993
COMPLETED BY	M. L. EDWARDS
TELEPHONE	(402) 636-2451

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: APRIL 1993
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

NOTES

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reason for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	719.0	2879.0	171793.0
12. Number of Hours Reactor was Critical	558.5	2718.5	133328.8
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	554.7	2714.7	131777.9
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	633532.0	3754275.9	173319751.9
17. Gross Elec. Energy Generated (MWH)..	209656.0	1265788.0	57106588.2
18. Net Elec. Energy Generated (MWH)....	199384.6	1207172.5	54478059.4
19. Unit Service Factor.....	77.1	94.3	76.7
20. Unit Availability Factor.....	77.1	94.3	76.7
21. Unit Capacity Factor (using MDC Net)	58.0	87.7	68.9
22. Unit Capacity Factor (using DER Net)	58.0	87.7	67.1
23. Unit Forced Outage Rate.....	.0	.0	4.3

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
REFUELING OUTAGE SCHEDULED TO BEGIN ON SEPTEMBER 18, 1993 AND LAST APPROXIMATELY 56 DAYS.

25. If shut down at end of report period, estimated date of startup: 05/01/93

26. Units in test status (prior to comm. oper.):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	N/A	_____
COMMERCIAL OPERATION	_____	_____

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	MAY 04, 1993
COMPLETED BY	M. L. EDWARDS
TELEPHONE	(402) 636-2451

MONTH APRIL 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>360</u>	17	<u>364</u>
2	<u>361</u>	18	<u>364</u>
3	<u>360</u>	19	<u>362</u>
4	<u>360</u>	20	<u>363</u>
5	<u>361</u>	21	<u>363</u>
6	<u>362</u>	22	<u>363</u>
7	<u>362</u>	23	<u>349</u>
8	<u>363</u>	24	<u>0</u>
9	<u>363</u>	25	<u>0</u>
10	<u>362</u>	26	<u>0</u>
11	<u>362</u>	27	<u>0</u>
12	<u>363</u>	28	<u>0</u>
13	<u>364</u>	29	<u>0</u>
14	<u>364</u>	30	<u>0</u>
15	<u>364</u>	31	<u>N/A</u>
16	<u>364</u>		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun
 DATE May 10, 1993
 COMPLETED BY M. L. Edwards
 TELEPHONE (402) 636-2451

REPORT MONTH April 1993

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ²	Cause & Corrective Action to Prevent Recurrence
2	04/24/93	S	164.3	H	1	N/A	N/A	N/A	On April 24, Fort Calhoun Station began a one-week maintenance outage to improve plant reliability and thermal performance for the peak summer season.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending April 1993

1. Scheduled date for next refueling shutdown. September 1993
2. Scheduled date for restart following refueling. November 1993
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?
Incorporate specific requirements resulting from reload safety analysis.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. N/A
 - c. If no such review has taken place, when is it scheduled? N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. June 1993
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. None Planned
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 529 Assemblies
 - c) spent fuel pool storage capacity 729 Assemblies
 - d) planned spent fuel pool storage capacity Planned to be increased with high density spent fuel racks.
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1995*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by *Jim Halter* Date 5-4-93