

NUREG/CR-4816
ORNL/TM-10328

PR-EDB: Power Reactor Embrittlement Data Base, Version 1

Program Description

Prepared by F. W. Stallmann, F. B. K. Kam, B. J. Taylor

Oak Ridge National Laboratory

Prepared for
U.S. Nuclear Regulatory Commission

9006290154 900630
PDR NUREG
CR-4816 R PDR

AVAILABILITY NOTICE

Availability of Reference Materials Cited in NRC Publications

Most documents cited in NRC publications will be available from one of the following sources:

1. The NRC Public Document Room, 2120 L Street, NW, Lower Level, Washington, DC 20555
2. The Superintendent of Documents, U.S. Government Printing Office, P.O. Box 37082, Washington, DC 20013-7082
3. The National Technical Information Service, Springfield, VA 22161

Although the listing that follows represents the majority of documents cited in NRC publications, it is not intended to be exhaustive.

Referenced documents available for inspection and copying for a fee from the NRC Public Document Room include NRC correspondence and internal NRC memoranda; NRC Office of Inspection and Enforcement bulletins, circulars, information notices, inspection and investigation notices; Licensee Event Reports; vendor reports and correspondence; Commission papers; and applicant and licensee documents and correspondence.

The following documents in the NUREG series are available for purchase from the GPO Sales Program: formal NRC staff and contractor reports, NRC-sponsored conference proceedings, and NRC booklets and brochures. Also available are Regulatory Guides, NRC regulations in the Code of Federal Regulations, and Nuclear Regulatory Commission Issuances.

Documents available from the National Technical Information Service include NUREG series reports and technical reports prepared by other federal agencies and reports prepared by the Atomic Energy Commission, forerunner agency to the Nuclear Regulatory Commission.

Documents available from public and special technical libraries include all open literature items, such as books, journal and periodical articles, and transactions. *Federal Register* notices, federal and state legislation, and congressional reports can usually be obtained from these libraries.

Documents such as theses, dissertations, foreign reports and translations, and non-NRC conference proceedings are available for purchase from the organization sponsoring the publication cited.

Single copies of NRC draft reports are available free, to the extent of supply, upon written request to the Office of Information Resources Management, Distribution Section, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

Copies of industry codes and standards used in a substantive manner in the NRC regulatory process are maintained at the NRC Library, 7920 Norfolk Avenue, Bethesda, Maryland, and are available there for reference use by the public. Codes and standards are usually copyrighted and may be purchased from the originating organization or, if they are American National Standards, from the American National Standards Institute, 1430 Broadway, New York, NY 10018.

DISCLAIMER NOTICE

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, or any of their employees, makes any warranty, expressed or implied, or assumes any legal liability of responsibility for any third party's use, or the results of such use, of any information, apparatus, product or process disclosed in this report, or represents that its use by such third party would not infringe privately owned rights.

NUREG/CR-4816
ORNL/TM-10328
RL, R5

PR-EDB: Power Reactor Embrittlement Data Base, Version 1

Program Description

Manuscript Completed: October 1989
Date Published: June 1990

Prepared by
F. W. Stallmann, F. B. K. Kam, B. J. Taylor

Oak Ridge National Laboratory
Operated by Martin Marietta Energy Systems, Inc.

Oak Ridge National Laboratory
Oak Ridge, TN 37831

Prepared for
Division of Engineering
Office of Nuclear Regulatory Research
U.S. Nuclear Regulatory Commission
Washington, DC 20555
NRC FIN B0415
Under Contract No. DE-AC05-84OR21400

ABSTRACT

Data concerning radiation embrittlement of pressure vessel steels in commercial power reactors have been collected from available surveillance reports. The purpose of this NRC-sponsored program is to provide the technical bases for voluntary consensus standards, regulatory guides, standard review plans, and codes. The data can also be used for the exploration and verification of embrittlement prediction models. The data files are given in dBASE III Plus format and can be accessed with any personal computer using the DOS operating system. Menu-driven software is provided for easy access to the data including curve fitting and plotting facilities. This software has drastically reduced the time and effort for data processing and evaluation compared to previous data bases.

The current compilation of the Power Reactor Embrittlement Data Base (PR-EDB, version 1) contains results from surveillance capsule reports of 78 reactors with 381 data points from 110 different irradiated base materials (plates and forgings) and 161 data points from 79 different welds. Results from heat-affected-zone materials are also listed. Electric Power Research Institute (EPRI), reactor vendors, and utilities are in the process of providing back-up quality assurance checks of the PR-EDB and will be supplementing the data base with additional data and documentation.

CONTENTS

	<u>Page</u>
ABSTRACT	iii
LIST OF FIGURES	vii
LIST OF TABLES.....	viii
PREFACE	xi
ACKNOWLEDGMENTS	xiii
1. INTRODUCTION	1
2. ARCHITECTURE	2
2.1 INTRODUCTION	2
2.2 KEY IDENTIFIERS	6
2.3 ORGANIZATION OF THE PR-EDB RAW DATA FILES	8
2.4 LIST OF PR-EDB FILES	10
APPENDIX A. PRELIMINARY SOFTWARE AND PROCESSING	55
A.1 INTRODUCTION	55
A.2 EDB-UTILITIES SOFTWARE PACKAGE	55
A.3 FILE MANIPULATION PROCEDURES	57
A.3.1 General Considerations	57
A.3.2 Retrieval of Files for Manipulation	59
A.3.3 Addition or Deletion of Fields	59
A.3.4 Calculations	59
A.3.5 Addition and Deletions of Records	59
A.3.6 Reordering	60
A.3.7 Display and Export	60
A.3.8 Save Working File	60
A.4 PLOTTING PROGRAM	61
A.5 CHARPY FITTING AND PLOTTING	63
A.5.1 General Considerations	63
A.5.2 Single-Curve Fitting and Plotting	64
A.5.3 Multiple-Curve Fitting and Plotting	64
A.5.4 Monte Carlo Uncertainty Analysis	64

CONTENTS
(Continued)

	<u>Page</u>
A.5.5 Extracting Selected Charpy Sets	66
A.5.6 Selection of Input Files and Data Sets	66
A.6 EXAMPLES	67
A.6.1 File-Manipulation Procedures with Plots	67
A.6.2 Charpy Fitting and Plotting	75
A.7 INSTALLATION	85

LIST OF FIGURES

	<u>Page</u>
1. Data flow in the PR-EDB	5
2. Architecture of the Power Reactor Embrittlement Data Base PR-EDB	9
A.1. Major options selected from first menu	56
A.2. EDB-Utilities file-manipulation procedures	58
A.3. Flowchart for EDB-Utilities plotting program	62
A.4. Procedures for fitting and plotting raw Charpy data	63
A.5. Graph of transition temperature shift versus upper shelf drop from example, Sect. A.6.1	75
A.6. Charpy fit for Zion Unit 2 weld material - baseline	77
A.7. Charpy fit for Zion Unit 2 weld material - Capsule T. Fit has not converged due to an outlier	78
A.8. Charpy fit for Zion Unit 2 weld material - Capsule U	78
A.9. Data from Fig. A.7 after the outlier at the top of the graph is eliminated	79
A.10. Same data as in Fig. A.7	83
A.11. Combination graph of Zion Unit 2 weld material, baseline plus Capsules T and U	84

LIST OF TABLES

	<u>Page</u>
1. Units used in PR-EDB files	3
2. Structure file for SPEC_LST.dbf	10
3. Structure file for SHFT_PR.dbf	11
4. Structure file for RAW_C_PR.dbf	12
5. Structure file for CV_RF_PR.dbf	13
6. Structure file for TEN_PR.dbf	13
7. Structure file for REAC_LST.dbf	14
8. Structure file for REAC_PR.dbf	15
9. Structure file for HEAT_LST.dbf	16
10. Structure file for CHEM_PR.dbf	17
11. Structure file for HEAT_PR.dbf	18
12. Structure file for WELD_PR.dbf	20
13. Structure file for HAZ_PR.dbf	21
14. Structure file for REF_TITL.dbf	21
15. Partial listing of SPEC_LST.dbf	24
16. Partial listing of SHFT_PR.dbf	25
17. Partial listing of RAW_C_PR.dbf	26
18. Partial listing of CV_RF_PR.dbf	27
19. Partial listing of TEN_PR.dbf	28
20. Listing of REAC_LST.dbf	29
21. Partial listing of REAC_PR.dbf	31
22. Listing of HEAT_LST.dbf	32

LIST OF TABLES
(Continued)

	<u>Page</u>
23. Partial listing of CHEM_PR.dbf	39
24. Partial listing of HEAT_PR.dbf	40
25. Partial listing of WELD_PR.dbf	41
26. Partial listing of HAZ_PR.dbf	42
27. Listing of REF_TITL.dbf	43
28. Partial listing of REF_LST.dbf	54
A.1. Transition temperature shifts versus upper shelf drop from example, Sect. A.6.1	70
A.2. File ZN2.dat obtained from RAW_C_PR.dat	76
A.3. Key to the information tag INFO in the screen output for the Monte Carlo uncertainty analysis	80
A.4. Sample screen output from the Monte Carlo uncertainty analysis - start	80
A.5. Sample screen output from the Monte Carlo uncertainty analysis - continuation	81
A.6. Sample output of the summary file	81
A.7. Sample output of the covariance file	82
A.8. Sample output of the EDB-dBASE file	83

PREFACE

This current compilation of the Power Reactor Embrittlement Data Base (PR-EDB, version 1) will be updated periodically and released to authorized users.

ACKNOWLEDGMENTS

The authors gratefully acknowledge the essential contributions of C. A. Baldwin who wrote the first version of the EDB software; R. B. Miller for providing its linkage to the fitting and plotting programs; T. J. Griesbach of EPRI and its subcontractors for their help in providing additional quality assurance for the EDB input data; Professor John Helm of Columbia University and Roger Stoller of ORNL for their valuable suggestions in reviewing this report; and C. Z. Serpan, Jr., Al Taboada, and N. Randall of the U.S. Nuclear Regulatory Commission for their financial support and guidance.

1. INTRODUCTION

Regulatory and research evaluations of embrittlement prediction models and of vessel integrity under load can be greatly expedited by the use of a well-designed, computerized embrittlement data base. The Power Reactor Embrittlement Data Base (PR-EDB) is a comprehensive collection of data from surveillance reports and other published reports of commercial nuclear reactors. The uses of the data base require that as many different data as available are collected from as many sources as possible with complete references and that subsets of relevant data can be easily retrieved and processed.

The objectives of this NRC-sponsored program are as follows:

1. to compile and to verify the quality of the PR-EDB;
2. to provide user-friendly software to access and process the data;
3. to explore or confirm embrittlement prediction models; and
4. to interact with standards organizations to provide the technical bases for voluntary consensus standards that can be used in regulatory guides, standard review plans, and codes.

To achieve these goals, the data base architecture was designed after much discussion and planning with prospective users. The PR-EDB is designed for use with any personal computer using any DOS system above Version 2.0. Updates will be issued periodically to authorized users. The data files in the PR-EDB are in dBASE format and can be accessed with any version of dBASE or compatible software, such as Clipper. A customized software package (EDB-Utilities), which handles most routine tasks, has been distributed with the data base and will be periodically updated. This report assumes that the reader has some familiarity with the use of personal computers, preferably with DOS and dBASE (III or IV).

The current compilation of the PR-EDB (Version 1) contains results from surveillance capsule reports of 78 reactors with 381 data points for 110 different irradiated base materials (plates and forgings) and 161 data points for 79 different welds. Results from heat-affected-zone materials are also listed. The menu-driven software, EDB-Utilities, facilitates maintenance, processing, and evaluations of the data. The time and effort required to process and evaluate different types of data in the PR-EDB have been drastically reduced from previous data bases.

Electric Power Research Institute (EPRI), reactor vendors, and utilities are in the process of providing back-up quality assurance checks of PR-EDB and will be supplementing the data base with additional data and documentation. Also EPRI, the Boiling Water Reactor Owner's Group (BWR-OG) Supplemental Material Surveillance Committee, and General Electric have agreed to release boiling-water-reactor data for insertion into the PR-EDB. This effort and coordination with the NRC research staff, EPRI, and industry have led to the adoption of PR-EDB as the basis for an industry-wide data base by the EPRI Reactor Vessel Embrittlement Management Project.

2. ARCHITECTURE

2.1 INTRODUCTION

The information in most data bases is contained in one or more "data files," with the data arranged in each file in the form of a table. Each row of such a table is called a "record," and each unit of information is stored in its assigned "data field" that corresponds to a particular column in the table. Older data bases, such as the ones published by MPC, consist of just one table, which means that all different data fields are contained in each record. In creating such a table, a decision must be made which types of data fields are to be included before data collection starts, and the data base must be redesigned whenever new types of data are added. Since every record contains all possible data fields, there are many duplications and a large number of empty fields. It is also fairly difficult to accommodate in such a table, multiple determinations of the same quantity such as chemistry and to provide complete references if the data in the same record come from different sources.

To avoid the problems of a single table data base, the PR-EDB is designed as a collection of many different data files, each of which closely resembles the data tables found in the original surveillance reports. For instance, most reports have tables containing transition temperatures and upper shelf energies for Charpy specimen before and after irradiation and/or the shift in these values during irradiation, and these data are collected in the file "SHF1_PR". Data are collected as reported (i.e., there are fields for unirradiated, irradiated, and shift values, depending on what is reported; fields are added for units because some reports use U.S. units, others, the new international units, and still others, the older European engineering units (see Table 1). Similarly, information about tensile tests is collected in the file TEN_PR, information about the irradiation, such as capsule fluence and temperature, in the file REAC_PR, and so on. A complete list of the data files in the PR-EDB is given in Sect. 2.4. This approach has many significant advantages:

1. The structure of the data files need not be predetermined; the data files are designed according to what is available in the original reports, and new data files can be added without disturbing the existing ones.

2. Because every record in a data file originates from a single report and, in most cases, from a single table in this report, a unique reference, including page number(s), can be given for each record.
3. Multiple determinations of the same quantity are given in different records, each with its proper references. Such multiple determinations occur, for instance, if the chemistry is determined by the manufacturer of the material as well as from broken specimens. Also, fluence determinations are frequently updated in subsequent reports using improved neutron physics calculations. All different determinations are kept in the PR-EDB, and it is up to the user to decide which determination to use for a particular application or, perhaps, calculate averages from several of them.

Table 1. Units used in PR-EDB files

Description	Symbol used in PR-EDB	Type of unit	Conversion factors			
			to U.S. unit		to Int'l unit	
Mil	MIL	Length	0.001	inch	0.00254	cm
mm	MM	Length	0.03937	inch	0.1	cm
cm	CM	Length	0.3937	inch	1.0	cm
Inch	IN.	Length	1.0	inch	2.54	cm
Fahrenheit	F	Temperature	F		(F-32)/1.8	= C
Centigrade	C	Temperature	1.8 C + 32	= F		C
Foot-pounds	FT-LB	Energy	1.0	ft-lb	1.3558	joules
Joules	J	Energy	0.7376	ft-lb	1.0	joules
Kilogram-meters	KGM	Energy	7.2330	ft-lb	9.8066	joules
mkp*	MKP	Energy	7.2330	ft-lb	9.8066	joules
mkp.cm ² **	MK/C2	Energy	5.78	ft-lb	7.92	joules
Kilogram/mm ²	K/MM	Stress	1.4223	ksi	9.8066	MPa***
Kip/inch ²	KSI	Stress	1.0	ksi	6.895	MPa***
Pound/inch ²	PSI	Stress	0.001	ksi	6.895	kPa***
Mega pascal	MPA	Stress	0.145	ksi	1.0	MPa***

*Same as kilogram-meters.

**Energy relative to the cross section of the Charpy minus notch $\approx 0.8 \text{ cm}^2$
(reported for the Beznau reactors).

***kPa = kilo pascal = 10^3 pascal, MPa = mega pascal = 10^6 pascal.

Because the data in the PR-EDB are distributed over many different files, means must be provided for combining data from several files. This is done with the techniques of "relational data bases," specifically dBASE by Ashton Tate. The format for data and auxiliary files and the techniques introduced by dBASE are now widely used in data base applications. The choice of the dBASE format for the PR-EDB allows the use of any version of the dBASE software as well as any of the other software packages which use the dBASE format, such as Clipper. The linkage between different data files in a relational data base is provided by "key identifiers" which are common to these files. For instance, all files with data concerning a specific material such as results of material property tests, material manufacture, heat treatment, and chemistry contain a field for the material identifier HEAT_ID. Similarly, files with data concerning irradiations contain fields for the identifiers of the reactor, PLANT_ID, and the surveillance capsule, CAPSULE. A detailed description of the key identifiers is given in Sect. 2.2. Extreme care must be exercised to assign the correct identifier to each record since otherwise connection between data from different records cannot be made or are made incorrectly. To assure correct identifications, numerous cross checks are made which have the additional advantage that many mistakes are caught in this manner that have eluded conventional proofreading.

The best designed data base is useless without convenient software to extract data for any given application. The dBASE-compatible software provides the necessary tools but requires some expertise from the user for successful application. For this reason, EDB-Utilities has been written to perform many processing steps such as retrieval and selection of data, calculations, and display to the screen or printer. (Output to a plotter is being considered for later release.) This software is menu-driven so that it can be used without extensive training. A description of the EDB-Utilities is given in Appendix A.

An overview of the data flow in the PR-EDB is given in Fig. 1. The source data are first transcribed to "Raw Data Files" in dBASE format as faithfully as possible with complete references. Data entry is currently done through the keyboard. Direct transfer will be used whenever computer-readable documents are available. Any deviation from the norm, which was either reported or noted during transcription (such as the correction of obvious typographical errors) is indicated in the NOTES field. Data from every available report are included, except when the information in a later report is simply a duplication of earlier data without any changes. The dBASE format distinguishes between "character," "numerical," and "date" fields, but all data in the PR-EDB are entered as characters. This somewhat complicates the numerical manipulation of data that have to be converted to numerical form first, but gives much greater freedom to data entry. Information such as <0.001 for an upper bound or RT for room temperature can be copied from the original report without change. Missing information is always indicated by a blank in the data field, which is not

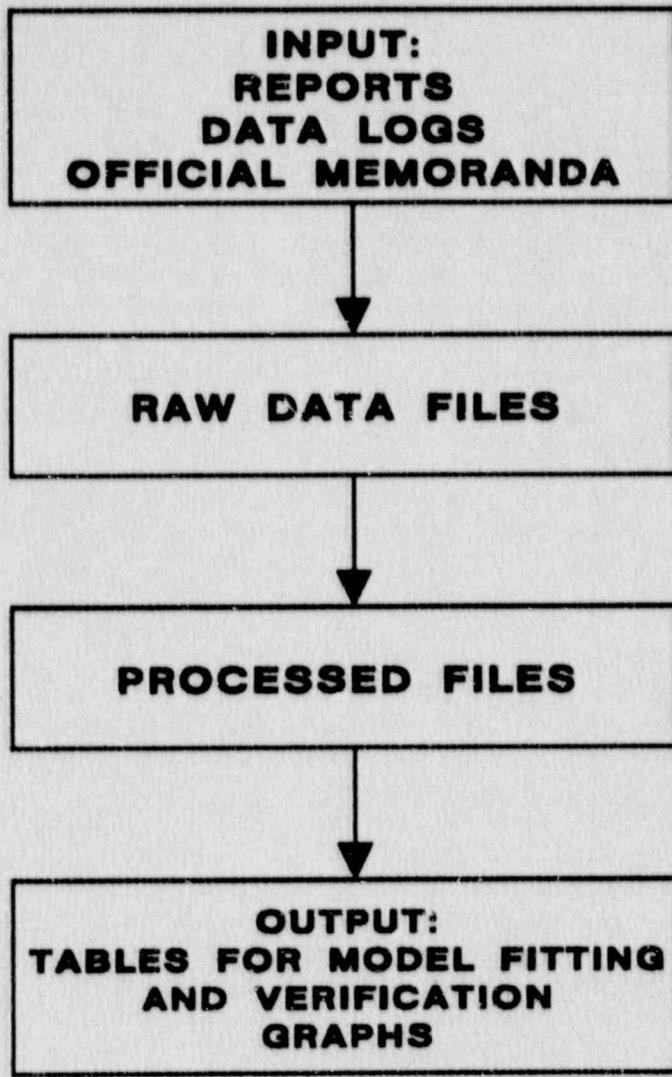


Fig. 1. Data flow in the PR-EDB.

possible in numerical or date formats. The Raw Data Files are next converted to processed files by the EDB-Utilities or other user-supplied software. Processed files will typically be in dBASE format, but ASCII-coded files can also be obtained from dBASE-compatible software for input into scientific software (e.g., numerical analysis programs written in FORTRAN), word processors, and spreadsheets. With the additional software, tables and graphs can be created for the purpose of model fitting, model verification, and other applications.

2.2 KEY IDENTIFIERS

One or more fields in each data file are occupied by "key identifiers," which provide the means for combining data from several files through "relations" that link the corresponding records in these files. These fields differ from other data fields in that the key identifiers are assigned by the manager of the data base in a manner that the same unique identifiers are used to label the data among the various data files that refer to the same experiments, materials, or source documents. The identifiers used in the reports do not always provide such unambiguous labelling. The following key identifiers are used in the current version of the PR-EDB:

1. PLANT_ID

Up to six characters are used to identify the reactor in which the irradiation was performed. Only up to three characters are used at present for the identification of commercial power reactors. A list of these identifiers with full name and other information about the reactor is given in the file REAC_LST.dbf (p. 20). The reactor code in PLANT_ID is also used to identify the surveillance program as a source of data, even if these data do not refer specifically to irradiation (as in files about chemistry or heat treatment; for details see Sect. 2.4).

2. CAPSULE

Up to six characters are used to identify the surveillance capsule. These are mostly identical to the identifications used in the surveillance program. Sometimes specimens are lumped together which come from different capsules with similar fluences, and in these cases special identifiers for data sets from combined capsules are assigned.

3. HEAT_ID

The material identifier can have up to ten characters. A simple scheme was devised which works as follows: The coding used for the PR-EDB assigns the first letter to the material type, namely P_late, F_orging, W_eeldment, H_eat-affected-zone material, or S_tandard reference material. The next 3 + 2 letters contain the reactor identifier (first three letters of PLANT_ID) plus some identification number, if more than one material is used in the same reactor (e.g., 01, 02, etc; if practical, some correspondence between the EDB and report identifications is retained, e.g., PCC101, PCC102, and PCC103 stand for D7206-1, D7206-2, and D7206-3, respectively). For standard reference materials that are not restricted to a particular reactor, other identification letters

are used, namely, SASTM for the A302B ASTM reference plate and SHSS02 for the HSST plate 02. The last four letters are reserved to distinguish between different parts of the same material (e.g., between surface and 1/2T, or between several sections of a plate) if different parts show markedly different material properties as documented in the reports. For instance, SASTM_S1 and SASTM_S2, denote pieces of the 6-in. ASTM A302B reference plate used in the Garigliano and Yankee Rowe reactors, respectively, whose baseline properties vary considerably from the same material used in Westinghouse reactors, which is denoted simply by SASTM. The file HEAT_LST.dbf (see p. 25) gives a complete list of identifiers used in HEAT_ID together with the corresponding identifiers in the surveillance reports.

It was first considered to use the identifiers given in the reports for HEAT_ID, a practice used in the MPC Data base. This, however, has proven to be impractical. Base material (plates or forgings) is usually characterized by the heat number (assigned to the ingot) or a manufacturer's number (assigned to the plate or forging after fabrication; see, for instance, BAW-1820) and one or the other, or neither, is used in reports, frequently with different choices of identification in different reports. For welds, a weld code such as SA-1585 or WF-232 is sometimes used, but the same code may be applied to different welds of the same type (BAW-1820). Alternatively, the wire heat number has been used for identification or just the heat numbers of the two plates joined by the weld. A distinct identification is rarely given for the heat-affected-zone material.

4. SPEC_ORI

Different orientations of the material test specimens may lead to substantially different property test results, thus this identifier is needed to correctly link the properties of irradiated specimens to the corresponding baseline values.

Orientations are assigned in the now customary T-L-S system as described in ASTM Standard E399*, with L the primary rolling or forging direction, or, for welds and HAZ, the direction of the weld seam; T is perpendicular to L and parallel to the plate surface, and S is perpendicular to the plate surface. The first letter describes the longitudinal direction of the specimen (perpendicular to the crack surface, if any), and the second letter describes the direction of the crack propagation (perpendicular to the notch). The orientation for each specimen set was determined as well as possible, preferably from

*Standard E399 applies formally only to CT specimens, but the extension to other types of specimens is straightforward (also see BAW-1820).

drawings, making sure that the same orientations are assigned to corresponding specimen sets. SPEC_ORI is left blank if no information is available.

5. REF_ID

In most files, each record is assigned a reference which indicates the source of the data. This is done by means of a 20-character field REF_ID. This identifier is usually a report number or a similar code which links it uniquely to the complete bibliographic information, i.e., author, title, and time of publication, which is given in the file REF_TITL.dbf (p. 43).

2.3 ORGANIZATION OF THE PR-EDB RAW DATA FILES

The current version of the PR-EDB is organized in the manner shown in Fig. 2. At the center is the file SPEC_LST.dbf (Table 2) containing a complete list of test specimen sets which are used in surveillance programs of commercial power reactors. Specimen sets are characterized by type and size of the specimen, given in SPEC_TYPE and SPEC_SIZE, and the four key identifiers PLANT_ID, CAPSULE, HEAT_ID, and SPEC_ORI, which link the sets to the other data files. Specimen sets for testing of the baseline properties of unirradiated materials are characterized by leaving the CAPSULE field blank. The data which are relevant for radiation embrittlement are distributed over three sets of data files. The first set, on the left of Fig. 2, consists of results of material property tests. Charpy, both individual tests and results of curve fittings, and tensile data are currently available (SHFT_PR.dbf, RAW_C_PR.dbf, and CV_RF_PR.dbf, for Charpy data and TEN_PR.dbf for tensile, Tables 4 through 11). Other test results, such as CT, WOL, hardness, and drop weight will be included later. The second set, at the bottom of Fig. 2, contains data describing the reactor and radiation environment for each surveillance capsule. Currently available is the file REAC_PR.dbf containing the fluence, irradiation temperature and irradiation time and the file REAC_LST.dbf, which is a list of power reactors as mentioned above. Under consideration is the addition of more detailed files containing the irradiation history, the group fluence spectra, and dosimetry to allow for fluence determination by independent investigators. Finally, the set on the right of Fig. 2 contains the information about the chemistry and fabrication of the materials used in the surveillance programs (HEAT_LST.dbf, CHEM_PR.dbf, HEAT_PR.dbf, WELD_PR.dbf, and HAZ_PR.dbf, discussed in pages 16-21).

The PR-EDB files with the suffix _LST are somewhat different from the other data files in that they provide a sort of directory of the other files and their relations to the key identifiers. SPEC_LST.dbf is a directory of surveillance capsules and baseline specimen sets. REAC_LST.dbf is a

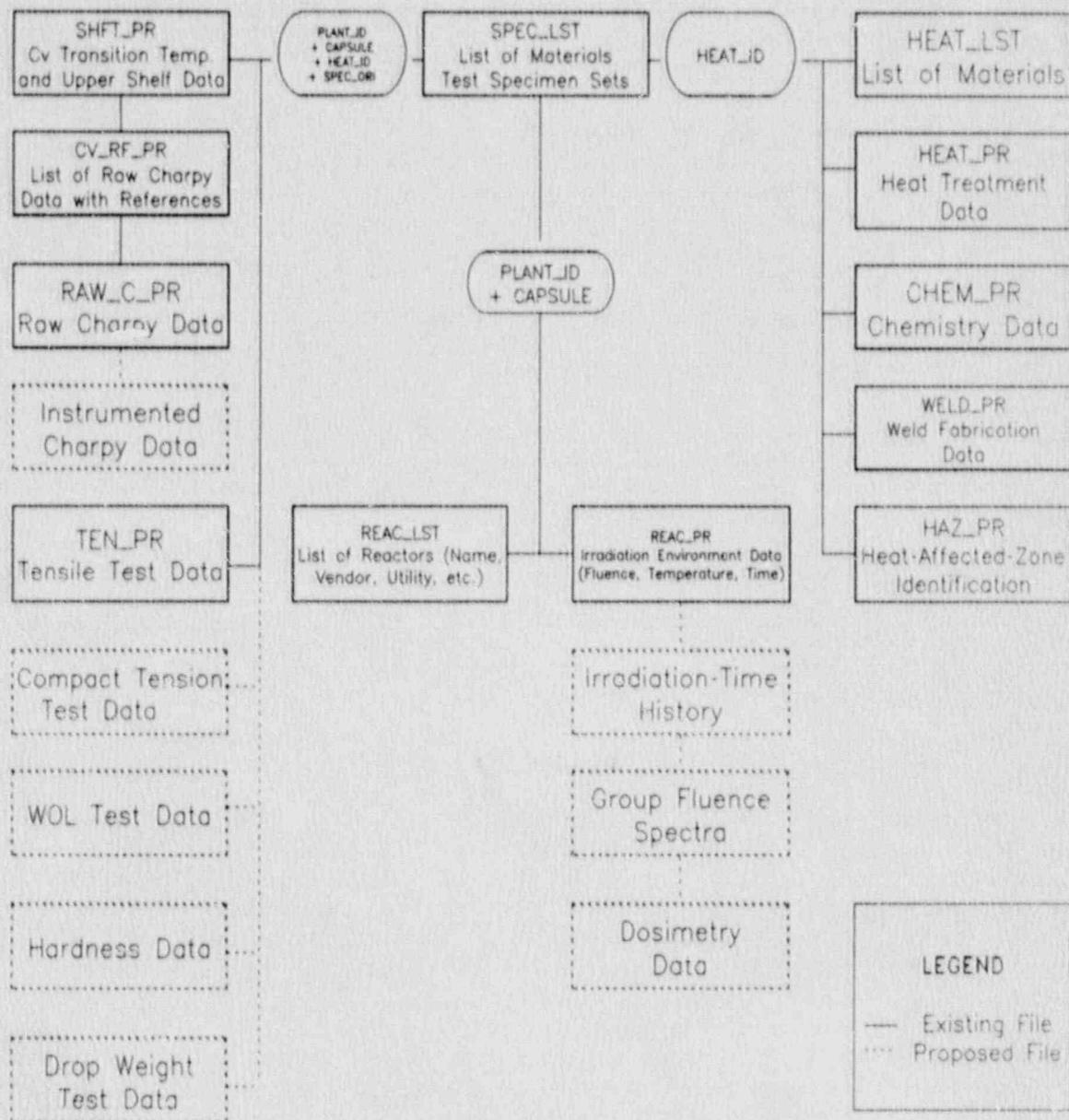


Fig. 2. Architecture of the Power Reactor Embrittlement Data Base PR-EDB.

directory of power reactors. And finally, HEAT_LST.dbf is a directory of the materials contained in the PR-EDB files. A more detailed description of these data files is given below.

2.4 LIST OF PR-EDB FILES

1. File SPEC_LST.dbf

Table 2. Structure file for SPEC_LST.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
CAPSULE	Surveillance Capsule or Experiment Identification
HEAT_ID	Identification Code for Given Material
SPEC_TYPE	Type of Specimen: Charpy (CV), TEN_size, C_ompact T_enison, WOL
SPEC_SIZE	Size of the Specimen
SPEC_ORI	Specimen Orientation: TL, LT, TS, etc.
SPEC_POS	Specimen Position: 1/4T, 1/2T, 3/4T, etc.
NO_OF_SPEC	Number of Specimens in Capsule or Experimental Set
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

The key identifiers PLANT_ID + CAPSULE + HEAT_ID + SPEC_ORI link the specimen sets to the corresponding records in the other data files. PLANT_ID + CAPSULE identify a particular experiment (i.e., the surveillance capsule in a given reactor, and HEAT_ID + SPEC_ORI a particular material). SPEC_TYPE indicates the type of specimen; its size, if nonstandard, is listed in SPEC_SIZE. SPEC_POS indicates the layer(s) relative to the surface from which the specimens were cut. (This is 1/4T in most cases; 1/4T + 3/4T means that the set consists of a mixture of specimens cut from the 1/4T and 3/4T layers, 1/4T - 3/4T means the specimens are cut from the whole range between 1/4T and 3/4T.) Also listed is the number of specimens in each set, if available.

A partial listing of the records in SPEC_LST.dbf is given at the end of the section (p. 24).

2. File SHFT_PR.dbf

SHFT_PR.dbf (Table 3) lists transition temperatures and upper shelf energies as determined by the evaluator of the report. It lists 30 ft-lb, 50 ft-lb, and 35-mil transition temperatures (irradiated and unirradiated), and shift (difference between the two), whatever is listed in the report, and similarly for the upper shelf energy with both absolute and relative shift values. Included are also fluences ($E > 1.0 \text{ MeV}$), taking into account the differences within the capsule between different specimen sets. Not included is the transition temperature at 50% shear since it is seldom reported and is difficult to be determined reliably. It can also be readily reconstructed from the individual Charpy test data. Also not included are transition temperatures at other energy levels or lateral expansion such as 71 ft-lb or 54 mil, which are sometimes listed in older reports. Shift values listed represent the actual differences between unirradiated and irradiated data; additional safety margins that are sometimes included in reports have been subtracted. Shift values are sometimes determined in separate graphs independent from the determination

Table 3. Structure file for SHFT_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
CAPSULE	Surveillance Capsule or Experiment Identification
HEAT_ID	Identification Code for Given Material
PROD_ID	Material Type: P_late, F_forging, W_eld, HAZ, or SRM
SPEC_ORI	Specimen Orientation: TL, LT, TS, etc.
CSP_F1	Fluence $> 1 \text{ MeV}$ at Charpy Specimen Location [n/cm^2]
UTT30	CVT at 30 ft-lb, Unirradiated Charpy Specimen
UTT50	CVT at 50 ft-lb, Unirradiated Charpy Specimen
ULE35	CVT at Lateral Expansion = 35 mils, Unirrad. Charpy Spec.
UUSE	Upper Shelf Energy, Unirradiated Charpy Specimen
ITT30	CVT at 30 ft-lb, Irradiated Charpy Specimen
ITT50	CVT at 50 ft-lb, Irradiated Charpy Specimen
ILE35	CVT at Lateral Expansion = 35 mils, Irrad. Charpy Spec.
IUSE	Upper Shelf Energy, Irradiated Charpy Specimen
DTT30	CVT Shift at 30 ft-lb
DTT50	CVT Shift at 50 ft-lb
DLE35	CVT Shift at Lateral Expansion = 35 mils
DUSE_ABS	Absolute Drop in Upper Shelf Energy
DUSE_REL	Percent Drop in Upper Shelf Energy
TEMP_U	Unit used for Temperature Data
USE_U	Unit used for Energy Data (in Upper Shelf Energy)
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

of unirradiated and irradiated test data and may, for this reason, be inconsistent with the latter. This is indicated in the notes. Fields for temperature and energy units have been added to allow entering of data in different units. (See sample output at the end of section.)

3. Files RAW_C_PR.dbf and CV_RF_PR.dbf

RAW_C_PR.dbf (Table 4) gives a complete list of individual Charpy test results (see p. 26) at the end of the section for a partial listing. The same key identifiers are used as in SHFT_PR.dbf and fluence and irradiation temperature are included which may be different for each specimen. Each set is listed only once, even if it appears in more than one report since corresponding data originate from the same Charpy test. An exception is the sets in which specimens from more than one capsule are combined; the same specimen may be listed under a single capsule and in a combination set, such as, under the capsules 1A and 1B and again under the combination A/B in the LaCrosse reactor. Fields for measuring units are again included to allow entering of data in different units. No references are listed in this file to save space since each specimen in the same set has the same reference. Instead, a third file, CV_RF_PR.dbf, is added to this set which contains only the references which are uniquely assigned to the data in RAW_C_PR.dbf by means of the key identifiers. The structure of this file is listed below (Table 5) and a partial listing of the records is given at the end of the section (p. 27).

Table 4. Structure file for RAW_C_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
CAPSULE	Surveillance Capsule or Experiment Identification
HEAT_ID	Identification Code for Given Material
PROD_ID	Material Type: P_late, F_orgeing, W_eld, HAZ, or SRM
SPEC_ORI	Specimen Orientation: TL, LT, TS, etc.
SPEC_ID	Specimen Identifier
TST_TEMP	Test Temperature of Specimen
TST_TEMP_U	Unit of Temperature used in TST_TEMP
IMP_E	Impact Energy of Charpy Specimen
IMP_E_U	Unit of Energy used in IMP_E
FRACT_APP	Fracture Appearance Value [% shear]
LAT_EXP	Lateral Expansion of Charpy Specimen
LAT_EXP_U	Unit of Length used in LAT_EXP
CSP_F1	Fluence > 1 MeV at Charpy Specimen Location [n/cm ²]
CSP_TEMP	Irradiation Temperature of Charpy Specimen
CSP_TEMP_U	Unit of Temperature used in CSP_TEMP

Table 5. Structure file for CV_RF_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
CAPSULE	Surveillance Capsule or Experiment Identification
HEAT_ID	Identification Code for Given Material
PROD_ID	Material Type: P_late, F_orging, W_eld, HAZ, or SRM
SPEC_ORI	Specimen Orientation: TL, LT, TS, etc.
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

4. File TEN_PR.dbf

The file TEN_PR.dbf (Table 6) lists the results of tensile tests with separate entries for each individual test. Averages from several experiments are included if no other information is available, but are omitted if individual test data are given. Such cases are indicated in NOTES.

Table 6. Structure file for TEN_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
CAPSULE	Surveillance Capsule or Experiment Identification
HEAT_ID	Identification Code for Given Material
PROD_ID	Material Type: P_late, F_orging, W_eld, HAZ, or SRM
SPEC_ORI	Specimen Orientation: TL, LT, TS, etc.
TSP_F1	Fluence > 1 MeV at Tensile Specimen Location [n/cm ²]
TSP_TEMP	Irradiation Temperature of Tensile Specimen
TSP_TEMP_U	Unit of Temperature used for TSP_TEMP
SPEC_ID	Specimen Identifier
TST_TEMP	Test Temperature of Specimen
TST_TEMP_U	Unit of Temperature used for TST_TEMP
YSL	Lower Yield Strength of Tensile Specimen
YSL_U	Unit of Stress used in YSL
YSU	Upper Yield Strength of Tensile Specimen
YSU_U	Unit of Stress used in YSU
UTS	Ultimate Tensile Strength
UTS_U	Unit of Stress used in UTS
ULG	Ultimate Elongation of Tensile Specimen [%]
TLG	Total Elongation of Tensile Specimen [%]
RA	Reduction in Area of Tensile Specimen [%]
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

Fluences and irradiation temperatures are included and may differ from specimen to specimen as in RAW_C_PR.dbf. Units from the original reports are used as specified in the unit fields. (See p. 28 at the end of the section for a partial listing of the records in TEN_PR.dbf.)

5. File REAC_LST.dbf

For each reactor code given in PLANT_ID, the file REAC_LST.dbf (Table 7) lists the full name of the reactor, the utility, the vendor, the pressure vessel manufacturer, and the architect/engineer of the plant. (A complete listing is given on p. 29 at the end of the section.) No references are included since these data come from many different sources and are easily verifiable.

Table 7. Structure file for REAC_LST.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
REAC_TYPE	Reactor Type: PWR, BWR, or TR
REAC_NAME	Reactor Name
LOCATION	Reactor Location
PLANT_OP	Reactor Operator or Utility
PLANT_DES	Reactor Designer or Vendor
ARCH_ENG	Reactor Architect/Engineer
VESSEL_MFG	Reactor Vessel Manufacturer

6. File REAC_PR.dbf

The file REAC_PR.dbf (Table 8) lists fluences in the form of $E > 1.0$ MeV, $E > 0.1$ MeV, dpa, as well as fluence rates, irradiation temperatures, startup and removal dates, and the full-power-equivalent irradiation times. Additional information for subintervals of the irradiation time is provided under the heading of CONFIG(uration), if the radiation environment has changed drastically during the course of irradiation. Methods for the determination of irradiation temperatures and fluence uncertainties are also listed, if available. (A partial listing is given on p. 31 at the end of the section.)

Table 8. Structure file for REAC_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
CAPSULE	Surveillance Capsule or Experiment Identification
START_DATE	Date at Start of Irradiation [MM/DD/YYYY]
STOP_DATE	Date at End of Irradiation [MM/DD/YYYY]
CONFIG	Indicator for Change in Irradiation Environment
EFP_TIME	Effective Full Power Time of Irradiation
EFP_T_U	Unit of Time used for EFP_TIME
CAP_T_MIN	Minimum Irradiation Temperature at Capsule Center
CAP_T_MAX	Maximum Irradiation Temperature at Capsule Center
CAP_T_U	Unit of Temperature used for CAP_T_MAX and CAP_T_MIN
TEMP_TAG	Temperature Measured by: M_elt Wire, T_hermocouples, N_ominal
CAP_F1	Fluence > 1 MeV at Capsule Center [n/cm ²]
F1_UNC	Uncertainty of Fluence > 1.0 MeV [% Standard Deviation]
F1_RATE	Fluence Rate > 1 MeV at Capsule Center [n/(cm ² ·s)]
CAP_FP1	Fluence > 0.1 MeV at Capsule Center [n/cm ²]
FP1_UNC	Uncertainty of Fluence > 0.1 MeV [% Standard Deviation]
CAP_DPA	Displacements per Atom of Iron at Capsule Center
DPA_UNC	Uncertainty of Displacements per Atom [% Standard Deviation]
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

7. File HEAT_LST.dbf

The file HEAT_LST.dbf (Table 9) relates the material codes given in HEAT_ID to the descriptions and heat numbers given in the surveillance reports and in the MPC data base. It also includes the ASTM material classification, the supplier of the material, and the thickness. SOURCE gives specific information about the origin of the surveillance material (if available) namely SCR-ap, if it was obtained from excess material during the fabrication of the vessel (this applies also to welds), CUTOUT for nozzle cutouts, and SIM-ulated welds, if the material was not obtained from an actual weld seam but fabricated from excess plate material using the same filler and flux. Finally, FABR-icated means, that the material was fabricated separately, as it is done for correlation materials. (A complete listing is given on p. 32 at the end of the section.)

Table 9. Structure file for HEAT_LST.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
HEAT_ID	Identification Code for Given Material
RPT_ID	Identifier Used in Surveillance Reports
HEAT_NO	Heat Number of Material Used by Supplier
PROD_ID	Material Type: P_late, F_orging, W_eld, HAZ, or SRM
MAT_ID	Material Classification: A302B, A5082, A533B1, etc.
SUPPLIER	Supplier of Material
THICKNESS	Thickness of Base Material [inches]
SOURCE	Source of Material: FABR-icated, SCR-ap, SIM-ulated Weld, CUTOUT
MPC_ID	Reference Number Assigned by Combustion Engineering (MPC)
MPC_HEAT	Material Heat Number (MPC)
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

8. File CHEM_PR.dbf

The file CHEM_PR.dbf (Table 10) lists the chemistries for the given materials together with information about the laboratory and method used, if given, and whether it is derived from test specimens or represents

Table 10. Structure file for CHEM_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
HEAT_ID	Identification Code for Given Material
PROD_ID	Material Type: P_late, F_orging, W_eld, HAZ, or SRM
MAT_ID	Material Classification: A302B, A5082, A533B1, etc.
CHEM_LAB	Chemistry Laboratory or Procedure Identification
METHOD	Method for Determining the Chemistry
SPEC_ID	Specimen Identifier
C	Weight Percent Carbon
MN	Weight Percent Manganese
P	Weight Percent Phosphorus
S	Weight Percent Sulfur
SI	Weight Percent Silicon
NI	Weight Percent Nickel
CR	Weight Percent Chromium
MO	Weight Percent Molybdenum
CU	Weight Percent Copper
V	Weight Percent Vanadium
B	Weight Percent Boron
CS	Weight Percent Cesium
TI	Weight Percent Titanium
CO	Weight Percent Cobalt
N	Weight Percent Nitrogen
O	Weight Percent Oxygen
SB	Weight Percent Antimony
AS	Weight Percent Arsenic
ZR	Weight Percent Zirconium
AL	Weight Percent Aluminum
AL_SOL	Weight Percent Aluminum in Solution
AL_INT	Weight Percent Aluminum Interstitial
PB	Weight Percent Lead
W	Weight Percent Tungsten
SN	Weight Percent Tin
ZN	Weight Percent Zinc
TA	Weight Percent Tantalum
H	Weight Percent Hydrogen
NB	Weight Percent Niobium
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

generic values given by the supplier of the material. Generic values are identified in SPEC_ID as LADLE, CHECK, or just HEAT, depending on what is revealed in the reports. The term WIRE is used if the chemistry of the filler wire rather than that of the actual weld material has been reported. Other terms listed in SPEC_ID are the identifiers of the test specimen whose chemistry was determined. Values are listed as reported without regard to consistency, but obvious duplications are sometimes omitted. (A partial listing is given on p. 39 at the end of the section.)

9. File HEAT_PR.dbf

The file HEAT_PR.dbf (Table 11) lists up to eight different steps of heat treatment with temperature ranges, duration, quench method, and an indication of whether the particular step was intended for normalizing, austenizing, tempering, or stress relief, as far as reported. The supplier of the material and the facility performing the heat treatment as well as the identification used for the ingot (HEAT_NO) and the finished material (SUPPL_ID) are also included. (A partial listing is given on p. 40 at the end of the section.)

Table 11. Structure file for HEAT_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
HEAT_ID	Identification Code for Given Material
PW_ID	Material Type: P_late, F_orging, W_eld, HAZ, or SRM
SUPPLIER	Supplier of Material
HEAT_TREAT	Facility Performing Heat Treatment
HEAT_NO	Heat Number of Material Used by Supplier
SUPPL_ID	Identifier Used by Supplier
MINTEMP_1	Heat Treatment Minimum Temperature, Run 1
MAXTEMP_1	Heat Treatment Maximum Temperature, Run 1
RANGE_1	Heat Treatment Temperature Range, Run 1
HOURS_1	Heat Treatment Duration, Run 1 [hours]
QCHM_1	Quench Method, Run 1
ID_1	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 1
MINTEMP_2	Heat Treatment Minimum Temperature, Run 2
MAXTEMP_2	Heat Treatment Maximum Temperature, Run 2
RANGE_2	Heat Treatment Temperature Range, Run 2
HOURS_2	Heat Treatment Duration, Run 2 [hours]
QCHM_2	Quench Method, Run 2
ID_2	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 2

Table 11 (continued)

MINTEMP_3	Heat Treatment Minimum Temperature, Run 3
MAXTEMP_3	Heat Treatment Maximum Temperature, Run 3
RANGE_3	Heat Treatment Temperature Range, Run 3
HOURS_3	Heat Treatment Duration, Run 3 [hours]
QCHM_3	Quench Method, Run 3
ID_3	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 3
MINTEMP_4	Heat Treatment Minimum Temperature, Run 4
MAXTEMP_4	Heat Treatment Maximum Temperature, Run 4
RANGE_4	Heat Treatment Temperature Range, Run 4
HOURS_4	Heat Treatment Duration, Run 4 [hours]
QCHM_4	Quench Method, Run 4
ID_4	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 4
MINTEMP_5	Heat Treatment Minimum Temperature, Run 5
MAXTEMP_5	Heat Treatment Maximum Temperature, Run 5
RANGE_5	Heat Treatment Temperature Range, Run 5
HOURS_5	Heat Treatment Duration, Run 5 [hours]
QCHM_5	Quench Method, Run 5
ID_5	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 5
MINTEMP_6	Heat Treatment Minimum Temperature, Run 6
MAXTEMP_6	Heat Treatment Maximum Temperature, Run 6
RANGE_6	Heat Treatment Temperature Range, Run 6
HOURS_6	Heat Treatment Duration, Run 6 [hours]
QCHM_6	Quench Method, Run 6
ID_6	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 6
MINTEMP_7	Heat Treatment Minimum Temperature, Run 7
MAXTEMP_7	Heat Treatment Maximum Temperature, Run 7
RANGE_7	Heat Treatment Temperature Range, Run 7
HOURS_7	Heat Treatment Duration, Run 7 [hours]
QCHM_7	Quench Method, Run 7
ID_7	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 7
MINTEMP_8	Heat Treatment Minimum Temperature, Run 8
MAXTEMP_8	Heat Treatment Maximum Temperature, Run 8
RANGE_8	Heat Treatment Temperature Range, Run 8
HOURS_8	Heat Treatment Duration, Run 8 [hours]
QCHM_8	Quench Method, Run 8
ID_8	N_ormalizing, A_ustenizing, T_empering, Stress R_elief, Run 8
TEMP_U	Unit for Temperature Data
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

10. File WELD_PR.dbf

The file WELD_PR.dbf (Table 12) gives additional information for weldments such as weld method, type and heat number of the filler material, and type and lot number of the flux used. The weld code (e.g., WF-209, plus the supplier of the weld) is also included. (A partial listing is given on p. 41 at the end of the section.)

Table 12. Structure file for WELD_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
HEAT_ID	Identification Code for Given Material
WLD_TYPE	Weld Type
WLD_CODE	Identification Code used by Weld Manufacturer
HEAT_1	HEAT_ID of the Plate on one side of the Weld
HEAT_2	HEAT_ID of the Plate on the other side of the Weld
WELD_SUPPLY	Supplier of Weld Material
WIRE_TYPE	Type of Weld Wire Used in the Weld
WIRE_HEAT	Weld Wire Heat Identifier
FLUX_TYPE	Type of Flux Used in the Weld
FLUX_LOT	Weld Flux Lot Identifier
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

11. File HAZ_PR.dbf

The file HAZ_PR.dbf (Table 13) identifies the base material and weldments used to prepare the heat-affected-zone specimens. (A partial listing is given on p. 42 at the end of the section.)

Table 13. Structure file for HAZ_PR.dbf

TAG	Used for Internal Operation
PLANT_ID	Reactor Identification
HEAT_ID	Identification Code for Given Material
HEAT_B	HEAT_ID of the Base Material Used in the HAZ
HEAT_W	HEAT_ID of the Weld Material Connected with the HAZ
REF_ID	Reference Identifier
PAGES	Page Number(s)
NOTES	Pertinent Information Related to Data Entries, If Needed

12. File REF_TITL.dbf

All PR-EDB files mentioned so far, with the exception of RAW_C_PR.dbf and REAC_LST.dbf, contain a reference in the field REF_ID plus page numbers for each record. The complete bibliographic listing, including author, title, report number, and date of publication, is located in the file REF_TITL.dbf (Table 14). A complete listing of this file is given on p. 43 at the end of the section.

Table 14. Structure file for REF_TITL.dbf

TAG	Used for Internal Operation
REF_ID	Reference Identifier
CONT	Continuation Tag for References Occupying More than One Record
REF_TITLE	Bibliographic Reference

13. File REF_LST.dbf

The file REF_LST.dbf lists all references used in the PR-EDB arranged according to the reactors for which the reports were written. The references are given as REF_ID; detailed information can be found in the file REF_TITL.dbf. The first two references describe the A302B ASTM and the A533B1 HSST-02 standard reference materials, respectively, and are not related to a specific reactor. A partial listing of this file is given on p. 54.

Tables 15 through 28 give partial or complete listings of the PR-EDB files discussed earlier.

Table 15. Partial listing of SPEC_LST.dbf

SPEC_LST.dbf: Page 1	TAG	PLANT_ID	CAPSULE	HEAT_ID	SPEC_TYPE	SPEC_SIZE	SPEC_ORI	SPEC_POS	NO_OF_SPEC	REF_ID	PAGES	NOTES
	M	AD1	FAD101	CV		LT	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	FAD101	CV		TL	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	HAD101	CV			1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	WAD101	CV		TL	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	FAD101	TEN		L	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	FAD101	TER		T	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	WAD101	TER			1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	FAD101	WXL		LT	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	FAD101	MOL		TL	1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	WAD101	WXL			1/4T			MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	CV		LT	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	HAD101	CV			1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TEN	L	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TER	T	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TEN		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TER		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	MOL		LT	1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	MOL		TL	1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	MOL			1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	P	FAD101	CV		LT	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	P	FAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	P	HAD101	CV			1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	P	WAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	P	WAD101	TEN	L	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	P	WAD101	TER	T	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	P	WAD101	TEN		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	P	WAD101	TER		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	P	FAD101	WXL		LT	1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	P	FAD101	WXL		TL	1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	CV		LT	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	HAD101	CV			1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	TEN	L	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	TEN	T	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TEN		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TER		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	MOL		LT	1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	R	FAD101	MOL		TL	1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	MOL			1/4T	3	MCAP-8957	3-3; 3-7	
	M	AD1	R	WAD101	TER		1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	S	FAD101	CV		LT	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	S	FAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	S	HAD101	CV			1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	S	WAD101	CV		TL	1/4T	12	MCAP-8957	3-3; 3-7	
	M	AD1	S	FAD101	TEN	L	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	S	FAD101	TER	T	1/4T	3		MCAP-8957	3-3; 3-7	
	M	AD1	S	WAD101	TER		1/4T	3		MCAP-8957	3-3; 3-7	

Table 16. Partial listing of SHFT PR. dbff

Table 17 Partial listing of RAW_C_PR.dbf

RAW_C_PR.dbf: Page 1											
	TAG	PLANT_ID	CAPSULE	HEAT_ID	PREDI_ID	SPEC_ID	SPEC_ID	1ST_TEMP	1ST_TEMP_U	TEMP_E	TEMP_E_U
#	AD1	FAD101	F	LT		-100.0	F	6.5	FT-LB	0	3.0
#	AD1	FAD101	F	LT		-100.0	F	4.0	FT-LB	0	1.0
#	AD1	FAD101	F	LT		-100.0	F	4.0	FT-LB	0	2.5
#	AD1	FAD101	F	LT		-26.0	F	54.0	FT-LB	14	39.0
#	AD1	FAD101	F	LT		-20.0	F	31.5	FT-LB	9	22.0
#	AD1	FAD101	F	LT		-20.0	F	38.5	FT-LB	14	28.0
#	AD1	FAD101	F	LT		20.0	F	70.0	FT-LB	39	52.0
#	AD1	FAD101	F	LT		20.0	F	77.0	FT-LB	42	57.0
#	AD1	FAD1C1	F	LT		20.0	F	84.0	FT-LB	39	61.0
#	AD1	FAD101	F	LT		50.0	F	75.0	FT-LB	20	58.0
#	AD1	FAD101	F	LT		50.0	F	75.5	FT-LB	25	56.0
#	AD1	FAD101	F	LT		50.0	F	60.0	FT-LB	30	47.0
#	AD1	FAD101	F	LT		110.0	F	111.0	FT-LB	77	79.0
#	AD1	FAD101	F	LT		110.0	F	117.5	FT-LB	80	86.0
#	AD1	FAD101	F	LT		110.0	F	113.0	FT-LB	77	78.0
#	AD1	FAD101	F	LT		210.0	F	158.0	FT-LB	100	90.0
#	AD1	FAD101	F	LT		216.0	F	145.0	FT-LB	100	88.0
#	AD1	FAD101	F	LT		210.0	F	123.0	FT-LB	100	90.0
#	AD1	FAD101	F	TL		50.0	F	25.0	FT-LB	10	17.0
#	AD1	FAD101	F	TL		-50.0	F	12.5	FT-LB	6	7.0
#	AD1	FAD101	F	TL		-50.0	F	9.0	FT-LB	5	4.6
#	AD1	FAD101	F	TL		0.0	F	53.0	FT-LB	30	39.0
#	AD1	FAD101	F	TL		0.0	F	49.5	FT-LB	30	36.0
#	AD1	FAD101	F	TL		0.0	F	42.0	FT-LB	25	29.0
#	AD1	FAD101	F	TL		50.0	F	64.0	FT-LB	36	49.0
#	AD1	FAD101	F	TL		50.0	F	70.0	FT-LB	40	52.0
#	AD1	FAD101	F	TL		50.0	F	80.0	FT-LB	42	50.0
#	AD1	FAD101	F	TL		110.0	F	107.0	FT-LB	73	73.0
#	AD1	FAD101	F	TL		110.0	F	131.0	FT-LB	100	84.0
#	AD1	FAD101	F	TL		110.0	F	100.0	FT-LB	73	67.0
#	AD1	FAD101	F	TL		160.0	F	123.0	FT-LB	100	81.0
#	AD1	FAD101	F	TL		160.0	F	132.0	FT-LB	100	89.0
#	AD1	FAD101	F	TL		160.0	F	131.0	FT-LB	100	87.0
#	AD1	FAD101	F	TL		210.0	F	129.0	FT-LB	100	89.0
#	AD1	FAD101	F	TL		210.0	F	130.0	FT-LB	100	82.0
#	AD1	FAD101	F	TL		210.0	F	126.0	FT-LB	100	85.0
#	AD1	FAD101	H02			-200.0	F	24.0	FT-LB	3	8.0
#	AD1	FAD101	H02			-200.0	F	46.0	FT-LB	50	23.0
#	AD1	FAD101	H02			-125.0	F	69.5	FT-LB	35	51.0
#	AD1	FAD101	H02			-125.0	F	89.5	FT-LB	20	40.0
#	AD1	FAD101	H02			-100.0	F	75.0	FT-LB	85	54.0
#	AD1	FAD101	H02			-100.0	F	61.0	FT-LB	85	50.0
#	AD1	FAD101	H02			-50.0	F	45.5	FT-LB	16	28.0
#	AD1	FAD101	H02			-120.0	F	120.0	FT-LB	51	49.0
#	AD1	FAD101	H02			-50.0	F	119.0	FT-LB	42	65.0
#	AD1	FAD101	H02			0.0	F	148.0	FT-LB	85	74.0
#	AD1	FAD101	H02			-100.0	F	127.0	FT-LB	60	80.0
#	AD1	FAD101	H02			0.0	F	195.0	FT-LB	100	86.0
#	AD1	FAD101	H02			62.0	F	100.0	FT-LB	90	67.0
#	AD1	FAD101	H02			62.0	F	190.0	FT-LB	100	92.0
#	AD1	FAD101	H02			62.0	F	200.0	FT-LB	100	85.0

Table 18. Partial listing of CV_RF_PR.dbf

TAG	PLANT_ID	CAPSULE	HEAT_ID	PROD_ID	SPEC_ORI	REF_ID	PAGES	NOTES
W	AD1		FAD101	F	LT	WCAP-8957	4-2	
W	AD1		FAD101	F	TL	WCAP-8957	4-3	
W	AD1		HAD101	HAZ		WCAP-8957	6-7	
W	AD1		WAD101	W	TL	WCAP-8957	4-6	
W	AD1	V	FAD101	F	LT	SWRI-06-8976	1; 42	
W	AD1	V	FAD101	F	TL	SWRI-06-8976	1; 41	
W	AD1	V	HAD101	HAZ		SWRI-06-8976	1; 44	
W	AD1	V	WAD101	W	TL	SWRI-06-8976	1; 43	
W	AL2		HAL201	HAZ	TL	WCAP-9228	3-8	
W	AL2		PAL201	P	LT	WCAP-9228	3-3	
W	AL2		PAL201	P	TL	WCAP-9228	3-4	
W	AL2		WAL201	W	TL	WCAP-9228	3-7	
B	AN1		HAN101	HAZ	TL	BAW-1440	C-4	
B	AN1		HAN101	HAZ	TS	BAW-1440	C-5	
B	AN1		HAN102	HAZ	TL	BAW-1440	C-8	
B	AN1		HAN102	HAZ	TS	BAW-1440	C-9	
B	AN1		PAN101	P	LT	BAW-1440	C-2	
B	AN1		PAN101	P	TL	BAW-1440	C-3	
B	AN1		PAN102	P	LT	BAW-1440	C-6	
B	AN1		PAN102	P	TL	BAW-1440	C-7	
B	AN1		WAN101	W	TL	BAW-1440	C-10	
B	AN1	A	HAN101	HAZ	TL	BAW-1836	5-3	
B	AN1	A	PAN101	P	LT	BAW-1836	5-4	
B	AN1	A	PAN101	P	TL	BAW-1836	5-4	
B	AN1	A	SHSS02	SRM	LT	BAW-1836	5-5	
B	AN1	A	WAN101	W	TL	BAW-1836	5-5	
B	AN1	B	HAN102	HAZ	TL	BAW-1698	5-3	
B	AN1	B	PAN102	P	LT	BAW-1698	5-4	
B	AN1	B	PAN102	P	TL	BAW-1698	5-3	
C	AN1	B	SHSS02	SRM	LT	BAW-1698	5-4	
B	AN1	E	HAN101	HAZ	TL	BAW-1440	5-4	
B	AN1	E	PAN101	P	LT	BAW-1440	5-4	
B	AN1	E	PAN101	P	TL	BAW-1440	5-4	
B	AN1	E	SHSS02	SRM	LT	BAW-1440	5-5	
B	AN1	E	WAN101	W	TL	BAW-1440	5-5	
W	AS1		HAS101	HAZ	TL	WCAP-9308	3-8	
W	AS1		PAS101	P	LT	WCAP-9308	3-3	
W	AS1		PAS101	P	TL	WCAP-9308	3-4	
W	AS1		WAS101	W	TL	WCAP-9308	3-7	
W	AS2		HAS201	HAZ	TL	WCAP-9330	3-8	
W	AS2		PAS201	P	LT	WCAP-9330	3-3	
W	AS2		PAS201	P	TL	WCAP-9330	3-4	
W	AS2		WAS201	W	TL	WCAP-9330	3-7	
G	BR		HBR_01	HAZ	TS	EPRI NP-2428	3-214	
G	BR		PBR_01	P	TL	EPRI NP-2428	3-207	
G	BR		WBR_01	W	TS	EPRI NP-2428	3-219	
G	BR	119	HBR_01	HAZ	TS	EPRI NP-2428	3-214	
G	BR	119	PBR_01	P	TL	EPRI NP-2428	3-208	
G	BR	119	WBR_01	W	TS	EPRI NP-2428	3-219	
G	BR	122	HBR_01	HAZ	TS	EPRI NP-2428	3-215	
G	BR	122	PBR_01	P	TL	EPRI NP-2428	3-208	
G	BR	122	WBR_01	W	TS	EPRI NP-2428	3-220	
G	BR	124	HBR_01	HAZ	TS	EPRI NP-2428	3-215	
G	BR	124	PBR_01	P	TL	EPRI NP-2428	3-208	
G	BR	124	WBR_01	W	TS	EPRI NP-2428	3-220	
G	BR	125	HBR_01	HAZ	TS	WCAP-9794	5-4	
G	BR	125	PBR_01	P	TL	WCAP-9794	5-4	

Table 19. Partial listing of TEN_PR.dbf

Table 20.

REAC_LST.dbf: Page 1				LOCATION	PLANT_OF
TAG	PLANT_ID	REAC_TYPE	REAC_NAME		
M	AD1	PWR	Angra Dos Reis Unit 1	Iteorne, RdJ, Brazil	Furnas-Centrais Electricas S.A.
M	AL2	PWR	Almerez Unit 2	Almerez, Caceres, Spain	Compania Sevillana de Electricidad
B	AN1	PWR	Arkansas Nuclear One, Unit 1	Russellville, Arkansas	Arkansas Power & Light Company
C	AN2	PWR	Arkansas Nuclear One, Unit 2	Russellville, Arkansas	Arkansas Power & Light Company
M	AS1	PWR	Asco Unit 1	Asco, Tarragona, Spain	Fuerzas Electricas de Cataluna,
M	AS2	PWR	Asco Unit 2	Asco, Tarragona, Spain	Fuerzas Electricas de Cataluna,
G	BF3	BWR	Browns Ferry Unit 3	Decatur, Alabama	Tennessee Valley Authority
G	BR	BWR	Big Rock Point Reactor	Charlevoix, Michigan	Consumers Power Company
M	BV1	PWR	Beaver Valley Unit 1	Shippingport, Pennsylvania	Duquesne Light Company
G	BW1	BWR	Brunswick Unit 1	Southport, North Carolina	Carolina Power & Light Company
G	BW2	BWR	Brunswick Unit 2	Southport, South Carolina	Carolina Power & Light Company
M	BZ1	PWR	Beznau Unit 1	Doettingen, Switzerland	Nordostschweizerische Kraftwerk
M	BZ2	PWR	Beznau Unit 2	Doettingen, Switzerland	Nordostschweizerische Kraftwerk
M	CAB	PWR	Jose Cabrera Reactor	Madrid, Spain	Union Electrica (UE)-Fenosa SA
M	CB1	PWR	Catawba Unit 1	Clover, South Carolina	Duke Power Company
C	CD1	PWR	Calvert Cliffs Unit 1	Lusby, Maryland	Baltimore Gas & Electric Company
E	CC2	PWR	Calvert Cliffs Unit 2	Lusby, Maryland	Baltimore Gas & Electric Company
M	CK1	PWR	Donald C. Cook Unit 1	Bridgman, Michigan	Indiana & Michigan Electric Co.
M	CK2	PWR	Donald C. Cook Unit 2	Bridgman, Michigan	Indiana & Michigan Electric Co.
M	CL1	PWR	Callaway Unit 1	Fulton, Missouri	Union Electric Company
G	CPR	BWR	Cooper	Brownville, Nebraska	Nebraska Public Power District
B	CR3	PWR	Crystal River Unit 3	Red Level, Florida	Florida Power Corporation
M	CTY	PWR	Kiddam Neck	Kiddam Neck, Connecticut	Connecticut Yankee Atomic Power
B	DB1	PWR	Davis-Besse Nuclear Power Station Unit 1	Oak Harbor, Ohio	Toledo Edison Company
M	DC1	PWR	Diablo Canyon Unit 1	Avila Beach, California	Pacific Gas & Electric Company
M	DC2	PWR	Diablo Canyon Unit 2	Avila Beach, California	Pacific Gas & Electric Company
G	DR1	BWR	Dresden Nuclear Plant Station Unit 1	Morris, Illinois	Commonwealth Edison Company
G	DR2	BWR	Dresden Nuclear Plant Station Unit 2	Morris, Illinois	Commonwealth Edison Company
G	DR3	BWR	Dresden Nuclear Plant Station Unit 3	Morris, Illinois	Commonwealth Edison Company
A	ERR	BWR	Elk River	Elk River, Minnesota	Rural Co-operative Power Assoc.
M	FA1	PWR	Joseph M. Farley Unit 1	Dothan, Alabama	Alabama Power Company
M	FA2	PWR	Joseph M. Farley Unit 2	Dothan, Alabama	Alabama Power Company
C	FC1	PWR	Fort Calhoun Station Unit 1	Fort Calhoun, Nebraska	Omaha Public Power District
G	FTZ	BWR	James A. FitzPatrick	Dawego, New York	New York Power Authority
G	GAR	BWR	Garigliano	Sessa Aurunca, Italy	Ente Nazionale per l'Energia El.
M	GTN	PWR	Robert E. Ginna Nuclear Plant Unit 1	Ontario, New York	Rochester Gas and Electric Corp.
G	HA1	BWR	Edwin L. Hatch Unit 1	Baxley, Georgia	Georgia Power Company
M	HB2	PWR	H. B. Robinson Unit 2	Hartsville, South Carolina	Carolina Power & Light Company
G	HM3	BWR	Humboldt Bay Power Plant Unit 3	Eureka, California	Pacific Gas & Electric Company
M	IP2	PWR	Indian Point Unit 2	Indian Point, New York	Consolidated Edison Company of
M	IP3	PWR	Indian Point Unit 3	Indian Point, New York	Consolidated Edison Company of
M	KO1	PWR	Korea Nuclear 1	Ko-Ri, Korea	Korea Electric Power Corporatio
M	KWE	PWR	Keweenaw Nuclear Power Plant	Carlton, Wisconsin	Wisconsin Public Service Corp.
A	LAC	BWR	Lacrosse Boiling Water Reactor	Genoa, Wisconsin	Dairyland Power Cooperative
M	LM2	PWR	Lemoniz Unit 2	Lemoniz, Vizcaya, Spain	Iberduero S. A.
M	MC1	PWR	M. B. McGuire Unit 1	Cornelius, North Carolina	Duke Power Company
M	MC2	PWR	M. B. McGuire Unit 2	Cornelius, North Carolina	Duke Power Company
B	MD1	PWR	Midland Unit 1	Midland, Michigan	Consumers Power Company
B	MD2	PWR	Midland Unit 2	Midland, Michigan	Consumers Power Company
G	ML1	BWR	Millstone Nuclear Power Station Unit 1	Waterford, Connecticut	Northeast Utilities Service Com.
C	ML2	PWR	Millstone Nuclear Power Station Unit 2	Waterford, Connecticut	Northeast Utilities Service Com.
G	MON	BWR	Monticello Nuclear Generating Plant	Monticello, Minnesota	Northern States Power Company
C	NY	PWR	Maine Yankee Nuclear Plant	Wiscasset, Maine	Maine Yankee Atomic Power Compa

Listing of REAC_LST.dbf

	PLANT_DES	ARCH_ENG	VESSEL_MFG
ad SA	Westinghouse/EBE	Gibbs & Hill/Promon	Babcock and Wilcox
	Westinghouse	Empresarios Agrupados/Gibbs & Hill Espanola	Combustion Engineering
	Babcock and Wilcox	Bechtel	Babcock and Wilcox
	Combustion Engineering	Bechtel	Combustion Engineering
S.A.	Westinghouse	Bechtel/Empresa Nacional de Ingenieria Tech (Spain)	Westinghouse
S.A.	Westinghouse	Bechtel/Empresa Nacional de Ingenieria Tech (Spain)	Westinghouse
	General Electric	Tennessee Valley Authority	General Electric
	General Electric	Bechtel	Combustion Engineering
	Westinghouse	Stone & Webster Engineering Corp.	Westinghouse
	General Electric	United Engineers and Constructors	Chicago Bridge and Iron
	General Electric	United Engineers and Constructors	Chicago Bridge and Iron
AG (NOK)	Westinghouse	Gibbs & Hill (US)/Brown Bovin et Cie (Switzerland)	Societe des Forges et Ateliers du Creusot
AG (NOK)	Westinghouse	Gibbs & Hill (US)/Brown Bovin et Cie (Switzerland)	Societe des Forges et Ateliers du Creusot
	Westinghouse	Gibbs & Hill	Combustion Engineering
	Westinghouse	Duke Power Company	Westinghouse
	Combustion Engineering	Bechtel	Combustion Engineering
	Combustion Engineering	Bechtel	Combustion Engineering
pany	Westinghouse	American Electric Power Service Corporation	Combustion Engineering
pany	Westinghouse	American Electric Power Service Corporation	Chicago Bridge and Iron
Company	Westinghouse	Bechtel	Combustion Engineering
	General Electric	Burns & Roe, Inc.	Combustion Engineering
	Babcock and Wilcox	Gilbert Associates, Inc.	Babcock and Wilcox
	Westinghouse	Jone & Webster Engineering Corp.	Combustion Engineering
	Babcock and Wilcox	Bechtel	Babcock and Wilcox
	Westinghouse	Pacific Gas & Electric Company/Bechtel	Combustion Engineering
	Westinghouse	Pacific Gas & Electric Company/Bechtel	Combustion Engineering
	General Electric	Bechtel	Babcock and Wilcox
	General Electric	Sargent & Lundy Engineers	Babcock and Wilcox
	General Electric	Sargent & Lundy Engineers	Babcock and Wilcox
	Allis-Chalmers	Sargent & Lundy Engineers	Pacific Coast Engineering
	Westinghouse	Southern Company Services, Inc./Bechtel	Combustion Engineering
	Westinghouse	Southern Company Services, Inc./Bechtel	Combustion Engineering
	Combustion Engineering	Gibbs & Hill	Combustion Engineering
	General Electric	Stone & Webster Engineering Corp.	Combustion Engineering
	General Electric	Ebasco Services, Inc.	Tenneco
	Westinghouse	Gilbert Associates, Inc.	Babcock and Wilcox
	General Electric	Southern Company Services, Inc./Bechtel	General Electric
	Westinghouse	Ebasco Services, Inc.	Combustion Engineering
	General Electric	Bechtel	Combustion Engineering
	Westinghouse	United Engineers and Constructors	Babcock and Wilcox
	Westinghouse	United Engineers and Constructors	Combustion Engineering
	Westinghouse	Gilbert Associates, Inc.	Westinghouse
	Westinghouse	Fluor Engineers, Inc.	Combustion Engineering
	Allis-Chalmers	Sargent & Lundy Engineers	Combustion Engineering
	Westinghouse	Iberduero S.A./Bechtel/Sener, S.A./INITEC	Babcock and Wilcox
	Westinghouse	Duke Power Company	Westinghouse
	Westinghouse	Duke Power Company	Westinghouse
	Babcock and Wilcox	Bechtel	Babcock and Wilcox
	Babcock and Wilcox	Bechtel	Babcock and Wilcox
	General Electric	Ebasco Services, Inc.	Combustion Engineering
	Combustion Engineering	Bechtel	Combustion Engineering
	General Electric	Bechtel	General Electric
	Combustion Engineering	Stone & Webster Engineering Corp.	Combustion Engineering

SI
APERTURE
CARD

Also Available On
Aperture Card

9006290154-01

REAC_LST.DBF: Page 2

TAG	PLANT_ID	REAC_TYPE	REAC_NAME	LOCATION	PLANT_OP
N	NA1	PWR	North Anna Unit 1	Mineral, Virginia	Virginia Electric & Power Company
N	NA2	PWR	North Anna Unit 2	Mineral, Virginia	Virginia Electric & Power Company
G	NM1	BWR	Nine Mile Point Unit 1	Scriba, New York	Kiagara Mohawk Power Corporation
B	OC1	PWR	Oconee Nuclear Station Unit 1	Seneca, South Carolina	Duke Power Company
B	OC2	PWR	Oconee Nuclear Station Unit 2	Seneca, South Carolina	Duke Power Company
B	OC3	PWR	Oconee Nuclear Station Unit 3	Seneca, South Carolina	Duke Power Company
G	OYS	BWR	Oyster Creek Nuclear Generating Station	Forked River, New Jersey	General Public Utilities Nuclear
C	PAL	PWR	Palisades Nuclear Plant	South Haven, Michigan	Consumers Power Company
N	PB1	PWR	Point Beach Nuclear Plant Unit 1	Two Creeks, Wisconsin	Wisconsin Electric Power Company
N	PB2	PWR	Point Beach Nuclear Plant Unit 2	Two Creeks, Wisconsin	Wisconsin Electric Power Company
N	PI1	PWR	Prairie Island Unit 1	Red Wing, Minnesota	Northern States Power Company
N	PI2	PWR	Prairie Island Unit 2	Red Wing, Minnesota	Northern States Power Company
G	PL1	BWR	Pilgrim Nuclear Power Station Unit 1	Plymouth, Massachusetts	Boston Edison Company
G	QC1	BWR	Quad Cities Nuclear Power Station Unit 1	Cordova, Illinois	Commonwealth Edison Company
G	QC2	BWR	Quad Cities Nuclear Power Station Unit 2	Cordova, Illinois	Commonwealth Edison Company
N	R12	PWR	Ringhals Unit 2	Varberg, Sweden	Swedish State Power Board (SSPB)
B	RS1	PWR	Echo Seco Unit 1	Clay Station, California	Sacramento Municipal Utility District
N	SA1	PWR	Salem Unit 1	Salem, New Jersey	Public Service Electric and Gas Company
N	SA2	PWR	Salem Unit 2	Salem, New Jersey	Public Service Electric and Gas Company
G	SH1	BWR	Susquehanna Unit 1	Berwick, Pennsylvania	Pennsylvania Power & Light Company
C	SL1	PWR	St. Lucie Unit 1	Hutchinson Island, Florida	Florida Power & Light Company
C	SL2	PWR	St. Lucie Unit 2	Hutchinson Island, Florida	Florida Power & Light Company
N	SO1	PWR	San Onofre Unit 1	San Clemente, California	Southern California Edison Company
N	SO1	PWR	Sequoiah Unit 1	Soddy Daisy, Tennessee	Tennessee Valley Authority
N	SO2	PWR	Sequoiah Unit 2	Soddy Daisy, Tennessee	Tennessee Valley Authority
N	SU1	PWR	Sunny Unit 1	Gravel Neck, Virginia	Virginia Electric & Power Company
N	SU2	PWR	Sunny Unit 2	Gravel Neck, Virginia	Virginia Electric & Power Company
B	TN1	PWR	Three Mile Island Nuclear Station Unit 1	Londonderry Twp., Pennsylvania	Metropolitan Edison Company
B	TN2	PWR	Three Mile Island Nuclear Station Unit 2	Three Mile Island, PA	Metropolitan Edison Company
N	TP3	PWR	Turkey Point Nuclear Power Station Unit 3	Florida City, Florida	Florida Power & Light Company
N	TP4	PWR	Turkey Point Nuclear Power Station Unit 4	Florida City, Florida	Florida Power & Light Company
N	TRD	PWR	Trojan Reactor	Prescott, Oregon	Portland General Electric Company
N	VS1	PWR	Virgil C. Summer Unit 1	Parr, South Carolina	Carolina Electric and Gas Company
G	VY	BWR	Vermont Yankee Nuclear Power Station	Vernon, Vermont	Vermont Yankee Nuclear Power Corporation
N	WB1	PWR	Watts Bar Unit 1	Spring City, Tennessee	Tennessee Valley Authority
N	WC1	PWR	Wolf Creek Unit 1	Burlington, Kansas	Kansas Gas & Electric
N	YR	PWR	Yankee Reactor, Rowe, Massachusetts	Rowe, Massachusetts	Yankee Atomic Electric Company
N	ZN1	PWR	Zion Nuclear Plant Reactor Unit 1	Zion, Illinois	Commonwealth Edison Company
N	ZN2	PWR	Zion Nuclear Plant Reactor Unit 2	Zion, Illinois	Commonwealth Edison Company

Table 20 (continued)

PLANT_DES	ARCH_ENG	VESSEL_MFG
Westinghouse	Stone & Webster Engineering Corp.	Rotterdamse Droogdok Maatschappij (Netherlands)
Westinghouse	Stone & Webster Engineering Corp.	Combustion Engineering
General Electric	Niagara Mohawk Power Corporation	Combustion Engineering
Babcock and Wilcox	Duke Power Company/Bechtel	Babcock and Wilcox
Babcock and Wilcox	Duke Power Company/Bechtel	Combustion Engineering
Babcock and Wilcox	Duke Power Company/Bechtel	Babcock and Wilcox
General Electric	Burns & Roe, Inc./General Electric Company	Combustion Engineering
Combustion Engineering	Bechtel	Combustion Engineering
Westinghouse	Bechtel	Babcock and Wilcox
Westinghouse	Bechtel	Combustion Engineering
Westinghouse	Fluor Engineers, Inc.	Societe des Forges et Ateliers du Creusot
Westinghouse	Fluor Engineers, Inc.	Societe des Forges et Ateliers du Creusot
General Electric	Bechtel	Combustion Engineering
General Electric	Sargent & Lundy Engineers	Babcock and Wilcox
General Electric	Sargent & Lundy Engineers	Babcock and Wilcox
Westinghouse	SSPB/Gibbs & Hill/Stal-Level Turbin AB	Rotterdamse Droogdok Maatschappij (Netherlands)
Babcock and Wilcox	Bechtel	Babcock and Wilcox
Westinghouse	Public Service Electric and Gas Company	Combustion Engineering
Westinghouse	Public Service Electric and Gas Company	Combustion Engineering
General Electric	Bechtel	Chicago Bridge and Iron
Combustion Engineering	Ebasco Services, Inc.	Combustion Engineering
Combustion Engineering	Ebasco Services, Inc.	C-E Avery
Westinghouse	Bechtel	Combustion Engineering
Westinghouse	Tennessee Valley Authority	Westinghouse
Westinghouse	Tennessee Valley Authority	Westinghouse
Westinghouse	Stone & Webster Engineering Corp.	Rotterdamse Droogdok Maatschappij (Netherlands)
Westinghouse	Stone & Webster Engineering Corp.	Rotterdamse Droogdok Maatschappij (Netherlands)
Babcock and Wilcox	Gilbert Associates, Inc.	Babcock and Wilcox
Babcock and Wilcox	Burns & Roe, Inc.	Babcock and Wilcox
Westinghouse	Bechtel	Westinghouse
Westinghouse	Bechtel	Babcock and Wilcox
Westinghouse	Gilbert Associates, Inc.	Chicago Bridge and Iron
General Electric	Ebasco Services, Inc.	Chicago Bridge and Iron
Westinghouse	Tennessee Valley Authority	Chicago Bridge and Iron
Westinghouse	Bechtel/Sargent & Lundy Engineers	Westinghouse
Westinghouse	Stone & Webster Engineering Corp.	Combustion Engineering
Westinghouse	Sargent & Lundy Engineers	Babcock and Wilcox
Westinghouse	Sargent & Lundy Engineers	Babcock and Wilcox

SI
APERTURE
CARD

Also Available On
Aperture Card

9006290154-02

Table 21. Partial listing of REAC_PR.dbf

Table 22. Listing of HEAT_LST.dbf

HEAT_LST_ID	PAGE_1	TAG_PLANT_ID	HEAT_ID	RPT_ID	HEAT_NO	PROD_ID	WT_ID	SUPPLIER	THICKNESS	SOURCE	WP_C_ID	WP_C_HEAT	REF_ID	PAGES	NOTES
SASTN		ASTM PLATE	A1021		SIN	A532B	U.S. STEEL CORP.	6"	FABR	E20 6	A0421	ASTM D333	2; 3		
SNSD01	C	HST1 01	A10201		SIN	A533B1	LURENS	12"	FABR	C01 6	MS3101	ORNL-4333	4; 5; 40		
SNSD02	S	HST1 02	A11951		SIN	A533B1	LURENS	12"	FABR			SWI-1698	3; 1		
AD1		AM161	A11951		SIN	A533B1	LURENS	12"	FABR			SWI-02-43770	9		
AD1		AM161	F		A5082	LUDISH		6 3/4"	SCR			WCAP-B957	3; 1; A-1	USED FOR MELLO	
AD1		AM153 (FOR WELD)	AM153		A5082	LUDISH		6 3/4"	SCR			WCAP-B957	3; 1; A-1		
AD1		AM161	AM161		A5082	LUDISH		6 3/4"	SCR			WCAP-B957	3; 1; A-1		
AD1		AM161 AND AM153	AM161		A5082	LUDISH		6 3/4"	SCR			WCAP-B957	3; 1; A-1		
AL2		HAL201	FROM 89405-1		HAZ	A533B1	LURENS	8 7/8"	SIN			WCAP-9228	2; 1		
AL2		PAL201	89405-1 (SURV.)		P	A533B1	LURENS	8 7/8"	SIN			WCAP-9228	2; 1		
AL2		PAL202	89405-2 (FOR WELD)		P	A533B1	LURENS	8 7/8"	SIN			WCAP-9228	2; 1; A-1		
AL2		PAL201	89405-1 AND 89405-2		HAZ	A533B1	LURENS	8 7/8"	SIN			WCAP-9228	2; 1; A-1; 2		
AN1		HAN101	FROM C5114-1		C5114-1	HAZ	A533B1	LURENS	8 3/4"	SIN			WCAP-9228	2; 1; A-1; 2	
AN1		HAN102	FROM C5114-2		C5114-2	HAZ	A533B1	LURENS	8 3/4"	SIN			WCAP-9228	2; 1; A-1; 2	
AN1		PAN101	C-5114-1 (SURV.)		P	A533B1	LURENS	8 3/4"	SIN			WCAP-9228	2; 1; A-1; 2		
AN1		PAN102	C-5114-2 (SURV.)		P	A533B1	LURENS	8 3/4"	SIN			WCAP-9228	2; 1; A-1; 2		
AN1		MP-193	MP-193		A06144	M	A533B1	UNION CARBIDE	8 3/4"	SIN			WCAP-9228	2; 1; A-1; 2	
AN2		HAN101	FROM CB009-3		CB009-3	HAZ	A533B						LINE 80; 8775	14	
AN2		PAN201	CB009-3 (SURV.)		CB009-3	P	A533B						LINE 80; 8775	14	
AN2		PAN202	CB009-1 (FOR WELD)		CB009-1	P	A533B						LINE 80; 8775	14	
AN2		PAN203	CB009-2 (FOR WELD)		CB009-2	P	A533B						LINE 80; 8775	14	
AN2		MAN201	CB009-1 AND C-8009-2		M	A533B							LINE 80; 8775	14	
AS1		PAS101	FROM 89405-2		HAZ	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1		
AS1		PAS102	50605-2 (SURV.)		P	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1		
AS1		PAS101	50605-1 (FOR WELD)		P	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1		
AS1		PAS102	89405-1 AND 89405-2		89405-1	P	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1	
AS2		PAS201	FROM R007-1		R007-1	HAZ	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1	
AS2		PAS202	R007-2 (FOR WELD)		P	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1		
AS2		PAS201	R007-1 AND R007-2		4P0072	M	A533B1	LURENS	8 7/8"	SIN			WCAP-9330	2; 1	
BM		HBR-01	FROM S-5503-2/3		192462/3	HAZ	A5028						GECR-4442	4	
BM		PBR-01	S-5503-2/3 (SURV.)		192462/3	P	A5028						GECR-4442	4	
BR		BR-01	FROM S-5503-2/3		192462/3	M	A5028						GECR-4442	4	
BR		BR-01	FROM B6907-1		HAZ	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7		
BV1		PBV101	B6907-1 (SURV.)		C6317-1	P	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7	
BV1		PBV102	B6607-1 (FOR WELD)		B6607-1	P	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7	
BV1		PBV103	B6607-2 (FOR WELD)		B6607-2	P	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7	
BV1		PBV101	B6607-1 AND B6607-2		304524	M	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7	
BV1		PBV101	PIECE # 301, SLAB 1		C449	P	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7	
BV1		PBV101	PIECE # 301, SLAB 1		C4887	P	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7	
BZ1		PB-101	12K007 TREATMENT H		HAZ	A5028							WCAP-9330	2; 1; A-7	
BZ1		PBZ102	12K007 TREATMENT D		P	A5028							WCAP-9330	2; 1; A-7	
BZ1		PBZ101	12K007 TREATMENT M		M	A533B1	LURENS	6"	SIN			WCAP-9330	2; 1; A-7		
CAB		HCB802	FROM B1105-2		HAZ	A533B1	LURENS	12"	FABR			WCAP-9330	2; 1; A-7		
CAB		PCAB01	B1105-1 (SURV.)		P	A533B1	LURENS	12"	FABR			WCAP-9330	2; 1; A-7		
CAB		PCAB02	B1105-2 (SURV.)		P	A533B1	LURENS	12"	FABR			WCAP-9330	2; 1; A-7		
CAB		WCA801	FORGING 05 FOR BASE		411363	F	A5082	KLOECKER-MERKE AG	8.5"	SCR			WCAP-9330	2; 1; A-7	
CB1		FCB102	FORGING 05 FOR WELD		527708	F	A5082	KLOECKER-MERKE AG	8.5"	SCR			WCAP-9330	2; 1; A-7	
CB1		FCB101	FORGING 05 FOR WELD		81105-1	P	A533B1	KLOECKER-MERKE AG	8.5"	SCR			WCAP-9330	2; 1; A-7	
CB1		WCB101	FORG. 05 & FORG. 04		890975	U	A1052	KLOECKER-MERKE AG	8.5"	SCR			WCAP-9330	2; 1; A-7	
CC1		HCC101	D7206-3 AND D7206-1		HAZ	A533B1	LURENS	12"	SIN			WCAP-9330	2; 1; A-7		
CC1		PCC101	D7206-1 (FOR WELD)		C4351-2	P	A533B1	LURENS	12"	SCR			WCAP-9330	2; 1; A-7	
CC1		PCC102	D7206-2 (FOR WELD)		C4441-2	P	A533B1	LURENS	12"	SCR			WCAP-9330	2; 1; A-7	
CC1		PCC103	D7206-3 (SURV.)		C4441-1	P	A533B1	LURENS	12"	SCR			WCAP-9330	2; 1; A-7	
CC1		PCC101	D7206-1 AND D7206-2		53A277	M	A533B1	LURENS	12"	SIN			WCAP-9330	2; 1; A-7	

Table 22 (continued)

HEAT_LIST_ID:	Page_2	SPF_10	HEAT_NO	PROD_10	MAT_ID	SUPPLIER	THICKNESS	SOURCE	MPC_ID	MPC_MATERIAL	REF_ID	PAGES	NOTES
TAG_PLANT_ID: HEAT_10													
C C1	WEC102	07206-1/07206-3	HAZ		A53381	LURENS	S18	S18	SH1-1280	SUM-1-7526	7		USED FOR HAZ
C C2	HCC201	08907-1 AND 08907-3	HAZ		A53381	LURENS	S18	S18	DODGET 50-318	50			
C C2	PCC201	08907-1 (FOR WELD)	C5804-1		A53381	LURENS	S18	S18	DODGET 50-318	62			
C C2	PCC202	08907-2 (SURV.)	C5805-1		A53381	LURENS	S18	S18	DODGET 50-318	61			
C C2	PCC203	08907-3 (FOR WELD)	C5803-3		A53381	LURENS	S18	S18	DODGET 50-318	63			
C C2	WEC201	08907-1 AND 08907-3	10137	H	A53381	COMBUSTION ENG.	S18	S18	DODGET 50-318	6			
C C2	HCK101	FROM B4406-3	B4406-3	HAZ	A53381	LURENS	S18	S18	SUM-1-7244-001/1	6			
C C2	PCK101	B4406-3 (SURV.)	B4406-3	P	A53381	LURENS	S18	S18	SUM-1-7244-001/1	6			
C C2	HCK201	FROM C521-2	C521-2	HAZ	A53381	CHICAGO BRIDGE	S18	S18	WCA-P-8512	2-1; A-1			
C C2	PCK201	C521-2 (SURV.)	C521-2	P	A53381	CHICAGO BRIDGE	S18	S18	WCA-P-8512	2-1; A-1			
C C2	PCK202	C5592-1 (FOR WELD)	C5592-1	P	A53381	CHICAGO BRIDGE	S18	S18	WCA-P-8512	2-1; A-1			
C C2	WCK201	C5592-1 AND C5592-1	H	A53381	CHICAGO BRIDGE	S18	S18	WCA-P-8512	2-1; A-1				
C L1	HCL101	FROM B2708-1	B2708-1	HAZ	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C L1	PCL101	B2708-1 (SURV.)	C45099-2	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C L1	PCL102	B2708-1 (FOR WELD)	C45044-1	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C L1	WCL101	B2708-1 AND B2707-1	000777	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C P8	HCPH01	G2802-1 AND G2802-2	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1				
C P8	PCHP01	G2802-2 (SURV.)	C2307-2	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C P8	PCHP02	G2802-1 (FOR HAZ & H)	C2331-2	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C P8	WCPH01	G2802-1 AND G2802-2	120086	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C R3	HCR301	FROM C5344-1	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1				
C R3	HCR302	FROM C5344-2	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1				
C R3	PCR301	C5344-1 (FOR WELD)	C5344-1	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C R3	PCR302	C5344-2 (SURV.)	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1				
C R3	WCR301	ATPIPC 1 MILD	H	A53381	AM. CHAIN AND CABLE	S18	S18	WCA-P-8542	2-1				
C R3	WCR302	WFC 259-1	72105	H	A53381	AM. CHAIN AND CABLE	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	WPH07-1 AND WPH07-6	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C T7	WTCTY01	WPH07-1 (FOR WELD)	A5892	P	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY02	WPH07-2 (SURV.)	A5892	P	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY04	WPH07-4 (SURV.)	A5877	P	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY07	WPH07-7 (SURV.)	A5911	P	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY08	WPH07-8 (FOR WELD)	A5911	P	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	WPH07-1 AND WPH07-8	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C T7	WTCTY01	AKA1233 (SURV.)	123-264	H	A5082	LAD15H	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY02	BCC241 (SURV.)	5P4086	F	A5082	LAD15H	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	FROM AM-2233	123-264	H	A5082	LAD15H	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY02	FROM BCC241	5P4086	H	A5082	LAD15H	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	WF-182-1	821544	H	A5082	UNION CARBOTE	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	FROM B4406-3	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1				
C T7	WTCTY01	B4106-1	84106-1	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY02	B4106-2	84106-2	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY03	B4106-3 (SURV.)	84106-3	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	84106-1 AND 84106-3	27206	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	FROM B5454-1	C5161-1	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	B5454-1 (SURV.)	C5161-1	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY02	B5454-2 (FOR WELD)	B5454-2	P	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C T7	WTCTY01	B5454-1 AND B5454-2	27206	H	A53381	LURENS	S18	S18	WCA-P-8542	2-1			
C D9	WTCTY01	19716	19716	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C D9	WTCTY01	WTCTY01	19716	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C D9	WTCTY01	ESU MAZ	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY02	SAU MAZ	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY01	SURVEILLANCE BASE	A91281	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1			
C D9	WTCTY02	ESU WELD	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY02	SAU WELD	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY01	ESU MAZ	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY02	ESU WELD	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY01	ESU MAZ	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				
C D9	WTCTY02	ESU WELD	H	A5028	LURENS	S18	S18	WCA-P-8542	2-1				

Table 22 (continued)

Table 23. Partial

CHEN_P0.dbf: Page 1								SPEC_ID	C	H	I	P	S	S1	M1	CR	MD	
TAG	PLANT_ID	HEAT_ID	PROD_ID	MAT_ID	CHEN_LAB	METHOD												
V	AD1	FAD101	F	A5082	WESTINGHOUSE		HEAT											
V	AD1	FAD101	F	A5082	B&W		HEAT											
V	AD1	FAD101	F	A5082	LADISH		HEAT	0.210	0.600	0.012	0.009	0.230	0.740	0.320	0.560			
V	AD1	MAD101	M	A5082	B&W		HEAT	0.130	1.400	0.005	0.005	0.310	0.560	0.150	0.490			
V	AL2	PAL201	P	A53361			HEAT	0.260	1.330	0.009	0.018	0.220	0.560	0.060	0.530			
V	AL2	WAL201	M	A53381			HEAT	0.150	1.130	0.004	0.011	0.190	0.040	0.050	0.620			
B	AN1	PAN101	F	A53381	LUKENS		HEAT	0.210	1.320	0.010	0.016	0.200	0.520	0.190	0.570			
B	AN1	PAN102	F	A53381	LUKENS		HEAT	0.210	1.320	0.010	0.016	0.200	0.520	0.190	0.570			
B	AN1	SHSS02	SRM	A53361			HEAT	0.230	1.390	0.013	0.013	0.210	0.640		0.500			
B	AN1	MAN101	M	A53381			HEAT	0.065	1.500	0.016	0.008	0.420	0.590		0.360			
B	AN1	MAN101	M	A53381	UCC		HEAT	0.090	1.490	0.016	0.016	0.510	0.590	0.060	0.390			
C	AN2	PAN201	P	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	131	0.078		0.012				0.559				
C	AN2	PAN201	P	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	14M	0.077		0.013				0.548				
C	AN2	PAN201	P	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	21U	0.075		0.015				0.545				
C	AN2	PAN201	P	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	26A	0.078		0.015				0.555				
C	AN2	PAN201	P	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	257	0.078		0.014				0.559				
C	AN2	MAN201	M	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	315	0.046		0.008				0.090				
C	AN2	MAN201	M	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	344	0.047		0.008				0.088				
C	AN2	MAN201	M	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	37J	0.045		0.008				0.075				
C	AN2	MAN201	M	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	37S	0.044		0.008				0.082				
C	AN2	MAN201	M	A5338	BATTELLE	ICAP, MD-BLUE-PHOTO FOR P	33J	0.045		0.008				0.081				
V	AS1	PAS101	F	A53381			HEAT	0.240	1.560	0.008	0.015	0.230	0.560	0.030	0.600			
V	AS1	MAS101	M	A53381			HEAT	0.150	1.250	0.011	0.013	0.170	0.270	0.040	0.500			
V	AS2	PAS201	F	A53381			HEAT	0.220	1.370	0.007	0.008	0.270	0.610	0.030	0.560			
V	AS2	MAS201	M	A53381			HEAT	0.150	1.290	0.005	0.007	0.150	0.050	0.020	0.530			
G	BR	PBR_01	P	A3028			HEAT	0.300	1.420	0.016	0.018	0.250	0.180	0.130	0.510			
G	BR	MBR_01	M	A3028			HEAT	0.120	1.250	0.014	0.012	0.280	0.100	0.190	0.530			
V	BV1	PBV101	P	A53381			HEAT	0.200	1.310	0.010	0.015	0.180	0.540	0.140	0.550			
V	BV1	WBV101	M	A53381			HEAT	0.110	1.370	0.018	0.006	0.270	0.620	0.015	0.480			
V	BV1	WBV101	M	A53381			DW63	0.124	1.420	0.008	0.004	0.277	0.637	0.029				
G	BW1	PBW101	P	A53381	LUKENS		HEAT	0.250	1.400	0.007	0.016	0.270	0.600		0.560			
G	BW2	PBW201	P	A53381	LUKENS		HEAT	0.230	1.500	0.010	0.015	0.270	0.560		0.570			
V	BZ1	PBZ101	P				HEAT	0.185	1.160	0.014	0.011	0.190	0.710	0.240				
V	BZ1	PBZ102	P				HEAT	0.185	1.160	0.014	0.011	0.190	0.710	0.240				
V	BZ1	PBZ102	P				HEAT											
V	BZ1	SASTM	SRM				HEAT	0.240	1.340	0.011	0.023	0.230			0.510			
V	CAB	PCAB01	P	A53381			HEAT	0.200	1.290	0.013	0.022	0.250	0.500		0.490			
V	CAB	PCAB02	P	A53381			HEAT	0.210	1.240	0.011	0.020	0.240	0.530		0.480			
V	CAB	SASTM	SRM	A3028			HEAT	0.240	1.340	0.011	0.023	0.230	0.180		0.510			
V	CAB	WCAB01	M	A53381			HEAT	0.170	1.250	0.015	0.011	0.320	0.110		0.530			
V	CAB	WCAB01	M	A53381			WS		1.510	0.021		0.245	0.063		0.210			

NOTE: Due to the large number of fields and limit of page space, not all fields are displayed here.

listing of CHEM_PR.dbf

SI
APERTURE
CARD

Also Available On
Aperture Card

9006290154-03

Table 24. Partial listing

HEAT_PR.DBF: Page 1

	TAG	PLANT_ID	HEAT_ID	PROD_ID	SUPPLIER	HEAT_TREAT	HEAT_NO	SUPPL_ID	MINTEMP_1	MAXTEMP_1	RANGE_1	HOURS_1	QCHP_1	ID_1	MINTEMP_2	MAXTEMP_2	RANGE_2
C		SRSS01	SRN	LUKENS		COMBUSTION ENG.	A10081	HSST01	1600	1750		4.00	AC	#	1500	1650	
M	AS1	FAD101	F	LADISH			ANH161		1590		10	3.00	MD	A	1250		10
M	AS1	FAD101	F	LADISH				ANH161	1590		10	3.00	MD	A	1250		10
M	AD1	WAD101	M	B&W					1150		25	21.00	FC	R			
M	AL2	PAL201	F	LUKENS				894051	1600		25	4.00	MD	A	1225		25
M	AL2	VAL201	M	LUKENS			83642		1100	1150		13.00	FC	R			
G	AB1	PAB101	P				C51141		1550	1600		4.50	BD	A	1200	1225	
G	AB1	PAB101	P	LUKENS	LUKENS		C5114-1	C5114-1	1650	1700		1 R/IN	MD	A	1200		
G	AB1	PAB101	P	LUKENS			C5114-1		1650	1770		1 R/IN	MD	A	1200		
G	AB1	PAB102	P				C51142		1550	1600		4.50	BD	A	1200	1225	
G	AB1	PAB102	P	LUKENS	LUKENS		C5114-2	C5114-2	1650	1700		1 R/IN	MD	A	1200		
G	AB1	PAB102	P	LUKENS			C5114-2		1650	1770		1 R/IN	MD	A	1200		
G	AB1	SHSS02	SRN	LUKENS		COMBUSTION ENG.	A11951	KSS102	1675		75			#	1600		75
G	AB1	SHSS02	SRN	LUKENS		COMBUSTION ENG.	A11951	KSS102	1600		75	4.00	MD	A	1225		25
M	AM1	MAN101	M				WF193		1100	1150		27.50	FC	R			
M	AM1	MAN101	M	UNION CARBIDE	UNION CARBIDE		406144	WF193	1100	1150		29.00	FC	R			
M	AM1	MAN101	M				WF193		1100	1150		29.00	FC	R			
M	AS1	PAS101	P	LUKENS			896052		1600		25	4.00	MD	A	1225		25
M	AS1	WAS101	M	LUKENS			83650		1140		25	22.00	FC	R			
M	AS2	PAS201	P	LUKENS				84081	1600		25	4.00	MC	A	1225		25
M	AS2	WAS201	M	LUKENS			4P6052		1140		25	6.00	FC	R			
G	BR	PBR_01	P	LUKENS		COMBUSTION ENG.	192462/3	855032	1650	1750			MD	A	1200	1250	
G	BR	WBR_01	M			COMBUSTION ENG.	COMBUSTION ENG.	192462	855032	1125		25	20.00	FC	R		
M	BV1	PBV101	P	LUKENS		COMBUSTION ENG.	C63171	B69031	1500	1650		4.00	MD	A	1200	1250	
M	BV1	PBV101	P	LUKENS			C63171	B69031	1550	1650		4.00	MD	A	1220	1250	
M	BV1	WBV101	M	LUKENS		COMBUSTION ENG.			1150		25	15.00	FC	R			
M	BV1	WBV101	M				305424		1125	1175		40.00	FC	R			
G	BW1	PBW101	P	LUKENS			04489	PIECE 301	1650		25	5.50	MD	A	1220		10
G	BW2	PBW201	P	LUKENS			04887	PIECE 301	1650		25	5.50	MD	A	1220		10
M	CA8	PCAB01	P					811051	1550	1600		4.00	MC	A	1225		
M	CA8	PCAB02	P					811052	1550	1600		4.00	MC	A	1225		
M	CA7	SASTR	SRN	U.S. STEEL	U.S. STEEL		C40071	ASTM PLATE	1650			4.00	MD	A	1200		
M	CA8	WCA801	M						1150			24.00	FC	R			
M	CB1	FCB101	F	ROTTERDAM			411343	FORGING DS	1679	1697		3.50	MD	A	1220	1247	
M	CB1	FCB101	F				411343	FORGI.	170	1697		3.50	MD	A	1220	1247	
M	CB1	WCB101	M	ROTTERDAM			895075				25	15.00	FC	R			
M	CB1	WCB101	M						1	1165		15.00	FC	R			
C	CC1	PCC103	P	LUKENS		COMBUSTION ENG.	C44611	D72063	1571		25	4.00	MD	A	1225		25
C	CC1	WCC101	M	COMBUSTION ENG.			33A277		1125		25	40.00	FC				
C	CC2	PCC202	P	LUKENS		COMBUSTION ENG.	C52861	D89072	1575		25	4.00	MD	A	1225		25
C	CC2	WCC201	M	COMBUSTION ENG.			10137		1125		25	40.00	FC				
M	CE1	PCX101	P					844063	1600			4.00	MD	A	1225		

NOTE: Due to the large number of fields and limited page space, not all fields are displayed here.

listing of HEAT_PR.dbf

SIZE_2	REFID_2	ACID_2	ID_2	MINTEMP_3	MAXTEMP_3	RANGE_3	HOURS_3	RCM_3	ID_3	TEMP_U	REF_ID	PAGES	NOTES	
4.00	80	A	1100	1250				4.00	AC	T	F	ORNL-4313	5; 19-32	
4.50	AC	T	1150		25		21.00	FC	R	F	MCAP-8957	A-1		
4.50	AC	T	1150		25		21.00	FC	R	F	SWR1-06-8976	9		
4.00	T	1150			25		40.00	FC	R	F	MCAP-9226	A-1		
											MCAP-9228	A-1		
5.00	80	T	1100	1250			40.00	FC	R	F	BAV-1440	3-3		
1 8/18	AC	*	1550	1600			4.50	80	A	F	BAV-1820	4-30	PER CERTIFICATE OF TEST PROBABLY SECOND AUS. AND TEMP.	
1 8/18	AC	T	1100	1150			60.00	FC	R	F	BAV-1543/R2	A-7		
5.00	80	T	1100	1150			40.00	FC	R	F	BAV-1698	3-3		
1 8/18	AC	T	1550	1600			4.50	80	A	F	BAV-1820	4-30	PER CERTIFICATE OF TEST PROBABLY SECOND AUS. AND TEMP.	
1 8/18	AC	T	1100	1150			60.00	FC	R	F	BAV-1543/R2	A-7		
4.00	80	A	1225		25		4.00	FC	T	F	BAV-1698	3-1; 3-6	PER IDENTIFICATION CARD	
4.00	FC	T	1125		25		40.00	FC	R	F	BAV-1820	4-30	PER MELD CERTIFICATION	
											BAV-1440	3-3		
											BAV-1820	4-30		
											BAV-1543/R2	A-7		
4.00	AC	T	1140		25		28.75	FC	R	F	MCAP-9308	A-1		
4.00	AC	T	1140		25		29.00	FC	R	F	MCAP-9308	A-1	PLATES B9605-2 AND B9605-1	
											MCAP-9330	A-1		
											MCAP-9330	A-1	PLATES B408-1 AND B407-2	
4.00	AC	T	1125		25		4.00	80	A	F	GECR-4442	C-2; C-3		
4.00	AC	T	1125	1175	25		40.00	FC	R	F	GECR-4442	C-3; D-1	PLATES 855032 AND 855033	
											MCAP-8457	A-1		
											DOCKET 50-334	TABLE 4		
											MCAP-8457	2-1; A-1		
5.50	AC	T	1150		25		50.00	FC	R	F	DOCKET 50-334	TABLE 1		
5.50	AC	T	1150		25		50.00	FC	R	F	NEJO-24161	6; 7; 13		
4.00	AC	T	1150				24.00	FC	R	F	NEJO-24157/R1	6; 7; 13		
4.00	AC	T	1150				24.00	FC	R	F	MENX/76/64	4-1		
6.00	AC	T	1150								MENX/76/64	4-3		
											MENX/76/64	4-1; 4-4		
											MCAP-10185	4-3		
6.00	AC	T	1140		25		22.00	FC	R	F	MCAP-9734	2-1; A-1		
6.00	AC	T	1115	1165	25		22.00	FC	R	F	MCAP-11527	4-1; 4-4		
											MCAP-9734	A-1		
											MCAP-11527	4-3; 4-4		
4.00	T	1150			25		40.00	FC	R	F	DOCKET 50-317	4; 6; 35		
4.00	T	1150			25		40.00	FC	R	F	DOCKET 50-317	6; 36		
4.00	AC	T	1150				40.00	FC	R	F	DOCKET 50-318	4; 6; 62		
											SWR1-02-4770	6; 63		
												9		

SI
APERTURE
CARD

Also Available On
Aperture Card

9006290154-04

Table 25. Partial listing of WELD_PR.dbf

Table 26. Partial listing of HAZ_PR.dbf

HAZ_PR.dbf: Page 1							
TAG	PLANT_ID	HEAT_ID	HEAT_B	HEAT_W	REF_ID	PAGES	NOTES
W	AD1	HAD101	FAD101	WAD101	WCAP-8957	3-1	W-HAZ OF ANH161
W	AL2	HAL201	PAL201	WAL201	WCAP-9228	2-1	FROM W-HAZ OF PLATE B9405-1
B	AN1	HAN101	PAN101	BAW-1440		3-2	FROM PLATE C-5114-1 LONG.
B	AN1	HAN102	PAN102	BAW-1698		3-2	FROM PLATE C-5114-2 LONG.
C	AN2	HAN201	PAN201	BM1-0584		14	FROM HEAT C-B009-3
W	AS1	HAS101	PAS101	WAS101	WCAP-9308	2-1	FROM W-HAZ OF PLATE B9605-2
W	AS2	HAS201	PAS201	WAS201	WCAP-9330	2-1	FROM W-HAZ OF PLATE R408-1
G	BR	HBR_01	PBR_01	MBR_01	GECR-4442	4	MADE FROM TEST WELD MATERIAL
W	BV1	HBV101	PBV101	WBV101	WCAP-10867	4-1	FROM WITHIN HAZ OF PL. B6903-1
W	BZ1	HBZ101			PE-ME 75/03	2	
W	CAB	HCAB02	PCAB02	WCAB01	WCAP-10185	4-1	FROM WELD HAZ MATERIAL
W	CB1	HCB101	FCB101	WC6101	WCAP-9734	2-1	FROM W-HAZ OF FORGING 05
C	CC1	HCC101	PCC103	WCC102	BM1-1280	7	FROM D7206-3 SIDE OF WELD
C	CC2	HCC201		WCC201	SWR1-7524	6; 29	FROM D8907-1 AND DB907-3
W	CK1	HCK101	PCK101	WCK101	SWR1-02-4770	7	FROM B4406-3 SIDE OF WELD
W	CK2	HCK201	PCK201	WCK201	WCAP-8512	2-1	FROM W-HAZ OF C5521-2
W	CL1	HCL101	PCL101	WCL101	WCAP-9842	2-1	FROM W-HAZ OF R2708-1
G	CPR	HCPRO1		WCPR01	NDE-103-0986	3-6	FROM G2802-1 AND G2802-2
B	CR3	HCR301	PCR301		BAW-1898	3-2	FROM C4344-1, HEAT NN
B	CR3	HCR302	PCR302		BAW-1898	3-2	FROM C4344-2, HEAT PP
W	CTY	HCTY04		MCTY01	WCAP-7036	5	FROM W9B07-1 AND W9B07-8
B	DB1	HDB101	FDB101		BAW-1882	3-2	FROM HEAT BCC241
B	DB1	HDB102	FDB102		BAW-1882	3-2	FROM HEAT AKJ233
W	DC1	HDC101	PDC103	WDC101	WCAP-8465	2-1	FROM W-HAZ OF B4106-3
W	DC2	HDC201	PDC201	WDC201	WCAP-8783	2-1	FROM W-HAZ OF B5454-1
G	DR1	HDR101			NEDC-12585	1	
G	DR2	HDR201		WDR201	BCL-585-10	5; 8	ESW HAZ
G	DR2	HDR202		WDR202	BCL-585-10	5; 8	SAW HAZ
G	DR3	HDR301	PDR301	WDR301	SWR1-06-7484-003	5	ESW HAZ
G	DR3	HDR302	PDR301	WDR302	SWR1-06-7484-003	5	SAW HAZ
W	FA1	HFA101	PFA101	WFA101	WCAP-8810	2-1	FROM W-HAZ OF B6919-1
W	FA2	HFA201	PFA201	WFA201	WCAP-8956	2-1	FROM W-HAZ OF B7212-1
C	FC1	HFC101		WFC102	TR-O-MCM-001/R1	4	FROM D4B02-2 AND D4B02-3
G	GAR	HGAR01		WGAR01	DPT/SN/041/R/79	3	
W	GIN	HGIN01	FGIN01	WGIN01	WCAP-7254	4	FROM W-HAZ OF 125P666
G	HA1	HHA101		WHA101	NEDC-30997	3-6; 3-19	FROM G-8404-2 AND G-8404-1
W	HB2	HHB201		WHB201	WCAP-7373	4	FROM W1D201-1 AND W1D201-3
G	HM3	HHM301			DOCKET 50-133	TABLE III	
W	IP2	HIP201	PIP203	WIP201	WCAP-7323	4	FROM HAZ OF B2002-3
W	IP3	HIP304	PIP304		WCAP-8475	2-1	FROM W-HAZ OF B2803-3
W	KO1	HKO101		WKO101	SWR1-17-5759-201	7	FROM W-HAZ
W	KWE	HKWE01	FKWE01	WKWE01	WCAP-8107	2-1	FROM W-HAZ OF 122X20BVA1
A	LAC	HLAC01	PLAC03	WLAC01	NEDC 8	96; 97; 99	FROM NP-1056
W	LR2	HLR201	FLR201	WLK201	WCAP-9329	2-1	FROM W-HAZ OF 123J39B
W	MC1	HMC101	PMC102	WMC101	WCAP-9195	2-1	FROM W-HAZ OF B5012-2
W	MC2	HMC201	FMC201	WMC201	WCAP-11029	4-1; 4-2; A-7	FROM W-HAZ OF FORGING 05
G	ML1	HML101	PML102	WML101	NEDC-30833	3-2; 3-3; 3-17	FROM PLATE G-2002-6
C	ML2	HML201	PML201	WML202	TR-N-MCM-008	3; 4; 8	FROM C-506-1 AND C-506-3
G	MON	HMON01	PMON01	WMON01	BM1-0483	6; 7	
C	MY	HY_01		HY_02	CR 75-317	8; A-4	FROM D-8406-1 AND D-8406-2
W	NA1	HNA101	FNA101	WNA101	WCAP-8771	2-1; A-1	FROM W-HAZ OF FORGING 03
W	NA2	HNA201	FNA201	WNA201	WCAP-8772	2-1; A-1	FROM W-HAZ OF FORGING 04
G	NN1	NNM101			DOCKET 50-220	11-1; 111-2	
B	OC1	HOC101	POC101		BAW-1421/R1	3-1; 3-2	FROM HEAT C2800-2
B	OC1	HOC102	POC102		BAW-1436	3-1; 3-2	FROM HEAT C3265-1
B	OC2	HOC201	POC201		BAW-1437	A-4	FROM HEAT AAW-163
B	OC2	HOC202	POC202		BAW-1437	A-4	FROM HEAT AWG-164

Table 27. Listing of REF_TITL.dbf

REF_TITL.dbf: Page 1	CONT	REF_TITLE
ACNP-66513	1	LaCrosse Boiling Water Reactor, Reactor Vessel Material Surveillance Program for
ACNP-66513	2	Evaluation of Radiation Effects, ACNP-66513, Allis-Chalmers, Bethesda, MD,
ACNP-66513	3	February 1966.
ASTM DS54	1	J. R. Hawthorne, Radiation Effects Information Generated on the ASTM Reference
ASTM DS54	2	Correlation Monitor Steels, ASTM DS54, American Society for Testing and
ASTM DS54	3	Materials, Philadelphia, PA, 1974.
ASTM STP 481	1	L. E. Steele and C. Z. Serpan, Jr., Analysis of Reactor Vessel Radiation Effects
ASTM STP 481	2	Surveillance Programs, Chapters 1 and 4, ASTM STP 481, American Society for
ASTM STP 481	3	Testing and Materials, Philadelphia, PA, 1966.
BAW-1421	1	A. L. Lowe, Jr., L. A. Hessler, H. S. Palme, and C. F. Zurlippe, Analysis of
BAW-1421	2	Capsule OCI-F from Duke Power Company Oconee Unit 1 Reactor Vessel Materials
BAW-1421	3	Surveillance Program, BAW-1421, Babcock & Wilcox, Lynchburg, VA, August 1975.
BAW-1421/R1	1	A. L. Lowe, Jr., L. A. Hessler, H. S. Palme, and C. F. Zurlippe, Analysis of
BAW-1421/R1	2	Capsule OCI-F from Duke Power Company Oconee Unit 1 Reactor Vessel Materials
BAW-1421/R1	3	Surveillance Program, BAW-1421, Rev. 1, Babcock & Wilcox, Lynchburg, VA,
BAW-1436	4	September 1975.
BAW-1436	1	A. L. Lowe, Jr., et al., Analysis of Capsule OCI-E Duke Power Company Oconee
BAW-1436	2	Nuclear Station Unit 1 Reactor Vessel Materials Surveillance Program, BAW-1436,
BAW-1436	3	Babcock & Wilcox, Lynchburg, VA, September 1977.
BAW-1437	1	Analysis of Capsule OCII-C from Duke Power Company Oconee Nuclear Station,
BAW-1437	2	Unit 2, Reactor Vessel Materials Surveillance Program, BAW-1437, Babcock &
BAW-1437	3	Wilcox, Lynchburg, VA, May 1977.
BAW-1438	1	A. L. Lowe, Jr., et al., Analysis of Capsule OCIII-A from Duke Power Company
BAW-1438	2	Oconee Nuclear Station Unit 3, BAW-1438, Babcock & Wilcox, Lynchburg, VA, July
BAW-1438	3	1977.
BAW-1439	1	A. L. Lowe, Jr., et al., Analysis of Capsule THI-1E from Metropolitan Edison
BAW-1439	2	Company Three Mile Island Nuclear Station - Unit 1, Reactor Vessel Materials
BAW-1439	3	Surveillance Program, BAW-1439, Babcock & Wilcox Co., Lynchburg, VA,
BAW-1439	4	January 1977.
BAW-1440	1	A. L. Lowe, Jr., et al., Analysis of Capsule ANI-E from Arkansas Power & Light
BAW-1440	2	Company Arkansas Nuclear One - Unit 1, Reactor Vessel Materials Surveillance
BAW-1440	3	Program, BAW-1440, Babcock & Wilcox, Lynchburg, VA, April 1977.
BAW-1543/R2	1	A. L. Lowe, K. E. Moore, and J. D. Aadland, Integrated Reactor Vessel Material
BAW-1543/R2	2	Surveillance Program, BAW-1543, Rev. 2, Babcock & Wilcox, Lynchburg, VA,
BAW-1543/R2	3	February 1984.
BAW-1638	1	A. L. Lowe, Jr., W. A. Pavinich, J. K. Schmotzer, and C. L. Whitmarsh, Analysis
BAW-1638	2	of Capsule V Virginia Electric & Power Company North Anna Unit No. 1 Reactor
BAW-1638	3	Vessel Materials Surveillance Program, BAW-1638, Babcock & Wilcox, Lynchburg,
BAW-1638	4	VA, March 1981.
BAW-1679/R1	1	A. L. Lowe, Jr. et al., Analyses of Capsule CR3-B, Florida Power Corporation,
BAW-1679/R1	2	Crystal River Unit 3, Reactor Vessel Materials Surveillance Program, BAW-1679,
BAW-1679/R1	3	Rev. 1, Babcock & Wilcox, Lynchburg, VA, June 1982.
BAW-1697	1	A. L. Lowe, Jr., et al., Analysis of Capsule OCIII-B from Duke Power Company
BAW-1697	2	Oconee Nuclear Station Unit 3 Reactor Vessel Materials Surveillance Program,
BAW-1697	3	BAW-1697, Babcock & Wilcox Co., Lynchburg, VA, October 1981.
BAW-1698	1	A. L. Lowe, Jr., et al., Analysis of Capsule ANI-B from Arkansas Power & Light
BAW-1698	2	Company's Arkansas Nuclear One, Unit 1, Reactor Vessel Materials Surveillance
BAW-1698	3	Program, BAW-1698, Babcock & Wilcox, Lynchburg, VA, November 1981.
BAW-1699	1	Analysis of Capsule OCII-A from Duke Power Company Oconee Nuclear Station, Unit
BAW-1699	2	2, Reactor Vessel Materials Surveillance Program, BAW-1699, Babcock & Wilcox,
BAW-1699	3	Lynchburg, VA, December 1981.
BAW-1701	1	A. L. Lowe, Jr., et al., Analyses of Capsule TE1-F, The Toledo Edison Company,
BAW-1701	2	Davis-Besse Nuclear Power Station Unit 1, Reactor Vessel Material Surveillance
BAW-1701	3	Program, BAW-1701, Babcock & Wilcox, Lynchburg, VA, January 1982 (Rev. 1, Toledo
BAW-1701	4	Edison, August 1982).
BAW-1702	1	A. L. Lowe, Jr., et al., Analyses of Capsule RS1-B, Sacramento Municipal Utility
BAW-1702	2	District, Rancho Seco Unit 1, Reactor Vessel Materials Surveillance Program,
BAW-1702	3	BAW-1702, Babcock & Wilcox, Lynchburg, VA, February 1982.
BAW-1718	1	A. L. Lowe, Jr., J. D. Aadland, W. A. Pavinich, and C. L. Whitmarsh, Fracture
BAW-1718	2	Toughness Test Results from Capsule CR3-B Florida Power Corporation Crystal
BAW-1718	3	River Unit 3, Reactor Vessel Material Surveillance Program, BAW-1718, Babcock &
BAW-1718	4	Wilcox Co., Lynchburg, VA, March 1982.
BAW-1719	1	A. L. Lowe, Jr., J. D. Aadland, J. E. Ewing, and W. A. Pavinich, Fracture
BAW-1719	2	Toughness Test Results from Capsule T11-F, the Toledo Edison Company,
BAW-1719	3	Davis-Besse Nuclear Power Station Unit 1, Reactor Vessel Material Surveillance
BAW-1719	4	Program, BAW-1719, Babcock & Wilcox, Lynchburg, VA, March 1982.
BAW-1792	1	A. L. Lowe, Jr., et al., Analyses of Capsule RS1-D, Sacramento Municipal Utility
BAW-1792	2	District, Rancho Seco Unit 1, Reactor Vessel Material Surveillance Program,
BAW-1792	3	BAW-1792, Babcock & Wilcox Co., Lynchburg, VA, October 1983.
BAW-1794	1	A. L. Lowe, Jr., et al., Analysis of Capsule V Virginia Electric & Power Company
BAW-1794	2	North Anna Unit No. 2 Reactor Vessel Materials Surveillance Program, BAW-1794,
BAW-1794	3	Babcock & Wilcox, Lynchburg, VA, October 1983.

Table 27 (continued)

REF_TITLE.d0f: Page 2	REF_ID	CONT	REF_TITLE
BAW-1803	1	A. S. Heller and A. L. Lowe, Jr., Correlations for Predicting the Effects of	
BAW-1803	2	Neutron Radiation on Linde 80 Submerged-Arc Welds, BAW-1803, Babcock & Wilcox	
BAW-1803	3	Co., Lynchburg, VA, January 1984.	
BAW-1820	1	J. D. Aadland, Babcock & Wilcox Owner's Group 177-Fuel Assembly Reactor Vessel	
BAW-1820	2	and Surveillance Program Materials Information, BAW-1820, Babcock & Wilcox,	
BAW-1820	3	Lynchburg, VA, December 1984.	
BAW-1834	1	A. L. Lowe, Jr., et al., Analyses of Capsule TE1-B, The Toledo Edison Company,	
BAW-1834	2	Davis-Besse Nuclear Power Station Unit 1, Reactor Vessel Material Surveillance	
BAW-1834	3	Program, BAW-1834, Babcock & Wilcox, Lynchburg, VA, May 1984.	
BAW-1836	1	A. L. Lowe, Jr., et al., Analyses of Capsule AN1-A Arkansas Power & Light	
BAW-1836	2	Company, Arkansas Nuclear One, Unit 1, Reactor Vessel Material Surveillance	
BAW-1836	3	Program, BAW-1836, Babcock & Wilcox, Lynchburg, VA, July 1984.	
BAW-1837	1	J. D. Aadland, et al., Analysis of Capsule OGI-A, Duke Power Company Occonee	
BAW-1837	2	Nuclear Station, Unit 1, BAW-1837, Babcock & Wilcox, Lynchburg, VA, August 1984.	
BAW-1880	1	A. L. Lowe et al., Analysis of Capsule W-83 Florida Power and Light Company	
BAW-1880	2	St. Lucie Plant Unit No. 2 Reactor Vessel Material Surveillance Program,	
BAW-1880	3	BAW-1880, Babcock & Wilcox, Lynchburg, VA, September 1985.	
BAW-1882	1	A. L. Lowe, Jr., et al., Analyses of Capsule TE1-A, The Toledo Edison Company,	
BAW-1882	2	Davis-Besse Nuclear Power Station Unit 1, Reactor Vessel Material Surveillance	
BAW-1882	3	Program, BAW-1882, Babcock & Wilcox, Lynchburg, VA, September 1985.	
BAW-1898	1	A. L. Lowe et al., Analysis of Capsule CR3-C Florida Power Corporation Crystal	
BAW-1898	2	River Unit 3 Reactor Vessel Material Surveillance Program, BAW-1898, Babcock &	
BAW-1898	3	Wilcox, Lynchburg, VA, March 1986.	
BAW-1899	1	A. L. Lowe et al., Analysis of Capsule CR3-D Florida Power Corporation Crystal	
BAW-1899	2	River Unit 3 Reactor Vessel Material Surveillance Program, BAW-1899, Babcock &	
BAW-1899	3	Wilcox, Lynchburg, VA, March 1986.	
BAW-2045	1	A. L. Lowe et al., Analysis of Capsule CR3-F Florida Power Corporation Crystal	
BAW-2049	2	River Unit-3, Reactor Vessel Material Surveillance Program, BAW-2049, Babcock &	
BAW-2049	3	Wilcox, Lynchburg, VA, September 1988.	
BCL-382-BS-1/R1	1	M. P. Mahanah, L. M. Lowry, R. D. Wooton, and M. P. Feiley, Final Report on	
BCL-382-BS-1/R1	2	Examination, Testing, and Evaluation of Specimens from the 210-degree Irradiated	
BCL-382-BS-1/R1	3	Pressure Vessel Surveillance Capsule for the Oyster Creek Nuclear Generating	
BCL-382-BS-1/R1	4	Station, BCL-382-BS-1, Rev. 1, Battelle Columbus Laboratories, Columbus, OH,	
BCL-382-BS-1/R1	5	October 1985.	
BCL-585-10	1	E. O. Fromm et al., Final Report on Dresden Nuclear Plant Reactor Pressure	
BCL-585-10	2	Vessel Surveillance Program: Unit No. 2 Capsule Basket Assembly No. 5,	
BCL-585-10	3	BCL-585-10, Battelle Columbus Laboratories, Columbus, OH, May 8, 1979.	
BCL-585-12	1	J. S. Perrin, et al., Final Report on Palisades Nuclear Plant Reactor Pressure	
BCL-585-12	2	Vessel Surveillance Program: Capsule A-240, BCL-585-12, Battelle Columbus	
BCL-585-12	3	Laboratories, Columbus, OH, March 1979.	
BCL-585-21	1	J. S. Perrin, et al., Final Report on Maine Yankee Nuclear Plant Reactor	
BCL-585-21	2	Pressure Vessel Surveillance Program: Capsule 263, BCL-585-21, Battelle	
BCL-585-21	3	Columbus Laboratories, Columbus, OH, December 1980.	
BCL-585-3	1	J. S. Perrin et al., Final Report on Dresden Nuclear Plant Reactor Pressure	
BCL-585-3	2	Vessel Surveillance Program: Unit No. 2 Neutron Dosimeter Monitor, Unit No. 2	
BCL-585-3	3	Capsule Basket Assembly No. 2, and Unit No. 3 Capsule Basket Assembly No. 12,	
BCL-585-3	4	BCL-585-3, Battelle Columbus Laboratories, Columbus, OH, September 15, 1977.	
BCL-585-4	1	J. S. Perrin, D. R. Farmelo, R. G. Jung, and E. O. Fromm, Final Report on Zion	
BCL-585-4	2	Nuclear Plant Reactor Pressure Vessel Surveillance Program: Unit No. 1 Capsule	
BCL-585-4	3	T and Unit No. 2 Capsule U, BCL-585-4, Battelle Columbus Laboratories, Columbus,	
BCL-585-4	4	OH, March 1978.	
BCL-585-84-3	1	L. M. Lowry, et al., Final Report on Examination, Testing, and Evaluation of	
BCL-585-84-3	2	Irradiated Pressure Vessel Surveillance Specimens from the Vermont Yankee	
BCL-585-84-3	3	Nuclear Power Station, BCL-585-84-3, Battelle Columbus Laboratories, Columbus,	
BCL-585-84-3	4	OH, May 1984.	
BNI-0275	1	J. S. Perrin and L. M. Lowry, Final Report on Quad Cities Nuclear Plant Unit No.	
BNI-0275	2	1 and Unit No. 2 Reactor Pressure Vessel Surveillance Program: Unirradiated	
BNI-0275	3	Mechanical Properties, Battelle Columbus Labs., Columbus, OH, February 1975.	
BNI-0275/DR3	1	J. S. Perrin and L. M. Lowry, Final Report on Dresden Nuclear Plant Unit No. 3	
BNI-0275/DR3	2	Vessel Surveillance Programs: Unirradiated Mechanical Properties, Battelle	
SMI-0275/DR3	3	Columbus Laboratories, Columbus, OH, February 15, 1975.	
BNI-0372	1	J. S. Perrin, J. W. Scheckerd, and V. G. Scotti, Final Report on Examination and	
BNI-0372	2	Evaluation of Capsule F for the Connecticut Yankee Reactor Pressure Vessel	
BNI-0372	3	Surveillance Program, Part A: Primary Investigations and Part B: Supplementary	
BNI-0372	4	Activities, Battelle Columbus Laboratories, Columbus, OH, March 1972.	
BNI-0375/DR3	1	J. S. Perrin et al., Final Report on Dresden Nuclear Plant Unit No. 3 Reactor	
BNI-0375/DR3	2	Pressure Vessel Surveillance Program: Capsule Basket No. 13, Capsule Basket	
BNI-0375/DR3	3	No. 14, and Neutron Dosimeter Monitor, Battelle Columbus Laboratories, Columbus,	
BNI-0375/DR3	4	OH, March 1, 1975.	
BNI-0375/QC1	1	J. S. Perrin, D. R. Farmelo, R. S. Denning, and L. M. Lowry, Final Report on	
BNI-0375/QC1	2	Quad Cities Nuclear Plant Unit No. 1, Reactor Pressure Vessel Surveillance	
BNI-0375/QC1	3	Program: Capsule Basket No. 2, Capsule Basket No. 3, and Neutron Dosimeter	
BNI-0375/QC1	4	Monitor, Battelle Columbus Laboratories, Columbus, OH, March 1975.	

Table 27 (continued)

REF_TITLE	CONT	REF_TITLE
BMI-0483	1	L. M. Lowry, et al., Interim Report on Examination, Testing, and Evaluation of Irradiated Pressure Vessel Surveillance Specimens from the Monticello Nuclear Generating Plant, Battelle Columbus Laboratories, Columbus, OH, April 1983.
BMI-0483	2	
BMI-0483	3	
BMI-0584	1	L. M. Lowry, et al., Summary Report on Examination, Testing, and Evaluation of Irradiated Pressure Vessel Surveillance Specimens from the Arkansas Nuclear One Unit 2 Generating Plant, Battelle Memorial Institute, Columbus, OH, May 1984.
BMI-0584	2	
BMI-0673	1	J. S. Perrin, J. W. Scheckherd, D. R. Farnelo, and L. M. Lowry, Final Report on Point Beach Nuclear Plant Unit No. 1 Pressure Vessel Surveillance Program:
BMI-0673	2	
BMI-0673	3	Evaluation of Capsule V, Battelle Columbus Laboratories, Columbus, OH, June 1973
BMI-0675	1	J. S. Perrin, et al., Final Report on Point Beach Nuclear Plant Unit No. 2 Pressure Vessel Surveillance Program: Evaluation of Capsule V, Battelle Columbus Laboratories, Columbus, OH, June 1975.
BMI-0675	2	
BMI-0675	3	
BMI-0975/QC2	1	J. S. Perrin et al., Final Report on Quad Cities Nuclear Plant Unit No. 2 Reactor Pressure Vessel Surveillance Program: Capsule Basket No. 12 and Capsule Basket No. 13, Battelle Columbus Laboratories, Columbus, OH, September 19, 1975.
BMI-0975/QC2	2	
BMI-0975/QC2	3	
BMI-0975/SU2	1	J. S. Perrin, et al., Final Report on Surry Unit No. 2, Pressure Vessel Irradiation Capsule Program: Examination and Analysis of Capsule X,
BMI-0975/SU2	2	
BMI-0975/SU2	3	Battelle Columbus Laboratories, Columbus, OH, September 1975.
BMI-1070	1	D. R. Ireland and V. G. Scotti, Final Report on Examination and Evaluation of Capsule A for the Connecticut Yankee Reactor Pressure Vessel Surveillance Program, Battelle Memorial Institute, Columbus, OH, October 1970.
BMI-1070	2	
BMI-1070	3	
BMI-1280	1	J. S. Perrin et al., Final Report on Calvert Cliffs Unit No. 1 Nuclear Plant Reactor Pressure Vessel Surveillance Program: Capsule 263, Battelle Columbus Laboratories, Columbus, OH, December 15, 1980.
BMI-1280	2	
BMI-1280	3	
CR 75-269	1	J. W. Scheckherd and R. A. Wullaert, Unirradiated Mechanical Properties of Maine Yankee Nuclear Pressure Vessel Materials, CR 75-269, Effects Technology, Inc., Santa Barbara, CA, February 1975.
CR 75-269	2	
CR 75-269	3	
CR 75-317	1	R. A. Wullaert and J. W. Scheckherd, Evaluation of the First Maine Yankee Accelerated Surveillance Capsule, CR 75-317, Effects Technology, Inc., Santa Barbara, CA, August 1975.
CR 75-317	2	
CR 75-317	3	
DOCKET 50-133	1	Mechanical Properties of Irradiated Reactor Material Surveillance Specimens Humboldt Bay Power Plant, Unit No. 3, Docket No. 50-133, Pacific Gas and Electric Company, San Francisco, CA, April 1972.
DOCKET 50-133	2	
DOCKET 50-133	3	
DOCKET 50-206	1	K. P. Baskin, Response to NRC inquiries regarding Docket No. 50-206, Provisional Operating License No. DPR-13 Reactor Vessel Material Surveillance Program San Onofre Nuclear Generating Station, Unit 1, Southern California Edison Company, Rosemead, CA, November 10, 1977.
DOCKET 50-206	2	
DOCKET 50-206	3	
DOCKET 50-220	1	C. V. Hangan, Report of the Examination, Testing, and Evaluation of Irradiated Reactor Vessel Surveillance Specimens from Nine Mile Point Unit 1, Docket No. 50-220, Niagara Mohawk Power Corporation, Syracuse, NY, August 1985.
DOCKET 50-220	2	
DOCKET 50-220	3	
DOCKET 50-245	1	W. G. Counsil and W. F. Fee, Response to NRC inquiries regarding Millstone Nuclear Power Station, Unit No. 1 Reactor Vessel Materials and Surveillance Program, Docket No. 50-245, Northeast Utilities, Hartford, CT, July 31, 1978.
DOCKET 50-245	2	
DOCKET 50-245	3	
DOCKET 50-247	1	W. J. Cahill, Jr., Response to NRC inquiries regarding Indian Point Unit 2 Reactor Vessel Material Surveillance Program, Docket No. 50-247, Consolidated Edison Company of New York, Inc., New York, NY, March 29, 1978.
DOCKET 50-247	2	
DOCKET 50-247	3	
DOCKET 50-247	4	
DOCKET 50-255	1	Summary of Findings Relative to Palisades Plant Reactor Vessel Materials, Attachment III Consumers Power Company Palisades Plant Docket 50-255, June 14, 1985.
DOCKET 50-255	2	
DOCKET 50-255	3	
DOCKET 50-261	1	B. J. Furr, Response to NRC inquiries regarding H. B. Robinson Steam Electric Plant, Unit No. 2 Docket No. 50-261 License No. DPR-23 Reactor Vessel Material Surveillance Program Data, Carolina Power & Light Company, Raleigh, NC, October 19, 1977.
DOCKET 50-261	2	
DOCKET 50-261	3	
DOCKET 50-261	4	
DOCKET 50-272	1	F. P. Librizzi, Response to NRC inquiries regarding Reactor Vessel Materials No. 1 Unit Salem Nuclear Generating Station Docket No. 50-272, Public Service Electric and Gas Company, Newark, NJ, November 16, 1977.
DOCKET 50-272	2	
DOCKET 50-272	3	
DOCKET 50-272	4	
DOCKET 50-280	1	J. E. Perrin, et al., Final Report on Surry Unit No. 1, Pressure Vessel Irradiation Capsule Program: Examination and Analysis of Capsule T, Docket 50280-462, Battelle Columbus Laboratories, Columbus, OH, June 1975.
DOCKET 50-280	2	
DOCKET 50-280	3	
DOCKET 50-280	4	
DOCKET 50-280/1	1	C. M. Stallings, Response to NRC inquiries regarding Surry Unit 1 and Unit 2 Reactor Vessel Material Surveillance Program Docket Nos. 50-280 and 50-281.
DOCKET 50-280/1	2	
DOCKET 50-281	1	C. M. Stallings, Response to NRC inquiries regarding Surry Unit 1 and Unit 2 Reactor Vessel Material Surveillance Program Docket Nos. 50-280 and 50-281.
DOCKET 50-281	2	
DOCKET 50-282	1	L. O. Mayer, Response to NRC inquiries regarding Prairie Island Nuclear Generating Plant, Docket Nos. 50-282 and 50-306, Reactor Vessel Material Surveillance Program (Units 1 and 2), Northern States Power Company,
DOCKET 50-282	2	
DOCKET 50-282	3	
DOCKET 50-282	4	
DOCKET 50-295	1	Minneapolis, MN, October 31, 1977.
DOCKET 50-295	2	
DOCKET 50-304	1	Zion Station Unit 1, Attachment I, NRC Docket No. 50-295, March 1973.
DOCKET 50-304	2	
DOCKET 50-304	3	
DOCKET 50-304	4	

Table 27 (continued)

REF_TITLE.dbf: Page 4

REF_ID	CONT	REF_TITLE
DOCKET 50-305/A	1	E. W. James, Preliminary response to NRC inquiries regarding Keweenaw Nuclear Power Plant Docket 50-305, Operating License DPR-43 Request for Information
DOCKET 50-305/A	2	Reactor Vessel Material Surveillance Program, Wisconsin Public Service Corp.
DOCKET 50-305/A	3	Green Bay, WI, October 11, 1977.
DOCKET 50-305/C	1	E. W. James, Response to NRC inquiries regarding Keweenaw Nuclear Power Plant
DOCKET 50-305/C	2	Docket 50-305 Operating License DPR-43 Reactor Vessel Material Surveillance
DOCKET 50-305/C	3	Program, Wisconsin Public Service Co.p., Green Bay, WI, February 1, 1978.
DOCKET 50-306	1	L. O. Mayer, Response to NRC inquiries regarding Prairie Island Nuclear
DOCKET 50-306	2	Generating Plant, Docket Nos. 50-282 and 50-306, Reactor Vessel Material
DOCKET 50-306	3	Surveillance Program (units 1 and 2), Northern States Power Company,
DOCKET 50-306	4	Minneapolis, MN, October 31, 1977.
DOCKET 50-317	1	A. E. Jundvall, Jr., Response to NRC inquiries regarding Calvert Cliffs Nuclear
DOCKET 50-317	2	Power Plant Unit No. 1 and 2, Docket No. 50-317 and 50-318 Reactor Vessel
DOCKET 50-317	3	Material Surveillance Program, Baltimore Gas and Electric Company, Baltimore,
DOCKET 50-317	4	MD, December 29, 1977.
DOCKET 50-318	1	A. E. Jundvall, Jr., Response to NRC inquiries regarding Calvert Cliffs Nuclear
DOCKET 50-318	2	Power Plant Unit No. 1 and 2, Docket No. 50-317 and 50-318 Reactor Vessel
DOCKET 50-318	3	Material Surveillance Program, Baltimore Gas and Electric Company, Baltimore,
DOCKET 50-318	4	MD, December 29, 1977.
DOCKET 50-334	1	C. W. Dunn, Response to NRC inquiries regarding Beaver Valley Power Station,
DOCKET 50-334	2	Unit No. 1, Docket 50-334, Reactor Vessel Material Surveillance Program,
DOCKET 50-334	3	Duquesne Light, Pittsburgh, PA, July 21, 1977.
DOCKET 50-335	1	R. E. Uhrig, Response to NRC inquiries regarding Reactor Vessel Materials of
DOCKET 50-335	2	Construction and Surveillance Programs for the St. Lucie Unit 1 Reactor Vessel
DOCKET 50-335	3	Material Surveillance Program, Florida Power & Light Company, Miami, FL,
DOCKET 50-335	4	September 30, 1977.
DOCKET 50-336	1	D. C. Switzer, Response to NRC inquiries regarding Millstone Nuclear Power
DOCKET 50-336	2	Station, Unit No. 2 Reactor 1, Pressure Vessel (RPV) Material Surveillance Program,
DOCKET 50-336	3	Docket 50-336, Northeast Nuclear Energy Company, Hartford, CT, December 9, 1977.
DOCKET 50-338	1	C. M. Stallings, Response to NRC inquiries regarding Pressure Vessel Fracture
DOCKET 50-338	2	Toughness Properties North Anna Power Station Unit Nos. 1 and 2, Docket Nos.
DOCKET 50-338	3	50-338 and 50-339, Virginia Electric and Power Company, Richmond, VA,
DOCKET 50-338	4	December 11, 1978.
DOCKET 50-339	1	C. M. Stallings, Response to NRC inquiries regarding Pressure Vessel Fracture
DOCKET 50-339	2	Toughness Properties North Anna Power Station Unit Nos. 1 and 2, Docket Nos.
DOCKET 50-339	3	50-338 and 50-339, Virginia Electric and Power Company, Richmond, VA,
DOCKET 50-339	4	December 11, 1978.
DOCKET 50-344	1	D. J. Broehl, Response to NRC inquiries regarding Trojan Reactor Vessel Material
DOCKET 50-344	2	Surveillance Program, Docket 50-344, Portland General Electric Company,
DOCKET 50-344	3	Portland, OR, May 22, 1978.
DOCKET 50-409	1	J. P. Madgett, Response to NRC inquiries regarding Dairyland Power Cooperative
DOCKET 50-409	2	LaCrosse Boiling Water Reactor (LACBWR) Provisional Operating License No. DPR-45
DOCKET 50-409	3	Reactor Vessel Material Surveillance Program, Docket No. 50-409, Dairyland Power
DOCKET 50-409	4	Cooperative, LaCrosse, WI, December 12, 1977.
DPT/SN/041/R/79	1	M. Galliani et al., Garigliano Nuclear Power Plant Pressure Vessel Surveillance
DPT/SN/041/R/79	2	Program Updating to 7th Operation Cycle, DPT/SN/041/R/79, Ente Nazionale per
DPT/SN/041/R/79	3	l'Energia Elettrica, Rome, Italy, June 1979.
EPRI NP-2428	1	P. McConnell et al., Irradiated Nuclear Pressure Vessel Steel Data Base,
EPRI NP-2428	2	EPRI NP-2428, Electric Power Research Institute, Palo Alto, CA, June 1982.
FP-RA-1	1	T. R. Hager, et al., Analysis of Capsule V from the Rochester Gas and Electric
FP-RA-1	2	R. E. Ginn Unit No. 1 Reactor Vessel Radiation Surveillance Program, FP-RA-1,
FP-RA-1	3	Westinghouse Electric Corporation, Pittsburgh, PA, March 1973.
GECR-4442	1	F. A. Brandt, Reactor Pressure Vessel Material Surveillance Program at the
GECR-4442	2	Consumers Power Company Big Rock Point Nuclear Plant, GECR-4442, General
GECR-4442	3	Electric, San Jose, CA, December 1963.
GECR-5492	1	F. A. Brandt, Humboldt Bay Power Plant Unit No. 3, Reactor Vessel Steel
GECR-5492	2	Surveillance Program, GECR-5492, General Electric, San Jose, CA, May 1967.
JBE 12	1	C. Z. Serpan, Jr. and J. R. Hawthorne, Yankee Reactor Pressure Vessel
JBE 12	2	Surveillance: Notch Ductility Performance of Vessel Steel and Maximum Service
JBE 12	3	Fluence Determined from Exposure During Cores II, III, and IV, J. Basic Eng.,
JBE 12	4	pp. 897-910, December 1967.
MDE-103-0986	1	T. A. Caine, Cooper Nuclear Station Reactor Vessel Surveillance Materials
MDE-103-0986	2	Testing and Fracture Toughness Analysis, MDE-103-0986, General Electric Company,
MDE-103-0986	3	San Jose, CA, May 1987.
N-NLM-011	1	J. J. Kozioł, Program for Irradiation Surveillance of Millstone Point Unit 2
N-NLM-011	2	Reactor Vessel Materials, N-NLM-011, Combustion Engineering, Inc., Windsor, CT,
N-NLM-011	3	October 15, 1970.

Table 27 (continued)

REF_TITLE.dbf: Page 5	CONT	REF_TITLE
REF_ID		
NEAD 11(3)	1	C. Z. Serpan, Jr. and H. E. Watson, Mechanical Property and Neutron Spectral
NEAD 11(3)	2	Analyses of the Big Rock Point Reactor Pressure Vessel, Naval Research
NEAD 11(3)	3	Laboratory, Washington, DC, Nucl. Eng. & Design 11(3), pp. 393-415, April 1970.
NEAD 17	1	J. R. Hawthorne, Postirradiation Dynamic Tear and Charpy-V Performance of
NEAD 17	2	12-in.-Thick A533-B Steel Plates and Weld Metals, Nucl. Eng. and Design, 17
NEAD 17	3	pp. 116-130, 1971.
NEAD 26	1	R. Galliani and C. Z. Serpan, Neutron Embrittlement Surveillance of the
NEAD 26	2	Garigliano Reactor Vessel Steel, Nucl. Eng. and Design 26, pp. 313-325, 1974.
NEAD 8	1	C. Z. Serpan, Jr., Neutron Radiation Embrittlement of LaCrosse Reactor Vessel
NEAD 8	2	Steel and Weldment: Properties and Directionality Considerations, Nucl. Eng.
NEAD 8	3	and Design 8, pp. 95-107, 1968.
NEDC-12585	1	G. F. Rieger and G. H. Henderson, Dresden Nuclear Power Station Unit One and
NEDC-12585	2	Unit Two Mechanical Properties of Irradiated Reactor Vessel Material Surveillance Specimens, NEDC-12585, General Electric, Pleasanton, CA, May 1975.
NEDC-12585	3	T. A. Caine, Millstone Nuclear Power Station, Unit 1, Reactor Pressure Vessel
NEDC-30833	1	Surveillance Materials Testing and Fracture Toughness Analysis, NEDC-30833,
NEDC-30833	2	DRF B13-01285, General Electric Company, San Jose, CA, December 1984.
NEDC-30833	3	T. A. Caine, Edwin I. Hatch Nuclear Power Plant, Unit 1, Reactor Pressure Vessel
NEDC-30997	1	Surveillance Materials Testing and Fracture Toughness analysis, NEDC-30997,
NEDC-30997	2	General Electric Company, San Jose, CA, October 1985.
NEDC-30997	3	Brunswick Steam Electric Plant Unit 2, Information on Reactor Vessel Material
NEDO-24157/R1	1	Surveillance Program, NEDO-24157, Rev. 1, General Electric Corporation, San
NEDO-24157/R1	2	Jose, CA, December 1978.
NEDO-24161	1	Brunswick Steam Electric Plant Unit 1, Information on Reactor Vessel Material
NEDO-24161	2	Surveillance Program, NEDO-24161, General Electric Company, San Jose, CA,
NEDO-24161	3	November 1978.
NEDO-24197	1	Monticello Nuclear Generating Plant Information on Reactor Vessel Material
NEDO-24197	2	Surveillance Program, NEDO-24197, General Electric Company, San Jose, CA, June
NEDO-24197	3	1979.
MUREG/CR-3319	1	W. N. McElroy, Ed., LWR Pressure Vessel Surveillance Dosimetry Improvement
MUREG/CR-3319	2	Program: LWR Power Reactor Surveillance Physics-Dosimetry Data Base Compendium,
MUREG/CR-3319	3	MUREG/CR-3319, NEDL-TME 85-3, U.S. Nuclear Regulatory Commission, Washington,
MUREG/CR-3319	4	DC, August 1985.
ORNL-4313	1	C. E. Childress, Fabrication History of the First Two 12-in.-thick ASTM A533
ORNL-4313	2	Grade B, Class 1 Steel Plates of the Heavy Section Steel Technology Program,
ORNL-4313	3	Documentary Report 1, ORNL-4313, Oak Ridge National Laboratory, Oak Ridge, TN,
ORNL-4313	4	February 1969.
ORNL-TM-3193	1	C. E. Childress, Manual for ASTM A-533 Grade B Class 1 Steel (HSST Plate 03)
ORNL-TM-3193	2	Provided to the International Atomic Energy Agency, ORNL-TM-3193, Oak Ridge
ORNL-TM-3193	3	National Laboratory, Oak Ridge, TN, March 1971.
P-CE-7752	1	W. J. Porter, Palisades Vessel Weld Documentation, Communication to Consumers
P-CE-7752	2	Power Company in reference to Letter P-CE-7747 dated September 25, 1984, Letter
P-CE-7752	3	P-CE-7752, Combustion Engineering, Inc., Windsor, CT, October 9, 1984.
PB-HE 75/02	1	G. Ullrich, B. Burgisser, Herrnberger, Nachbestrahlungsuntersuchungen an
PB-HE 75/02	2	Reaktordruckgefassmaterial Beznau I/Kapsel V. Ermittlung der Neutronenfluens
PB-HE 75/02	3	sowie Lateral-Expansion und Root-Notch-Contraction, PB-HE-75/02, Addendum zu
PB-HE 75/02	4	PB-HE-73/9, Eidg. Institut fur Reaktorforschung, November 1975.
PB-HE 75/03	1	G. Ullrich and B. Burgisser, Nachbestrahlungsuntersuchungen an
PB-HE 75/03	2	NOK-Reaktordruckgefassmaterial der Kernkraftwerke Beznau 1/2 - Kapsel R,
PB-HE 75/03	3	PB-HE-75/03, Eidg. Institut fur Reaktorforschung, November 1975.
PB-HE 78/06	1	G. Ullrich, B. Burgisser, E. Hegedues, and T. Aerne/jem,
PB-HE 78/06	2	Nachbestrahlungsuntersuchungen an NOK-Reaktordruckgefassmaterial des KKB
PB-HE 78/06	3	I/Kapsel S, PB-HE 78/06, Eidg. Institut fur Reaktorforschung, August 1978.
STUDSVIK/MS-78/226	1	S. Rao and Y. Haag, Surveillance Test Results. Ringhals 2, Studsvik Energiteknik
STUDSVIK/MS-78/226	2	AB, Sweden, July 17, 1978.
SWRI-02-3467	1	E. B. Norris, Analysis of the First Vessel Material Surveillance Capsule
SWRI-02-3467	2	Withdrawal from Lacrosse Boiling Water Reactor, SWRI Project 02-3467, Southwest
SWRI-02-3467	3	Research Institute, San Antonio, TX, March 1973.
SWRI-02-3574	1	E. B. Norris, Analysis of the First Material Surveillance Capsule from H. B.
SWRI-02-3574	2	Robinson Unit No. 2, SWRI Project 02-3574, Southwest Research Institute, San
SWRI-02-3574	3	Antonio, TX, July 1973.
SWRI-02-4074-001	1	E. B. Norris, Analysis of the Vessel Material Surveillance Capsules Withdrawn
SWRI-02-4074-001	2	from Lacrosse Boiling Water Reactor During the 1975 Refuelling, SWRI Project
SWRI-02-4074-001	3	02-4074-001, Southwest Research Institute, San Antonio, TX, April 1977.
SWRI-02-4074/2	1	E. B. Norris, Radiation and Corrosion Surveillance Programs for LACBWR Power
SWRI-02-4074/2	2	Plant, SWRI Project 02-4074, Progress Report No. 2, Southwest Research
SWRI-02-4074/2	3	Institute, San Antonio, TX, June 15, 1976.

Table 27 (continued)

REF_TITLE	REF_ID	COST	REF_TITLE
SWRI-02-4221	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Turkey Point Unit	
SWRI-02-4221	2	No. 4 Analysis of Capsule Y, SwRI Project 02-4221, Southwest Research Institute,	
SWRI-02-4221	3	San Antonio, TX, June 1976.	
SWRI-02-4397	1	E. B. Norris, Reactor Vessel Material Surveillance Program for W. B. Robinson	
SWRI-02-4397	2	Unit No. 2, Analysis of Capsule V, SwRI Project No. 02-4397, Southwest Research	
SWRI-02-4397	3	Institute, San Antonio, TX, October 1976.	
SWRI-02-4531	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Indian Point Unit	
SWRI-02-4531	2	No. 2 Analysis of Capsule T, SwRI Project 02-4531, Southwest Research Institute,	
SWRI-02-4531	3	San Antonio, TX, June 1977.	
SWRI-02-4531/\$	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Indian Point	
SWRI-02-4531/\$	2	Unit No. 2 Analysis of Capsule T, Supplement to Final Report, SwRI Project No.	
SWRI-02-4531/\$	3	02-4531, Southwest Research Institute, San Antonio, TX, December 1980.	
SWRI-02-4770	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Donald C. Cook	
SWRI-02-4770	2	Unit No. 1, Analysis of Capsule T, SwRI Project 02-4770, Southwest Research	
SWRI-02-4770	3	Institute, San Antonio, TX, December 1977.	
SWRI-02-5131	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Capsule S -	
SWRI-02-5131	2	Turkey Point Unit No. 3, Capsule S - Turkey Point Unit No. 4, SwRI Projects	
SWRI-02-5131	3	02-5131 and 02-5380, Southwest Research Institute, San Antonio, TX, May 1979.	
SWRI-02-5212	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Indian Point Unit	
SWRI-02-5212	2	No. 2 Analysis of Capsule Y, SwRI Project No. 02-5212, Southwest Research	
SWRI-02-5212	3	Institute, San Antonio, TX, November 1980.	
SWRI-02-5380	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Capsule S -	
SWRI-02-5380	2	Turkey Point Unit No. 3, Capsule S - Turkey Point Unit No. 4, SwRI Projects	
SWRI-02-5380	3	02-5131 and 02-5380, Southwest Research Institute, San Antonio, TX, May 1979.	
SWRI-02-5951	1	E. B. Norris, Pilgrim Nuclear Power Station Unit 1 Reactor Vessel Irradiation	
SWRI-02-5951	2	Surveillance Program, SwRI Project 02-5951, Southwest Research Institute, San	
SWRI-02-5951	3	Antonio, TX, July 1981.	
SWRI-02-6208-001	1	E. B. Norris, Analysis of the Vessel Material Surveillance Capsules Withdrawn	
SWRI-02-6208-001	2	from LaCrosse Boiling Water Reactor During the 1980 Refuelling, SwRI Project	
SWRI-02-6208-001	3	No. 02-6208-001, Southwest Research Institute, San Antonio, TX, October 9, 1981.	
SWRI-06-6901-001	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Zion Unit No. 2	
SWRI-06-6901-001	2	Analysis of Capsule T, SwRI Project 06-6901-001, Southwest Research Institute,	
SWRI-06-6901-001	3	San Antonio, TX, July 1983.	
SWRI-06-6901-002	1	E. B. Norris, Dresden Nuclear Power Station Unit 2 Reactor Vessel Irradiation	
SWRI-06-6901-002	2	Surveillance Program, Analysis of Capsule No. 8, SwRI Project No. 06-6901-002,	
SWRI-06-6901-002	3	Southwest Research Institute, San Antonio, TX, March 1983.	
SWRI-06-7484-001	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Zion Unit No. 1	
SWRI-06-7484-001	2	Analysis of Capsule X, SwRI Project 06-7484-001, Southwest Research Institute,	
SWRI-06-7484-001	3	San Antonio, TX, March 1984.	
SWRI-06-7484-003	1	E. B. Norris, Dresden Nuclear Power Station Unit 3 Reactor Vessel Irradiation	
SWRI-06-7484-003	2	Surveillance Program, Analysis of Capsule No. 18, SwRI Project No. 06-7684-003,	
SWRI-06-7484-003	3	Southwest Research Institute, San Antonio, TX, February 1984.	
SWRI-06-8575	1	P. K. Nair and E. B. Norris, Reactor Vessel Material Surveillance Program for	
SWRI-06-8575	2	Turkey Point Unit No. 3: Analysis of Capsule V, SwRI Project No. 06-8575,	
SWRI-06-8575	3	Southwest Research Institute, San Antonio, TX, August 1986.	
SWRI-06-8658	1	P. K. Nair, E. B. Norris, and M. L. Williams, Susquehanna Unit 1 Dosimeter	
SWRI-06-8658	2	Testing, SwRI Project No. 06-8658, Southwest Research Institute, San Antonio,	
SWRI-06-8658	3	TX, September 1986.	
SWRI-06-8851	1	P. K. Nair and M. L. Williams, Reactor Vessel Material Surveillance Program for	
SWRI-06-8851	2	Sequoia Unit No. 1: Analysis of Capsule U, SwRI Project 06-8851, Southwest	
SWRI-06-8851	3	Research Institute, San Antonio, TX, October 1986.	
SWRI-06-8888	1	P. K. Nair and M. L. Williams, Reactor Vessel Material Surveillance Program for	
SWRI-06-8888	2	Donald C. Cook Unit No. 2: Analysis of Capsule X, SwRI Project 06-888, Southwest	
SWRI-06-8888	3	Research Institute, San Antonio, TX, May 1987.	
SWRI-06-8976	1	P. K. Nair and M. L. Williams, Reactor Vessel Material Surveillance Program for	
SWRI-06-8976	2	Angra Dos Reis Unit No. 1: Analysis of Capsule V, SwRI Project 06-8976,	
SWRI-06-8976	3	Southwest Research Institute, San Antonio, TX, October 1987.	
SWRI-07-1599	1	D. R. Ireland and E. B. Norris, Influence of Neutron Irradiation on the	
SWRI-07-1599	2	Properties of Steels and Welds Typical of the EPR Pressure Vessel After Two	
SWRI-07-1599	3	Power Years Operation, SwRI Project 07-1599, Southwest Research Institute,	
SWRI-07-1599	4	San Antonio, TX, March 1988.	
SWRI-07-2892/A	1	E. B. Norris, Analysis of First Surveillance Material Capsule from San Onofre	
SWRI-07-2892/A	2	Unit 1, SwRI Project 07-2892, Southwest Research Institute, San Antonio, TX,	
SWRI-07-2892/A	3	May 1971.	
SWRI-07-2892/B	1	E. B. Norris, Analysis of Second Surveillance Material Capsule from San Onofre	
SWRI-07-2892/B	2	Unit 1, SwRI Project No. 07-2892, Southwest Research Institute, San Antonio, TX,	
SWRI-07-2892/B	3	June 5, 1972.	
SWRI-17-2108	1	F. A. Iddings, D. G. Cadene, and M. L. Williams, Reactor Vessel Material	
SWRI-17-2108	2	Surveillance Program for Indian Point Unit No. 2 Analysis of Capsule V,	
SWRI-17-2108	3	Final Report, SwRI Project No. 17-2108, Southwest Research Institute, San	
SWRI-17-2108	4	Antonio, TX, October 1988.	

Table 27 (continued)

REF_TITLE.dof: Page 7	CONT	REF_TITLE
SWRI-17-5759-201	1	E. B. Norris, Reactor Vessel Material Surveillance Program for KD-RI Unit 1
SWRI-17-5759-201	2	Analysis of Capsule V, SWRI Project No. 17-5759-201, Southwest Research
SWRI-17-5759-201	3	Institute, San Antonio, TX, June 1980.
SWRI-17-7517-219	1	E. B. Norris, P. K. Nair, and R. J. Dexter, Reactor Vessel Material Surveillance
SWRI-17-7517-219	2	Program for Ko-RI Unit No. 1: Analysis of Capsule T, SWRI Project 17-7517-219,
SWRI-17-7517-219	3	Southwest Research Institute, San Antonio, TX, July 1986.
SWRI-7244-001/1	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Donald C. Cook,
SWRI-7244-001/1	2	Unit No. 1, Analysis of Capsule Y, SWRI-7244-001/1, Southwest Research
SWRI-7244-001/1	3	Institute, San Antonio, TX, January 1984.
SWRI-7244-002/1	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Donald C. Cook,
SWRI-7244-002/1	2	Unit No. 2, Analysis of Capsule Y, SWRI-7244-002/1, Southwest Research
SWRI-7244-002/1	3	Institute, San Antonio, TX, February 1984.
SWRI-7279-001/3	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Indian Point Unit
SWRI-7279-001/3	2	No. 2 Analysis of Capsule Z, SWRI-7279-001/3, Southwest Research Institute, San
SWRI-7279-001/3	3	Antonio, TX, April 1984.
SWRI-7484-002/1	1	E. B. Norris, Quad Cities Nuclear Power Station Unit 2 Reactor Vessel
SWRI-7484-002/1	2	Irradiation Surveillance Program, Analysis of Capsule No. 18, SWRI-7484-002/1,
SWRI-7484-002/1	3	Southwest Research Institute, San Antonio, TX, March 1984.
SWRI-7524	1	E. B. Norris, Reactor Vessel Material Surveillance Program for Calvert Cliffs
SWRI-7524	2	Unit 2 Analysis of 263-Deg. Capsule, SWRI-7524, Southwest Research Institute,
SWRI-7524	3	San Antonio, TX, September 1985.
SWRI-7857	1	E. B. Norris, Quad Cities Nuclear Power Station Unit 1 Reactor Vessel
SWRI-7857	2	Irradiation Surveillance Program, Analysis of Capsule No. 8, SWRI-7857,
SWRI-7857	3	Southwest Research Institute, San Antonio, TX, August 1984.
TR-F-MCM-004	1	S. T. Byrne, Florida Power & Light Company, St. Lucie Unit No. 1,
TR-F-MCM-004	2	Post-Irradiation Evaluation of Reactor Vessel Surveillance Capsule W-97,
TR-F-MCM-004	3	TR-F-MCM-004, Combustion Engineering, Inc., Windsor, CT, December 1983.
TR-N-MCM-008	1	S. T. Byrne, Northeast Utilities Service Company, Millstone Nuclear Unit No. 2,
TR-N-MCM-008	2	Evaluation of Irradiated Capsule W-97, Reactor Vessel Materials Irradiation
TR-N-MCM-008	3	Surveillance Program, TR-N-MCM-008, Combustion Engineering, Inc., Windsor, CT,
TR-O-MCD-001	4	April 1982.
TR-O-MCD-001	1	A. Ragl, Omaha Public Power District Fort Calhoun Station Unit No. 1 Evaluation
TR-O-MCD-001	2	of Baseline Specimens Reactor Vessel Materials Irradiation Surveillance Program,
TR-O-MCD-001	3	TR-O-MCD-001, Combustion Engineering, Inc., Windsor, CT, March 1977.
TR-O-MCM-001	1	S. T. Byrne, Omaha Public Power District Fort Calhoun Station Unit No. 1,
TR-O-MCM-001	2	Post-Irradiation Evaluation of Reactor Vessel Surveillance Capsule W-225,
TR-O-MCM-001	3	Reactor Vessel Materials Irradiation Surveillance Program, TR-O-MCM-001,
TR-O-MCM-001	4	Combustion Engineering, Inc., Windsor, CT, May 1979.
TR-O-MCM-001/R1	1	S. T. Byrne, Omaha Public Power District Fort Calhoun Station Unit No. 1,
TR-O-MCM-001/R1	2	Post-Irradiation Evaluation of Reactor Vessel Surveillance Capsule W-225,
TR-O-MCM-001/R1	3	TR-O-MCM-001, Rev. 1, Combustion Engineering, Inc., Windsor, CT, August 1980.
WCAP-10015	1	L. R. Singer, Kansas Gas & Electric Company Wolf Creek Generating Station Unit
WCAP-10015	2	No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-10015, Westinghouse
WCAP-10015	3	Electric Corporation, Pittsburgh, PA, June 1982.
WCAP-10030	1	S. E. Yanichko, S. L. Anderson, R. P. Shogan, and R. G. Lott, Analysis of the
WCAP-10030	2	Fourth Capsule from the Commonwealth Edison Company Dresden Unit 3 Nuclear Plant
WCAP-10030	3	Reactor Vessel Radiation Surveillance Program, WCAP-10030, Westinghouse Electric
WCAP-10030	4	Corporation, Pittsburgh, PA, January 1982.
WCAP-10064	1	S. E. Yanichko, S. L. Anderson, R. P. Shogan, and R. G. Lott, Analysis of the
WCAP-10064	2	Third Capsule from the Commonwealth Edison Company Quad Cities Unit 2 Nuclear
WCAP-10064	3	Plant Reactor Vessel Radiation Surveillance Program, WCAP-10064, Westinghouse
WCAP-10064	4	Electric Corporation, Pittsburgh, PA, April 1982.
WCAP-10086	1	S. E. Yanichko et al., Analysis of Capsule T from the Rochester Gas and Electric
WCAP-10086	2	Corporation R. E. Ginna Nuclear Plant Reactor Vessel Radiation Surveillance
WCAP-10086	3	Program, WCAP-10086, Westinghouse Electric Corporation, Pittsburgh, PA, April
WCAP-10086	4	1982.
WCAP-10102	1	S. E. Yanichko, K. C. Tran, and W. T. Kaiser, Analysis of Capsule P from
WCAP-10102	2	Northern States Power Company Prairie Island Unit 1, Reactor Vessel Radiation
WCAP-10102	3	Surveillance Program, WCAP-10102, Westinghouse Electric Corporation, Pittsburgh,
WCAP-10102	4	PA, May 1982.
WCAP-10185	1	S. E. Yanichko, K. C. Tran, R. P. Shogan, and R. G. Lott, Analysis of Capsule N
WCAP-10185	2	from the Union Electrica, S.A., Jose Cabrera Reactor Vessel Radiation
WCAP-10185	3	Surveillance Program, WCAP-10185, Westinghouse Electric Corporation, Pittsburgh,
WCAP-10185	4	PA, October 1982.
WCAP-10236	1	S. E. Yanichko, S. L. Anderson, R. P. Shogan, and R. G. Lott, Analysis of
WCAP-10236	2	Capsule D from the Connecticut Yankee Reactor Vessel Radiation Surveillance
WCAP-10236	3	Program, WCAP-10236, Westinghouse Electric Corporation, Pittsburgh, PA, January
WCAP-10236	4	1983.
WCAP-10300	1	S. E. Yanichko and S. L. Anderson, Analysis of Capsule Y from the Power
WCAP-10300	2	Authority of the State of New York Indian Point Unit 3 Reactor Vessel Radiation
WCAP-10300	3	Surveillance Program, WCAP-10300, Westinghouse Electric Corporation, Pittsburgh,
WCAP-10300	4	PA, March 1983.

Table 27 (continued)

REF_TITLE.dbf: Page 8

REF_ID

CONT REF_TITLE

WCAP-10304	1	S. E. Yanichko, S. L. Anderson, P. P. Shogan, and R. G. Lott, Analysis of
WCAP-10304	2	Capsule T from the H. B. Robinson Unit 2 Reactor Vessel Radiation Surveillance
WCAP-10304	3	Program, WCAP-10304, Westinghouse Electric Corporation, Pittsburgh, PA, March
WCAP-10304	4	1983.
WCAP-10340	1	S. E. Yanichko, S. L. Anderson, C. A. Cheney, and W. T. Kaiser, Analysis of
WCAP-10340	2	Capsule T from the Tennessee Valley Authority, Sequoyah Unit 1 Reactor Vessel
WCAP-10340	3	Radiation Surveillance Program, WCAP-10340, Westinghouse Electric Corporation,
WCAP-10340	4	Pittsburgh, PA, May 1983.
WCAP-10425	1	M. K. Kunka, S. E. Yanichko, C. A. Cheney, and W. T. Kaiser, Analysis of Capsule
WCAP-10425	2	U from the Alabama Power Company, Joseph M. Farley, Unit 2, Reactor Vessel
WCAP-10425	3	Radiation Surveillance Program, WCAP-10425, Westinghouse Electric Corporation,
WCAP-10425	4	Pittsburgh, PA, October 1983.
WCAP-10474	1	R. S. Boggs, S. E. Yanichko, C. A. Cheney, and W. T. Kaiser, Analysis of Capsule
WCAP-10474	2	U from the Alabama Power Company, Joseph M. Farley, Unit 1, Reactor Vessel
WCAP-10474	3	Radiation Surveillance Program, WCAP-10474, Westinghouse Electric Corporation,
WCAP-10474	4	Pittsburgh, PA, February 1984.
WCAP-10492	1	R. S. Boggs, S. E. Yanichko, C. A. Cheney, and W. T. Kaiser, Analysis of Capsule
WCAP-10492	2	T from the Public Service Electric and Gas Company Salem Unit 2 Reactor Vessel
WCAP-10492	3	Radiation Surveillance Program, WCAP-10492, Westinghouse Electric Corporation,
WCAP-10492	4	Pittsburgh, PA, March 1984.
WCAP-10509	1	R. S. Boggs, S. E. Yanichko, C. A. Cheney, and W. T. Kaiser, Analysis of Capsule
WCAP-10509	2	T from the Tennessee Valley Authority, Sequoyah Unit 2 Reactor Vessel Radiation
WCAP-10509	3	Surveillance Program, WCAP-10509, Westinghouse Electric Corporation, Pittsburgh,
WCAP-10509	4	PA, April 1984.
WCAP-10637	1	M. K. Kunka and C. A. Cheney, Analysis of Capsules T-330 and W-290, Consumers
WCAP-10637	2	Power Company, Palisades Reactor Vessel Radiation Surveillance Program,
WCAP-10637	3	WCAP-10637, Westinghouse Electric Corporation, Pittsburgh, PA, September 1984.
WCAP-10694	1	R. S. Boggs, C. A. Cheney, and W. T. Kaiser, Analysis of Capsule Y from the
WCAP-10694	2	Public Service Electric and Gas Company Salem Unit 1, Reactor Vessel Radiation
WCAP-10694	3	Surveillance Program, WCAP-10694, Westinghouse Electric Corporation, Pittsburgh,
WCAP-10694	4	PA, December 1984.
WCAP-10736	1	S. E. Yanichko, V. A. Perone, and W. T. Kaiser, Analysis of Capsule T from the
WCAP-10736	2	Wisconsin Electric Power Company Point Beach Nuclear Plant Unit No. 1,
WCAP-10736	3	WCAP-10736, Westinghouse Electric Corporation, Pittsburgh, PA, December 1984.
WCAP-10786	1	S. E. Yanichko, T. V. Congedo, and W. T. Kaiser, Analysis of Capsule U from the
WCAP-10786	2	Duke Power Company McGuire Unit 1 Reactor Vessel Radiation Surveillance Program,
WCAP-10786	3	WCAP-10786, Westinghouse Electric Corporation, Pittsburgh, PA, February 1985.
WCAP-10814	1	R. S. Boggs, A. H. Fero, and W. T. Kaiser, Analysis of Capsule U from the South
WCAP-10814	2	Carolina Electric and Gas Company Virgil C. Summer Unit 1 Reactor Vessel
WCAP-10814	3	Radiation Surveillance Program, WCAP-10814, Westinghouse Electric Corporation,
WCAP-10814	4	Pittsburgh, PA, June 1985.
WCAP-10861	1	S. E. Yanichko, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule X from
WCAP-10861	2	Portland General Electric Company Trojan Reactor Vessel Radiation Surveillance
WCAP-10861	3	Program, WCAP-10861, Westinghouse Electric Corporation, Pittsburgh, PA, June
WCAP-10861	4	1985.
WCAP-10867	1	R. S. Boggs, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule U from the
WCAP-10867	2	Duquesne Light Company Beaver Valley Unit 1 Reactor Vessel Radiation
WCAP-10867	3	Surveillance Program, WCAP-10867, Westinghouse Electric Corporation, Pittsburgh,
WCAP-10867	4	PA, September 1985.
WCAP-11006	1	R. S. Boggs, T. V. Congedo, and H. Gong, Analysis of Capsule R from the
WCAP-11006	2	Northern States Power Company Prairie Island Unit 1 Reactor Vessel Radiation
WCAP-11006	3	Surveillance Program, WCAP-11006, Westinghouse Electric Corporation, Pittsburgh,
WCAP-11006	4	PA, February 1986.
WCAP-11029	1	S. E. Yanichko, T. V. Congedo, and W. T. Kaiser, Analysis of Capsule V from
WCAP-11029	2	Duke Power Company McGuire Unit 2 Reactor Vessel Radiation Surveillance Program,
WCAP-11029	3	WCAP-11029, Westinghouse Electric Corporation, Pittsburgh, PA, January 1986.
WCAP-11343	1	S. E. Yanichko and J. C. Schmertz, Analysis of Capsule R from the Northern
WCAP-11343	2	States Power Company Prairie Island Unit 2 Reactor Vessel Radiation Surveillance
WCAP-11343	3	Program, WCAP-11343, Westinghouse Electric Corporation, Pittsburgh, PA,
WCAP-11343	4	December 1986.
WCAP-11374/R1	1	R. G. Lott et al., Analysis of Capsule U from the Union Electric Company
WCAP-11374/R1	2	Callaway Unit 1 Reactor Vessel Radiation Surveillance Program, WCAP-11374,
WCAP-11374/R1	3	Rev. 1, Westinghouse Electric Corporation, Pittsburgh, PA, June 1987.
WCAP-11415	1	S. E. Yanichko and V. A. Perone, Analysis of Capsule V from the Virginia
WCAP-11415	2	Electric Power Company Surry Unit 1 Reactor Vessel Radiation Surveillance
WCAP-11415	3	Program, WCAP-11415, Westinghouse Electric Corporation, Pittsburgh, PA,
WCAP-11415	4	February 1987.
WCAP-11438	1	R. P. Shogan et al., Analysis of Capsule W from the Alabama Power Company Joseph
WCAP-11438	2	M. Farley Unit 2 Reactor Vessel Radiation Surveillance Program, WCAP-11438,
WCAP-11438	3	Westinghouse Electric Corporation, Pittsburgh, PA, April 1987.

Table 27 (continued)

REF_TITLE.dbf: Page 9	REF_ID	CONT	REF_TITLE
WCAP-11499	1	S. E. Yanichko and V. A. Perone, Analysis of Capsule V from the Virginia	
WCAP-11499	2	Electric and Power Company Surry Unit 2 Reactor Vessel Radiation Surveillance	
WCAP-11499	3	Program, WCAP-11499, Westinghouse Electric Corporation, Pittsburgh, PA, June	
WCAP-11499	4	1987.	
WCAP-11527	1	S. E. Yanichko and S. L. Anderson, Analysis of Capsule 2 from the Duke Power	
WCAP-11527	2	Company Catawba Unit 1 Reactor Vessel Radiation Surveillance Program,	
WCAP-11527	3	WCAP-11527, Westinghouse Electric Corporation, Pittsburgh, PA, June 1987.	
WCAP-11553	1	S. E. Yanichko, E. P. Lippincott, L. Albertin, and J. C. Schmertz, Analysis of	
WCAP-11553	2	Capsule U from the Wolf Creek Nuclear Operating Corporation Wolf Creek Reactor	
WCAP-11553	3	Vessel Radiation Surveillance Program, WCAP-11553, Westinghouse Electric	
WCAP-11553	4	Corporation, Pittsburgh, PA, August 1987.	
WCAP-11563/R1	1	R. P. Shogan et al., Analysis of Capsule X from the Alabama Power Company Joseph	
WCAP-11563/R1	2	M. Farley Unit 1 Reactor Vessel Radiation Surveillance Program, WCAP-11563,	
WCAP-11563/R1	3	Rev. 1, Westinghouse Electric Corporation, Pittsburgh, PA, September 1987.	
WCAP-11567	1	S. E. Yanichko, S. L. Anderson, J. C. Schmertz, and L. Albertin, Analysis of	
WCAP-11567	2	Capsule S from Pacific Gas and Electric Company Diablo Canyon Unit 1 Reactor	
WCAP-11567	3	Vessel Radiation Surveillance Program, WCAP-11567, Westinghouse Electric	
WCAP-11567	4	Corporation, Pittsburgh, PA, December 1987.	
WCAP-11851	1	S. E. Yanichko, S. L. Anderson, and L. Albertin, Analysis of Capsule U from the	
WCAP-11851	2	Pacific Gas and Electric Company Diablo Canyon Unit 2 Reactor Vessel Radiation	
WCAP-11851	3	Surveillance Program, Westinghouse Electric Corporation, Pittsburgh PA, May 1988	
WCAP-2834/R1	1	S. E. Yanichko, San Onofre Reactor Vessel Radiation Surveillance Program,	
WCAP-2834/R1	2	WCAP-2834-R1, Westinghouse Electric Corporation, Pittsburgh, PA, November 1966.	
WCAP-7036	1	S. E. Yanichko, Connecticut Yankee Reactor Vessel Radiation Surveillance Pro-	
WCAP-7036	2	gram, WCAP-7036, Westinghouse Electric Corporation, Pittsburgh, PA, April 1967.	
WCAP-7254	1	S. E. Yanichko, Rochester Gas and Electric Robert E. Ginna Unit No. 1 Reactor	
WCAP-7254	2	Vessel Radiation Surveillance Program, WCAP-7254, Westinghouse Electric	
WCAP-7254	3	Corporation, Pittsburgh, PA, May 1969.	
WCAP-7323	1	S. E. Yanichko, Consolidated Edison Co., Indian Point Unit No. 2 Reactor Vessel	
WCAP-7323	2	Radiation Surveillance Program, WCAP-7323, Westinghouse Electric Corporation,	
WCAP-7323	3	Pittsburgh, PA, May 1969.	
WCAP-7373	1	S. E. Yanichko, Carolina Power and Light Co., W. B. Robinson, Unit No. 2,	
WCAP-7373	2	Reactor Vessel Radiation Surveillance Program, WCAP-7373, Westinghouse Electric	
WCAP-7373	3	Corporation, Pittsburgh, PA, January 1970.	
WCAP-7513	1	S. E. Yanichko, Wisconsin Michigan Power Co. Point Beach Unit No. 1 Reactor	
WCAP-7513	2	Vessel Radiation Surveillance Program, WCAP-7513, Westinghouse Electric	
WCAP-7513	3	Corporation, Pittsburgh, PA, June 1970.	
WCAP-7656	1	S. E. Yanichko, Florida Power and Light Co., Turkey Point Unit No. 3 Reactor	
WCAP-7656	2	Vessel Radiation Surveillance Program, WCAP-7656, Westinghouse Electric	
WCAP-7656	3	Corporation, Pittsburgh, PA, May 1971.	
WCAP-7712	1	S. E. Yanichko and G. C. Zula, Wisconsin Michigan Power Co. and the Wisconsin	
WCAP-7712	2	Electric Power Co. Point Beach Unit No. 2 Reactor Vessel Radiation Surveillance	
WCAP-7712	3	Program, WCAP-7712, Westinghouse Electric Corporation, Pittsburgh, PA, June	
WCAP-7712	4	1971.	
WCAP-7723	1	S. E. Yanichko, Virginia Electric & Power Co. Surry Unit No. 1 Reactor Vessel	
WCAP-7723	2	Radiation Surveillance Program, WCAP-7723, Westinghouse Electric Corporation,	
WCAP-7723	3	Pittsburgh, PA, July 1971.	
WCAP-8064	1	S. E. Yanichko and D. J. Lege, Commonwealth Edison Co., Zion Unit No. 1 Reactor	
WCAP-8064	2	Vessel Radiation Surveillance Program, WCAP-8064, Westinghouse Electric	
WCAP-8064	3	Corporation, Pittsburgh, PA, March 1973.	
WCAP-8085	1	S. E. Yanichko and D. J. Lege, Virginia Electric & Power Co. Surry Unit No. 2	
WCAP-8085	2	Reactor Vessel Radiation Surveillance Program, WCAP-8085, Westinghouse Electric	
WCAP-8085	3	Corporation, Pittsburgh, PA, June 1973.	
WCAP-8086	1	S. E. Yanichko and D. J. Lege, Northern States Power Co. Prairie Island Unit	
WCAP-8086	2	No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-8086, Westinghouse	
WCAP-8086	3	Electric Corporation, Pittsburgh, PA, June 1973.	
WCAP-8107	1	S. E. Yanichko, D. J. Lege, and G. C. Zula, Wisconsin Public Service Corporation	
WCAP-8107	2	Keweenaw Nuclear Power Plant Reactor Vessel Radiation Surveillance Program,	
WCAP-8107	3	WCAP-8107, Westinghouse Electric Corporation, Pittsburgh, PA, April 1973.	
WCAP-8132	1	S. E. Yanichko and D. J. Lege, Commonwealth Edison Co., Zion Unit No. 2 Reactor	
WCAP-8132	2	Vessel Radiation Surveillance Program, WCAP-8132, Westinghouse Electric	
WCAP-8132	3	Corporation, Pittsburgh, PA, May 1973.	
WCAP-8193	1	S. E. Yanichko and D. J. Lege, Northern States Power Co. Prairie Island Unit	
WCAP-8193	2	No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-8193, Westinghouse	
WCAP-8193	3	Electric Corporation, Pittsburgh, PA, September 1973.	
WCAP-8216	1	S. E. Yanichko and D. J. Lege, Swedish State Power Board Ringhals Unit No. 2	
WCAP-8216	2	Reactor Vessel Radiation Surveillance Program, WCAP-8216, Westinghouse Electric	
WCAP-8216	3	Corporation, Pittsburgh, PA, November 1973.	

Table 27 (continued)

REF_TITL.dbf: Page 10

REF_ID

CONT REF_TITLE

WCAP-8233	1	S. E. Yanichko, D. J. Lege, and J. H. Phillips, Tennessee Valley Authority
WCAP-8233	2	Sequoia Unit No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-8233,
WCAP-8233	3	Westinghouse Electric Corporation, Pittsburgh, PA, December 1973.
WCAP-8249	1	S. E. Yanichko, D. J. Lege, S. L. Anderson, and T. R. Mager, Analysis of Capsule
WCAP-8249	2	S from Carolina Power and Light Company K. B. Robinson Unit No. 2 Reactor Vessel
WCAP-8249	3	Radiation Surveillance Program, WCAP-8249, Westinghouse Electric Corporation,
WCAP-8249	4	Pittsburgh, PA, December 1973.
WCAP-8421	1	S. E. Yanichko, T. R. Mager, and S. Kang, Analysis of Capsule R from the
WCAP-8421	2	Rochester Gas & Electric Corporation R. E. Ginne Unit No. 1 Reactor Vessel
WCAP-8421	3	Radiation Surveillance Program, WCAP-8421, Westinghouse Electric Corporation,
WCAP-8421	4	Pittsburgh, PA, November 1974.
WCAP-8457	1	J. A. Davidson, J. H. Phillips, and S. E. Yanichko, Duquesne Light Company
WCAP-8457	2	Beaver Valley Unit No. 1 Reactor Vessel Radiation Surveillance Program,
WCAP-8457	3	Westinghouse Electric Corporation, Pittsburgh, PA, October 1974.
WCAP-8465	1	J. A. Davidson, J. H. Phillips, and S. E. Yanichko, Pacific Gas and Electric Co.
WCAP-8465	2	Diablo Canyon Unit No. 1 Reactor Vessel Radiation Surveillance Program,
WCAP-8465	3	WCAP-8465, Westinghouse Electric Corporation, Pittsburgh, PA, January 1975.
WCAP-8475	1	S. E. Yanichko and J. A. Davidson, Consolidated Edison Co. of New York Indian
WCAP-8475	2	Point Unit No. 3, Reactor Vessel Radiation Surveillance Program, WCAP-8475,
WCAP-8475	3	Westinghouse Electric Corporation, Pittsburgh, PA, January 1975.
WCAP-8511	1	J. A. Davidson, J. H. Phillips, and S. E. Yanichko, Public Service Electric and
WCAP-8511	2	Gas Co. Salem Unit No. 1 Reactor Vessel Radiation Surveillance Program,
WCAP-8511	3	WCAP-8511, Westinghouse Electric Corporation, Pittsburgh, PA, November 1975.
WCAP-8512	1	J. A. Davidson, S. E. Yanichko, and J. H. Phillips, American Electric Power
WCAP-8512	2	Company Donald C. Cook Unit No. 2 Reactor Vessel Radiation Surveillance Program,
WCAP-8512	3	WCAP-8512, Westinghouse Electric Corporation, Pittsburgh, PA, November 1975.
WCAP-8513	1	J. A. Davidson, J. H. Phillips, and S. E. Yanichko, Tennessee Valley Authority
WCAP-8513	2	Sequoia Unit No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-8513,
WCAP-8513	3	Westinghouse Electric Corporation, Pittsburgh, PA, November 1975.
WCAP-8631	1	S. E. Yanichko, J. H. Phillips, and S. L. Anderson, Analysis of Capsule T from
WCAP-8631	2	the Florida Power and Light Company Turkey Point Unit No. 3 Reactor Vessel
WCAP-8631	3	Radiation Surveillance Program, WCAP-8631, Westinghouse Electric Corporation,
WCAP-8631	4	Pittsburgh, PA, December 1975.
WCAP-8739	1	S. E. Yanichko and S. L. Anderson, Analysis of Capsule S from the Wisconsin
WCAP-8739	2	Electric Power Company and Wisconsin Michigan Power Company Point Beach Nuclear
WCAP-8739	3	Plant Unit No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-8739,
WCAP-8739	4	Westinghouse Electric Corporation, Pittsburgh, PA, November 1976.
WCAP-8771	1	J. A. Davidson and J. H. Phillips, Virginia Electric and Power Company North
WCAP-8771	2	Anne Unit No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-8771,
WCAP-8771	3	Westinghouse Electric Corporation, Pittsburgh, PA, September 1976.
WCAP-8772	1	J. A. Davidson, Virginia Electric and Power Company North Anne Unit No. 2
WCAP-8772	2	Reactor Vessel Radiation Surveillance Program, WCAP-8772, Westinghouse Electric
WCAP-8772	3	Corporation, Pittsburgh, PA, November 1976.
WCAP-8783	1	J. A. Davidson and S. E. Yanichko, Pacific Gas and Electric Company Diablo
WCAP-8783	2	Canyon Unit No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-8783,
WCAP-8783	3	Westinghouse Electric Corporation, Pittsburgh, PA, December 1976.
WCAP-8810	1	J. A. Davidson, J. H. Phillips, and S. E. Yanichko, Southern Alabama Power
WCAP-8810	2	Company Joseph M. Farley Nuclear Plant Unit No. 1 Reactor Vessel Radiation
WCAP-8810	3	Surveillance Program, WCAP-8810, Westinghouse Electric Corporation, Pittsburgh,
WCAP-8810	4	PA, December 1976.
WCAP-8908	1	S. E. Yanichko, S. L. Anderson, and K. V. Scott, Analysis of Capsule V from the
WCAP-8908	2	Wisconsin Public Service Corporation Keweenaw Nuclear Plant Reactor Vessel
WCAP-8908	3	Radiation Surveillance Program, WCAP-8908, Westinghouse Electric Corporation,
WCAP-8908	4	Pittsburgh, PA, January 1977.
WCAP-8916	1	J. A. Davidson, S. L. Anderson, and K. V. Scott, Analysis of Capsule V from
WCAP-8916	2	Northern States Power Company Prairie Island Unit No. 1, Reactor Vessel
WCAP-8916	3	Radiation Surveillance Program, WCAP-8916, Westinghouse Electric Corporation,
WCAP-8916	4	Pittsburgh, PA, August 1977.
WCAP-8956	1	J. A. Davidson and S. E. Yanichko, Alabama Power Company Joseph M. Farley
WCAP-8956	2	Nuclear Plant Unit No. 2 Reactor Vessel Radiation Surveillance Program,
WCAP-8956	3	WCAP-8956, Westinghouse Electric Corporation, Pittsburgh, PA, August 1977.
WCAP-8957	1	J. A. Davidson and S. E. Yanichko, Furnas-Centrais Electricas S.A. Angra Dos
WCAP-8957	2	Reis Unit No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-8957,
WCAP-8957	3	Westinghouse Electric Corporation, Pittsburgh, PA, May 1977.
WCAP-9195	1	J. A. Davidson and S. E. Yanichko, Duke Power Company William B. McGuire Unit
WCAP-9195	2	No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-9195, Westinghouse
WCAP-9195	3	Electric Corporation, Pittsburgh, PA, November 1977.
WCAP-9212	1	J. A. Davidson, S. E. Yanichko, and S. L. Anderson, Analysis of Capsule V from
WCAP-9212	2	Northern States Power Company Prairie Island Unit No. 2, Reactor Vessel
WCAP-9212	3	Radiation Surveillance Program, WCAP-9212, Westinghouse Electric Corporation,
WCAP-9212	4	Pittsburgh, PA, November 1977.
WCAP-9228	1	P. J. Fields, J. A. Davidson, and S. E. Yanichko, Central Nuclear de Almaraz
WCAP-9228	2	Almaraz Unit No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-9228,
WCAP-9228	3	Westinghouse Electric Corporation, Pittsburgh, PA, December 1977.

Table 27 (continued)

REF_TITLE.dbf: Page 11	REF_ID	CONT	REF_TITLE
WCAP-9234	1	J. A. Davidson and S. E. Yanichko, South Carolina Electric and Gas Company	
WCAP-9234	2	Virgil C. Summer Nuclear Plant Unit No. 1 Reactor Vessel Radiation Surveillance	
WCAP-9234	3	Program, WCAP-9234, Westinghouse Electric Corporation, Pittsburgh, PA,	
WCAP-9234	4	January 1978.	
WCAP-9298	1	J. A. Davidson, Tennessee Valley Authority Watts Bar Unit No. 1 Reactor Vessel	
WCAP-9298	2	Radiation Surveillance Program, WCAP-9298, Westinghouse Electric Corporation,	
WCAP-9298	3	Pittsburgh, PA, July 1978.	
WCAP-9308	1	P. J. Fields, J. A. Davidson, and S. E. Yanichko, Fuerzas Electricas de Cataluna	
WCAP-9308	2	ASCO Unit No. 1 Reactor Vessel Radiation Surveillance Program, WCAP-9308,	
WCAP-9308	3	Westinghouse Electric Corporation, Pittsburgh, PA, July 1978.	
WCAP-9329	1	P. J. Fields, J. A. Davidson, and S. E. Yanichko, Iberduero S.A. Lemoniz Unit	
WCAP-9329	2	No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-9329, Westinghouse	
WCAP-9329	3	Electric Corporation, Pittsburgh, PA, August 1978.	
WCAP-9330	1	P. J. Fields, J. A. Davidson, and S. E. Yanichko, Fuerzas Electricas de Cataluna	
WCAP-9330	2	ASCO Unit No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-9330,	
WCAP-9330	3	Westinghouse Electric Corporation, Pittsburgh, PA, August 1978.	
WCAP-9331	1	J. A. Davidson, S. L. Anderson, and R. P. Shogan, Analysis of Capsule T from the	
WCAP-9331	2	Wisconsin Electric Power Company Point Beach Nuclear Plant Unit No. 2 Reactor	
WCAP-9331	3	Vessel Radiation Surveillance Program, WCAP-9331, Westinghouse Electric	
WCAP-9331	4	Corporation, Pittsburgh, PA, August 1978.	
WCAP-9339	1	P. J. Fields and S. L. Anderson, Analysis of Capsule H from the Connecticut	
WCAP-9339	2	Yankee Reactor Vessel Radiation Surveillance Program, WCAP-9339, Westinghouse	
WCAP-9339	3	Electric Corporation, Pittsburgh, PA, September 1978.	
WCAP-9357	1	S. E. Yanichko and S. L. Anderson, Analysis of Capsule R from the Wisconsin	
WCAP-9357	2	Electric Power Company Point Beach Nuclear Plant Unit No. 1 Reactor Vessel	
WCAP-9357	3	Radiation Surveillance Program, WCAP-9357, Westinghouse Electric Corporation,	
WCAP-9357	4	Pittsburgh, PA, August 1978.	
WCAP-9469	1	J. A. Davidson, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule U from	
WCAP-9469	2	Portland General Electric Company Trojan Reactor Vessel Radiation Surveillance	
WCAP-9469	3	Program, WCAP-9469, Westinghouse Electric Corporation, Pittsburgh, PA, May 1979.	
WCAP-9491	1	J. A. Davidson, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule Y from the	
WCAP-9491	2	Indian Point Unit No. 3 Reactor Vessel Radiation Surveillance Program, WCAP-9491	
WCAP-9491	3	Westinghouse Electric Corporation, Pittsburgh, PA, April 1979.	
WCAP-9520	1	S. E. Yanichko, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule F from the	
WCAP-9520	2	Southern California Edison Company San Onofre Reactor Vessel Radiation	
WCAP-9520	3	Surveillance Program, WCAP-9520, Westinghouse Electric Corporation, Pittsburgh,	
WCAP-9520	4	PA, May 1979.	
WCAP-9635	1	S. E. Yanichko, S. L. Anderson, R. P. Shogan, and R. G. Lott, Analysis of	
WCAP-9635	2	Capsule R from the Wisconsin Electric Power Company Point Beach Nuclear Plant	
WCAP-9635	3	Unit No. 2 Reactor Vessel Radiation Surveillance Program, WCAP-9635,	
WCAP-9635	4	Westinghouse Electric Corporation, Pittsburgh, PA, December 1979.	
WCAP-9678	1	S. E. Yanichko, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule Y from the	
WCAP-9678	2	Public Service Electric and Gas Company Salem Unit No. 1 Reactor Vessel	
WCAP-9678	3	Radiation Surveillance Program, WCAP-9678, Westinghouse Electric Corporation,	
WCAP-9678	4	Pittsburgh, PA, February 1980.	
WCAP-9717	1	S. E. Yanichko, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule Y from the	
WCAP-9717	2	Alabama Power Company Farley Unit No. 1, Reactor Vessel Radiation Surveillance	
WCAP-9717	3	Program, WCAP-9717, Westinghouse Electric Corporation, Pittsburgh, PA, June 1980	
WCAP-9734	1	S. E. Yanichko, Duke Power Company Catawba Unit No. 1 Reactor Vessel Radiation	
WCAP-9734	2	Surveillance Program, WCAP-9734, Westinghouse Electric Corporation, Pittsburgh,	
WCAP-9734	3	PA, July 1980.	
WCAP-9794	1	S. E. Yanichko, S. L. Anderson, R. P. Shogan, and R. G. Lott, Analysis of	
WCAP-9794	2	Capsule 125 from the Consumers Power Company Big Rock Point Nuclear Plant	
WCAP-9794	3	Reactor Vessel Radiation Surveillance Program, WCAP-9794, Westinghouse Electric	
WCAP-9794	4	Corporation, Pittsburgh, PA, September 1980.	
WCAP-9842	1	L. R. Singer, Union Electric Company Callaway Unit No. 1 Reactor Vessel	
WCAP-9842	2	Radiation Surveillance Program, WCAP-9842, Westinghouse Electric Corporation,	
WCAP-9842	3	Pittsburgh, PA, May 1981.	
WCAP-9860	1	S. E. Yanichko et al., Analysis of Capsule V from the Duquesne Light Company	
WCAP-9860	2	Beaver Valley Unit No. 1 Reactor Vessel Radiation Surveillance Program,	
WCAP-9860	3	WCAP-9860, Westinghouse Electric Corporation, Pittsburgh, PA, January 1981.	
WCAP-9875	1	S. E. Yanichko, S. L. Anderson, R. P. Shogan, and R. G. Lott, Analysis of the	
WCAP-9875	2	Maine Yankee Reactor Vessel Second Accelerated Surveillance Capsule, WCAP-9875,	
WCAP-9875	3	Westinghouse Electric Corporation, Pittsburgh, PA, March 1981.	
WCAP-9877	1	S. E. Yanichko, S. L. Anderson, and W. T. Kaiser, Analysis of Capsule T from	
WCAP-9877	2	Northern States Power Company Prairie Island Unit No. 2, Reactor Vessel	
WCAP-9877	3	Radiation Surveillance Program, WCAP-9877, Westinghouse Electric Corporation,	
WCAP-9877	4	Pittsburgh, PA, March 1981.	

Table 28. Partial listing of REF_LST.dbf

REF_LST.dbf:	Page 1	
TAG	PLANT_ID	REF_ID
*		ASTM DS54
*		ORNL-4313
*	AD1	SWRI-06-8976
*	AD1	WCAP-8957
*	AL2	WCAP-9228
*	AN1	BAW-1440
*	AN1	BAW-1543/R2
*	AN1	BAW-1698
*	AN1	BAW-1803
*	AN1	BAW-1820
*	AN1	BAW-1836
*	AN1	NUREG/CR-3319
*	AN2	BMI-0584
*	AS1	WCAP-9308
*	AS2	WCAP-9330
*	BR	EPRI NP-2428
*	BR	GECR-4442
*	BR	NE&D 11(3)
*	BR	WCAP-9794
*	BV1	DOCKET 50-334
*	BV1	WCAP-10867
*	BV1	WCAP-8457
*	BV1	WCAP-9860
*	BW1	NEDO-24157/R1
*	BW1	NEDO-24161
*	BW2	NEDO-24157/R1
*	BZ1	PB-ME 75/02
*	BZ1	PB-ME 75/03
*	BZ1	PB-ME 78/06
*	BZ2	NUREG/CR-3319
*	BZ2	PB-ME 75/02
*	CAB	EPRI NP-2428
*	CAB	WCAP-10185
*	CAB	WENX/76/64
*	CB1	WCAP-11527
*	CB1	WCAP-9734
*	CC1	BMI-1280
*	CC1	DOCKET 50-317
*	CC2	DOCKET 50-318
*	CC2	SWRI-7524

APPENDIX A. PRELIMINARY SOFTWARE AND PROCESSING

A.1 INTRODUCTION

The software described in this appendix is the current implementation of a system that provides the user of the PR-EDB with the necessary tools to process the data and to create a variety of tables, fits, and graphs for reports and verification of irradiation embrittlement predictions. The present version is far from complete; many important tasks remain which are not yet part of this software package and must be performed through the dBASE or compatible software. In particular, any direct editing of data and the elimination of duplicate records can currently be done only through calls to dBASE. Also, the combination of data from several different files by means of key identifiers needs to be done through dBASE. Software to facilitate these tasks will be part of later updates.

The current version of the software package EDB-Utilities has been written in the Clipper language, which allows compilation of dBASE procedures and has facilities for menu and help screens so that the user can usually run the program without additional instructions. A detailed user's manual will be available in the near future. The program package provides the means for a number of file-manipulation tasks, including the display of data on the computer screen and hardcopy to a printer. The dBASE and related software, such as Clipper, lack the facility for extensive mathematics/statistics calculations and scientific graphs. Some plotting and fitting programs are written in FORTRAN using the IMSL and GRAFMATIC libraries and require ASCII files as input. The ASCII files for the plotting program can be created through the EDB-Utilities file-manipulation feature. Another ASCII file that is needed for input to the fitting of raw Charpy data, RAW_C_PR.dat, has been created from the file RAW_C_PR.dbf and is part of the PR-EDB set of diskettes. Also included is the ASCII file RAW_C_RS.dat, which contains the results of the Monte Carlo uncertainty analysis and can be used as input for the multiple fitting program.

The primary output device for graphic presentations by the EDB-Utilities is the monitor screen in EGA format. (Other formats, e.g., VGA, are considered for future releases.) Utilities are available to transfer the screen picture to the printer, for instance, GRAFPLUS (Sect. A.7). Creating output for plotting devices (e.g., HP plotters) is not part of the current version of the EDB_Utilities but is being considered for future releases.

A.2 EDB-UTILITIES SOFTWARE PACKAGE

The EDB-Utilities package has been designed to provide the end user of the Embrittlement Data Base with convenient means to manipulate, view, plot, and fit the data that are given in dBASE format. There are four major options that can be selected from the first menu (Fig. A.1).

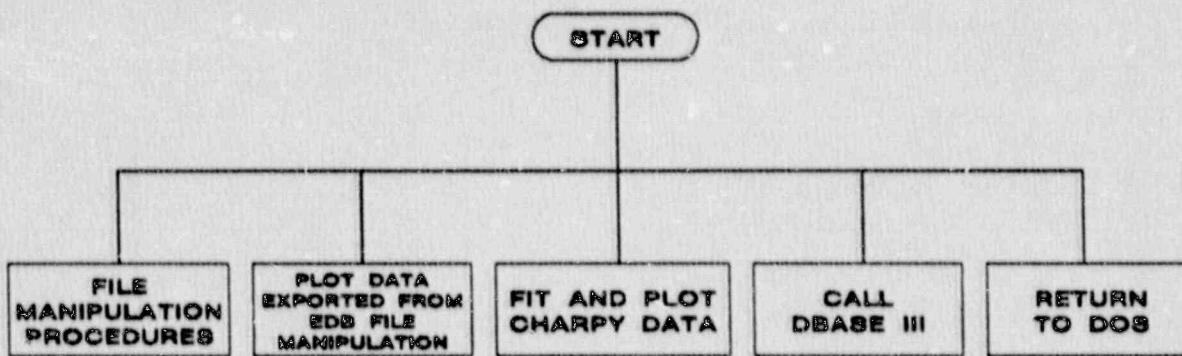


Fig. A.1. Major options selected from first menu.

1. File-Manipulation Procedures. This option is not restricted to PR-EDB files; any file in dBASE format can be processed. Specifically, the following operations can be performed:

- a. Retrieve a file for manipulation
- b. Add or delete fields
- c. Use numerical data for calculations and place results in user-defined fields
- d. Add or delete records
- e. Reorder records
- f. Display or export data
- g. Save working file

A detailed explanation of the different operations will be given in Sect. A.3.

2. Plot Data Exported from EDB files. Numerical data from dBASE files can be exported to ASCII files, as described in Sect. A.3.7, and can be represented in scatter plots with automatic scaling and labeling. Specific curves can be added to the graph as an option. Details are given in Sect. A.4.

3. Fit and Plot Charpy Impact Data from EDB files. Raw Charpy impact data from the file RAW_C_PR.dbf, once in ASCII format, can be fitted to a hyperbolic tangent curve and the results plotted. A Monte Carlo uncertainty analysis program is included which determines the uncertainties in the fitting parameters given the uncertainties in impact energy and test temperature of the original data. The following options are provided:

- a. Single-curve fitting and plotting
- b. Multiple-curve fitting and plotting
- c. Monte Carlo uncertainty analysis
- d. Extracting selected Charpy sets

Details are given in Sect. A.5.

4. Run dBASE III PLUS. This option allows the user to run the dBASE program without exiting EDB-Utilities, provided dBASE III PLUS has been properly installed. The user can, in this manner, easily switch between different options and edit or perform other tasks in dBASE for which no provisions are given in the EDB-Utilities package. Note that all fields in the PR-EDB files are character fields, and any numerical manipulation of data must first use the VAL(...) function to obtain numbers, and convert the results back to character strings using STR(...).

A.3 FILE-MANIPULATION PROCEDURES

File-manipulation procedures are shown schematically in Fig. A.2.

A.3.1 General Considerations

The PR-EDB consists of a number of data files in dBASE format. Each file can be considered as a table of data; the columns of the table are called Data Fields and the rows, Records. Each data field has a given length FIELD_LEN, a unique identifier FIELD_NAME, and a FIELD_TYPE, which declares the data as either "character," or "numeric," or "date" fields, and this information is coded in a special manner at the head of the dBASE files.

All data fields in the PR-EDB are of character type; this allows one to use blanks for missing data and also the introduction of scientific notation, for which dBASE III PLUS has no provisions. However, the actual data types must be provided in some way to processing codes in order to perform calculations, comparisons, and orderings. For this reason, a "structure file" is assigned to each data file which has the same name but the extension .str instead of .dbf (e.g., REAC_PR.dbf has the structure file REAC_PR.str). A structure file has the same first five fields as a dBASE structure file, but in addition a field F_T for the actual field type, which can be C for character, N for numeric, S for scientific notation, and

D for date. It also has a field DESC which contains a detailed description of the data field plus the type of units used in brackets [...]. This information is displayed in menu screens concerning data fields and is used to label the axes in the plots that are generated in the plotting option of EDB-Utilities.

All file-manipulation procedures, such as changing data fields or records, reordering, and displaying are never performed on the original data file; the input file is first copied to a "working file" WORK.dbf, and the associated structure file, to STRUCT.dbf. A dBASE-type structure file TMPS.dbf is also needed for some file-manipulation procedures. After performing the desired procedures, the working files may be saved to new files, or the old files may be overwritten. The working files remain in the directory and can be accessed again by the EDB-Utilities, even after the program has been temporarily terminated. The original files remain as they are unless overwritten by the user.

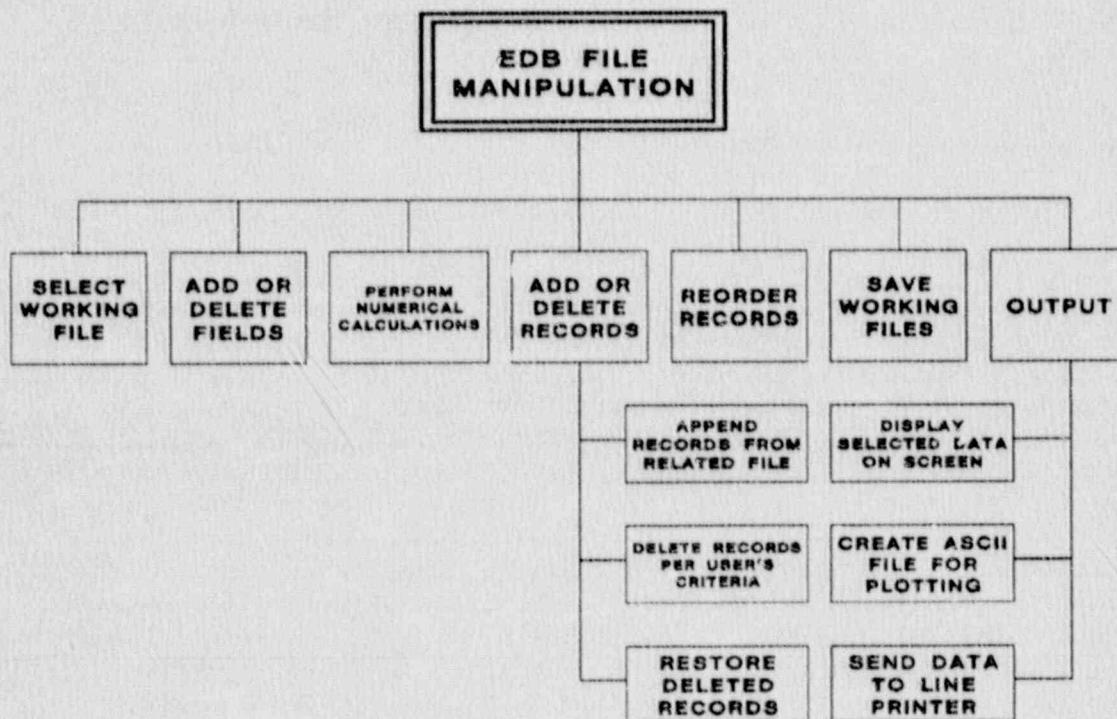


Fig. A.2. EDB-Utilities file-manipulation procedures.

A.3.2 Retrieval of Files for Manipulation

Any dBASE file can be processed by the manipulation option of EDB, including the structure files. The input files need not be PR-EDB files (i.e., containing only character fields), but the working file WORK.dbf has this property. A new structure file is created if an associated structure file is not present or is incompatible with the input file. Field names used in the EDB files are listed in a file STR_ALL.dbf, and this file is used to put information in the new structure file for such field names. Other information must be entered by the user, who can also change the information from STR_ALL.dbf, if necessary. The new structure file is saved if none was previously available, but old files are not overwritten, even if they are incompatible. The working file WORK.dbf has no associated index file. However, an index file can be used with the input file to ensure the proper order in the working file.

A.3.3 Addition or Deletion of Fields

Fields can be added to the working file for storing the results of calculations, and only these fields can be used for this purpose. However, fields added in a previous run to a file that was subsequently saved can still be used for results after later retrieval of the file. Any field can also be deleted including ones that had been added previously. Both additions and deletions become final only after the modification command is given; tentative additions and/or deletions can be aborted, if necessary.

A.3.4 Addition and Deletion of Records

Records can be deleted from the working file according to conditions entered by the user. This step is done using the option "Select Records From Working File." Note that the deletion of records in dBASE is a two-step procedure; first records to be deleted are marked and following that the marked records are completely eliminated. The marked records can be restored as long as the final elimination step has not been executed. Consequently, there are two options in the selection procedure, namely deletion (marking) of records and restoring (unmarking). All records can be deleted if the restoration option is entered before deleting any records. Both procedures are done for records that satisfy conditions that are entered by the user in the menu screens provided. Using suitable sequences of deletions and restorations, practically all selection criteria can be satisfied (see Examples, Sect. A.6). After the selection procedure is completed, all deleted (i.e., marked) records are removed (eliminated) and can no longer be restored. Selections can also be aborted in case of an error.

Records can be added to the working file by appending them from another dBASE file. This file need not be a PR-EDB file or a saved working file and may have numerical or date fields. All data are again converted into character fields before appending. Data are appended according to the field names, which must be the same in the working and the added file.

A.3.5 Calculations

A variety of numerical operations can be performed on fields of numerical- or scientific-type data, with the results entered into user-defined scientific fields. Only one operation at a time can be performed, namely addition, subtraction, multiplication, division, exponentiation plus exponential function, and logarithm. More complicated formulas can be calculated in a properly chosen sequence of operations using, perhaps, some auxiliary fields for temporary storage. A warning is given for improper operations such as division by zero, and a blank record is given as a result. A blank is also given if one of the operations has a blank record, indicating missing data.

A.3.6 Reordering

No index files are associated with the working file, but records can be sorted in any manner by entering the ordering criteria in the menu screen provided. The chosen arrangement can be saved in an output file, if desired.

A.3.7 Display and Export

The data in the working file can be displayed on the screen or printed. A menu is provided which allows the selection of fields to be displayed or printed in any desired order. For printing, the user must supply the number of characters per line and the number of lines per page. The fields will be distributed over several pages if the width of the output exceeds the number of lines permitted, and, of course, if there are more records than the number of lines per page. The data can also be saved in an output file that is formatted in a manner suitable for subsequent plotting with the EDB-Utilities plotting program.

A.3.8 Save Working File

The working file and the associated structure file can be saved by copying them under a user-specified name. A warning will be given if a file under this name already exists, but overwriting an existing file is permitted, destroying the old file in the process.

A.4 PLOTTING PROGRAM

The data input for the plotting program (Fig. A.3) is prepared by the file-manipulation procedures and exported to special plot files as described in the previous Sect. A.2. Data from up to ten different files can be put in the same plot with different symbols assigned to each data set. Once the plotting program has been called from the main menu, the following information must be given:

1. The name of the structure file that was associated with the working file from which the data set was prepared. The sole purpose of this file is to provide the information about which data fields should be used for the x- and y-coordinates, respectively. Thus, any structure file may be used which has the corresponding numerical data fields in it, even if another working file was used to create the plot input. The field names used for coordinates must be common to all plot files for the same plot but other fields may differ.
2. The plot title. The title will appear in large letters on top of the plot.
3. Data fields for the coordinates. The user selects these fields from an input screen that shows the numerical fields in the structure file. The descriptions and units given in the structure file will be used for labeling the coordinate axes.
4. Names of the plot files and associated descriptions. The data descriptions will be used for (optional) legends that can be placed at any desired location in the plot and will appear in the same sequence as they are entered in the screen. Blanks as descriptions will be ignored in the legend.
5. Curves. Certain types of curves can be placed on the plots in addition to the data points. The user selects the desired type of curves from the menu screen and may also enter legends for these curves.
6. Symbols and colors. An option is provided for selecting symbols and colors (for color monitors and printers) for the different data sets and the curves. Dashes of different densities can be selected for the curves.

Once the input is given, a picture of the plot will appear on the screen. Scaling and labeling of the axes are automatic, using the information from the input data and structure files. The user will be asked to place the legend at some location in the plot where it does not interfere with the data points and curves. The plot may then be sent to the printer if the necessary connections are in place.

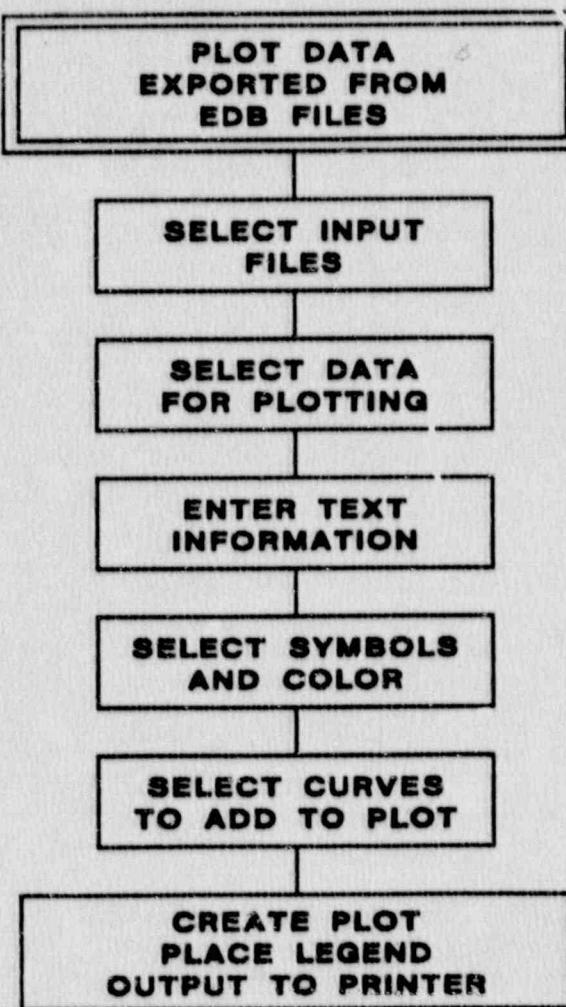


Fig. A.3. Flowchart for EDB-Utilities plotting program.

All input data for the plot, including information about curves, symbols, and colors, are saved and can be used with subsequent plots. The user can choose to use the same plot data again, with a possible change in curves, symbols, colors, and the placement of the legend. The input information may also be changed selectively or completely erased (this applies only to the information that is generated during the plotting procedure, such as name of input files, title, and legends, but not to the data files which remain intact).

A.5 CHARPY FITTING AND PLOTTING

Procedures for fitting and plotting of raw Charpy data are shown schematically in Fig. A.4.

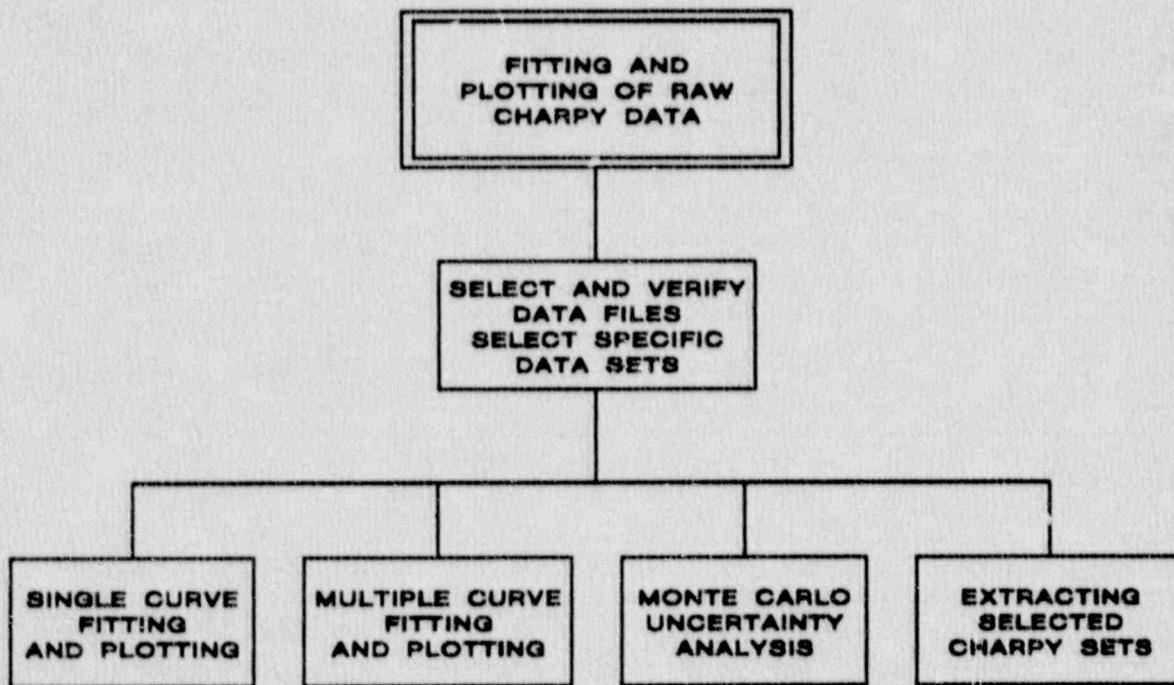


Fig. A.4. Procedures for fitting and plotting raw Charpy data.

A.5.1 General Considerations

The EDB-Utilities software allows the creation of fits and plots for any given set of raw Charpy data using the hyperbolic tangent function. Each set of raw Charpy data is uniquely identified by the keys PLANT_ID + CAPSULE + HEAT_ID + SPEC_ORI, and all data points having the same combination of these four identifiers are combined into the same set, unless the sequence of data records is interrupted by a record that contains different key identifiers. The program requires an input file in ASCII format, RAW_C_PR.dat, that was created from the PR-EDB file RAW_C_PR.dbf. All data are converted to U.S. units, the unit fields are removed, and a one-character field is added in front of SPEC_ID. If this field contains an asterisk, the record will be excluded from processing; this feature is useful for removing outliers. No outliers are tagged in

this way in the attached files, but an example is given below (see Figs. A.7 and A.8). The modified file was then converted to ASCII format using the dBASE command "Copy to RAW_C_PR.dat SDF."

The fitting procedure is completely automated. Upper and lower shelf values are restricted by adding a penalty proportional to the deviation from initial estimates to avoid unphysical fits. A first inspection of the points is made to obtain a rough estimate for the initial values of the curve parameters. A nonlinear least-squares fitting program ZSSQ from the IMSL library is used to determine the best fit, which is then plotted and appears on the screen, including data points with automatic scaling and labeling of the axes in both U.S. and international units. The plots can be sent to the printer, including a summary of transition temperatures and upper shelf data (in international units). All internal calculations are done in international units, centigrade and joule. Currently, no options are available to fit and plot lateral expansion or fracture appearance versus test temperature data. A detailed report of the fitting and uncertainty analysis will be available in the near future.

As stated in Sect. A.2, Item 3, there are four options for this part of the program.

A.5.2 Single-Curve Fitting and Plotting

In this option, the selected data sets are fitted and plotted, one at a time, with optional printing of the results. Titles and subtitles for the plots can be either given individually or at the start of the procedure, with the key identifiers serving as subtitles. The user has the option to skip any data set and to terminate the procedure without reading the whole input file.

A.5.3 Multiple-Curve Fitting and Plotting

With this option, up to ten different fits can be placed into one plot which is useful for comparing data before and after irradiation. The user enters the overall title and legends for the individual fits. Default legends can be used which contain the name of the capsule and the total fluence (>1.0 MeV). Legends can be placed by the user at a suitable free spot in the plot.

A.5.4 Monte Carlo Uncertainty Analysis

Uncertainties for the fitting parameters are needed to determine accuracy and credibility of the transition temperature and upper shelf data. A covariance matrix of the fitting parameters is part of any least-squares procedure, but these covariances are not used for uncertainty analysis in the EDB-Utilities. The unavoidable linearization used for determining the covariances disregards second-order effects, and there is no possibility to account for uncertainties in test temperature. A more reliable procedure is the use of random variations of the input data (Monte Carlo procedure); such variations can be applied to both impact energy and test temperature,

and the results reflect more accurately the influences of nonlinearities. The necessary computing time is, of course, increased by a large factor but remains manageable for today's computers. Because this option is completely automated, including printing the plots, a fairly large amount of data sets can be processed overnight or over a weekend. Nonphysical results and results that deviate substantially from the mean are eliminated from the sampling. This has the added advantage that fits can be obtained even after some tries have failed initially.

The user enters the (one-standard deviation) uncertainties for the impact energy and test temperature and the number of iterations. Unsuccessful iterations (i.e., the ones rejected by the program as nonphysical or inconsistent with the rest) are not counted; however, the total number of tries may not exceed five times the specified iteration number. Also needed is the number of sets to be skipped at the beginning of the input data file and the number to be processed. Processing is done, one set at a time, in sequence, starting after the specified number of sets have been skipped. A more specific selection can be obtained by using, as the input data file, the one created by the selection procedure (Sect. A.5.5). Continuous plotting and printing can be chosen as an option, with the user providing the common title and the key identifiers as a subtitle.

Three output files are created by the procedure, the names of which are either entered by the user or assigned by default. The "Summary Output File" (default name FORT15; the 15 in FORT15 is the unit number of the FORTRAN output file) is a list which contains the set number, key identifiers, fluence, irradiation temperature, transition temperature at 41 J and 68 J (30 ft-lb and 50 ft-lb), upper and lower shelf energy (all in international units), the number of specimens in the set, and the number of successful iterations for each processed set. The "Covariance Output File" (default name FORT16) contains mean values, standard deviations, and correlations for all fitting parameters which include the transition temperature at the center of the curve and 1/slope, which is one-half of the impact energy range of the transition region. The "EDB_dBASE Output File" (default name FORT17) is intended for conversion to a dBASE file whose data are an alternative to the file SHFT_PR. This file can also be used as an "auxiliary" input file in the multiple fitting option (Sect. 5.3). Its data are given in U.S. units. A "status report" listing the results from all successful and unsuccessful iterations is placed on the screen during the procedure. The screen output can be redirected to a file as an option.

The complete file RAW_C_PR.dat has been processed with this program, with 10 J and 4°C as input uncertainties for impact energy and test temperature, respectively, and 200 iterations. The summary and dBASE output files, RAW_C_PR.sum and RAW_C_RS.dat, respectively, are included in the data disks. The dBASE version of RAW_C_RS.dat (RAW_C_RS.dbf) is also included in the package. The covariance file, which is very large and of limited usefulness, is not included, but a sample printout is given with the examples.

A.5.5 Extracting Selected Charpy Sets

The raw data files for individual Charpy sets are quite large and thus require long search times in sequential access. It is, therefore, convenient and saves time to copy small subsets from a larger file, if such subsets are processed repeatedly. This is accomplished through the data-selection option. The user specifies the selection criteria and may, in addition, skip certain sets and terminate the procedure without going through the rest of the input file. No processing is done during this option, but the user-specified output file (default FORT20) can now serve as an input file for any subsequent processing step.

A.5.6 Selection of Input Files and Data Sets

The four options require essentially the same input information that is requested in several input screens:

1. Names of input data files, primary and auxiliary. The primary input file is RAW_C_PR.dat which is included in the package and given as default at the menu. The user may change this name to that of any other data file that contains raw Charpy data in the same format, for instance, one obtained from the selection procedure described in Sect. A.5.5. The multiple-fitting option permits the use of an additional, "auxiliary" input file that contains the values of the fitting parameters as generated by the Monte Carlo uncertainty analysis as EDB-dBASE output file. The file RAW_C_RS.dat is included in the package and appears as default. Its use is optional and allows the user to bypass the least-squares fitting, speeding up the process and avoiding problems with convergence. The data sets in the primary and auxiliary files need not be identical or in the same sequence.
2. Selection criteria. Reactors and materials can be selected for processing by entering the selection criteria in the appropriate menu screen. These criteria are not used in Monte Carlo uncertainty analysis. The Monte Carlo uncertainty analysis is designed to process a large number of data sets in sequence without user intervention. Consequently, only the starting number and the number of sets to be processed can be given. The number associated with each set can be found in the (ASCII) file RAW_CPY.SUM. These numbers are entered in the input screen as discussed in Sect. A.5.4.
3. Selection of symbols and colors. This is essentially the same procedure as for plots in Sect. A.4, Item 6. Data points that are excluded from the fitting procedure can be plotted using different symbols. Such points can also be completely eliminated from the plot by using the empty symbol (zero symbol) for rejected data points.

A.6 EXAMPLES

The following examples are intended as exercises for the user of the EDB-Utilities to guide the user through the various options and to show typical applications of these processing steps. It is assumed that the user will try to duplicate the sequence of processing steps which are listed in the following pages and to verify the results. The reader of this report who is only interested in the capabilities of the EDB-Utilities may have difficulty in understanding the listing of processing steps without running the program and may safely skip these parts, concentrating on the general introductions and resulting tables and graphs.

The EDB-Utilities program is started by typing EDB. The user then selects the various options as indicated below.

A.6.1 File-Manipulation Procedures with Plots

In the following example, the relation between shift in transition temperature and upper-shelf energy drop will be investigated. This example was chosen because it makes use of most of the file-manipulation features and needs only one PR-EDB raw data file SHFT_PR.dbf. For more in-depth investigations, chemistry data need to be added which would require additional processing with dBASE software. Also, the proposed manipulation of the data file could be done, perhaps faster, directly in dBASE by someone familiar with the software. However, the creation of the plot file and the actual plotting requires the EDB-Utilities software.

This example compares the values given in DTT30 for the shift of transition temperature at 30 ft-lb with the relative upper-shelf drop given in DUSE_REL in percent. However in many reports, these values are not listed directly and only the transition temperatures, UTT30 and ITT30, and upper shelf energies, UUSE and IUSE, for unirradiated and irradiated conditions are given, from which the shift values can easily be calculated. Thus, two new fields are added to the input file, DTT and DUSE, which will contain either the reported or calculated value, whatever is available, with preference for the directly reported value. The values for base material (plates, forgings, and standard reference materials) and welds will be separated. Heat-affected-zone data will be ignored.

Using the file-manipulation option, the following sequence of procedures will be performed:

```

Retrieve:      SHFT_PR.dbf
Delete records: PROD_ID = HAZ
               TEMP_U <> F*
               USE_U <> FT-LB
               (data with non-U.S. units are eliminated;
                conversion to U.S. units is another possibility)
  
```

*The symbol <> (abbreviation for < or >) stands for "not equal". Symbols are listed as they appear on the menu screen (and dBASE commands).

Add two user-defined fields: Name and description as follows

DTT Shift in Transition Temperature @ 30 ft-lb
 [degree F]
 DUSE Upper Shelf Drop [Percent]
 (text in brackets [] are units for the plot)

Save working file: to a temporary file (e.g., TMPO)

Delete records: DUSE_REL <= 0
 (use reported USE_REL with positive drop only)

Calculate: DUSE = DUSE_REL + 0 (put DUSE_REL into DUSE)

Save working file: to another temporary file (e.g., TMP01)

Retrieve: TMPO.dbf

Delete records: DUSE_REL > 0 (retain records without DUSE_REL)
 UUSE <= 0
 IUSE <= 0 (eliminate missing UUSE and IUSE values)

Calculate: DUSE = UUSE - IUSE
 DUSE = DUSE / UUSE
 DUSE = DUSE * 100
 (put calculated values (UUSE - IUSE)/UUSE * 100 to
 DUSE)

Delete records: DUSE <= 0 (eliminate negative USE drop)

Add records: From TMP01.dbf

The working file now contains both reported and calculated relative drop values for USE with preference for reported values and negative drop values eliminated. A similar sequence can be performed for shift in transition temperature:

Save working file: to a temporary file (e.g., TMP1)

Delete records: DTT30 <= 0

Calculate: DTT = DTT30 + 0 (puts DTT30 into DTT)

Save working file: to another temporary file (e.g., TMP11)

Retrieve: TMP1.dbf

Delete records: DTT30 > 0 (eliminate reported DTT30)

Calculate: DTT = ITT30 - UTT30 (put calculated shift into DTT)

Delete records: DTT <= 0 (eliminate negative shift)

Add records: From TMP11.dbf

The working file now contains all the desired values for transition temperature shift and upper shelf drop in the two newly created fields DTT and DUSE. These values are ready for printing and plotting. Through the processes of deleting and adding, the original sequence of records has been destroyed. It may be desirable to put the records in some new order before printing (the order of records is immaterial for plotting), for instance, through:

Reorder records: DTT - ASCENDING

By sending selected fields of the working file to the printer, one obtains the list in Table A.1. For plotting, it may be desirable to have different symbols for base materials and welds. This can be accomplished by selecting subsets of records from the file before exporting them to the plot files. First, the working file must be saved to a permanent file:

Save working file: to DTT_US.dbf

Delete records: PROD_ID = W (for base material)

Export to plot file: to BASE.dat

Retrieve: DTT_US.dbf

Delete records: PROD_ID \diamond W (for weld material)

Export to plot file: to WELD.dat

The data are now ready for plotting. After selecting the plotting option, the following input must be entered:

Title: Shift in Transition Temperature versus Upper Shelf Drop

Structure File: DTT_US (any of the temporary files will do also)

Coordinates: DTT for x-coordinate, DUSE for y-coordinate

Input files and legends: BASE.dat - Base Material
WELD.dat - Weld Material

The present plot needs no additional curves, but suitable symbols and colors may be selected. Following that, the plot is created automatically. The last step is to put the legends at an appropriate place after prompting. The finished plot is shown in Fig. A.5. There seems to be a near linear relation between transition temperature shift and upper shelf drop for low shift values, although the USE drop appears to level off at high shift values. No obvious distinctions between base metal and welds can be seen. The data scatter is rather large and suggests further investigation of additional factors which may influence the relation between CVT shift and USE drop.

Table A.1. Transition temperature shifts versus upper shelf drop from example, Sect. A.6.1. Data are arranged according to increasing transition temperature shift

Shift in Transition Temperature vs. Upper Shelf: Page 1													
PLANT_ID	CAPSULE	HEAT_ID	PROD_ID	SPEC_ORI	CSP_F1	UTT30	ITT30	DTT30	DTT	UUSE	IUSE	DUSE_REL	DUSE
DB1	A	FDB101	F	TL	1.290E+19	-26	-24	2	2.000E+00	140	127	9.286E+00	
DR2	3	MDR201	W	TL	1.300E+16	-10	-8	2	2.000E+00	101	96	4.950E+00	
NA2	V	MNA201	W	TL	2.410E+18	-26	-24	2	2.000E+00	115	92	2.000E+01	
CB1	Z	WCB101	W	TL	3.080E+18	-35	-30	5	5.000E+00	128	123	3.906E+00	
NA2	V	FNA201	F	TL	2.410E+18	60	69	9	9.000E+00	75	62	1.733E+01	
AD1	V	WAD101	W	TL	4.580E+18	-80	-70	10	1.000E+01	171	155	9.357E+00	
FA2	U	WFA201	W	TL	5.610E+18	-30	-20	10	1.000E+01	144	132	8.333E+00	
AN2	97	MAN201	W	TL	3.340E+18		10	10	1.000E+01	142	2.7	2.700E+00	
BR	119	SHR3	SRM	LT	1.500E+18	0	15	15	1.500E+01	96	70	2.700E+01	
BR	121	WBR_01	W	TS	0.0	-70	-55	15	1.500E+01	95	92	3.158E+00	
OC3	B	FOC302	F	TL	3.120E+18	-9	10	19	1.900E+01	112	100	1.071E+01	
VT	300	PVY_01	P	LT	4.300E+16	8	27	19	1.900E+01	148	128	1.351E+01	
KME	R	FKME02	F	LT	2.070E+19	-50	-30	20	2.000E+01	157	153	2.548E+00	
PB2	V	FPB201	F	LT	4.740E+18	-55	-35	20	2.000E+01	145	135	6.897E+00	
P11	P	FP1101	F	LT	1.250E+19	-25	-5	20	2.000E+01	158	142	1.013E+01	
WC1	U	WWC101	W	TL	3.390E+18	-50	-30	20	2.000E+01	100	92	8.000E+00	
SL2	WB3	PSL201	P	TL	1.600E+18	40	61	21	2.100E+01	117	111	5.128E+00	
CR3	B	PCR301	P	TL	1.000E+18	16	37	21	2.100E+01	94	89	5.319E+00	
NA1	V	FNA101	F	TL	2.490E+18	46	65	21	2.100E+01	83	75	1.176E+01	
AN2	97	PAN201	P	LT	3.410E+18		24	21	2.100E+01	142	8.4	8.400E+00	
TR0	U	WTRO01	W	TL	3.880E+18	2	24	22	2.200E+01	111	81	2.410E+00	
ZN1	T	PZN101	P	TL	1.800E+18	25	50	25	2.500E+01	117	114	2.564E+00	
AD1	V	FAD101	F	LT	4.580E+18	-25	0	25	2.500E+01	141	127	9.929E+00	
AD1	V	FAD101	F	TL	4.580E+18	-22	3	25	2.500E+01	128	119	7.031E+00	
GIN	V	FGIN01	F	LT	4.900E+18	-40	-15	25	2.500E+01	183	160	1.257E+01	
SQ2	T	FSQ201	F	TL	2.200E+18	0	25	25	2.500E+01	80	82	6.818E+00	
PL1	1	PPL101	P	LT	2.300E+17		25	25	2.500E+01		10.0	1.000E+01	
CR3	B	WCR301	W	TL	1.170E+18	36	64	28	2.800E+01	76	70	7.895E+00	
CR3	B	WCR301	W	TL	1.170E+18	36	64	28	2.800E+01	79	70	1.139E+01	
DB1	A	FDB102	F	TL	1.290E+19	16	44	28	2.800E+01	127	118	7.087E+00	
QC2	13	WQC201	W	TL	1.690E+16	-30	0	30	3.000E+01	125	107	1.440E+01	
CL1	U	PCL101	P	LT	3.270E+18	-20	10	30	3.000E+01	104	93	1.058E+01	
G1K	T	FG1K01	F	LT	1.750E+19	-40	-10	30	3.000E+01	183	143	2.186E+01	
VS1	U	WVS101	W	TL	6.390E+18	-55	-25	30	3.000E+01	91	87	4.396E+00	
WC1	U	WPC101	P	LT	3.390E+18	-20	10	30	3.000E+01	148	145	2.027E+00	
OC3	B	FOC301	F	TL	3.120E+18	-9	12	32	3.200E+01	148	125	1.554E+01	
OC1	A	POC102	P	TL	8.950E+18	18	39	34	3.400E+01	108	106	1.852E+00	
SL2	WB3	PSL201	P	LT	1.600E+18	3	38	35	3.500E+01	133	118	1.128E+01	
AN1	8	PAN102	P	LT	4.280E+18	-13	22	35	3.500E+01	147	123	1.633E+01	
ML1	2	WML101	W	TS	3.300E+17	-45	-10	35	3.500E+01	112	75	3.304E+01	
PB2	R	FPB201	F	LT	2.010E+19	-45	-10	35	3.500E+01	145	140	3.448E+00	
P12	T	FP1201	F	TL	1.050E+19	0	35	35	3.500E+01	108	93	1.389E+01	
KO1	T	FKO101	F	TL	9.800E+18		35	35	3.500E+01	168	139	1.730E+01	
P11	P	FP1101	F	TL	1.250E+19	-27	10	37	3.700E+01	143	136	4.895E+00	
ZN2	U	PZN201	P	LT	2.000E+18	37	75	38	3.800E+01	128	121	5.469E+00	
DR2	4	PDR201	P	LT	9.500E+18	0	39	39	3.900E+01	153	117	2.353E+01	
NA1	V	FNA101	F	LT	2.490E+18	-6	33	39	3.900E+01	135	122	9.630E+00	
OC3	B	SHSS02	SRM	LT	3.120E+18	56	95	39	3.900E+01	130	96	2.615E+01	
BR	127	SHR3	SRM	LT	7.100E+18	0	40	40	4.000E+01	96	82	1.458E+01	
QC2	12	PQC201	P	LT	1.270E+19	-15	25	40	4.000E+01	135	123	8.889E+00	
VS1	U	PVS101	P	LT	6.390E+18	-20	20	40	4.000E+01	133	131	1.504E+00	
DR1	G4-1	WDR101	W	TS	1.000E+19	0	43	43	4.300E+01	110	91	1.727E+01	
QC2	18	WQC201	W	TL	6.600E+16		43	43	4.300E+01		27.0	2.700E+01	
QC2	C	WQC201	W	TL	1.020E+18	4	49	45	4.500E+01	67	54	1.940E+01	
MC1	U	PMC101	P	LT	4.140E+18	5	50	45	4.500E+01	140	133	5.000E+00	
SU2	X	PSU201	P	TL	3.020E+18	5	59	45	4.500E+01	105	95	9.524E+00	
OC3	A	WOC301	W	TL	8.100E+17	45	93	48	4.800E+01	66	54	1.818E+01	
AH1	A	PAN101	P	LT	1.030E+19	-31	17	48	4.800E+01	132	108	1.818E+01	
ZN2	U	SHSS02	SRM	LT	2.000E+18	50	100	50	5.000E+01	124	111	1.048E+01	
DR2	S	PDR201	P	LT	1.900E+19	0	50	50	5.000E+01	139	127	8.633E+00	
MC1	U	PMC101	P	TL	4.140E+18	0	50	50	5.000E+01	101	100	9.901E+01	
SA2	T	PSA201	P	LT	2.560E+18	30	80	50	5.000E+01	122	115	5.738E+00	
SU1	T	PSU101	P	LT	2.500E+18	-10	40	50	5.000E+01	125	120	4.000E+00	
AN2	97	PAN201	P	TL	3.330E+18		60	50	5.000E+01	124	1.2	1.200E+00	
CK2	Y	WCK201	W	TL	5.550E+18		50	50	5.000E+01		9.0	9.000E+00	
CTY	H	PCTY07	P	LT	1.790E+18	-30	23	53	5.300E+01	120	117	2.500E+00	
BR	119	WBR_01	W	TS	1.500E+18	-70	-15	55	5.500E+01	95	80	1.579E+01	
P12	T	FP1201	F	LT	1.050E+19	-25	30	55	5.500E+01	150	133	1.133E+01	
QC2	G14	PQC201	P	LT	4.140E+19	-15	40	55	5.500E+01	135	119	1.185E+01	
SU2	X	PSU201	P	LT	3.020E+18	-5	50	55	5.500E+01	125	120	4.000E+00	
CPR	1	WCP01	W	TL	2.300E+17	-10	45	55	5.500E+01	112	85	2.400E+01	
PL1	1	WPL101	W	TL	2.300E+17		55	55	5.500E+01		18.0	1.800E+01	
CTY	H	PCTY02	P	LT	1.790E+18	-50	7	57	5.700E+01	135	130	3.704E+00	
DR2	2	WDR202	W	TL	1.870E+19	-13	45	58	5.800E+01	71	70	1.408E+00	
DR3	G10	PDR301	P	LT	9.250E+18	-10	48	58	5.800E+01	135	115	1.481E+01	
NA1	1	PHA101	P	LT	2.400E+17	-70	-12	58	5.800E+01	139	131	5.755E+00	
ML1	2	PML101	P	LT	3.300E+17	15	73	58	5.800E+01	106	88	1.698E+01	

Table A.1 (continued)

Shift in Transition Temperature vs. Upper Shelf: Page 2

PLANT_ID	CAPSULE	HEAT_ID	PROD_ID	SPEC_ORI	CSP_F1	UTT30	ITT30	DTT30	DTT	UUSE	IUSE	DUSE_REL	DUSE
RS1	D	PRS101	P	TL	6.600E+18	4	62	58	5.800E+01	90	88		2.22E+00
CC1	263	WCC101	W	TL	6.000E+18	-50	9	59	5.900E+01	160	119		2.563E+01
DR3	14	PD8301	P	LT	1.020E+18	-10	50		6.000E+01	135	112		1.704E+01
ZN1	T	PZN101	P	LT	1.800E+18	-5	55		6.000E+01	140	132		5.714E+00
BR	127	PBR_01	P	TL	7.100E+18	-5	55	60	6.000E+01	82	75		1.098E+01
CC1	263	PCC103	P	LT	6.000E+18	8	68	60	6.000E+01	138	115		1.667E+01
OC1	A	POC102	P	LT	8.950E+18	-5	55	60	6.000E+01	141	137		2.637E+00
P12	T	WP1201	W	TL	1.050E+19	-75	-15	60	6.000E+01	103	92		1.068E+01
P12	V	WP1201	W	TL	5.490E+18	-75	-15	60	6.000E+01	103	100		2.913E+00
SA1	T	SHSS02	SRM	LT	2.560E+18	50	110	60	6.000E+01	123	115		6.504E+00
SQ2	T	FSQ201	F	LT	2.200E+18	-70	-10	60	6.000E+01	134	118		1.194E+01
SU2	X	SHSS02	SRM	LT	3.020E+18	40	100	60	6.000E+01	125	115		8.000E+00
ZN1	U	PZN101	P	TL	8.920E+18	25	85	60	6.000E+01	117	102		1.282E+01
CK1	T	PCK101	P	LT	1.800E+18	5	65	60	6.000E+01	130	108	16.0	1.600E+01
CK1	T	SHSS02	SRM	LT	1.806E+18	45	105	60	6.000E+01	120	120	15.0	1.500E+01
FC1	S225	PFC101	P	LT	5.100E+18	22	82	60	6.000E+01	138	119	13.0	1.300E+01
DR2	4	MDR202	W	TL	6.400E+18	-13	50	63	6.300E+01	71	51		2.817E+01
DC3	B	MDC301	W	TL	3.120E+18	45	109		6.400E+01	66	49		2.576E+01
DC3	B	MDC301	W	TL	3.120E+18	45	109	64	6.400E+01	66	49		2.576E+01
HC2	V	FMC201	F	LT	3.060E+18	-75	-10	65	6.500E+01	156	134		1.410E+01
DC2	U	POC201	P	LT	3.510E+18	5	70	65	6.500E+01	144	124	15	1.500E+01
ZN1	T	SHSS02	SRM	LT	1.800E+18	42	108		6.600E+01	124	106		1.452E+01
AM1	A	PAN101	P	TL	1.930E+19	17	83	66	6.600E+01	96	83		1.354E+01
DC1	S	SHSS02	SRM	LT	2.980E+18	66	112	66	6.600E+01	124	123		8.065E-01
CTY	H	PCTY04	P	LT	1.790E+18	-24	43	67	6.700E+01	126	122		3.175E+00
CAB	P	PCAB02	P	LT	1.430E+19	-24	44	68	6.800E+01	125	123		1.600E+00
SL1	97	PSL101	P	LT	5.500E+18	8	76	68	6.800E+01	139	107	23.0	2.300E+01
CC2	263	WCC201	W		8.060E+18			69	6.900E+01			23	2.300E+01
BR	122	PBR_01	P	TL	2.300E+19	-5	65	70	7.000E+01	82	62		2.439E+01
CL1	U	WGL101	W	TL	3.270E+18	-55	15	70	7.000E+01	112	101		9.821E+00
NC2	V	FMC201	F	TL	3.060E+18	-25	45	70	7.000E+01	94	85		9.574E+00
SA2	T	PSA201	P	TL	2.560E+18	10	80	70	7.000E+01	97	89		8.247E+00
SO1	T	FSQ101	F	LT	2.740E+18	-35	35	70	7.000E+01	116	98		1.552E+01
SU1	T	SHSS02	SRM	LT	2.500E+18	45	115	70	7.000E+01	123	105		1.463E+01
CK1	T	PCK101	P	TL	1.800E+18	20	90	70	7.000E+01	94	84	10.0	1.000E+01
CK2	X	WCK201	W	TL	8.530E+18			70	7.000E+01			15	1.500E+01
ML2	W97	PML201	P	LT	3.780E+18	38	108	70	7.000E+01	131	94	28.0	2.800E+01
SL1	97	PSL101	P	TL	5.500E+18	16	86	70	7.000E+01	103	78	24.0	2.400E+01
DC2	U	POC201	P	TL	3.510E+18	26	99	73	7.300E+01	95	94	1	1.000E+00
CPR	1	PCPR01	P	LT	2.300E+17	-66	8	74	7.400E+01	129	112	13	1.300E+01
SL1	97	WSL101	W	TL	5.500E+18	-53	21	74	7.400E+01	144	100	31.0	3.100E+01
DR1	G4-1	SNH3	SRM	LT	1.000E+19	10	85	75	7.500E+01	70	58		1.714E+01
HB2	T	PHB203	P	LT	4.110E+19	30	105	75	7.500E+01	114	105		7.895E+00
SO1	U	FSQ101	F	TL	1.080E+19	45	120	75	7.500E+01	72	58		1.944E+01
SU2	V	PSU201	P	LT	1.880E+19	-5	70	75	7.500E+01	125	121		3.200E+00
SU2	V	PSU201	P	LT	1.880E+19	5	80	75	7.500E+01	105	94		1.048E+01
ZN2	T	PZN201	P	LT	8.700E+18			75	7.500E+01			19.0	1.900E+01
ML2	W97	WML201	W	TL	3.780E+18	-30	46	76	7.600E+01	132	98	26.0	2.600E+01
CTY	D	PCTY04	P	LT	2.220E+19	-24	54	78	7.800E+01	126	110		1.270E+01
DC1	E	WDC101	W	TL	1.500E+18	-5	75		8.000E+01	64	55		1.406E+01
SC1	G1	WQC101	W	TL	8.900E+18	10	90		8.000E+01	100	85		1.500E+01
FA1	U	WFA101	W	TL	1.650E+19	-80	0	80	8.000E+01	149	108		2.752E+01
FA1	Y	WFA101	W	TL	5.830E+18	-80	0	80	8.000E+01	149	130		1.275E+01
P11	R	FP1101	F	LT	4.030E+19	-25	55	80	8.000E+01	158	145		8.228E+00
SO2	T	MSQ201	W	TL	2.200E+18	-75	5	80	8.000E+01	112	110		1.786E+00
CK1	T	WCK101	W		1.800E+18	-90	-10	80	8.000E+01	110	80	27.0	2.700E+01
ZN1	X	PZN101	P	TL	1.500E+19			80	8.000E+01			21.0	2.100E+01
CC2	263	PCC202	P	LT	8.060E+18			84	8.400E+01			21	2.100E+01
FA1	Y	PFA101	P	LT	5.830E+18	-30	55	85	8.500E+01	140	128		8.571E+00
P12	R	FP1201	F	TL	4.420E+19	0	85	85	8.500E+01	108	98		9.259E+00
ZN1	U	PZN101	P	LT	8.920E+18	-5	80	85	8.500E+01	140	120		1.429E+01
CK2	X	PCK201	P	LT	1.050E+19			85	8.500E+01			33	3.300E+01
YR	W1	SASTM_S1	SRM		0.220E+19	15		85	8.500E+01	87	62	17	1.700E+01
P11	R	FP1101	F	TL	4.030E+19	-27	60	87	8.700E+01	143	129		9.790E+00
CC1	263	SHSS01	SRM	LT	5.900E+18	39	127	88	8.800E+01	135	109		1.926E+01
IP3	T	PIP301	P	LT	2.920E+18	-2	87	89	8.900E+01	132	119		9.848E+00
DR2	2	PDR201	P	LT	2.980E+19	0	90		9.000E+01	153	109		2.876E+01
FA1	U	PFA101	P	TL	1.650E+19	15	105	90	9.000E+01	90	82		8.889E+00
GIN	V	SASTM	SRM	LT	4.900E+18	40	130	90	9.000E+01	78	65		1.667E+01
PB1	S	PPB101	P	LT	7.050E+18	-45	45	90	9.000E+01	107	92		1.462E+01
PB2	V	SHSS02	SRM	LT	4.740E+18	45	135	90	9.000E+01	125	95		2.400E+01
TRO	X	PTRO01	P	LT	1.770E+19	-15	75	90	9.000E+01	116	102		1.207E+01
ZN1	X	PZN101	P	LT	1.200E+19			90	9.000E+01			23.0	2.300E+01
ZN2	T	PZN201	P	TL	1.100E+19			90	9.000E+01			15.0	1.500E+01
NY	W263	PMY_01	P	TL	6.600E+18	15	108	93	9.300E+01	115	96		1.652E+01
KW	V	SHSS02	SRM	LT	5.590E+18	45	140	95	9.500E+01	123	109		1.138E+01
PB1	S	SASTM	SRM	LT	7.050E+18	40	135	95	9.500E+01	78	68		1.282E+01

Table A.1 (continued)

Shift in Transition Temperature vs. Upper Shelf: Page 3														
PLANT_ID	CAPSULE	HEAT_ID	PROD_ID	SPEC_ORI	CSP_F1	UTT30	ITT30	DTT30	DTT	UUSE	IUSE	DUSE_REL	DUSE	
GC2	12	MOC201	M	TL	1.250E+19	-30	65	95	9.500E+01	125	89	2.680E+01		
SU2	X	WSU201	M	TL	3.020E+18	-18	75	95	9.500E+01	90	70	2.222E+01		
TR2	X	PTR001	P	TL	1.770E+19	-8	103	95	9.500E+01	84	76	9.524E+00		
ML2	W97	PML201	P	TL	3.780E+18	-17	113	96	9.600E+01	108	79	2.700E+01		
CR3	D	PCR301	P	TL	7.500E+18	-16	113	97	9.700E+01	94	72	2.340E+01		
HY	W263	PMY_01	P	LT	6.600E+18	0	97	97	9.700E+01	140	113	1.929E+01		
RS1	B	WRS101	M	TL	3.990E+18	-14	85		9.900E+01	68	51	2.500E+01		
RS1	B	WRS101	M	TL	3.990E+18	-14	85	99	9.900E+01	68	51	2.500E+01		
FA1	X	WFA101	M	TL	2.800E+19	-80	20	100	1.000E+02	149	115	2.282E+01		
PI1	T	PPB101	P	LT	2.110E+19	-45	55	100	1.000E+02	107	95	1.121E+01		
PI2	R	FPI201	F	LT	4.420E+19	-25	75	100	1.000E+02	150	127	1.533E+01		
PI2	R	WP1201	M	TL	4.420E+19	-75	25	100	1.000E+02	103	91	1.165E+01		
SA1	T	PSA101	P	LT	2.560E+18	-20	80	100	1.000E+02	108	89	1.759E+01		
SA1	T	PSA102	P	LT	2.560E+18	-25	75	100	1.000E+02	116	103	1.121E+01		
CK2	Y	PCK201	P	TL	7.010E+18			100	1.000E+02					
ZN2	T	SHSS02	SRM	LT	1.000E+19			100	1.000E+02		20.0	2.000E+01		
ZN1	T	WZN101	M	TL	1.800E+18	7	108		1.010E+02	63	56	2.900E+01		
DR2	4	MDR201	M	TL	9.500E+18	-10	93	103	1.030E+02	101	80	2.079E+01		
FA2	U	PFA201	P	LT	5.610E+18	-23	80	103	1.030E+02	130	94	2.769E+01		
CK2	X	PCK201	P	TL	1.050E+19			103	1.030E+02		27	2.700E+01		
AN1	E	WAN101	M	TL	7.270E+17	5	110		1.050E+02	73	58	2.055E+01		
DR3	G14	PDR301	P	LT	2.060E+19	-10	95	105	1.050E+02	135	106	2.148E+01		
FA1	U	PFA101	P	LT	1.650E+19	-30	85	105	1.050E+02	140	110	2.143E+01		
FA1	X	PFA101	P	TL	2.800E+19	15	120	105	1.050E+02	90	80	1.111E+01		
PI1	R	PPB101	P	LT	2.220E+19	-45	60	105	1.050E+02	107	100	6.542E+00		
PS2	T	SHSS02	SRM	LT	9.450E+18	49	154	105	1.050E+02	124	109	1.210E+01		
CK1	Y	PCK101	P	LT	1.340E+19			105	1.050E+02		20.0	2.000E+01		
TR	W2	SASTM_S1	SRM	LT	0.254E+19	15		105	1.050E+02	87	62	1.700E+01		
PS1	V	WPB101	M	TL	3.500E+18	-45	65		1.100E+02	65	54	1.692E+01		
CA8	K	WCA801	M	TL	1.400E+19	-20	90	110	1.100E+02	102	85	1.667E+01		
CTY	D	WCTY01	M	TL	2.200E+19	-50	60	110	1.100E+02	105	83	2.095E+01		
DC1	S	WDC101	M	TL	2.980E+18	-67	43	110	1.100E+02	98	87	1.122E+01		
PI1	V	SHSS02	SRM	LT	5.210E+18	49	159	110	1.100E+02	124	91	2.661E+01		
SA1	Y	PSA103	P	LT	8.910E+18	-50	60	110	1.100E+02	130	113	1.308E+01		
SU1	V	PSU101	P	LT	1.940E+19	-10	100	110	1.100E+02	125	116	7.200E+00		
CK1	Y	SHSS02	SRM	LT	1.200E+19			110	1.100E+02		28.0	2.800E+01		
TR	W1	PYR_01	P	LT	0.220E+19	10		110	1.100E+02	77	62	1.800E+01		
TR	W2	PYR_01	P	LT	0.254E+19	10		110	1.100E+02	77	62	1.800E+01		
DC2	A	MOC201	M	TL	3.370E+18	4	118	114	1.140E+02	67	47	2.985E+01		
CK1	Y	PCK101	P	TL	1.060E+19	4		115	1.150E+02	67		19.0		
ZN1	T	WZN101	M	TL	3.060E+18	4	116	116	1.160E+02	64	56	1.250E+01		
PI1	R	WP1101	M	TL	4.030E+18	-57	60	117	1.170E+02	79	75	5.063E+00		
IP3	T	PIP304	P	TL	2.920E+18	60	178	118	1.180E+02	67	58	1.343E+01		
CR3	D	WCR301	M	TL	7.500E+18	36	155	119	1.190E+02	79	68	1.392E+01		
DR3	G10/11	WDR301	M	TL	8.640E+18	40	160		1.200E+02	70	60	1.429E+01		
BV1	U	PBV101	P	LT	6.540E+18	-5	115	120	1.200E+02	134	99	2.612E+01		
CA8	P	PCAB01	P	LT	1.430E+19	-50	70	120	1.200E+02	103	85	1.748E+01		
CR3	F	MCR301	M	TL	1.080E+19	36	156	120	1.200E+02	79	64	1.899E+01		
PS1	T	SASTM	SRM	LT	2.110E+19	40	160	120	1.200E+02	78	69	1.154E+01		
SO1	F	PSO102	P	LT	5.140E+19	10	130	120	1.200E+02	97	71	2.680E+01		
SU2	V	SHSS02	SRM	LT	1.880E+19	45	165	120	1.200E+02	125	102	1.840E+01		
HY	1	PMY_01	P	LT	1.300E+19	0	120	120	1.200E+02	140	96	31.0		
ZN1	X	SHSS02	SRM	LT	1.400E+19			120	1.200E+02		32.0	3.200E+01		
CR3	C	WCR301	M	TL	6.560E+18	36	158	122	1.220E+02	79	63	2.025E+01		
TR1	E	WTM101	M	TL	1.080E+18	-56	68		1.240E+02	81	64	2.099E+01		
FC1	W225	SHSS01	SRM	LT	5.100E+18	27	151	124	1.240E+02	128	102	2.100E+01		
QC1	G1	PQC101	P	LT	1.190E+19	-30	95		1.250E+02	105	85	1.905E+01		
DB1	B	WDB101	M	TL	5.920E+18	-11	114	125	1.250E+02	70	57	1.857E+01		
PI2	V	SHSS02	SRM	LT	5.490E+18	45	170	125	1.250E+02	123	102	1.707E+01		
SA1	Y	SHSS02	SRM	LT	8.910E+18	50	175	125	1.250E+02	123	103	1.626E+01		
TP3	V	SASTM	SRM	LT	1.229E+19	35	160	125	1.250E+02	76	60	2.105E+01		
DB1	F	WDB101	M	TL	2.290E+18	-11	116		1.270E+02	72	65	9.722E+00		
DB1	F	WDB101	M	TL	2.290E+18	-11	116		1.270E+02	70	64	8.571E+00		
DR1	F	WDB101	M	TL	2.290E+18	-11	116		1.270E+02	75	65	1.333E+01		
DR1	G2	POR101	P	TS	1.200E+19	85	212	127	1.270E+02	70	56	2.000E+01		
ZN2	U	WZN201	M	LT	2.000E+18	-10	118		1.280E+02	69	50	2.754E+01		
CR3	F	PCR301	P	TL	1.080E+19	16	144	128	1.280E+02	94	71	2.447E+01		
CC2	263	SHSS01	SRM	LT	8.060E+18			128	1.280E+02		37	3.700E+01		
PS2	T	WPB201	M	TL	9.470E+18	0	130		1.300E+02	66	56	1.515E+01		
BV1	V	PBV101	P	LT	2.550E+18	-5	125	130	1.300E+02	134	114	1.493E+01		
DR3	G14	WDR301	M	TL	1.780E+19	40	170	130	1.300E+02	70	59	1.571E+01		
SO1	F	SASTM	SRM	LT	5.140E+19	40	170	130	1.300E+02	78	62	2.051E+01		
SO1	U	FSQ101	F	LT	1.080E+19	-35	95	130	1.300E+02	116	92	2.069E+01		
ZN1	U	SHSS02	SRM	LT	8.920E+18	42	172	130	1.300E+02	124	88	2.903E+01		
FA2	U	PFA201	P	TL	5.610E+18	-13	120	133	1.330E+02	95	69	2.737E+01		
DR1	G2	POR101	P	LS	1.200E+19	60	194	134	1.340E+02	110	80	2.727E+01		

Table A.1 (continued)

Shift in Transition Temperature vs. Upper Shelf: Page 4														
PLANT_ID	CAPSULE	HEAT_ID	PROD_ID	SPEC_ORI	CSF_F1	UTT30	ITT30	DTT30	DTT	UUSE	IUSE	DUSE_REL	DUSE	
BR	127	WBR_01	W	TS	7.100E+18	-70	65	135	1.350E+02	95	70	2.632E+01		
BV1	U	PBV101	P	TL	6.540E+18	-20	155	135	1.350E+02	80	78	2.500E+00		
FA1	X	PFA101	P	LT	2.800E+19	-30	105	135	1.350E+02	140	116	1.857E+01		
IP3	T	PIP304	P	LT	2.920E+18	33	170	137	1.370E+02	105	96	8.571E+00		
CAB	P	SASTH	SRM	LT	1.430E+19	40	176	138	1.380E+02	65	62	4.615E+00		
GIN	V	WG1N01	W	TL	5.900E+18	-25	115		1.400E+02	80	53	3.379E+01		
BV1	V	PBV101	P	TL	2.550E+18	20	160	140	1.400E+02	80	75	6.250E+00		
CAB	K	PCAB01	P	LT	1.400E+19	-50	90	140	1.400E+02	103	89	1.356E+01		
CTT	D	SASTH	SRM	LT	2.200E+19	40	180	140	1.400E+02	78	61	2.179E+01		
GIN	T	SASTH	SRM	LT	1.750E+19	40	180	140	1.400E+02	78	61	2.179E+01		
GIN	V	WG1N01	W	TL	4.900E+18	-25	115	140	1.400E+02	80	51	3.625E+01		
IP3	Y	SHSS02	SRM	LT	8.050E+18	50	190	140	1.400E+02	123	98	2.033E+01		
KME	R	SHSS02	SRM	LT	2.070E+19	45	185	140	1.400E+02	123	95	2.276E+01		
SO1	T	WSP101	W	TL	2.740E+18	-85	55	140	1.400E+02	111	78	2.973E+01		
IP3	T	WIP301	W	TL	2.920E+18	-55	88	143	1.430E+02	120	91	<4.17E+01		
RS1	D	WRS101	W	TL	6.600E+18	-14	130		1.440E+02	68	53	2.206E+01		
ZN2	U	WZN201	W	LT	2.820E+18	-23	122		1.450E+02	68	48	2.961E+01		
ZB2	U	WZB201	W	LT	2.820E+18	-23	122		1.450E+02	68	48	2.961E+01		
PS2	T	WPB201	W	TL	9.450E+18	0	145	145	1.450E+02	65	56	1.385E+01		
SO1	F	WSO101	W	TL	5.140E+19	-20	125	145	1.450E+02	99	80	1.919E+01		
SU1	V	SHSS02	SRM	LT	1.940E+19	45	190	145	1.450E+02	123	100	1.870E+01		
SU2	V	WSU201	W	TL	1.840E+19	-20	125	145	1.450E+02	90	60	3.333E+01		
IP2	Y	PIP203	P	LT	4.720E+18		145		1.450E+02		29.0	2.900E+01		
GIN	T	WG1B01	W	TL	1.750E+19	-25	125		1.500E+02	80	53	3.375E+01		
BR	124	PBR_01	P	TL	1.070E+20	-5	145	150	1.500E+02	82	70	1.663E+01		
BV1	V	WBV101	W	TL	2.550E+18	-60	90	150	1.500E+02	112	88	2.143E+01		
GIN	T	WG1N01	W	TL	1.750E+19	-25	125	150	1.500E+02	80	55	3.125E+01		
HB2	T	SASTH	SRM	TL	4.110E+19	65	215	150	1.500E+02	40	37	7.500E+00		
IP3	Y	PIP304	P	TL	8.050E+18	60	210	150	1.500E+02	67	57	1.493E+01		
NY	1	SHSS01	SRM	LT	1.300E+19	15	165	150	1.500E+02	130	100	23.0	2.300E+01	
AN1	A	WAN101	W	TL	1.030E+19	5	156	151	1.510E+02	73	45	3.836E+01		
PS2	R	SHSS02	SRM	LT	2.010E+19	49	200	151	1.510E+02	124	98	2.097E+01		
RS1	D	WRS101	W	TL	6.600E+18	-14	138	152	1.520E+02	68	53	2.206E+01		
DR3	14	WDR301	W	TL	7.710E+18	40	195		1.550E+02	70	64	8.571E+00		
QC1	G1	WQC102	W	TL	7.200E+18	-30	125		1.550E+02	72	52	2.776E+01		
BV1	U	WBV101	W	TL	6.540E+18	-60	95	155	1.550E+02	112	83	2.549E+01		
CAB	K	PCAB01	P	LT	3.680E+19	-50	105	155	1.550E+02	105	83	2.095E+01		
CAB	N	SASTH	SRM	LT	3.680E+19	40	195	155	1.550E+02	78	55	2.949E+01		
PAL	W290	PPAL01	P	TL	1.090E+19	25	180	155	1.550E+02	102	84	1.765E+01		
SA2	T	WSA201	W	TL	2.560E+18	-30	125	155	1.550E+02	111	79	2.883E+01		
P11	P	SHSS02	SRM	LT	1.250E+19	49	205	156	1.560E+02	124	85	3.145E+01		
RC1	U	WRC101	W	TL	4.140E+18	-5	155	160	1.600E+02	112	75	3.304E+01		
P12	T	SHSS02	SRM	LT	1.050E+19	45	205	160	1.600E+02	123	88	2.846E+01		
SO1	U	WSO101	W	TL	1.080E+19	-85	75	160	1.600E+02	111	82	2.613E+01		
TP3	T	WTP301	W	TL	7.050E+18	12	176		1.640E+02	66	62	6.061E+00		
GIN	R	WG1N01	W	TL	1.170E+19	-25	140		1.650E+02	80	51	3.625E+01		
PB1	R	WPB101	W	TL	2.170E+19	-45	120		1.650E+02	65	53	1.846E+01		
PB1	S	WPB101	W	TL	8.510E+18	-45	120		1.650E+02	65	54	1.692E+01		
PB2	V	WPB201	W	TL	7.330E+18	0	165		1.650E+02	66	42	3.636E+01		
FA2	W	PFA201	P	LT	1.540E+19	-23	142	165	1.650E+02	130	102	2.154E+01		
FA2	W	PFA201	P	TL	1.540E+19	-13	152	165	1.650E+02	95	76	2.000E+01		
PB1	R	WPB101	W	TL	2.220E+19	-45	120	165	1.650E+02	65	51	2.154E+01		
PB1	S	WPB101	W	TL	7.050E+18	-45	120	165	1.650E+02	65	52	2.000E+01		
PB2	V	WPB201	W	TL	4.740E+18	0	165	165	1.650E+02	65	42	3.538E+01		
SA1	Y	WSA101	W	TL	8.910E+18	-135	30	165	1.650E+02	104	75	2.788E+01		
SU1	T	WSU101	W	T	2.500E+18	-15	150	165	1.650E+02	70	53	2.429E+01		
SU1	T	WSU101	W	TL	2.880E+18	-15	152	165	1.670E+02	70	53	2.429E+01		
QC2	G15	WQC201	W	TL	3.820E+19	-30	140	170	1.700E+02	125	80	3.600E+01		
DC2	U	WQC201	W	TL	3.510E+18	-13	160	174	1.740E+02	121	85	3.000E+01		
DB1	A	WDB101	W	TL	1.290E+19	-11	164	175	1.750E+02	72	62	1.389E+01		
KME	V	WKMEC1	W	TL	5.590E+18	-50	125	175	1.750E+02	126	82	3.492E+01		
PAL	W290	PPAL01	P	LT	1.090E+19	0	175	175	1.750E+02	155	112	2.774E+01		
QC1	G6	WQC101	W	TL	3.560E+19	10	185	175	1.750E+02	100	75	2.500E+01		
ZN2	T	WZN201	W	LT	1.100E+19			175	175	1.750E+02		36.0	3.600E+01	
IP3	Y	WIP301	W	TL	8.050E+18	-55	125	180	1.800E+02	120	68	4.333E+01		
PB1	T	WPB101	W	TL	2.110E+19	-45	135	180	1.800E+02	65	55	1.538E+01		
P12	R	SHSS02	SRM	LT	4.420E+19	45	225	180	1.800E+02	123	77	3.740E+01		
QC1	G5	PQC101	P	LT	4.040E+19	-30	150	180	1.800E+02	106	73	3.113E+01		
TP3	V	WTP301	W	TL	1.229E+19	10	190	180	1.800E+02	65	48	2.615E+01		
RY	2	PMY_01	P	LT	8.830E+19	0	185	185	1.850E+02	140	85	3.929E+01		
P11	R	SHSS02	SRM	LT	4.030E+19	49	235	186	1.860E+02	124	86	3.065E+01		
ZN1	U	WZN101	W	TL	8.920E+18	7	195	188	1.880E+02	68	49	2.794E+01		
BR	122	WBR_01	W	TS	2.300E+19	-70	120	190	1.900E+02	95	57	4.000E+01		
OC1	A	WOC101	W	TL	8.950E+18	-5	186	191	1.910E+02	64	52	1.875E+01		
RT	2	PMY_01	P	TL	8.840E+19	15	210	195	1.950E+02	115	71	3.826E+01		
IP2	Y	WIP201	W	TL	5.890E+18			195	1.950E+02	42.0		4.200E+01		
KD1	T	WKD101	W	TL	9.800E+18			195	1.950E+02	66	47	2.900E+01		

Table A.1 (continued)

Shift in Transition Temperature vs. Upper Shelf: Page 5														
PLANT_ID	CAPSULE	HEAT_ID	PROD_ID	SPEC_OBI	CSP_F1	UTT30	ITT30	DTT30	DTT	LUSE	IUSE	DUSE_REL	DUSE	
ZN1	X	WZN101	W	TL	1.500E+19		195	1.950E+02		31.0		3.100E+01		
ZN1	U	WZB101	W	TL	1.020E+19	4	203	1.990E+02	64	52		1.875E+01		
CK1	T	CKX101	W		1.060E+19			2.000E+02			36.0	3.600E+01		
DR3	14	WDR302	W	TL	6.150E+18	-45	160	2.050E+02	65	45		3.077E+01		
PAL	A240	PPAL01	P	LT	4.500E+19	0	205	205	2.050E+02	165	95		4.242E+01	
PAL	A240	PPAL01	P	TL	4.600E+19	25	230	205	2.050E+02	105	68		3.524E+01	
GC2	G16	WGC202	W	TL	2.630E+19	15	220	205	2.050E+02	87	41		5.287E+01	
DR3	G12	WDR302	W	TL	2.060E+19	-45	163	2.080E+02	65	42		3.538E+01		
DR2	5	WDR202	W	TL	4.600E+19	-13	195	208	2.080E+02	71	52		2.676E+01	
GC2	12	WGC202	W	TL	9.000E+18	15	195	210	2.100E+02	90	51		4.333E+01	
CA8	N	WCAB01	W	TL	3.680E+19	-20	200	220	2.200E+02	102	62		3.922E+01	
NY	W263	WPY_01	W	TL	6.800E+18	-30	192	222	2.220E+02	105	59		4.381E+01	
TP4	T	WTP401	W		7.580E+18	0	224		2.240E+02	68	43		3.676E+01	
TR	08	SASTH S1	SRH		5.000E+19	15	240	225	2.250E+02	87	66		4.713E+01	
TR	08	SASTH S1	SRH		5.000E+19	15		225	2.250E+02	87	66	47	4.700E+01	
BR	124	WBR_01	W	TS	1.070E+20	-70	160	230	2.300E+02	95	65		3.158E+01	
PB2	R	WPB201	W	TL	2.010E+19	0	230	230	2.300E+02	65	47		2.769E+01	
PB2	R	WPB201	W	TL	2.540E+19	0	235		2.350E+02	66	68		2.727E+01	
KME	R	WME201	W	TL	2.070E+19	-50	185	235	2.350E+02	126	78		3.810E+01	
FC1	W225	UFC101	W	TL	5.100E+18	-28	210	238	2.380E+02	98	54	34.0	3.400E+01	
BU1	V	WSU101	W	TL	1.940E+19	-15	225	240	2.400E+02	70	50		2.857E+01	
DR3	G16	WDR302	W	TL	1.200E+19	-45	205	250	2.500E+02	65	41		3.692E+01	
TR	06	SASTH S1	SRH		7.000E+19	15	275	260	2.600E+02	87	44		4.963E+01	
TR	06	SASTH S1	SRH		7.000E+19	15		260	2.600E+02	87	44	49	4.900E+01	
GC1	G7	WGC102	W	TL	2.370E+19	-30	235	265	2.650E+02	72	49		3.194E+01	
NY	1	WPY_01	W	TL	1.300E+19	-30	240	270	2.700E+02	105	57	45.0	4.500E+01	
PAL	W290	WPA01	W	LT	1.090E+19	-85	205	290	2.900E+02	118	64		4.576E+01	
YE	01	SASTH S1	SRH		9.000E+19	15	325	310	3.100E+02	87	42		5.172E+01	
TR	02	SASTH S1	SRH		9.000E+19	15	325	310	3.100E+02	87	42		5.172E+01	
TR	02	SASTH S1	SRH		9.000E+19	15		310	3.100E+02	87	42	52	5.200E+01	
TR	08	PYR_01	P	LT	5.000E+19	10	330	320	3.200E+02	76	46		3.947E+01	
TR	08	PYR_01	P	LT	5.000E+19	10		320	3.200E+02	77	46	40	4.000E+01	
NY	2	WPY_01	W	TL	8.840E+19	-30	315	345	3.450E+02	51	50		5.327E+01	
PAL	A240	WPA01	W	LT	4.600E+19	-85	265	350	3.500E+02	54			5.500E+01	
TR	06	PYR_01	P	LT	7.000E+19	10	370	360	3.600E+02	77	45		4.079E+01	
TR	06	PYR_01	P	LT	7.000E+19	10		360	3.600E+02	77	44	42	4.200E+01	
TR	01	PYR_01	P	LT	9.000E+19	10	410	400	4.000E+02	76	45		4.079E+01	
TR	02	PYR_01	P	LT	9.000E+19	10	410	400	4.000E+02	76	45		4.079E+01	
TR	02	PYR_01	P	LT	9.000E+19	10		400	4.000E+02	77	42	45	4.500E+01	

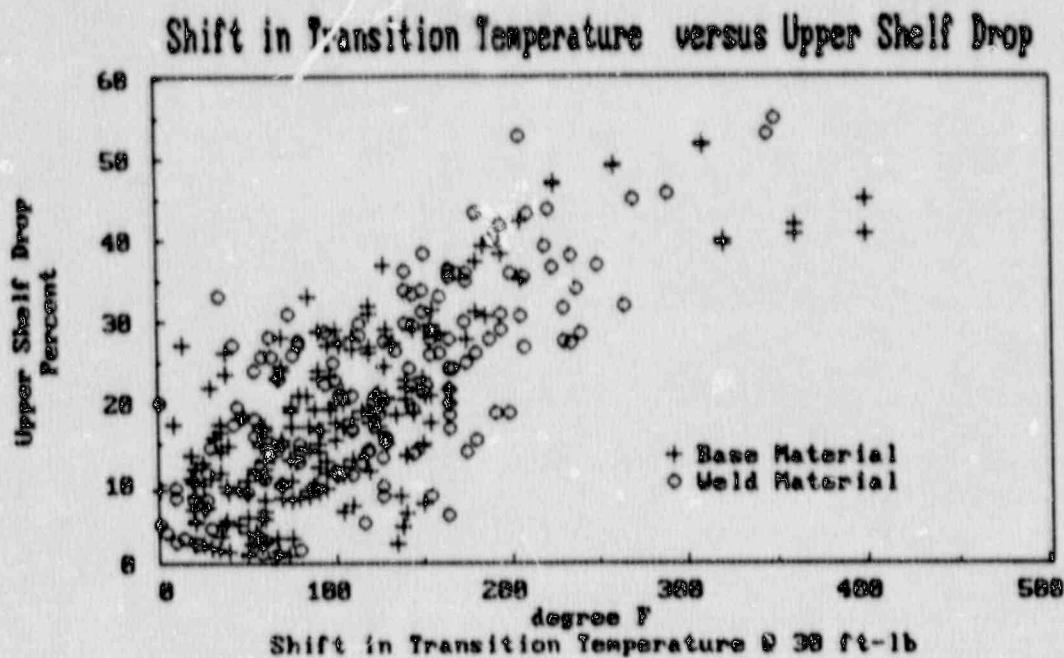


Fig. A.5. Graph of transition temperature shift versus upper shelf drop from example, Sect. A.6.1.

A.6.2 Charpy Fitting and Plotting

In this example, the Charpy fitting and plotting procedures will be applied to the weld data for the Zion Unit 2 reactor. The corresponding raw Charpy data are at the end of the file RAW_C_PR.dat, resulting in long waiting times when these sets are processed. For this reason - and, of course, to demonstrate this feature - these sets are first copied to a file ZN2.dat with the aid of the option "Extracting Selected Charpy Sets." After choosing this option, the output file name ZN2.DAT is entered, followed by entering, in the menu screen for selecting sets, the key ZN2 for PLANT_ID and the letter W for PROD_ID (or HEAT_ID). During the run of the program, the three sets baseline (CAPSULE is blank) and capsules T and U appear on the screen for confirmation. The output file (with a modification, see below) is listed in Table A.2.

Table A.2. File ZN2.dat obtained from RAW_C_PR.dat.
 Note the added asterisk at the front of
 specimen W40 tagging it as an outlier

WZN2	WZN201	W	LT	-100.0	10.0	17	7				
WZN2	WZN201	W	LT	-100.0	11.0	13	8				
WZN2	WZN201	W	LT	-60.0	17.0	23	14				
WZN2	WZN201	W	LT	-60.0	20.0	33	16				
WZN2	WZN201	W	LT	-60.0	14.0	23	12				
WZN2	WZN201	W	LT	-25.0	35.0	58	32				
WZN2	WZN201	W	LT	-25.0	25.0	47	23				
WZN2	WZN201	W	LT	-25.0	32.0	42	27				
WZN2	WZN201	W	LT	10.0	37.0	55	33				
WZN2	WZN201	W	LT	10.0	40.0	61	38				
WZN2	WZN201	W	LT	10.0	41.0	71	40				
WZN2	WZN201	W	LT	40.0	47.0	65	43				
WZN2	WZN201	W	LT	40.0	51.0	71	48				
WZN2	WZN201	W	LT	40.0	76.0	100	63				
WZN2	WZN201	W	LT	40.0	43.5	57	40				
WZN2	WZN201	W	LT	75.0	60.0	95	58				
WZN2	WZN201	W	LT	75.0	62.0	97	57				
WZN2	WZN201	W	LT	75.0	56.5	97	53				
WZN2	WZN201	W	LT	210.0	69.0	100	69				
WZN2	WZN201	W	LT	210.0	69.5	100	70				
WZN2	WZN201	W	LT	210.0	62.0	100	61				
WZN2	WZN201	W	LT	300.0	68.0	100	65				
WZN2	WZN201	W	LT	300.0	72.0	100	68				
WZN2	WZN201	W	LT	300.0	70.0	100	61				
WZN2	T	WZN201	W	LT	W34	71.0	12.5	15	1.100E+19	579	
WZN2	T	WZN201	W	LT	V33	135.0	22.0	35	1.100E+19	579	
WZN2	T	WZN201	W	LT	*W40	180.0	88.0	100	76	1.100E+19	579
WZN2	T	WZN201	U	LT	W35	185.0	40.0	80	38	1.100E+19	579
WZN2	T	WZN201	W	LT	W39	235.0	36.5	80	34	1.100E+19	579
WZN2	T	WZN201	U	LT	W38	300.0	43.5	100	43	1.100E+19	579
WZN2	T	WZN201	W	LT	W37	350.0	44.5	100	47	1.100E+19	579
WZN2	T	WZN201	W	LT	W36	400.0	42.5	100	45	1.100E+19	579
WZN2	U	WZN201	W	LT	W45	50.0	16.5	10	13	2.800E+18	550
WZN2	U	WZN201	W	LT	W47	73.0	20.5	20	21	2.800E+18	550
WZN2	U	WZN201	W	LT	W42	125.0	31.0	65	30	2.800E+18	550
WZN2	U	WZN201	W	LT	W41	200.0	45.0	99	42	2.800E+18	550
WZN2	U	WZN201	W	LT	W44	285.0	48.0	100	..	2.800E+18	550
WZN2	U	WZN201	W	LT	W43	345.0	50.0	100	49	2.800E+18	550
WZN2	U	WZN201	W	LT	W46	350.0	46.5	100	71	2.800E+18	550
WZN2	U	WZN201	W	LT	W48	400.0	50.0	100	53	2.800E+18	550

The user can now test the other three options using the newly created file as an input file instead of the given default RAW_C_PR.dat. Figures A.6, A.7, and A.8 show the resulting plots for the three sets, with appropriate titles and subtitles entered by the user on prompts from the screen. Note that the fit for capsule T did not converge; inspection shows an outlier of 88-ft-lb impact energy at 180°F, which is inconsistent with the rest of the data. This point can be eliminated from the fitting by placing an asterisk in front of the specimen identifier W40, as shown in Table A.2. This can be done with any editor that works on ASCII files (e.g., EDLIN) if no better is available. Figure A.9 shows the resulting fit after this change. Note that the excluded point appears as a black square at the top of the graph.

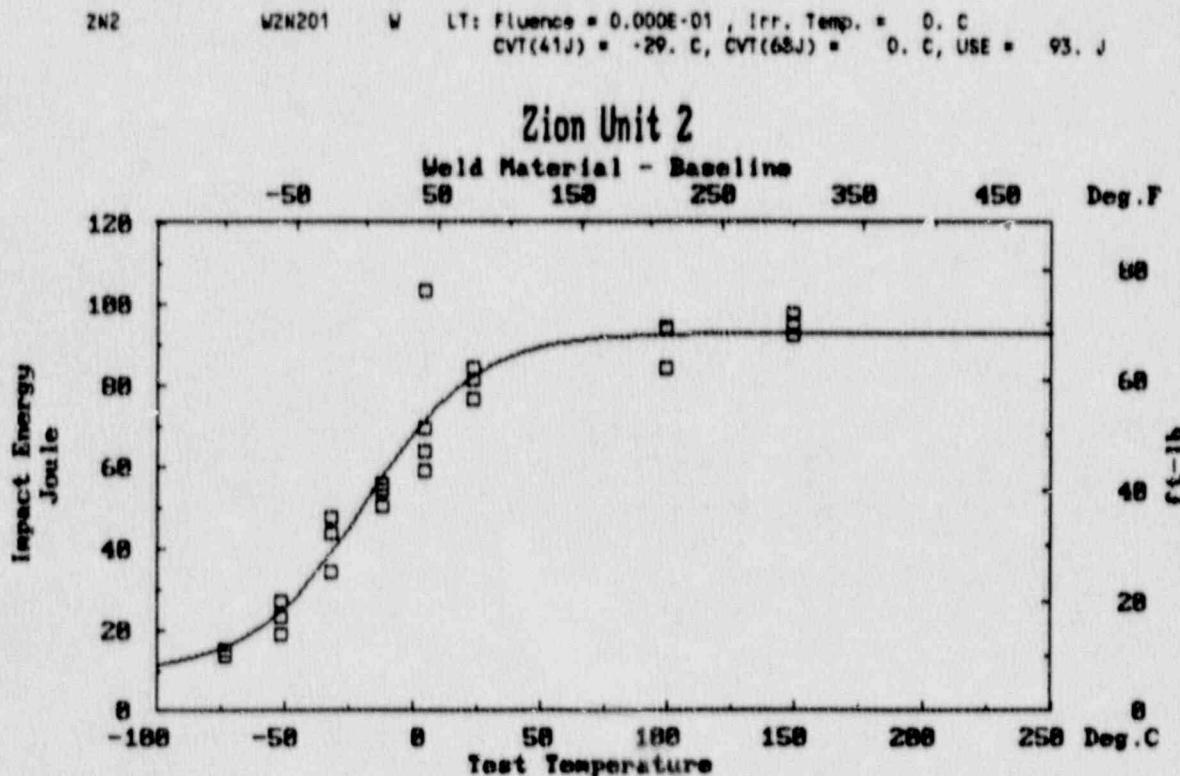


Fig. A.6. Charpy fit for Zion Unit 2 weld material - baseline.

ZN2 T WZN201 V LT: Fluence = 1.100E+19 , Irr. Temp. = 304. C
 CVT(41J) = -7585. J, CVT(68J) = ***** C, USE = 56. J

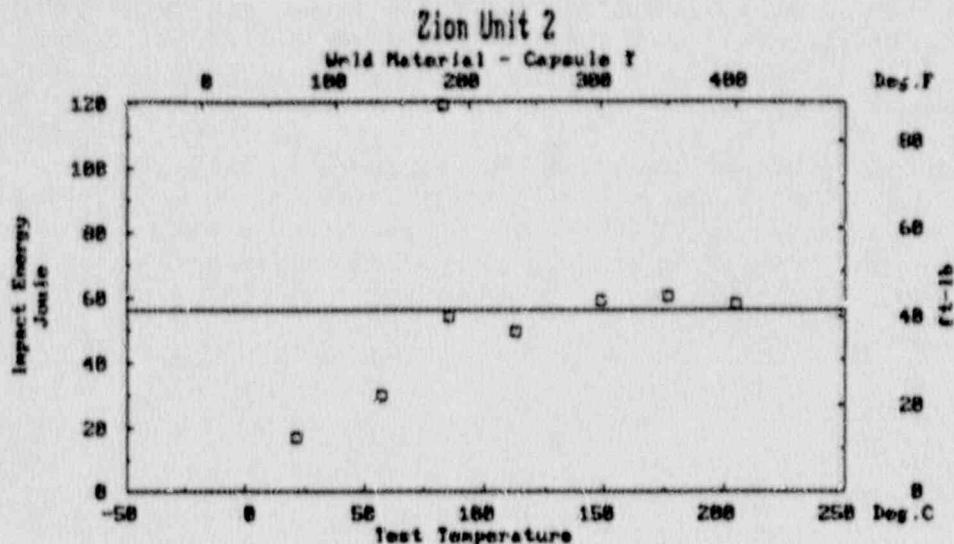


Fig. A.7. Charpy fit for Zion Unit 2 weld material - Capsule T.
 Fit has not converged due to an outlier.

ZN2 U WZN201 V LT: Fluence = 2.800E+18 , Irr. Temp. = 288. C
 CVT(41J) = 48. C, CVT(68J) = ***** C, USE = 66. J

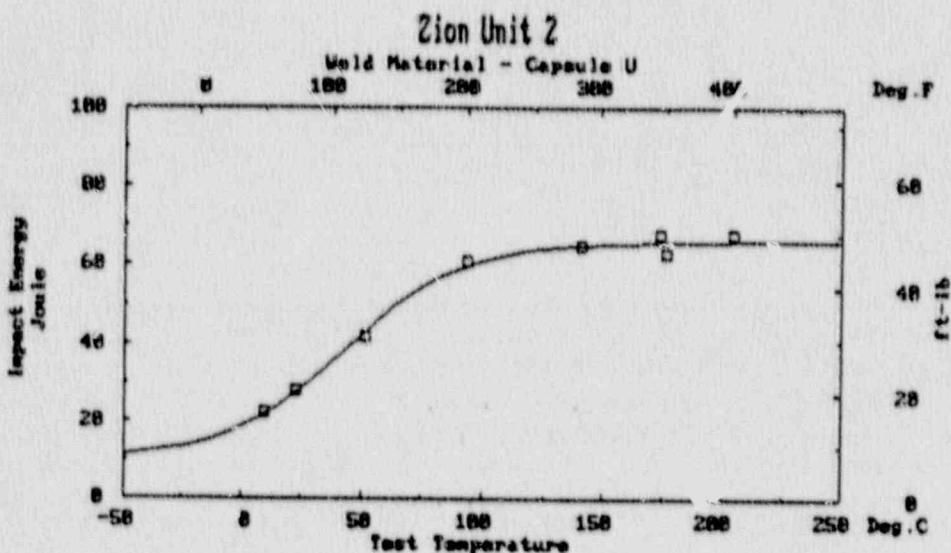


Fig. A.8. Charpy fit for Zion Unit 2 weld material - Capsule U.

ZN2 T WZN201 W LT: Fluence = 1.100E+19 , Irr. Temp. = 304. C
 CVT(41J) = 70. C, CVT(68J) =***** C, USE = 58. J

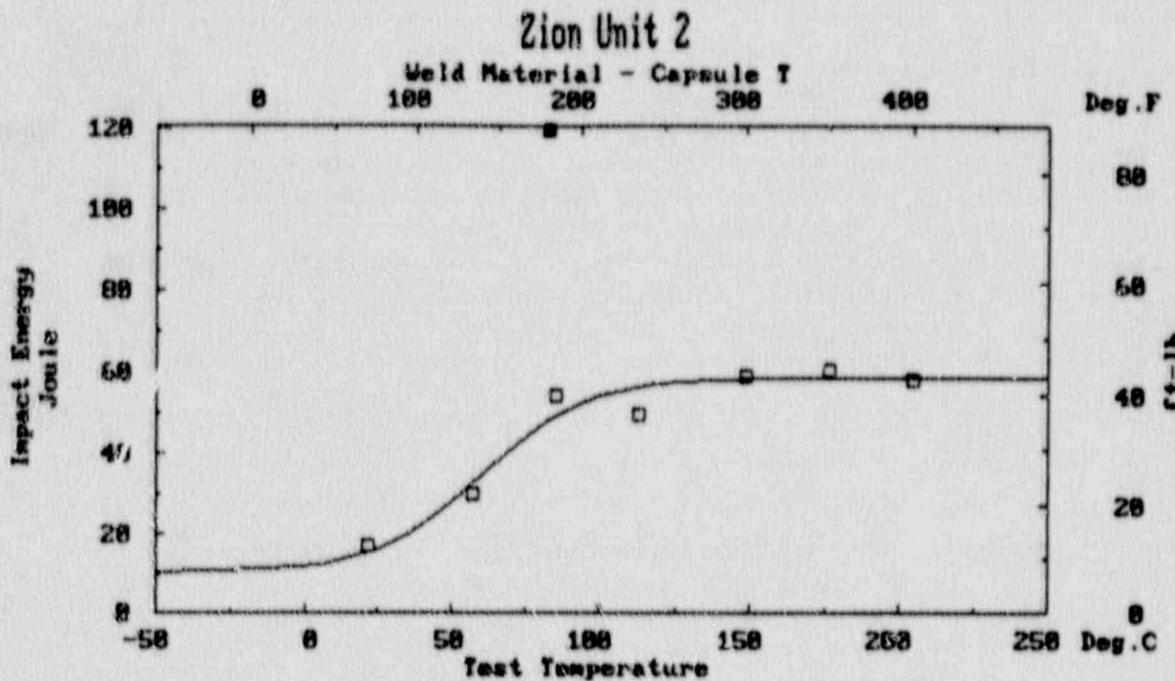


Fig. A.9. Data from Fig. A.7 after the outlier at the top of the graph is eliminated.

The modified input file can now be used for the two other options. On the input screens for the Monte Carlo uncertainty analysis, several default options are already provided including the input file, title, input uncertainties, number of iterations, and output files. To process all the sets, the number of skipped sets must, of course, be 0 and the number of processed sets 3. Please make sure that the correct input file ZN2.dat is chosen. During the run of the program, a summary of each single fit appears on the screen including a parameter "INFO" which indicates whether the fit was successful or not and the reason for failing. A list explaining the different INFO values is given in Table A.3. Sample screen outputs are shown in Tables A.4 and A.5. The first output at the start for the set in capsule T shows the failure to determine CV" at 50 ft-lb. Such data are first rejected for 20% of the prescribed iterations; once it becomes established that this is due to the low upper shelf, CVT at 50 ft-lb is ignored, as shown in the subsequent screen output; there is still a high failure rate due to other causes, as indicated in Table A.5. The three output files are listed in Tables A.6, A.7, and A.8. The EDB-dBASE file may be used as an auxiliary file for the multiple-curve fitting and plotting procedure. The user may choose to obtain a plot for each processed set. A sample is shown in Fig. A.10.

Table A.3. Key to the information tag INFO in the screen output for Monte Carlo uncertainty analysis

INFO > 0 :	Regular Output
- 0 :	Fitting did not converge
- -1 :	Inconsistent with previous values, skipped
- -2 :	Inconsistent with first value, first value ignored
- -3 :	CVT @ 30 ft-lb or 50 ft-lb could not be determined (ignored after failure in 20% of the iterations)
- -4 :	Failure to find initial value for fitting, fitting aborted
- -5 :	Fit not physically possible, rejected

Table A.4. Sample screen output from the Monte Carlo uncertainty analysis - start.

The first 20 fits are rejected because of failure to obtain transition temperature at 68 J (50 ft-lb), INFO = -3

ZN2	T	WZX201	W	LT:	Fluence = 1.100E+19	, Irr. Temp. = 304.
LSE-	10., USE-	58., SLOPE-	0.0286, TMID-	60., CVT41-	70., CVT68-*****	-3
LSE-	10., USE-	59., SLOPE-	0.0213, TMID-	45., CVT41-	58., CVT68-*****	-3
LSE-	9., USE-	59., SLOPE-	0.0252, TMID-	64., CVT41-	75., CVT68-*****	-3
LSE-	10., USE-	62., SLOPE-	0.0146, TMID-	76., CVT41-	90., CVT68-*****	-3
LSE-	9., USE-	61., SLOPE-	0.0741, TMID-	71., CVT41-	73., CVT68-*****	-3
LSE-	10., USE-	44., SLOPE-	0.0002, TMID-*****	CVT41-*****	CVT68-*****	-5
LSE-	11., USE-	56., SLOPE-	0.0587, TMID-	51., CVT41-	57., CVT68-*****	-3
LSE-	10., USE-	53., SLOPE-	0.0203, TMID-	67., CVT41-	91., CVT68-*****	-3
LSE-	10., USE-	62., SLOPE-	0.0167, TMID-	86., CVT41-	97., CVT68-*****	-3
LSE-	10., USE-	64., SLOPE-	0.0083, TMID-	41., CVT41-	59., CVT68-*****	-3
LSE-	10., USE-	55., SLOPE-	0.0141, TMID-	18., CVT41-	46., CVT68-*****	-3
LSE-	10., USF-	76., SLOPE-	0.0000, TMID-*****	CVT41-*****	CVT68-*****	-5
LSE-	10., USE-	57., SLOPE-	0.0147, TMID-	54., CVT41-	75., CVT68-*****	-3
LSE-	10., USE-	56., SLOPE-	0.0198, TMID-	31., CVT41-	55., CVT68-*****	-3
LSE-	10., USE-	56., SLOPE-	0.0198, TMID-	31., CVT41-	50., CVT68-*****	-3
LSE-	10., USE-	49., SLOPE-	0.0340, TMID-	46., CVT41-	65., CVT68-*****	-3

ZN2	T	WZN201	W	LT :	Iteration number	0 out of 100
-----	---	--------	---	------	------------------	--------------

Table A.5. Sample screen output from the Monte Carlo uncertainty analysis - continuation. Failure to obtain transition temperatures at 68 J (50 ft-lb) is now ignored

```

LSE= 11., USE= 63., SLOPE= 0.0204, TMID= 65., CVT41= 65., CVT68=***** 2
LSE= 9., USE= 68., SLOPE= 0.0184, TMID= 76., CVT41= 76., CVT68=***** 2
LSE= 10., USE= 61., SLOPE= 0.0147, TMID= 60., CVT41= 60., CVT68=***** 2
LSE= 9., USE= 56., SLOPE= 0.0703, TMID= 82., CVT41= 82., CVT68=***** -1
LSE= 17., USE= 53., SLOPE= 2.9786, TMID= 60., CVT41= 60., CVT68=***** -5
LSE= 30., USE= 51., SLOPE=19.1053, TMID= 110., CVT41= 110., CVT68=***** -5
LSE= 10., USE= 61., SLOPE= 0.0207, TMID= 56., CVT41= 67., CVT68=***** 2
LSE= 23., USE= 56., SLOPE=67.3105, TMID= 68., CVT41= 68., CVT68=***** -5
LSE= 10., USE= 69., SLOPE= 0.0160, TMID= 97., CVT41= 68., CVT68=***** 0
LSE= 10., USE= 51., SLOPE= 0.0412, TMID= 40., CVT41= 54., CVT68=***** -1
LSE= 9., USE= 51., SLOPE= 1.7939, TMID= 41., CVT41= 41., CVT68=***** -5
LSE= 10., USE= 59., SLOPE= 0.0424, TMID= 50., CVT41=**56*, CVT68=***** -1
LSE= 10., USE= 59., SLOPE= 0.0169, TMID= 59., CVT41= 75., CVT68=***** 2
LSE= 10., USE= 76., SLOPE= 0.0118, TMID= 80., CVT41= 75., CVT68= 164. 2
LSE= 9., USE= 76., SLOPE= 0.0118, TMID= 80., CVT41= 75., CVT68= 164. 2
LSE= 9., USE= 60., SLOPE= 0.0632, TMID= 63., CVT41= 67., CVT68=***** -1
LSE= 23., USE= 49., SLOPE= 2.0703, TMID= 94., CVT41= 94., CVT68=***** -5
LSE= 10., USE= 54., SLOPE= 0.0195, TMID= 49., CVT41= 72., CVT68=***** 2
LSE= 10., USE= 76., SLOPE= 0.0125, TMID= 73., CVT41= 68., CVT68= 152. 2
LSE= 10., USE= 58., SLOPE= 0.0318, TMID= 43., CVT41= 53., CVT68=***** -1
LSE= 10., USE= 57., SLOPE= 0.0226, TMID= 48., CVT41= 63., CVT68=***** 2
LSE= 10., USE= 59., SLOPE= 0.0252, TMID= 59., CVT41= 70., CVT68=*****

```

ZN2 T WZN201 W LT : Iteration number 59 out of 100

Table A.6. Sample output of the summary file

```

PR-EDB - Hyperbolic Tangent Fits of Raw Charpy Data
Impact Energy Uncertainty = 10.0 J
Test Temperature Uncertainty = 4.0 Degree C
Number of Successful Iterations = 10, Maximum = 50

```

Set No.	Reactor ID	Caps Type	Material Or.	Fluence	Irr.Temp.	CVT(41J) Deg.C	CVT(68J) Deg.C	USE Deg.C	LSE Joule	Spec Joule	Iter No.
1 ZN2		WZN201	W LT	0.000E-01	0.	-21.6 ± 7.2	1.6 ± 3.4	92.6 ± 3.2	10.1 ± 2.5	24	10
2 ZN2	T	WZN201	W LT	1.100E+19	304.	71.6 ±11.9	0.0 ±-1.0	61.5 ± 8.9	10.4 ± 0.7	7	10
3 ZN2	U	WZN201	W LT	2.800E+18	286.	52.6 ±14.4	0.0 ±-1.0	70.5 ± 8.8	10.2 ± 0.3	6	10

Table A.7. Sample output of the covariance file

PR-EDB - Hyperbolic Tangent Fits of Raw Charpy Data
 Impact Energy Uncertainty = 10.0 J
 Test Temperature Uncertainty = 4.0 Degree C
 Number of Successful Iterations = 10, Maximum = 50

ZN2 WZN201 W LT: Fluence = 0.000E-01 , Irr. Temp. = 0. C
 No. of Specimen = 24, No. of Iterations = 10

Parameter	Mean	Std.Dev.	Correlation Coefficients		
Lower Shelf Energy	10.1	2.5			
Upper Shelf Energy	92.6	2.2	-0.258		
CVT at Midpoint	16.8	6.5	0.341	0.460	
1/slope (degree C)	42.6	9.4	-0.793	0.248	-0.634
CVT at 41 Joule	27.6	7.2	0.415	0.185	0.953 -0.786
CVT at 68 Joule	1.6	3.4	-0.332	0.332	0.632 -0.006 0.585

ZN2 T WZN201 W LT: Fluence = 1.100E+19 , Irr. Temp. = 304. C
 No. of Specimen = 7, No. of Iterations = 10

Parameter	Mean	Std.Dev.	Correlation Coefficients		
Lower Shelf Energy	10.4	0.7			
Upper Shelf Energy	61.5	8.9	-0.055		
CVT at Midpoint	59.8	16.9	0.056	0.707	
1/slope (degree C)	52.0	15.4	-0.188	0.281	0.106
CVT at 41 Joule	71.8	11.9	-0.039	0.048	0.657 0.278
CVT at 68 Joule	0.0	-1.0	0.000	0.000	0.000 0.000

ZN2 U WZN201 W LT: Fluence = 2.800E+18 , Irr. Temp. = 288. C
 No. of Specimen = 8, No. of Iterations = 10

Parameter	Mean	Std.Dev.	Correlation Coefficients		
Lower Shelf Energy	10.2	0.3			
Upper Shelf Energy	70.5	8.8	-0.030		
CVT at Midpoint	50.6	22.2	-0.113	0.800	
1/slope (degree C)	82.6	22.6	-0.220	0.504	0.490
CVT at 41 Joule	52.6	14.4	-0.146	0.411	0.873 0.348
CVT at 68 Joule	0.0	-1.0	0.000	0.000	0.000 0.000

Table A.8. Sample output of the EDB-dBASE file

ZN2	WZN201	W	LT	0.000E-01	0	-17	13	35	6	68	2	7.	2.0	0.0130	24	10	
ZN2	T	WZN201	W	LT	1.100E+19	579	161	21	0	-1	45	7	8.	140.0	0.0107	7	10
ZN2	U	WZN201	W	LT	2.800E+18	550	127	26	0	-1	52	6	7.	123.0	0.0067	8	10

ZN2 T WZN201 W LT: Fluence = 1.100E+19, Irr. Temp. = 304. C
 CVT(41J) = 72. C, CVT(68J) = 0. C, USE = 62. J

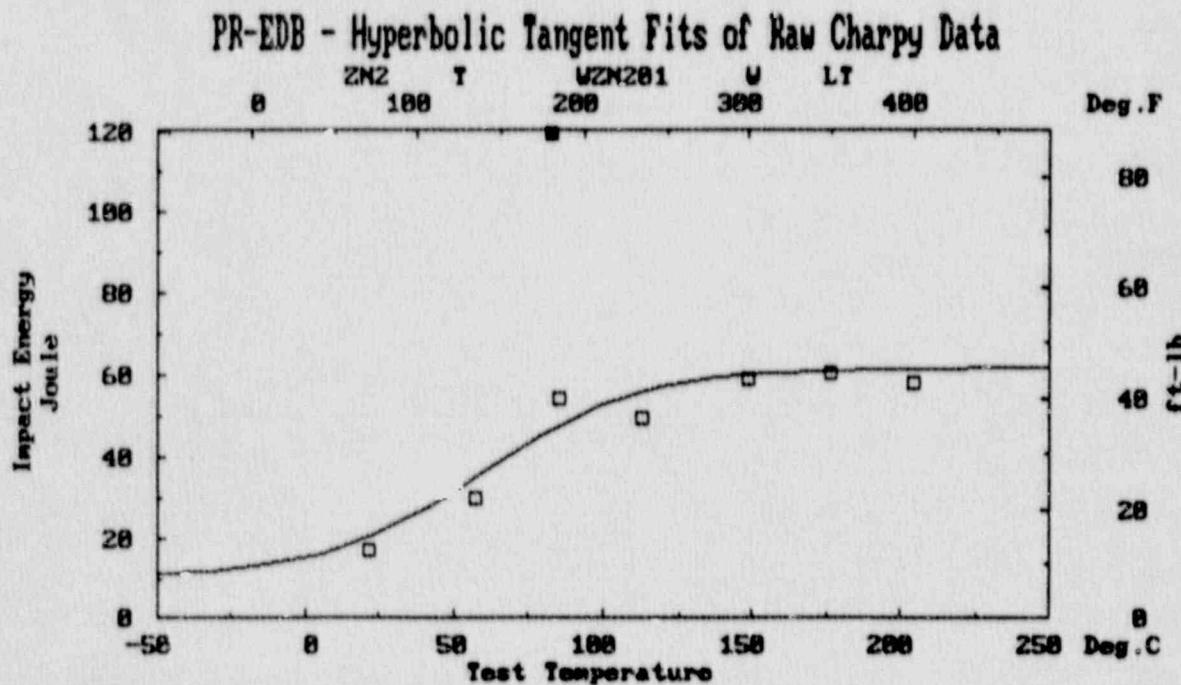


Fig. A.10. Same data as in Fig. A.7. Note the slight difference in the fitting curve as a result of the Monte Carlo Procedure.

All three fitting curves and the corresponding data points can be combined in one plot with the multiple fitting and plotting procedure. Figure A.11 shows a sample output for appropriately chosen text in title and legends. No auxiliary input file was used for this example.

```

ZN2      WZN201   W   LT: Fluence = 0.000E-01 , Irr. Temp. = 0. C
                  CVT(41J) = -29. C, CVT(68J) = 0. C, USE = 93. J

ZN2      T      WZN201   W   LT: Fluence = 1.100E+19 , Irr. Temp. = 304. C
                  CVT(41J) = 70. C, CVT(68J) =***** C, USE = 58. J

ZN2      U      WZN201   W   LT: Fluence = 2.800E+18 , Irr. Temp. = 288. C
                  CVT(41J) = 48. C, CVT(68J) =***** C, USE = 66. J
  
```

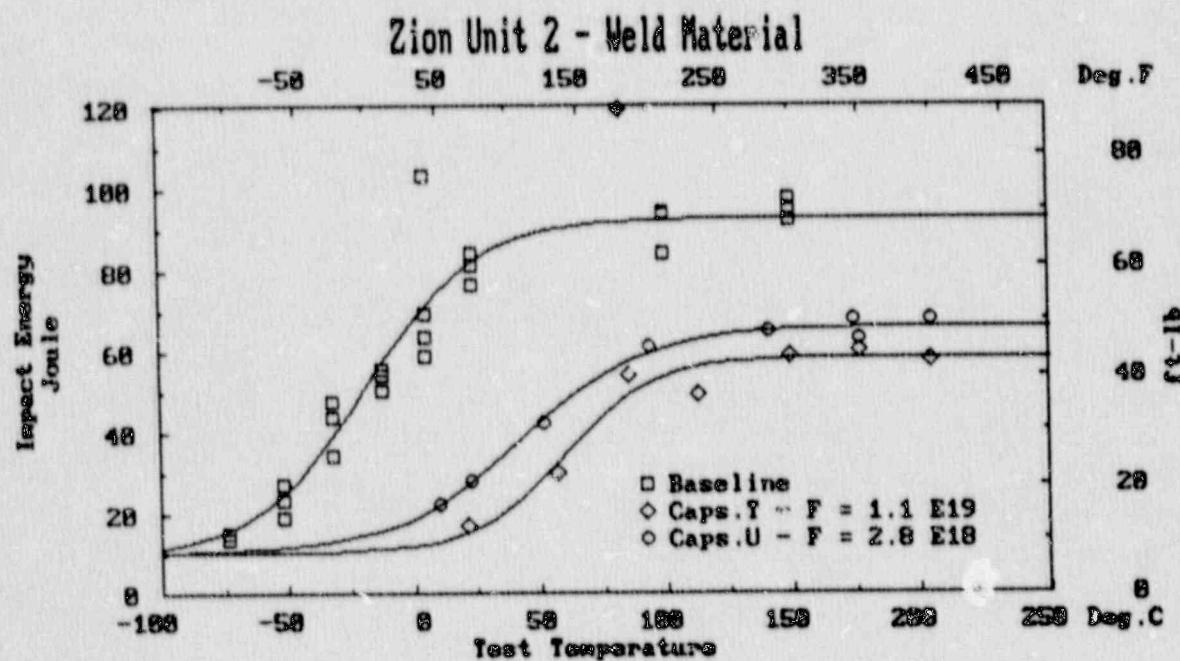


Fig. A.11. Combination graph of Zion Unit 2 weld material, baseline plus Capsules T and U.

A.7 INSTALLATION

Running the EDB-Utilities with data files requires an IBM-compatible PC, preferably with 80286 (AT) or 80386 processor, and a hard disk. At least 512 kb RAM is recommended. A matching co-processor 80287/80387 is also required. The graphics programs are written for an EGA screen with 640 x 350 pixel resolution. Versions for monochrome and color monitors are available. For hardcopy of the graphics, a utility is necessary which transfers the EGA screen to the printer on the Shift-PrtScr command. (This command is part of the software, and it does not need to be given externally.) The GRAFPLUS utility by Jewell Technologies, Inc., is used in our installation.

The PR-EDB package consists of two disks for the software including the essential data files for running the EDB-Utilities labeled System #1 and System #2. Four disks contain the data files of the PR-EDB, two of which contain the large raw Charpy data file RAW_C_PR.dbf and its ASCII equivalents RAW_C_PR.dat and RAW_C_RS.dat which are the primary and auxiliary files, respectively, for use in the Charpy fitting and plotting programs. The System disks and Data disks should be copied into a suitable directory on the hard disk. Also loaded should be the dBASE programs (dBASE III PLUS or IV) or a path specified so that dBASE can be reached from the EDB directory. About 10 Mb disk space should be available for running the EDB-Utilities plus data, but less may be needed if only part of the data files are used. A disk containing the results of the examples is also included. These data should be loaded after all other input data, and the plots and fits can be obtained without the preprocessing steps described in Sect. A.6.

NUREG/CR-4816
ORNL/TM-10328
Dist. Category RL, R5

INTERNAL DISTRIBUTION

1-2.	C. A. Baldwin	36-39.	F. W. Stallmann
3.	J. A. Bucholz	40.	J. S. Tang
4.	R. D. Cheverton	41.	J. A. Wang
5.	R. L. Childs	42.	R. M. Westfall
6.	D. G. O'Connor	43.	G. E. Whitesides
7.	W. R. Corwin	44.	A. Zucker
8.	S. K. Iskander	45.	Document Reference Section
9-31.	F. B. K. Kam	46.	Central Research Library
32.	R. P. Leinius	47-48.	Laboratory Records Department
33.	R. K. Nanstad	49.	Laboratory Records - ORNL R.C.
34.	W. E. Pennell	50.	ORNL Patent Office
35.	C. E. Pugh		

NUREG/CR-4816
ORNL/TM-10328
Dist. Category RL, R5

EXTERNAL DISTRIBUTION

T. J. Griesbach, EPRI, 3412 Hillview Avenue, P. O. Box 10412, Palo Alto,
California 94303

W. L. Server, TENERA Engineering Services, 1995 University Avenue,
Berkeley, California 94704

J. Hice, TENERA Engineering Services, 1995 University Avenue, Berkeley,
California 94704

J. Helms, Department of Applied Physics and Nuclear Engineering, 202 Mudd,
Columbia University, New York, New York 10027

C. Z. Serpan, Jr., U.S. Nuclear Regulatory Commission, Division of
Engineering, Mail Stop NL S 217 C, Washington, DC 20555

Al Taboada, U.S. Nuclear Regulatory Commission, Division of Engineering,
Mail Stop NL S 217 C, Washington, DC 20555 (7 copies)

Assistant Manager for Energy Research and Development, U.S. Department of
Energy, Oak Ridge Operations Office, P. O. Box E, Oak Ridge, TN 37831

U.S. Department of Energy, Technical Information Center, Office of
Information Services, P. O. Box 62, Oak Ridge, TN 37831

BIBLIOGRAPHIC DATA SHEET

(See instructions on the reverse)

1. REPORT NUMBER
(Assigned by NRC. Add Vol., Supp., Rev.,
and Addendum Numbers, if any.)

NUREG/CR-4816
ORNL/TM-10328

2. TITLE AND SUBTITLE

PR-EDB: Power Reactor Embrittlement Data Base,
Version 1

Program Description

3. DATE REPORT PUBLISHED
MONTH YEAR
June 1990

4. FIN OR GRANT NUMBER
B0145

5. TYPE OF REPORT

Technical

7. PERIOD COVERED (Inclusive Dates)

6. AUTHOR(S)

F. W. Stallmann, F. B. K. Kam, B. J. Taylor

8. PERFORMING ORGANIZATION - NAME AND ADDRESS (If NRC, provide Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address; If contractor, provide name and mailing address.)

Oak Ridge National Laboratory
P. O. Box 2008
Oak Ridge, TN 37831

9. SPONSORING ORGANIZATION - NAME AND ADDRESS (If NRC, type "Same as above"; if contractor, provide NRC Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address.)

Division of Engineering
Office of Nuclear Regulatory Research
U.S. Nuclear Regulatory Commission
Washington, DC 20555

10. SUPPLEMENTARY NOTES

11. ABSTRACT (200 words or less)

Data concerning radiation embrittlement of pressure vessel steels in commercial power reactors have been collected from available surveillance reports. The purpose of this NRC-sponsored program is to provide the technical bases for voluntary consensus standards, regulatory guides, standard review plans, and codes. The data can also be used for the exploration and verification of embrittlement prediction models. The data files are given in dBASE III Plus format and can be accessed with any personal computer using the DOS operating system. Menu-driven software is provided for easy access to the data including curve fitting and plotting facilities. This software has drastically reduced the time and effort for data processing and evaluation compared to previous data bases.

The current compilation of the Power Reactor Embrittlement Data Base (PR-EDB, version 1) contains results from surveillance capsule reports of 78 reactors with 381 data points from 110 different irradiated base materials (plates and forgings) and 161 data points from 79 different welds. Results from heat-affected-zone materials are also listed. Electric Power Research Institute (EPRI), reactor vendors, and utilities are in the process of providing back-up quality assurance checks of the PR-EDB and will be supplementing the data base with additional data and documentation. Periodic updates of data and software will be released to authorized users. Future updates will also include results from irradiations in materials test reactors.

12. KEY WORDS/DESCRIPTORS (List words or phrases that will assist researchers in locating the report.)

Power Reactor, Data Base, Embrittlement

13. AVAILABILITY STATEMENT

Unlimited

14. SECURITY CLASSIFICATION

(This Page)

Unclassified

(This Report)

Unclassified

15. NUMBER OF PAGES

16. PRICE

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

OFFICIAL BUSINESS
PENALTY FOR PRIVATE USE, \$300

SPECIAL FOURTH-CLASS RATE
POSTAGE & FEES PAID
USNRC
PERMIT No. G-67

120555139531 1 14N19L1RS
US NRC-OADM
DIV FOIA & PUBLICATIONS SVCS
TPO PDR-NUREG
P-123
WASHINGTON DC 20555