



**Florida
Power**
CORPORATION

June 21, 1990
3F0690-10

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Response to Generic Letter 90-04,
Regarding Generic Safety Issues (GSIs)

Dear Sir:

Florida Power Corporation (FPC) hereby submits the attached enclosure as our response to Generic Letter 90-04, "Request For Information On The Status Of Licensee Implementation Of Generic Safety Issues Resolved With Imposition Of Requirements Or Corrective Actions".

Sincerely,

P. M. Beard, Jr.
Senior Vice President
Nuclear Operations

PMB/GMF/sdr

Enclosure

xc: Regional Administrator, Region II
Senior Resident Inspector

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Enclosure 1

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions

FACILITY NAME: Crystal River Unit 3
 DOCKET NO.: 50-302
 LICENSEE: Florida Power Corporation

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	N/A	
43 (B107)	Reliability Of Air Systems	All Plants	I	All actions requested by Generic Letter 88-14 have been completed except for testing of accumulators on HVAC dampers in the Control Complex. That testing will be completed before the end of 1990.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	C	FPC to NRC letter: Jan. 30, 1990 (3F0190-17)
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	Completed in Refuel 7; official correspondence forthcoming.
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	FPC to NRC letters: Nov. 25, 1985 (3F1185-27) and May 21, 1986 (3F0586-23)
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and telecon on May 23, 1985

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and July 31, 1984 (3F0784-21)
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03), Dec. 22, 1987 (3F1183-19) and Jan. 22, 1988 (3F0188-21)
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C	FPC considers this item closed as it relates to GL 83-28. However, the NRC by GL 90-03 has relaxed the requirements for item 2.2.2 and requires licensees to review its current practices. Submittal is scheduled for Sept. 24, 1990. FPC to NRC letters: July 31, 1984 (3F0784-21) and Feb. 25, 1986 (3F0286-05)
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and July 31, 1984 (3F0784-21)
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and July 31, 1984 (3F0784-21)
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03), July 31, 1984 (3F0784-21) and July 10, 1986 (3F0786-09)
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and July 31, 1984 (3F0784-21)

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and July 31, 1984 (3F0784-21)
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03), July 31, 1984 (3F0784-21), Oct. 11, 1985 (3F1085-05), and July 8, 1986 (3F0786-07)
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) July 31, 1984 (3F0784-21), and Mar. 29, 1985 (3F0385-22)
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W and B&W Plants	I	FPC to NRC letters: Mar. 29, 1985 (3F0385-22, TSCRN 134), April 11, 1989 (3F0489-05); NRC to FPC letter April 20, 1989 (3N0489-21, accepts withdrawal of TSCRN 134). Proposed changes will be incorporated as applicable in TSIP.
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B & W Plants	I	FPC to NRC letters: Nov. 4, 1983 (3F1183-03), Jan. 16, 1984 (3F0184-15, TSCRN 111), July 31, 1984, Mar. 31, 1989 (3F0389-22), April 11, 1989 (3F0489-06); NRC to FPC letter April 20, 1989 (3N0489-21, accepts withdrawal of TSCRN 111). Proposed changes will be incorporated as applicable in TSIP.
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	FPC to NRC letters: Nov. 4, 1983 (3F1183-03) and July 31, 1984 (3F0784-21)

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75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	FPC to NRC letter: Nov. 4, 1983 (3F1183-03)
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	N/A	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	FPC to NRC letters: Feb. 25, 1986 (3F0286-13) and Mar. 18, 1988 (3F0388-13)
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C	FPC to NRC letter: Jan. 4, 1989 (3F0189-02)
124	Auxiliary Feedwater System Reliability	ANO - 1 & 2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	I	A third auxiliary feedwater pump will be installed during Refuel 8 (currently scheduled for March 1992). FPC to NRC letter: May 31, 1990 (3F0590-13)
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	FPC to NRC letters: April 3, 1981 (3-041-07) and June 1, 1981 (3-051-49)
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	FPC to NRC letters: April 3, 1981 (3-041-07) and June 1, 1981 (3-051-49)
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	I	Per FPC's letter dated Mar. 31, 1989 (3F0389-21) this item will be incorporated in TSIP.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	
B-36	Develop, Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Application After 4/1/80	N/A	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	FPC to NRC letter: March 14, 1980 and NRC to FPC letter: April 20, 1981 (3N-81-141)