Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

June 10, 1990

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for May, 1990.

Very truly yours,

W R. Arnett

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/JWT/msh

Enclosure

cc: A. B. Davis, NRC, Region III
NRC Resident Inspector LaSalle
Ill. Dept. of Nuclear Safety
R. Pulsifer, NRR Project Manager
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
D. R. Eggett, NED
J. E. Kuskey, GE Resident
T. J. Kovach, Manager of Nuclear Licensing
W. F. Naughton, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle
Terry Novotney, INPO Coordinator, LaSalle Station
T. A. Hammerich, Regulatory Assurance Supervisor
J. W. Gieseker, Technical Staff Supervisor
Central File

ZCADTS/5

IE24
1/1

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MAY 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

ZCADTS/5

TABLE OF CONTENTS
(UNIT 1)

1. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761, extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, generator on-line at 3113 MWT, CRD exercising in progress
	1200	Returned to full-power.
6	0430	Reduced power to 3077 MWT for "B" Main Turbine Control Valve work.
	0900	Increased power to 3277 MWT.
8	0130	Reduced power to 2966 MWT for CRD exercising.
	1100	Returned to full power.
15	0000	Reduced power to 3170 MWT for CRD exercising.
	0700	Returned to full power.
16	0300	Reduced load to 3100 MWT to insert FCL rods.
17	0700	Returned to full power.
22	0100	Reduced power to 2910 MWT for CRD exercising, FW Heater Bay work, and monthly surveillances.
	1600	Returned to full power.
29	0000	Reduced load to 3120 for CRD exercising.
	1000	Returned to full power.
31	2400	Reactor Critical, generator on-line at full power.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.(Unit 1)

1. Amendments to the Facility License or Technical Specification.
   a. Revised Snubber surveillance interval tolerance.

2. Changes to procedures which are described in the Safety Analysis Report.
   a. LaSalle Special Test, LST-90-053 "Operation of the 1B Reactor Recirc Flow Control Valve without LVDT Feedback". This procedure will provide the instructions necessary to perform initial calibration, installation tuning, testing, and operation of the special signal card for simulating RR valve position based on the RR loop flow signal. This is necessary due to erratic feedback signals from the 1B FCV LVDT and still use a flow signal feedback to allow continued operation of the 1B FCV.
   b. LaSalle Special Test, LST-90-054 "Control Room/Auxiliary Electric Room HVAC System Initial Damper Adjustment'. The purpose of this test is to provide a methodology for ensuring that the "A" train Control Room (VC) and Auxiliary Electric Room (VE) HVAC systems are operable during the initial adjustment of balancing dampers. This will increase system flowrate to design by removing excess system resistance.
   c. LaSalle Special Test, LST-90-061 "Manipulation of the Reactor Recirculation 1B Loop's Flow Control Valve Without LVDT Feedback". The purpose of the procedure is to provide a method of controlling the 1B loop's flow control valve and thus change reactor power without LVDT feedback on the valve. The purpose is due to the position feedback signal is unstable and the drywell is inaccessible.

3. Tests and Experiments not described in the Safety Analysis Report.
   (None)

4. Major corrective maintenance to Safety-Related Equipment, including any SOR switch failure reports.
   (None, See attached 2 pages for SOR-switch reports)

5. Completed Safety-Related Modifications.
   (None)

C. LICENSEE EVENT REPORTS (Unit 1)

LER Number	Date	Description
90-007-00	5/11/90	Failed RCIC Hi Steam Flow DP Switch due to torn diaphram
90-008-00	5/25/90	Missed hourly fire watch on Refuel Floor

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (attached)

2. Average Daily Unit Power Level (attached)

3. Unit Shutdowns and Significant Power Reductions (none)

SOR dp Switch Failure Data Sheet

Equipment Piece Number: 1E31-N012AA Model Number: 103AS-B203-NX-JJTTX6

Serial Number: 87-12-602

Application: RHR Shutdown Cooling High Suction Flow Isolation Switch

Date and Time of Discovery: 05/05/90 1140 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 167.4 - 169.4 "WC

Nominal Setpoint: 168.4 "WC

Action Limits: <163.0 or >173.8 "WC

Reject Limits: <158.8 or >178.0 "WC

Technical Specification Limit: 186.0 "WC

As Found Setpoint: 186.0 "WC

Date and Time of Return to Service: 05/06/90 0540 hours

Model Number of Replacement Switch: 103AS-B203-NX-JJTTX6

Serial Number of Replacement Switch: 88-6-119

Cause: Instrument setpoint drift.

Corrective Action: Replaced switch. Inspected failed switch with nothing abnormal found.

DVR Number: 1-1-90-039

SOR dp Switch Failure Data Sheet

Equipment Piece Number: 1E31-N013AA Model Number: 103AS-B203-NX-JJTTX6

Serial Number: 85-4-86

Application: RCIC High Flow Isolation Switch

Date and Time of Discovery: 05/11/90 0240 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 165.4 - 167.4 "WC

Nominal Setpoint: 166.4 "WC

Action Limits: <116.0 or >171.8 "WC

Reject Limits: <155.8 or >177.0 "WC

Technical Specification Limit: 185.0 "WC

As Found Setpoint: --- "WC

Date and Time of Return to Service: 05/11/90 1900 hours

Model Number of Replacement Switch: 103AS-B203-N[illegible]JJTTX6

Serial Number of Replacement Switch: 88-6-121

Cause: Switch would not hold pressure during functional testing. During inspection the switch was found to have a tear in the diaphram between the cylinder disk outside diameter and O-ring seating surface (approx 3/8").

Corrective Action: Replaced switch. Inspected failed switch.

DVR Number: 1-1-90-040

B.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE June 10, 1989
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: MAY 1990 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net) 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (None)

4. REASONS FOR RESTRICTION (IF ANY): (N/A)

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	744.0	3,452.0	35,586.1
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,641.2
7. TIME GENERATOR ON-LINE (HOURS)	744.0	3,325.1	34,743.4
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWHt-Gross)	2,450,832	10,510,137.4	98,343,705.4
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	840,903	3,596,Y79	32,665,520
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	816,957	3,483,416	31,216,118
12. REACTOR SERVICE FACTOR (%)	100.0	95.3	63.3
13. REACTOR AVAILABILITY FACTOR (%)	100.0	95.3	66.2
14. SERVICE FACTOR (%)	100.0	91.8	61.8
15. AVAILIBILITY FACTOR (%)	100.0	91.8	61.8
16. CAPACITY FACTOR (USING MDC) (%)	106.0	92.8	53.6
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	101.9	89.2	51.5
18. FORCED OUTAGE FACTOR (%)	0.0	3.8	9.6

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
(None)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
(N/A)

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE June 10, 1989
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

REPORT PERIOD: MAY 1990

DAY	POWER	DAY	POWER
1	1,083	17	1,103
2	1,101	18	1,102
3	1,106	19	1,100
4	1,108	20	1,098
5	1,109	21	1,102
6	1,092	22	1,051
7	1,095	23	1,105
8	1,081	24	1,100
9	1,099	25	1,099
10	1,108	26	1,100
11	1,105	27	1,101
12	1,111	28	1,096
13	1,111	29	1,079
14	1,109	30	1,096
15	1,104	31	1,097
16	1,090		

E. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

(None)

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE

(Note: the year and unit numbers have been ommitted from the Outage Number)

(Unit 0)

116	0DG01P	Repair
117	0DG08CB	Check hold-downs
118	0DG08CA	Check hold-downs
119	0 D/G	Replace air-start motors
120	0 D/G	Replace filter

(Unit 1)

221	1E31-N012AA(RH)	Replace switch
224	1E12-D300B (RH)	Lubricate
226	1E12-F011B (RH)	(Administrative, per T.S. 3.6.3.)
229	1E51-F008 (RI)	(Administrative, per T.S.)
230	1E51-F038 (RI)	Repack valve
234	1E21-C002 (LP)	Lubricate
235	1E22-C302A (HP)	Repair head-gasket

3. Changes to the Off-Site Dose Calculation Manual
(None).

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MAY, 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

ZCADTS/7

TABLE OF CONTENTS
(Unit 2)

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761 extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor subcritical, generator off-line, L2R03 in progress.
31	2400	Reactor subcritical, generator off-line, L2R03 in progress.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.(Unit 2)

1. Amendments to the Facility License or Technical Specification.
   a. Revised Snubber surveillance interval tolerance.

2. Changes to procedures which are described in the Safety Analysis Report.
   (None)

3. Tests and Experiments not described in the Safety Analysis Report.
   (None)

4. Major corrective maintenance to Safety-Related Equipment, including any SOR switch failures.
   (See Table 1)

5. Completed Safety-Related Modifications.
   (None)

B.4 TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L86754	2C41-D304	Foreign material in orfice	Test loop pressure low	Removed obstruction
L98490	2E12-FC48B	Faulty Limitorque operator	Breaker trips when valve strokes closed	Repalced operator

(No SOR Switch Failures)

C. LICENSEE EVENT REPORTS (Unit 2)

LER Number	Date	Description
90-008-00	05/02/90	ESF Actuation of the Control Room B Emergency Circuit Ventilation Makeup Fan/Procedural Deficiency.
90-009-00	05/10/90	RWCU Isolation during Surveillance Testing.

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (Attached)

2. Average Daily Unit Power Level (Attached)

3. Unit Shutdowns and Significant Power Reductions (Attached)

ZCADTS/7

D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE #2
DATE June 10, 1990
COMPLETED BY J.W.THURSTEDT
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: MAY 1990 — GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,030
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 0

4. REASONS FOR RESTRICTION (IF ANY): Refueling (L2R03)

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	0.0	1,768.5	81,811.2
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,716.0
7. TIME GENERATOR ON-LINE (HOURS)	0.0	1,748.7	31,481.8
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWHt-Gross)	0	5,384,967.2	91,907,615.2
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	0	1,797,042	30,358,768
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	-6,409	1,682,598	29,856,843
12. REACTOR SERVICE FACTOR (%)	0.0	63.7	64.8
13. REACTOR AVAILABILITY FACTOR (%)	0.0	63.7	66.3
14. SERVICE FACTOR (%)	0.0	63.3	63.0
15. AVAILABILITY FACTOR (%)	0.0	63.3	63.8
16. CAPACITY FACTOR (USING MDC) (%)	-1.1	49.5	57.0
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	-1.0	46.6	54.7
18. FORCED OUTAGE FACTOR (%)	0.0	2.9	15.7

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
Refueling (L2R03 Continuation)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
June 9, 1990

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE June 10, 1989
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

REPORT PERIOD: MAY 1989

DAY	POWER	DAY	POWER
1	-11	17	-11
2	-11	18	-11
3	-11	19	-11
4	-11	20	-11
5	-11	21	-11
6	-11	22	-11
7	-11	23	-11
8	-11	24	-11
9	-12	25	-12
10	-11	26	-12
11	-11	27	-12
12	-11	28	-12
13	-11	29	-12
14	-11	30	-12
15	-11	31	-12
16	-11		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON CODE	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
6	3-17-90	S	744	C	4	L2R03

ZCADTS/7

E. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

(None)

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE
1224	2E51-F073A (RH)	Repair position indicator.
1228	2RHO24B/27B	(Administrative, perform LOP-RH-07)
1229	LPCS D/G, Div III battery	(Part of SAT outage)
1233	2E22-F038 (HP)	Perform LTS-500-211
1269	"A" RHR/O D/G	Perform LES-RH-200
1278	2B RHR Pump seal-cooler	(Administrative)
1281	Div I ECCS Injection valves	Perform LTS-500-209
1282	2E51-C004 (RI)	Determinate cable
1290	A RHR miscellaneous valves	Perform LOP-RH-07
1297	LPCS spool-piece	Remove spool-piece
1298	HPCS spool-piece	Remove spool-piece
1299	2E51-F025 (RI)	Replace solenoid valve
1301	2H13-P621 (RI)	Perform signature trace
1309	2E51-F076 (RI)	Repack valve
1330	2E51-F357 (RI)	Perform hydro static test
1379	2E12-F302 (RH)	Perform LOP-SF-05
1398	2E12-F024A/27A (RH)	Perform LOP-RH-07
1407	2E12-F024B/27B (RH)	Perform LOP-RH-07
1410	2E12-F047B (RH)	Adjust packing

3. Changes to the Off-Site Dose Calculation Manual.
(None).

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)