



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 31, 1990

Docket No. 50-327

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. Kingsley:

SUBJECT: RELIEF FROM CODE HYDROSTATIC PRESSURE TEST REQUIREMENTS FOR REACTOR
COOLANT SYSTEM (TAC 75030) - SEQUOYAH NUCLEAR PLANT, UNIT 1

By letter dated May 29, 1990, you requested relief from the hydrostatic pressure test requirements of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, 1980 Edition through the Winter 1981 Addenda (Code). The relief was requested for the reactor coolant system (RCS) and a section of an associated safety injection line of the emergency core cooling system (ECCS) for Unit 1. The relief is associated with the Unit 1 modification, during the restart from the Cycle 4 refueling outage, to replace Check Valve 1-VLV-63-551 in a safety injection line.

You stated that the need to replace the check valve was only recently identified during testing performed to support restart of Unit 1 from its Cycle 4 refueling outage. Because the replacement and Code weld examination of this check valve is presently scheduled to be completed in the early morning of May 30, 1990 and the unit will then be ready to begin heatup to return to power, you requested that NRC expedite its review of the relief request.

You proposed, as an alternative method to the Code test, to perform a system leakage test of the new valve, welds, and adjoining piping in Mode 3 and a Code examination of the welds. As discussed in the enclosed Safety Evaluation (SE), the staff has evaluated your request for relief from the Code requirements and concludes that the alternative method of testing proposed by you provides an acceptable level of quality and safety at Sequoyah.

Granting relief from Code requirements is authorized by law, pursuant to 10 CFR 50.55a(a)(3)(i), where the proposed alternative to the Code would provide an acceptable level of quality and safety at the plant. Therefore, based on the enclosed SE and pursuant to 10 CFR 50.55a(a)(3)(i), Tennessee Valley Authority is granted the requested relief from the Code for Sequoyah, Unit 1, for the modification to replace Check Valve 1-VLV-63-551, as documented in the enclosed SE. As you explained, a Code hydrostatic pressure test of the

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RCS will be performed near the end of the 10-year inservice inspection interval for Sequoyah Unit 1 as part of the Sequoyah inservice inspection program.

Because the staff concluded that the alternative method of testing provides an acceptable level of quality and safety and the check valve was scheduled to be replaced in the early morning of May 30, 1990, you were granted the requested relief from the Code on the evening of May 29, 1990. This letter also documents that this relief was granted on this date.

This relief for the RCS and the associated ECCS piping is in addition to the relief for the RCS granted to Sequoyah by the staff in its letter dated, May 11, 1990 for the modifications in the Cycle 4 refueling outage for each unit to remove the upper head injection system and the RCS resistance temperature device (RTD) manifolds. The Cycle 4 refueling outage for Unit 2 is scheduled to begin in October 1990.

Sincerely,

Original signed by

Suzanne Black, Assistant Director
for Projects
TVA Projects Division
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation

cc w/enclosure:
See next page

5/31/90 *Janice Mone*

OFC	NRR:TVA/PM	NRR:MTEB	OGC	TVA:AD/P	TVA:AD	ADSP:AD
NAME	JDonohew	CY Cheng	SBlack	BDLew	DCrutchfield	
DATE	5/30/90	5/30/90	5/31/90	5/31/90	5/31/90	5/31/90

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