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US Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Response to Generic Letter 90-04

Dear Sirs:

In response to Generic Letter 90-04, attached is a completed tabulation of Generic Safety Issues (GSI) for TMI-2. As expected, these GSIs do not apply to TMI-2 in its current shutdown condition. However, as stated in the attached, if a decision is made to restart the plant, these issues will be reviewed for applicability.

Sincerely,

M. B. Roche
Director, TMI-2

EDS/mkk

Attachment

cc: T. T. Martin - Regional Administrator, Region I
J. F. Stolz - Director, Plant Directorate I-4
L. H. Thonus - Project Manager, TMI Site
F. I. Young - Senior Resident Inspector, TMI

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FACILITY NAME: Three Mile Island Unit 2
 DOCKET NO.: 50-320
 LICENSEE: GPU Nuclear

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	NA	NA - This issue will be reviewed for applicability prior to restart.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	NA	NA - This issue will be reviewed for applicability prior to restart.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	NA	NA - This issue will be reviewed for applicability prior to restart.
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	NA	NA - This issue will be reviewed for applicability prior to restart.
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	NA	NA - This issue will be reviewed for applicability prior to restart.

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	NA	- This issue will be reviewed for applicability prior to restart.
75 (B089)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	NA	- This issue will be reviewed for applicability prior to restart.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	NA - This issue will be reviewed for applicability prior to restart.	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	NA - This issue will be reviewed for applicability prior to restart.	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	NA - This issue will be reviewed for applicability prior to restart.	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA - This issue will be reviewed for applicability prior to restart.	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	NA	This issue will be reviewed for applicability prior to restart.
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	NA	This issue will be reviewed for applicability prior to restart.
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	NA	This issue will be reviewed for applicability prior to restart.
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	NA	This issue will be reviewed for applicability prior to restart.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NA	This issue will be reviewed for applicability prior to restart.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NA	This issue will be reviewed for applicability prior to restart.
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NA	This issue will be reviewed for applicability prior to restart.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NA	This issue will be reviewed for applicability prior to restart.