

BOSTON EDISON
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

K. L. Highfill
Station Director &
Vice President, Nuclear Operations

May 14, 1990
BECo Ltr. #90-068

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

License No. DPR-35
Docket No. 50-293

Subject: April 1990 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

K. L. Highfill
K.L. Highfill

WJM:bal

Attachment

cc: Regional Administrator, Region 1
U.S. Nuclear Regulatory Commission
475 Allendale Rd.
King of Prussia, PA 19406

Senior Resident Inspector

9005210260 900430
PDR ADOCK 05000293
R PIC

Case No 2082896078
IE29
11

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
 UNIT Pilgrim 1
 DATE May 14, 1990
 COMPLETED BY W. Munro
 TELEPHONE (508) 747-8474

MONTH April 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	123
13	0	29	381
14	0	30	574
15	0	31	N/A
16	0		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE May 14, 1990
 COMPLETED BY W. Munro
 TELEPHONE (508) 747-8474

OPERATING STATUS

Notes

1. Unit Name Pilgrim 1
2. Reporting Period April 1990
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None
9. Power Level To Which Restricted, If Any (Net MWe) None
10. Reasons For Restrictions, If Any N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>719.0</u>	<u>2879.0</u>	<u>152447.0</u>
12. Number Of Hours Reactor Was Critical	<u>164.8</u>	<u>1831.1</u>	<u>87236.3</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>65.5</u>	<u>1726.4</u>	<u>83855.7</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>82392.0</u>	<u>3341256.0</u>	<u>143826168.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>27130.0</u>	<u>1152930.0</u>	<u>48389754.0</u>
18. Net Electrical Energy Generated (MWH)	<u>25874.0</u>	<u>*1109586.0</u>	<u>*46492770.0</u>
19. Unit Service Factor	<u>9.1</u>	<u>60.0</u>	<u>55.0</u>
20. Unit Availability Factor	<u>9.1</u>	<u>60.0</u>	<u>55.0</u>
21. Unit Capacity Factor (Using MDC Net)	<u>5.4</u>	<u>57.5</u>	<u>45.5</u>
22. Unit Capacity Factor (Using DER Net)	<u>5.5</u>	<u>58.8</u>	<u>46.6</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>12.7</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup - _____
26. Units In Test Status (Prior to Commercial Operation): N/A

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

(9/77)

* Includes adjustment from March 1990 Report

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Operational Summary for April 1990

The unit started the reporting period with the mid cycle outage in progress. On 4-24-90 the reactor was made critical at 0310 hours. During the increase to power the HPCI and RCIC operability tests were successfully performed at both 150 psig and 1000 psig. The power increase continued, and at 0633 hours on 4-28-90 the unit was synchronized to the grid and remained on line for the remainder of the reporting period.

Safety Relief Valve Challenges
Month of April 1990

Requirement: NUREG-0737 T.A.P. II.K.3.3

Date: April 25, 1990

Valve # 203-3A, 3B, 3C and 3D

Reason: Startup Testing (ref. Procedure 8.5.6.2)

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo Ltr. #81-01 dated 01/05/81.

Month April 1990

PILGRIM NUCLEAR POWER STATION
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Recirculation System Motor Generator Set (MG)	Recirc. MG Set "A"	Instability in the speed control function loop. (F&MR 90-49)	Aging of the electronic components of the MG Set.	Recirculation Speed Control Modification PDC 90-14 has been incorporated into "A" Recirc. MG Set.	N/A	N/A
Electric Power System	Circuit Breaker 52-202 (G.E. type AK-2A-50)	Breaker 52-202 failed to trip (open) automatically during bus transfer. (F&MR 90-64)	Latch prop misalignment due to missing retaining ring resulting in failure of its trip coil.	Breaker sent to G.E. for inspection and overhaul. Breaker re-installed and post work tested satisfactorily.	Inspection, overhaul and testing of similar (480V) breakers performed. (Refer to Associated LER).	LER 90-005-00
Residual Heat Removal System (RHR)	RHR Injection Check Valve CK-1001-68B.	Check valve CK-1001-68B failed hydrodynamic leakage test. (F&MR 90-68).	Misalignment of the valve under investigation.	Valve disassembled and inspected. Replaced hinge pin bushings, hinge pins, and machined disc and seat surfaces. Performed hydrodynamic leakage test successfully.	To be determined.	N/A
Residual Heat Removal System (RHR)	LPCI Manual Isolation Valve 1001-HO-33A.	Leakage developed through pressure seal.	Normal wear.	Furmanite injected per PDC 89-49 and FRN 89-49-11.	Pressure seals to be repaired during RFO-8.	N/A
Residual Heat Removal System (RHR)	Reactor Head Spray Vent Valve 1001-HO-177.	Developed external body to bonnet leakage.	Normal wear.	Furminate injected per PDC 90-031, to seal valve in its normally closed position.	Valve to be replaced during RFO-8.	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293

NAME Pilgrim 1DATE May 14, 1990COMPLETED BY W. MunroTELEPHONE (508) 747-8474REPORT MONTH April 1990

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODES	CAUSE & CORRECTIVE ACTION TO PREVENTIVE RECURRENCE
02	04/01/90	S	653.5	B	1	N/A	N/A	N/A	Continuation of shut- down for midcycle maintenance outage.

1	2	2	3	4&5
F-forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)

Month April 1990

PILGRIM NUCLEAR POWER STATION
 MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Residual Heat Removal System (RHR)	RHR Containment Isolation Valve. MO-1001-29B	Valve failed Local Leak Rate Test (LLRT) (F&MR 90-73)	Under investigation	Disassembly and inspection of valve. Lapped the body, seal, and seats. Replaced valve stem. Performed LLRT successfully.	To be determined	N/A

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that, each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next Refueling Shutdown: Second Quarter 1991
3. Scheduled date for restart following refueling: Second Quarter 1991
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1986/87 refueling outage was of the same design as loaded in the previous outage, and consisted of 192 assemblies.
7. (a) There are 580 fuel assemblies in the core.
(b) There are 1320 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1000 fuel assemblies.