



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TEXAS ENGINEERING EXPERIMENT STATION/TEXAS A&M UNIVERSITY SYSTEM

DOCKET NO. 50-128

AMENDMENT TO FACILITY OPERATING LICENSE

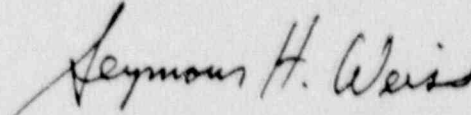
Amendment No. 12
License No. R-83

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to facility Operating License No. R-83, filed by the Texas Engineering Experiment Station/Texas A&M University System (the licensee), dated September 4, 1989 and October 24, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Publication of notice of this amendment is not required since it does not involve a significant hazards consideration nor amendment of a license of the type described in 10 CFR Section 2.106(a)(2).

2. The license is amended by designating the Texas Engineering Experiment Station/Texas A&M University System as the licensee instead of the Texas A&M University. This amendment deletes the name Texas A&M University wherever it appears in the license or amendments thereto and replaces it with Texas Engineering Experiment Station/Texas A&M University System wherever it appears in the license or amendments thereto.
3. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 11.C.(2) of Facility Operating License No. R-83 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 12, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.
4. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactor, Decommissioning and
Environmental Project Directorate
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A Technical
Specifications Changes

Date of Issuance: January 2, 1990

ENCLOSURE TO LICENSE AMENDMENT NO. 12

FACILITY OPERATING LICENSE NO. R-83

DOCKET NO. 50-128

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove Page</u>	<u>Insert Page</u>
46	46
47	47
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51	51
57	57

6.0 Administrative Controls

6.1 Organization

6.1.1 Structure

- (a) A line management organizational structure shall provide for the administration and operation of the reactor facility. The Deputy Director of the Texas Engineering Experiment Station (TEES) and the Director of the Nuclear Science Center (NSC) shall have line management responsibility for adhering to the terms and conditions of the Nuclear Science Center Reactor (NSCR) license and technical specifications and for safeguarding the public and facility personnel from undue radiation exposure. The facility shall be under the direct control of the Director (NSC) or a licensed senior reactor operator designated by him to be in direct control.
- (b) Management Levels
- Level 1 - Deputy Director TEES (Licensee) - Responsible for the NSCR facility license.
- Level 2 - Director (NSC) - Responsible for reactor facility operation and shall report to Level 1.
- Level 3 - Senior Reactor Operator on Duty - Responsible for the day-to-day operation of the NSCR on shift operation and shall report to Level 2.
- Level 4 - Reactor Operating Staff - Licensed reactor operators and senior reactor operators and trainees. These individuals shall report to Level 3.
- (c) Radiation Safety - A qualified, health physicist shall be assigned responsibility for implementation of the radiation protection program at the NSCR. The individual shall report to Level 1 management.
- (d) Reactor Safety Board (RSB) - Responsible to the Licensee for providing an independent review and audit of the safety aspects of the NSCR.

6.1.2 Responsibility - Responsibility for the safe operation of the reactor facility shall be in accordance with the line organization established in 6.1.1.

6.2 Review and Audit

6.2.1 Reactor Safety Board - The Reactor Safety Board (RSB) shall consist of at least three (3) voting members knowledgeable in fields which relate to nuclear safety. The RSB shall review, evaluate and make recommendations on safety standards associated with the operational use of the facility. NSCR operations and health physics shall be represented as ex-officio members on the RSB. The review and advisory functions of the RSB shall include NSCR operations, radiation protection, and the facility license.

6.2.2 RSB Charter and Rules - The operations of the RSB shall be in accordance with a written charter, including provisions for:

- (a) Meeting frequency - not less than once per calendar year and as frequent as circumstances warrant consistent with effective monitoring of facility activities.
- (b) Voting rules
- (c) Quorums
- (d) Use of subcommittees
- (e) Review, approval and dissemination of minutes

6.2.3 RSB Review Function - The responsibilities of the RSB or a designated subcommittee thereof include, but are not limited to the following:

- (a) Review and evaluation of whether a proposed change, test, or experiment would constitute an unreviewed safety question or a change in Technical Specifications.
- (b) Review of new procedures, major revisions of procedures, and proposed changes in reactor facility equipment or systems having safety significance.
- (c) Review of new experiments or classes of experiments that could affect reactivity or result in the release of radioactivity.
- (d) Review of proposed changes in technical specifications, license, or charter.

- (e) Review of the NSCR radiation protection program.
- (f) Review of violations of technical specifications, license, or charter, and violations of internal procedures or instructions having safety significance.
- (g) Review of operating abnormalities having safety significance.
- (h) Review of reportable occurrences listed in 6.0.2.
- (i) Review of audit reports.

6.2.4 RSB Audit Function - The RSB or a subcommittee thereof shall audit reactor operations and radiation protection programs at least quarterly, but at intervals not to exceed four months. Audits shall include but are not limited to the following:

- (a) Facility operations, including radiation protection, for conformance to the technical specifications, applicable license conditions, and standard operating procedures at least once per calendar year (interval between audits not to exceed 15 months).
- (b) The retraining and requalification program for the operating staff at least once per calendar year (interval between audits not to exceed 15 months).
- (c) The facility security plan and records at least once per calendar year (interval between audits not to exceed 15 months).
- (d) The reactor facility emergency plan and implementing procedures at least once per calendar year (interval between audits not to exceed 15 months).

6.2.5 Audit of ALARA Program - The licensee or his designated alternate (excluding anyone whose normal job function is within the NSCR) shall conduct an audit of the reactor facility ALARA program at least once per calendar year (interval between audits not to exceed 15 months). The results of the audit shall be transmitted by the licensee to the RSB at the next scheduled meeting.

- (b) A written report within 30 days of:
 - (1) Personnel changes in the facility organization involving Level 1 and Level 2.
 - (2) Significant changes in the transient or accident analysis as described in the Safety Analysis Report.

6.7 Records - Records of facility operations in the form of logs, data sheets or other suitable forms shall be retained for the period indicated as follows:

6.7.1 Records to be Retained for a Period of at Least Five Years or for the Life of the Component Involved

- (a) Normal reactor facility operation.
- (b) Principal maintenance operations.
- (c) Reportable occurrences.
- (d) Surveillance activities required by the Technical Specifications.
- (e) Reactor facility radiation and contamination surveys where required by applicable regulations.
- (f) Experiments performed with the reactor.
- (g) Fuel inventories, receipts, and shipments.
- (h) Approved changes in operating procedures.
- (i) Records of meeting and audit reports of the RSB.

6.7.2 Records to be Retained for at Least One Training Cycle - Re-training and requalification of certified operations personnel: Records of the most recent complete cycle shall be maintained at all times the individual is employed.

6.7.3 Records to be Retained for the Lifetime of the Reactor Facility

- (a) Gaseous and liquid radioactive effluents released to the environs.