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Docket No. 50-334

Mr. J. J. Carey, Vice President
Duquesne Light Company
Nuclear Division
Post Office Box 4
Shippingport, Pennsylvania 15077

Dear Mr. Carey:

SUBJECT: UPGRADED SRO AND RO TRAINING AND TRAINING FOR MITIGATING CORE DAMAGE - REQUEST FOR ADDITIONAL INFORMATION

In our review of NUREG-0737 Item Nos. I.A.2.1 and II.B.4 for Beaver Valley Power Station, Unit No. 1, we have identified additional information which we will need in order to complete our review. Science Applications, Inc. under contract to the NRC has developed the attached request for additional information. We request that you respond within 30 days of receipt of this letter. Please send a copy of your submittal directly to Dr. R. T. Liner, Science Applications, Inc., 1710 Goodridge Drive, McLean, Virginia 22102.

Sincerely,

Original signed by:
S. A. Varga
Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

OFFICE	ORB#1:DL	ORB#1:DL	ORB#1:DL				
SURNAME	DChaney:ds	DWigginton	SVarga				
DATE	03/15/82	03/16/82	03/16/82				

Mr. J. J. Carey
Duquesne Light Company

cc: Mr. H. P. Williams
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Ronald C. Haynes
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Enclosure 1

Subject: Improvements in Operator Training and Requalification Programs

Plant: Beaver Valley

Date: 23 February 1982

LICENSING ACTION REQUEST FOR ADDITIONAL INFORMATION

The U.S. Nuclear Regulatory Commission and its technical assistance contractor, Science Applications, Inc. (SAI), are performing a review to ascertain the acceptability of your response to certain requirements contained in post-TMI Action Items set forth in NUREG-0660 and NUREG-0737:

I.A.2.1 Immediate Upgrading of Reactor Operator and Senior Reactor Operator Training and Qualifications

II.B.4 Training for Mitigating Core Damage

Specifically, this review addresses the following items from Enclosure 1 of Harold Denton's letter of March 28, 1980, contained in NUREG-0737. Item A.2.c which addresses operator training requirements, item A.2.e which addresses instructor requalification, and Section C which addresses operator requalification. Some of these items are further elaborated in Enclosures 2, 3, and 4 of the Denton letter and in post-TMI Action Item II.B.4 (also in NUREG-0737).

Our review is presently based on your submittal of September 2, 1980, which incudes the following:

1. Letter dated 2 September, 1980, J. A. Werling to Harold R. Denton.

2. Duquesne Light Company, Beaver Valley Power Station Training Manual Section 2.0, Operations Training, Issue 2, Rev. 1, (3 subsections, no date).

We have assumed in our review that these submittals reflect your current training program. We need answers to the following questions before we can complete our evaluation:

1. The training programs for licensed operating personnel consists of two options. Under what conditions has Option I been exercised? Under what conditions has Option II been exercised? Are both options in effect now?

2. The training programs for reactor operators (Option I and Option II) and the requalification program for reactor operators (Section 2.2) include lectures which appear to have the potential for covering the subjects of heat transfer, fluid flow and thermodynamics as called out in enclosure 1 of Denton's March 28, 1980, letter. Do these lectures in fact cover these materials and is the coverage to the level spelled out in enclosure 2 of the Denton letter?

3. The training program for licensed operating personnel Option I appears to contain no training in the area of accident mitigation and training in dealing with transients as called out in enclosure 1 of Denton's letter. Does your training program Option I cover these areas and is the coverage to the level spelled out in enclosure 3 of Denton's letter?

4. The training program for licensed operating personnel Option II appears to have some potential for covering the training in the area of accident mitigation and training in dealing with transients as called out in enclosure 1 of Denton's letter. Does your training program Option II cover these areas and is the coverage to the level spelled out in enclosure 3 of Denton's letter?

5. Do the training programs for Options I and II each involve 80 contact hours under the subjects of heat transfer, fluid flow, thermodynamics and accident mitigation? Are there similarly 80 contact hours of heat transfer, fluid flow, thermodynamics and accident mitigation in the requalification program (Section 2.2)? (A contact hour of instruction is a one-hour period in which the course instructor is present or available for instructing or assisting students; lectures, seminars, discussions, problem-solving sessions, and examinations are considered contact periods under this definition.)

6. The reactor operator requalification program, Section 2.2.4.3, identifies nine reactor manipulations (Items (a) to (h) and (j)) along with ambiguous item (i), plant and reactor operation, that involve emergency or transient procedures where reactivity is changing. Do these ten items cover all the twenty-seven manipulations identified in enclosure 4 of the Denton letter?

7. Are your instructors enrolled in appropriate requalification programs to assure they are cognizant of current operating history, problems, and changes to procedures and administrative limitations?

For item II.B.4 provide an outline of the training program for mitigating core damage, including the number of training hours involved. Your outline can include any training program which relates to the training for mitigating core damage. Follow the guidelines given in the enclosure 3 of H. R. Denton's letter dated March 28, 1980 and INPO Guidelines for Training to Recognize and Mitigate the Consequences of Core Damage (Document Number STG-01, Rev. 1, January 15, 1981). NRC requires minimum of 80 contact hours of training for mitigating core damage.