AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Unit #1
DATE March 12, 1982
COMPLETED BY R. W. Short
TELEPHONE (402)536-4543

MONTH February, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	462.9	17	321.6
2	466.1	18	376.9
3	476.7	19	457.2
4	477.3	20	475.9
5	475.8	21	483.6
6	473.4	22	485.8
7	476.8	23	486.5
8	478.4	24	486.5
9	479.1	25	485.2
10	228.4	26	486.2
11	0.0	27	485.7
12	37.3	28	485.4
13	384.2	29	
14	435.7	30	
15	452.7	31	
16	457.3		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE March 12, 1982
COMPLETED BY R. W. Short
TELEPHONE (402)536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: February, 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672.0	1,416.0	73,897.0
12. Number Of Hours Reactor Was Critical	618.5	1,362.5	57,601.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	613.0	1,357.0	56,447.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	876,324.7	1,969,834.5	68,671,664.6
17. Gross Electrical Energy Generated (MWH)	297,153.9	668,441.9	22,742,387.5
18. Net Electrical Energy Generated (MWH)	282,681.3	637,276.5	21,485,145.0
19. Unit Service Factor	91.2	95.8	76.4
20. Unit Availability Factor	91.2	95.8	76.4
21. Unit Capacity Factor (Using MDC Net)	88.0	94.2	63.4
22. Unit Capacity Factor (Using DER Net)	88.0	94.2	63.1
23. Unit Forced Outage Rate	8.8	4.2	3.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): None

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982

DOCKET NO. 50-285
UNIT NAME Fort Calhoun #1
DATE March 12, 1982
COMPLETED BY R. W. Short
TELEPHONE (402)536-4543

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-01	820210	F	50.0	A	1	82-005	RB	CONROD	Control rod inserted into the core due to a faulty magnetic clutch. Replaced magnetic clutch.
82-02	820217	F	9.0	A	3	N/A	NA	INSTRU-C	A circuit card in the electro-hydraulic turbine control system malfunctioned. Replaced faulty circuit card.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending February 1982.

1. Scheduled date for next refueling shutdown. — January 7, 1983

2. Scheduled date for restart following refueling. — April 1, 1983

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      A Technical Specification change.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. —

   c. If no such review has taken place, when is it scheduled? —

4. Scheduled date(s) for submitting proposed licensing action and support information. — October 1, 1982

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:
   a) in the core — 133 assemblies
   b) in the spent fuel pool — 237 "
   c) spent fuel pool storage capacity — 483 "
   d) planned spent fuel pool storage capacity — 483 "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1985

Prepared by J. K. Gasper — Date March 1, 1982

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

February, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station maintained a nominal 100% power during February with the following exceptions:

(1) February 10, 1982 / CEDM #24 clutch failed causing a control rod to drop into the core. A controlled power reduction was initiated. The unit was returned to service February 12, 1982.

(2) February 17, 1982 a malfunction of the turbine control valves caused a automatic reactor shutdown. The unit was returned to service early on February 18, 1982.

A unusual event was declared on February 3, 1982 due to a radioactive gas leak from the auxiliary building vent header. Conditions returned to normal the same day and no Technical Specifications release limits were exceeded.

One additional reactor operator's license was received during February.

The operations staff completed the annual radiation/security refresher during February.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
LER 82-01	During normal operation, while performing ST-FP-9(F.1), two fire barrier penetrations were found to be non-functional. The Shift Supervisor was immediately notified. Contrary to the requirements of Tech. Spec. 2.19(7), a fire watch was not posted within one hour, nor was an hourly fire watch patrol established. The fire detector zones adjacent to the degraded fire barriers were operable.
LER 82-02	During power operations at approximately 99% power, lockout relay 86B1/CRHS (Containment Radiation High Signal) failed to actuate on demand by the plant radiation monitoring system. This resulted in the failure of one of the Engineered Safety Feature Channels to operate. However, during the incident all redundant CRHS lockout relays (86A/CRHS, 86A1/CRHS and 86B/CRHS) functioned as designed and initiated appropriate safeguards equipment as required.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operation Incidents and are not reported elsewhere in the report:

Operation Incidents		Deficiency
OI-1496	ST-DG-2	Diesel Generator #2 devices, out of tolerance. Fuel Oil Transfer Pump #2 low pressure alarm, Fuel Oil Transfer Pump #2 high pressure alarm, local tachometer, lube oil pressure idle pressure low alarm & engine trip.
OI-1500	ST-ESF-14	Not completed on time. Engineered Safeguards.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-STRETCH-2	The purpose of Special Procedure SP-STRETCH-2 was to measure and compare dose rates in various areas at 1500 MW_{th} with dose rates at 1420 MW_{th}. This test was performed following licensing approval to measure reactor power from 1420 to 1500 MW. This procedure did not involve an unreviewed safety question per 10CFR50.59 since plant operation was totally unaffected. The test consisted entirely of measuring dose rates at various locations.
SP-FAUD-1	2-4-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSTION APPROVAL (Continued)

Procedure	Description
SP-FAUD-1	2-5-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-6-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-8-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-9-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-10-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-11-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

Procedure	Description
SP-FAUD-1	2-12-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-16-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-17-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-18-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
SP-FAUD-1	2-19-82 / Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF

Effective March 1, the following people will be appointed to the designated areas.

Fred Smith - Plant Chemist

Gary Roach - Plant Health Physics

G. TRAINING

Training for the month of February included General Employee Training in Security, Radiation Protection, First Aid and Driver's Training.

Systems Training was held for Maintenance. Operators Requalification Training continued as scheduled.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No. 64 revised the reactor coolant system pressure-temperature limits for operation through 6.1 effective full power years.

for W. Gary Gates

S. C. Stevens
Manager
Fort Calhoun Station

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
13020	12-6-81	AC-334 leaks.	Replaced bonnet gasket and repacked valve.
13450	12-23-81	CH-181 has flange leak.	Lapped disc and seat.
12530	1-27-82	480V Load Shed lockout switches did not operate properly.	Tested.
13020	12-6-81	AC-334 leaks.	Removed bonnet, cleaned internals & seat repacked.
13450	12-23-81	CH-181 flange leak.	Lapped disc & seat.
12530	11-27-81	OPLS 480V Load Shed.	Checked circuitry.
12142	11-27-81	Excessive ripple on power supply for power supply for D-122C.	Found ground & calibrated.
13549	1-4-82	NP-4 Circuit #21 tripped & will not reset.	Replaced NP-2 #2.
13711	1-21-82	FCV-1101 valve-flow spiked low & returned.	Found loose terminals and repaired.
13126	12-7-81	CH-396 suction relief on B Boric Acid Tank leaked.	Replaced gaskets.
12547	1-25-82	HCV-306 stroke time slow.	Solenoid valve replaced.
10468	1-24-82	HCV-2968 valve leaks.	Repaired per procedure.
12977	12-3-81	HCV-402B will not open from control switch.	Replaced solenoid valve.
11843	12-6-81	HCV-509A, B leaking through.	Installed new diaphrams and leak tested.
13428	12-23-81	LCV-1173 leaked through.	Lapped seat & face.
12844	11-20-81	FM-212 reads low.	Replaced square root extractor.
11905	10-20-81	RC-319 packing leaks.	Replaced valve.
9823	12-3-81	HCV-335 valve leaks around flange gaskets.	Replaced gaskets.
13275	12-14-81	PCV-103-2 repair or replace valve operator.	Replaced actuator.

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
13264	12-16-81	TI-A/122H Reactor Coolant Loop Z pegged low.	Replaced butt splices on containment side of electrical penetration.
9943	12-4-81	Component Cooling Water from VA-8A no flow alarm.	Blown fuse in power supply.
12642	12-4-82	Containment Cooling Flow Ind. VA-8B pegged high.	Bad connection in amplifier board.
11426	9-18-81	CH-1B leaking through primary packing.	Maintenance performed per MP-CH-1.
13276	12-16-81	A-112H Reading high.	Replaced butt splices on containment side of electrical penetration.