OPERATING DATA REPORT

DOCKET NO. 050-0331
DATE March 12, 1982
COMPLETED BY Sidney L. Brown
TELEPHONE 319-851-5611

OPERATING STATUS

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: February, 1982
3. Licensed Thermal Power (MWt): 1658
*4. Nameplate Rating (Gross MWe): 565 (Turbine Rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672	1,416	62,040
12. Number Of Hours Reactor Was Critical	672	1,416	44,894.7
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	672	1,416	43,805.4
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,031,146	2,002,899	55,056,297
17. Gross Electrical Energy Generated (MWH)	349,234	679,475	18,454,515
18. Net Electrical Energy Generated (MWH)	329,880	641,183	17,272,769
19. Unit Service Factor	100%	100%	70.6%
20. Unit Availability Factor	100%	100%	70.6%
21. Unit Capacity Factor (Using MDC Net)	95.3%	87.9%	54.1%
22. Unit Capacity Factor (Using DER Net)	91.2%	84.2%	51.7%
23. Unit Forced Outage Rate	0	0	16.7%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Maintenance; April 1, 1982; two weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

* Turbine Rating: 565.7 MWe
Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

AVERAGE DAILY UNIT POWER LEVEL

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MONTH February, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	506	17	509
2	502	18	505
3	507	19	510
4	509	20	479
5	506	21	494
6	472	22	503
7	481	23	500
8	494	24	506
9	508	25	311
10	507	26	414
11	499	27	499
12	504	28	506
13	509	29	
14	505	30	
15	494	31	
16	506		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982

DOCKET NO. 050-0331
UNIT NAME Duane Arnold Energy Center
DATE March 12, 1982
COMPLETED BY Sidney L. Brown
TELEPHONE 319-851-5611

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	820225	F	0	A	4	LER 82-015	CF	VALVOP A	Worn stem nut on the "B" RHR SW MOV 1947 prevented valve operation. This resulted in a 24 hour LCO and a forced power reduction because the "B" containment spray system was compromised and the "A" RHR system had previously been declared inoperable. The forced power reduction ended 20 hours into the 24 hour LCO after the "A" RHR system was made operable. The stem nut on MOV-1947 was replaced and satisfactorily tested.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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Docket No. 050-0331
Unit Duane Arnold Energy C
Date March 12, 1982
Completed by S. L. Brown
Telephone 319-851-5611

REFUELING INFORMATION

1. Name of facility.

   A. Duane Arnold Energy Center

2. Scheduled date for next refueling shutdown.

   A. September 4, 1982

3. Scheduled date for restart following refueling.

   A. Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   A. Yes. New MAPLHGR tables will have to be included in Technical Specifications.

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

   A. Unknown at this time.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   A. New fuel assemblies to be placed in the reactor will be more highly enriched than those currently in use.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   A. a) 368 b) 448

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   A. 2050

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   A. 1998

Docket No. 055-0331
Unit Duane Arnold Energy Ctr
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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

2-1 At the beginning of this reporting period, the plant was operating at 511 MWe.

2-4 During a normal operation surveillance test, RCIC injection valve, MOV 2512, was opened and would not close. The valve was repaired by replacing the motor operator circuit breaker close contactor coil.

RO Report 82-007

2-5 During normal operation while moving new fuel through the opened outer reactor building railroad airlock door, the inner railroad airlock personnel door was inadvertently opened, violating secondary containment integrity. A defective solenoid was replaced, the interlock design will be reviewed, and personnel will be instructed about secondary containment requirements.

RO Report 82-009

2-6 During normal operations, the 'B' oxygen analyzer for the containment atmosphere dilution system was reading upscale. The analyzer was repaired by retightening a connection.

RO Report 82-010

2-8 During normal operation, a self-cleaning strainer IS-85B on the suction line to the 'B' screen wash pump for the river water supply system became plugged. The plugged strainer was cleared and a failed strainer shear pin replaced.

RO Report 82-011

During normal operation while performing surveillance testing, the main steam isolation valve leakage control system bypass valve, MOV-8403C, opened but failed to close. The fault was caused by a failed motor starter contactor coil auxiliary switch which was replaced.

RO Report 82-012

2-14 New fuel arrived on site.

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

2-15 During normal operation, control building isolation damper, IV-AD-31A, was stuck open. The damper was shut, but it was determined that it would not automatically close if isolation was required.

RO Report Pending

2-19 During normal operation, the failures of one snubber, GBB-4-SS-211 and one hanger on the 'A' RHR discharge line were identified.

RO Report Pending

2-20 During normal operation, snubber GBB-3-SS-235 on the 'B' RHR system was removed from service for testing after it was identified that the pipe clamp was rotated and the snubber extension was bent. The snubber was replaced and the pipe clamp properly positioned.

RO Report 82-013

2-21 During normal operations, the torus level transmitter, LT-4363, started behaving erratically.

RO Report Pending

2-24 During a normal operation surveillance test RHR service water valve MOV-1947 failed to close. The valve stem nut and gaskets were replaced which corrected the problem.

RO Report 82-015

2-25 During a normal operation surveillance test, the main steam high temperature switch would not trip.

RO Report Pending

2-28 The plant was operating at 528 MWe at 2253 hours.

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MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
2-4-82	RCIC	MOV-2512	Replaced motor operator circuit breaker close contactor coil
2-5-82	Containment Atmosphere Control	SV 8105A, SV 8105B and SV 8102B	Disassembled, inspected and reassembled
2-5-82	Primary Containment	LR 4385 (B Torus level indicator)	Calibrated and re-installed
2-8-82	MSIV-LCS	MOV-8403C	Replaced auxiliary switch
2-11-82	River Water Supply	1S-85B (Strainer for screen wash pump)	Replaced shear pin and cleaned strainer
2-20-82	RHR	Snubber GBB-3-SS-235	Replaced snubber
2-23-82	Control Bldg. H&V	RE 6101A (Air intake monitor)	Lubricated
2-23-82	Containment Atmosphere Control	RE 8102B (Gaseous Monitor)	Replaced tube
2-23-82	Containment Atmosphere Control	RE 8103A (Iodine Monitor)	Replaced detector
2-26-82	RHR	GBB-5-H-18 Hanger Clamp	Repositioned clamp
2-26-82	RHR	GBB-5-SR-130 Clamp	Repositioned clamp
2-26-82	RHR	GBB-4-H-20D Pipe Support	Retorqued bolts
2-26-82	RHR	GBB-4-SS-216 Snubber Support	Retorqued bolts
2-26-82	RHR	GBB-5-SS-215 Pipe Hanger	Retorqued bolts and repositioned clamp
2-26-82	RHR	GBB-4-SS-211 Snubber Support	Replaced snubber support
2-26-82	RHR	GBB-3-SS-237 & 238 Pipe Clamp	Clamp inspected and realigned
2-26-82	RHR	GBB-4-SS-213 Pipe Clamp	Repositioned

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MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
2-26-82	RHR	GBB-4-H7 Pipe Hanger	Repositioned support
2-26-82	RHR	GBB-4-H7, GBB-4-H20D, GBB-4-H4, GBB-4-H10, GBB-4-H11, GBB-4-H12, GBB-4-H13, GLE-7-H14, GBB-5-H18 and DBB-1-H20A Pipe Hangers	Adjusted hangers
2-26-82	RHR Cooling Water	MOV-1947	Replaced stem nut and gaskets