American Electric Power Service Corporation D. C. Cook 1, 2 (50-315, 50-316)

Cincinnati Gas and Electric Company Zimmer (50-358)

The Cleveland Electric Illuminating Company Perry 1, 2 (50-440, 50-441)

Commonwealth Edison Company
Braidwood 1, 2 (50-456, 50-457)
Byron 1, 2 (50-454, 50-455)
Dresden 1, 2, 3 (50-10, 50-237, 50-249)
La Salle 1, 2 (50-373, 50-374)
Quad-Cities 1, 2 (50-254, 50-265)
Zion 1, 2 (50-295, 50-304)

Consumers Power Company
Big Rock Point (50-155)
Palisades (50-255)
Midland 1, 2 (50-329, 50-330)

Dairyland Power Corporation LACBWR (50-409)

The Detroit Edison Company Fermi 2 (50-341)

Illinois Power Company Clinton 1, 2 (50-461, 50-462)

Iowa Electric Light and Power Company
Duane Arnold (50-331)

Northern Indiana Public Service Company Bailly (50-367)

Northern States Power Company Monticello (50-263) Prairie Island 1, 2 (50-282, 50-306)

Public Service of Indiana Marble Hill 1, 2 (50-546, 50-547)

Toledo Edison Company Davis-Besse 1 (50-346)

Union Electric Company Callaway 1, 2 (40-483, 50-486)

Wisconsin Electric Power Company Point Beach 1, 2 (50-266, 50-301)

Wisconsin Public Service Corporation Kewaunee (50-305)

Illinois Department of Nuclear Safety
Garv N. Wright. Manager, Nuclear Facility Safety
3203190285 820312
DR ADOCK 05000010



SSINS No.: 6835 Accession No. 8202040130 IN 82-06

UNITED STATES NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT WASHINGTON, D.C. 20555

March 12, 1982

IE INFORMATION NOTICE NO. 82-06: FAILURE OF STEAM GENERATOR PRIMARY SIDE MANWAY CLOSURE STUDS

Description of Circumstances:

The Nuclear Regulatory Commission (NRC) was notified by Maine Yankee Atomic Power Company and by Combustion Engineering (CE) that during routine disassembly of a steam generator primary side manway at Maine Yankee, 6 of the 20 manway closure studs failed and another 5 were found by ultrasonic testing to be cracked. These are $1\frac{1}{2} \times 10$ inch studs of SA 540 grade B 24 alloy steel. The studs had been exposed to boric acid from a small primary coolant leak and to Furmanite sealing compound (primary grade) applied in an attempt to seal this leak. The studs exhibited evidence of surface corrosion attack possibly as a result of an interaction associated with stud preload, lubricant, Furmanite and primary coolant leakage environment. A metallurgical analysis to determine the failure mechanism is currently underway at CE. The entire set of studs on the affected steam generator (SG #2) have been replaced and an ultrasonic examination of all primary manway studs on steam generator units 2 and 3 s being performed. Further corrective actions are pending stud failure analysis and its applicability to other primary boundary closures.

In the last few years there have been a significant number of incidents of failed or severly degraded bolts and studs. Examples of the latter; primary coolant pump stud-bolts (Calvert Cliffs and Ft Calhoun) and steam generator primary manway closures studs (Oconee and ANO-1). The failures described were attributed to stress corrosion cracking and corrosion wastage of high strength studs that are difficult to detect.

The NRC has contacted the CE Regulatory Response Group and requested a review of the problem.

This IE information notice is provided as an early notification of a potentially significant matter that is still under review by the NRC staff. If NRC evaluation so indicates, further licensee action may be requested. In the interim, we expect that licensees will review this information for applicability to their facilities.

No written response to this information notice is requested. If you need additional information, please contact the Regional Administrator of the appropriate NRC Regional Office.

Attachment:

Recently issued IE Information Notices

RECENTLY ISSUED IE INFORMATION NOTICES

B2-05 Increasing Frequency of Drug-Related Incidents facilities he an OL or CP B2-04 Potential Deficiency of Certain AGASTAT E-7000 Facilities he an OL or CP B2-03 Environmental Tests of Electrical Terminal Blocks facilities he an OL or CP B2-01 Auxiliary Feedwater Pump O2/26/82 All power regression of Circuitry Modification B0-32 Clarification of Certain O2/26/82 All facilities he an OL or CP B2-02 Westinghouse W-2 Switch Certain Part 50 licer active Materials B2-02 Westinghouse NBFD Relay O1/27/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain Nuclear Power Plants B2-01 Auxiliary Feedwater Pump O1/22/82 All power regression of Certain	Information		Date of	
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Rev. 1 Lockout Resulting from Westinghouse W-2 Switch Circuitry Modification 80-32 Clarification of Certain 02/26/82 All facility, materials and Use Shipements for Exlusive-Use Shipements of Radio-active Materials 82-02 Westinghouse NBFD Relay 01/27/82 All power reactive Materials 82-02 Westinghouse NBFD Relay 01/27/82 All power reactive in Reactor facilities he an OL or CP Certain Nuclear Power Plants 82-01 Auxiliary Feedwater Pump 01/22/82 All power reactive Mestinghouse W-2 Switch Circuit Modification 81-39 EPA Crosscheck Program - 12/23/81 All power reactive Mater Test Program an OL or CP	82-03		03/04/82	All power reactor facilities holding an OL or CP
Rev. 1 Requirements for Exlusive- Use Shipements of Radio- active Materials 82-02 Westinghouse NBFD Relay 01/27/82 All power reaction Facilities in Reactor Protection Systems at an OL or CP Certain Nuclear Power Plants 82-01 Auxiliary Feedwater Pump 01/22/82 All power reaction Systems at an OL or CP Lockout Resulting from facilities in Westinghouse W-2 Switch an OL or CP Circuit Modification 81-39 EPA Crosscheck Program - 12/23/81 All power reactions in Water Test Program an OL or CP		Lockout Resulting from Westinghouse W-2 Switch	02/26/82	All power reactor facilities holding an OL or CP
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	81-38	from Contamination of Air-	12/16/81	All power reactor facilities holding an OL or CP

OL = Operating License CP = Construction Permit