



GULF STATES UTILITIES COMPANY

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February 24, 1982

RBG- 12,163

File G9.5

Mr. Darrell G. Eisenhut, Director
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Eisenhut:

River Bend Station-Units 1 and 2
Docket Nos. 50-458/50-459



Enclosed are sixty (60) copies of Amendment 2 to the River Bend Station Final Safety Analysis Report. This amendment provides a response or schedule for each request for additional information attached to your letter of December 31, 1981; responses to various previous requests; and various corrections, changes and additional information which have resulted from our continued review of the FSAR. Also enclosed is a status summary for each of the items noted in Enclosure 4 of your August 5, 1981 letter.

Sincerely,

W. J. Cahill, Jr.
Senior Vice President
River Bend Nuclear Group

WJC/WJR/kt

Enclosures

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5/1/60*

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)
)
GULF STATES UTILITIES COMPANY) DOCKET NOS. 50-458
) 50-459
)
River Bend Station, Units 1 and 2)
_____)

AFFIDAVIT OF SERVICE

Frank P. Libertowicz, being duly sworn, on oath, deposes
and says that the following document:

AMENDMENT 2 TO THE FINAL SAFETY ANALYSIS REPORT

in the above-captioned proceeding has been served upon the persons
shown on the attached list by depositing copies thereof in the United
States mail on February 23, 1982, with proper postage affixed for
first class mail.

DATED: February 23, 1982

Frank P. Libertowicz

Stone & Webster Engineering Corporation
P.O. Box 2325
Boston, Massachusetts 02107

Subscribed and sworn to before me this 23rd day of February, 1982.

John O. Hall
Notary Public in and for the
Commonwealth of Massachusetts

My Commission Expires September 21, 1984

AMENDMENT 2 TO THE
FINAL SAFETY ANALYSIS REPORT
Distribution List

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Livermore, CA 94550

ATTACHMENT I

As requested by Mr. Bob Perch of your staff, this attachment is provided to address the areas identified in Enclosure 4 of D. G. Eisenhower's letter of August 5, 1981.

- (1) Environmental Qualification of Safety Related Electrical Equipment - Commission Memorandum and Order of May 23, 1980 defines the current staff requirements for qualification of this equipment. Additional guidance on this matter was provided in a subsequent NRR Order dated November 26, 1980 (concerning record requirements), Supplements 2 and 3 dated September 30, 1980 and October 24, 1980, respectively to IE Bulletin No. 79-01B, and a generic letter to all holders of CPs and OLs dated October 1, 1980.

STATUS SUMMARY

This guidance is reflected in the Environmental Qualification Document (EQD) submitted as part of the Final Safety Analysis Report (FSAR). The EQD presently addresses Class 1E electrical equipment purchased through Stone & Webster, the Architect Engineer for River Bend Station. A qualification program for equipment purchased through General Electric, the NSSS supplier, is currently under development and will utilize this guidance.

- (2) Emergency Preparedness - Guidance on the preparation of emergency plans is presented in NUREG-0654 (FEMA-REP-1), "Criteria for Preparedness in Support of Nuclear Power Plants". The requirements for the emergency response facilities are included in NUREG-0696, "Functional Criteria for Emergency Response Facilities." Further guidance on emergency preparedness is provided in the revised Appendix E to 10 CFR Part 50.

STATUS SUMMARY

FSAR Section 13.3, Emergency Planning, has been developed in accordance with the provisions of 10CFR50.34, Appendix E to 10CFR50 and NUREG-0654 (FEMA-REP-1).

Section 13.3.6 is currently under revision to include details of the emergency facilities provided for River Bend Station as required by NUREG-0696.

- (3) Safety-Related Structures, Systems and Components (Q-list) Controlled by the QA Program - Staff requests for additional information regarding this issue have been sent to a number of OL applicants. A recent request regarding the Diablo Canyon is provided as Enclosure 5.

STATUS SUMMARY

The structures, systems and components controlled by the QA program are those identified as such in Section 3.2, and are controlled as described in Section 17.2.2.1. Other items, such as fire protection equipment and instrument calibration, are controlled by the QA program as described in the appropriate sections of Chapter 17.

- (4) Fracture Prevention of Containment Pressure Boundary (GDC 51) - Enclosure 6 provides clarification on how the staff determines compliance with GDC 51.

STATUS SUMMARY

The following toughness requirements were specified for the ferritic materials of the containment pressure boundary to meet GDC-51, which requires that under operating, maintenance, testing and postulated accident conditions the containment pressure boundary material shall behave in a nonbrittle manner.

Toughness tests (Charpy V-Notch) were required for all ferritic materials greater than 5/8 in. thickness which form part of the containment, mat liner, mat embedments, containment vessel, containment vessel equipment hatch and personnel air locks, drywell combination equipment hatch and personnel door, drywell personnel air lock, and attachments to these components. The tests were performed at 0 F for all ferritic materials except bolts, which were tested at 30 F. The tests conformed to ASME III, Class MC, NE-2300 requirements.

All ferritic steel plates for the above components that are less than 5/8 in. thick (except for backing strips and gas test channels) were impact tested in accordance with ASTM A-20 and the test temperature was not more than 0 F.

Class 1 and Class 2 ferritic forgings and piping with process piping (thicker portion) thickness greater than 5/8 in. were toughness tested (Charpy V-Notch) for acceptance purposes at 40 F or lower in accordance with ASME III, subsection NB-2300. Class 1 and Class 2 ferritic forgings and piping with process pipe (thicker portion) thickness less than or equal to 5/8 in. were toughness tested (Charpy V-Notch) for information only at 40 F or lower in accordance with ASME III, subsection NB-2300, except that subsize specimens were allowed in accordance with SA-370. Ferritic forging penetrations with process piping (thicker portion) thickness greater than 5/8 in. were drop weight tested at 10 F in accordance with ASME III, subsection NB-2300. The test specimens were taken from each end of the penetration and in the vicinity of the outside diameter of the flued head of the penetrations produced as singles. For penetrations produced as multiples, the test specimens were taken at each end of the forging and in the vicinity of the outside diameter of one of the flued heads.

The isolation valves on Class 1 piping comply with the NB-2300 criteria. The isolation valves, of the containment pressure boundary, for all the Class 2 piping did not require toughness testing since the walls of the connecting pipes were less than 5/8 in. thick.

By imposing the above requirements, the River Bend Station design addresses the toughness criteria of subarticle NC-2300 of ASME Section III, 1977 Summer addenda.

Additionally, information to support your review of this topic has been provided in our letter of February 19, 1982, as requested in your letter of December 17, 1981.

- (5) Seismic Qualification of Instrumentation and Electrical Equipment Including Dynamic Loading for BWR's - A staff request for additional information in the review area has been sent to a number of pending OL applicants. A copy of that request is provided as Enclosure 7.

STATUS SUMMARY

Seismic qualification of safety-related instrumentation and electrical equipment is discussed in FSAR Section 3.10.

Seismic qualification of safety-related mechanical equipment is discussed in FSAR Section 3.9.2.2.

Data sheets similar to the "Qualification Summary of Equipment" sheets included in Enclosure 7 will be compiled and provided for each selected piece of equipment, upon request of the SQRT.

- (6) Fire Protection - The current requirements for the fire protection programs are defined in the new Appendix R to 10 CFR Part 50. As further guidance, a copy of a recent staff request for additional information on pending OL applications is provided as Enclosure 8.

STATUS SUMMARY

An appendix to the FSAR is currently under preparation to address Appendix R to 10CFR50. This appendix is expected to be submitted prior to May 1, 1982.

- (7) Core Thermocouple Readouts in the Control Room (TMI Action Item II.F.2 in NUREG-0737) - Discussion of this item should address how core thermocouple readouts are provided in the control room including location and rate of printout (see Part (4) of Attachment 1 to Item II.F.2). Additional information on this

item is provided in the Safety Evaluation Report Related to the Operation of La Salle County Station, Units 1 and 2, NUREG-0519.

STATUS SUMMARY

Our review of this issue is provided as part of our TMI evaluation outlined in Appendix 1A. GSU endorses the BWROG position on this issue, and as a member of the BWROG, is working toward a final resolution. FSAR revisions will be provided as they are developed.

- (8) Preservice and Inservice Inspections - Staff guidance in this review area has been sent to a number of pending OL applicants. A copy of that guidance is provided as Enclosure 9.

STATUS SUMMARY

This guidance will be utilized in development of the River Bend Station preservice and inservice inspection programs.

- (9) Preservice Inspection and Testing of Snubbers - The staff has recently established requirements to ensure snubber operability which have been transmitted to pending OL applicants. A copy of those requirements is provided as Enclosure 10.

STATUS SUMMARY

This guidance is currently under evaluation for use in the development of the River Bend Station preservice and inservice inspection programs identified in Item 8.

- (10) Containment Integrity Regarding Ability to Withstand a Hydrogen Detonation - Staff requests for detailed analysis and results regarding this issue have been sent to a number of OL applicants. A copy of that request is provided as Enclosure 11.

STATUS SUMMARY

This guidance was previously provided by Mr. R. L. Tedesco's letter to our E. L. Draper dated March 30, 1981. Any FSAR changes required will be provided following completion of the program identified in our letter dated June 22, 1981.