NRC FOR	IM 366 U. S. NUCLEAR REGULATORY COMMISSION
	LICENSEE EVENT REPORT
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	REPORT L 6 0 5 0 0 3 6 6 0 0 1 1 1 1 8 2 8 0 2 0 9 8 2 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80
02	On 1-11-82, with Unit 2 at steady state power it was discovered that a
0 3	setpoint change from 133 to 111" WC, per DCR 79-376, for the RCIC stm.
04	line dP inst. (2E51-NO18), represented a Tech Spec conflict by it not
05	having been implemented. Tech Spec Table 3.3.2-2 limits the RCIC stm.
06	line flow to 300% (dP setpoint of 111"). RCIC was declared inop and per
07	Tech Spec 3.7.3.a HPCI was operable. There were no effects upon public
08	health & safety due to this non-repetitive event. Unit 1 is not affected.
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10	setpoint conflicted with Tech Specs. The 2E51-N018 was recalibrated to
	comply with table 3.3.2-2 (< 300%) & the calib. procedure "RCIC Stm. Line
	dP Instr. FT&C" (HNP-2-3410) was revised to reflect the new setpoint
	limit. The RCIC system was proven operable.
7 8	BO FACILITY STATUS LE (28) 0 7 4 (29) NA OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32) L A (31) Design Review (32)
7 8 1 6 7 8	ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 3 10 11 44 45 46 ROLLOCATION OF RELEASE 36 NA LOCATION OF RELEASE 36 NA 80
17 78	PERSONNEL EXPOSURES 0 0 0 0 0 37 Type 9 PERSONNEL INJURIES 13 0 10 12 13 13 10 10 10 10 10 10 10 10 10 10 10 10 10
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20	PUBLICITY ISSUED DESCRIPTION 45 9 10 PDR ADOCK 05000366 PDR ADOCK 05000366 PDR 46 PDR 68 69 NRC USE ONLY
	NAME OF PREPARERC. L. Coggin - Supt. Plt. Eng. Serv. PHONE 912-367-7851

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LER #: 50-366/1982-5 Licensee: Georgia Power Company Facility Name: Edwin I. Hatch Docket #: 50-366

Narrative Report for LER 50-366/1982-5

On 1/11/82 with Unit 2 at steady state 74% thermal power, it was discovered that the RCIC inboard high steam line differential pressure instrument setpoint (2E51-N018) was above the 300% rated flow Tech Spec limit (Table 3.3.2-2). The RCIC system was declared inoperable and the inboard steam supply isolation valve was closed. Per Tech Spec 3.7.3.a the HPCI system was operable. This event was determined to be reportable per Tech Spec Section 6.9.1.9.c (reduction of redundant engineering safety features). There were no effects upon public health and safety due to this event. This is a non-repetitive occurrence. Unit 2 RCIC outboard high dP isolation is not affected, nor is Unit 1 RCIC in/outboard isolations affected by this event.

As a result of Unit 2 RCIC testing (TER 79-03) GE issued Revision 1 to FDDR HT2-396 on 8-21-79, which changed the 2E51-N018 instrument setpoint from 133" WC to 111" WC. DCR 79-376 was written to implement FDDR HT2-396, R1, but was not expeditiously implemented by the site, nor was the calibration procedure for the instrument ("RCIC Steam Line Hi dP FT&C", HNP-2-3410) revised to reflect the DCR changes until 1-11-82.

This event was due to the site personnel failing to recognize that the reduction of the RCIC instrument (2E51-N018) setpoint from 111" WC to 133" WC, implied that the 133" setpoint was non-conservative with respect to Tech Specs, and thus constituted a Tech Spec conflict per Tech Spec Table 3.3.2-2. The RCIC steam line isolation valves are to isolate at less than or equal to 300% of rated steam line flow, in which the new 300% setpoint was 111".

On 1-11-82, when a Tech Spec conflict was discovered, the Unit 2 RCIC was declared inoperable and the RCIC dP instrument (2E51-N018) was recalibrated along with the "RCIC Steam Line Hi dP Instrument FT&C" (HNP-2-3410) procedure being revised to the new 111" setpoint (300% flow limit). The Unit 2 RCIC was proven operable following the recalibration of 2E51-N018.