#### OPERATING DATA REPORT

050-0331
February 2, 1982
J. Van Sickel
319-851-5611

#### **OPERATING STATUS**

1. Unit Name: Duane Arnold Energy Center	Notes
2. Reporting Period: January, 1982	
3. Licensed Thermal Power (MWt):1658	이 것은 이 것이 같은 것이 같은 것이 같아?
4. Nameplate Rating (Gross MWe): 565 (Turbine Rating)	1993년 - 11일 - 11일 · 11일 · 11일 · 11일
5. Design Electrical Rating (Net MWe): 538	
6. Maximum Dependable Capacity (Gross Mwe): 545	
7. Maximum Dependable Capacity (Net MWe): 515	L]

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any: \_\_\_\_

승규는 것은 것은 것은 것이 있다.	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	744	744	61,368
12. Number Of Hours Reactor Was Critical	744	744	44,222.7
13. Reactor Reserve Shutdown Hours	0	0	. 0 .
14. Hours Generator On-Line	744	744	43,133.4
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	971,753	971,753	54,025,151
17. Gross Electrical Energy Generated (MWH)	330,241	330,241	18,105,281
18. Net Electrical Energy Generated (MWH)	311,303	311,303	16,942,889
19. Unit Service Factor	100%	100%	70.3%
20. Unit Availability Factor	100%	100%	70.3%
21. Unit Capacity Factor (Using MDC Net)	81.2%	81.2%	52.9%
22. Unit Capacity Factor (Using DER Net)	77.8%	77.8%	51.3%
23. Unit Forced Outage Rate	0.0%	0.0%	16:9%

Maintenance, April 1, 1982, Two weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_

\* Turbine Rating: 565.7 MWe

8202180398 820209 PDR ADOCK 05000331 R

Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	050-0331
UNIT	Duane Arnold Energy Center
DATE	February 9, 1982
COMPLETED BY	J. Van Sickel
TELEPHONE	319-851-5611

MONTH \_\_January, 1982

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AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
220	17	443
223	18	472
233	10	502
328	20	448
. 399	20	425
444	22	369
451	23	421
439 .	24	461
307	25	481
386	26	458
455	27	501
452	28	502
444	50	499
446	30	375
444	31	494
449	51	

#### INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

TELEPHONE 319-851-5611 Method of Shutting Down Reactor<sup>3</sup> Component Code<sup>5</sup> Reason? Duration (Hours) System Code<sup>4</sup> Typel Licensee Cause & Corrective No. Date Event Action to Report # Prevent Recurrence 1. 820109 S 0 H 4 Power was reduced to perform control rod withdrawals. 2. 820130 S 4 Power was reduced to allow repair 0 B work on the condenser air ejectors. 7 3 4 F: Forced Reason: Method: Exhibit G - Instructions A-Equipment Failure (Explain) B-Maintenance or Test S: Scheduled 1-Manual for Preparation of Data 2-Manual Scram. Entry Sheets for Licensee C-Refueling 3-Automatic Scram. Event Report (LER) File (NUREG-D-Regulatory Restriction 4-Other (Explain) 0161) E-Operator Training & License Examination **F**-Administrative 5 G-Operational Error (Explain) Exhibit 1 - Same Source (9/77) H-Other (Explain)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982

050-0331

DATE February 2, 1982

Duane Arnold Energy Ctr.

DOCKET NO.

UNIT NAME

COMPLETED BY J. Van Sickel

#### REFUELING INFORMATION

Unit Duane Arnold Energy O Date February 2, 1982 Completed by J. Van Sicke Telephone 319-851-5611

1. Name of facility.

A. Duane Arnold Energy Center

2. Scheduled date for next refueling shutdown.

A: September 4, 1982

3. Scheduled date for restart following refueling.

A. Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

A. Yes. New MAPLHGR tables will have to be included in Technical Specifications.

5. Scheduled date(s) for submitting proposed licensing action and supporting information.
A. Unknown at this time.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

A. New fuel assemblies to be placed in the reactor will be more highly enriched than those currently in use.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

A. a) 368 b) 448

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

A. 2050

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

A. 1998

# MAJOR SAFETY RELATED MAINTENANCE

Docket No. 050-0331 Unit Duane Arnold Energy Center Date February 2, 1982 Completed by J. Van Sickel Telephone 319-851-5611

DATE	SYSTEM	COMPONENT	DESCRIPTION
1-4-82	Containment Atmospheric Control	RE-8101B	Replaced detector.
1-21-82	RHR Service Water	Snubber GBC-1-SS-56	Replaced snubber
1-25-82	Primary Containment	TDS-2260A	Replaced switch on temperature module
1-27-82	RHR	CV-2037 ·	Replaced valve voke
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Docket No.	050-0331	
Unit Duane	Arnold Energy Center	
Date Februa	ary 2, 1982	
Completed	by J. Van Sickel	
Telephone_	319-851-5611	

## NARRATIVE SUMMARY OF OPERATING EXPERIENCE

- 1-1 At the beginning of the report period the plant was operating at 244 Mwe.
- 1-5 A power increase was begun.
- 1-6 The plant was operating at 474 MWe at 1327 hours.
- 1-7 During normal operation, an offgas stack high/low flow alarm was received. The cause was a frozen sample return line.

#### ETSV Report 82-1

1-8 During surveillance testing, turbine building main steam supply line area high temperature switch TIS 4477, Subchannel B, was found to have a nonconservative out of specification setpoint.

#### RO Report 82-1

- 1-9 Plant load was reduced to allow control rod withdrawals to be performed.
- 1-10 A load increase was begun.
- 1-11 The plant was operating at 486 MWe at 1430 hours.
- 1-12 During surveillance testing, RCIC steam supply high flow PDIS 2441 and PDIS 2442 were found to have out of tolerance setpoints.

#### RO Report Pending

1-13 During surveillance testing, reactor low pressure (core spray and RHR valve opening permissive) switch PS 4529 was found to have an out of tolerance setpoint.

#### RO Report Pending

1-14 During normal operation torus level recorder LR 4385 was found indicating downscale.

# RO Report Pending

During normal operation the "B" containment oxygen analyzer was declared inoperable. The redundant analyzer was out of service for design change work.

### RO Report Pending

1-15 During normal operation, offgas stack sample pump 1P-247A was found inoperable. The redundant sample pump had been found inoperable earlier.

ETSV Report 82-2

Docket No.	050-0331
Unit Duane	Arnold Energy Center
Date Februa	ary 2, 1982
Completed 1	by J. Van Sickel
Telephone	319-851-5611

### NARRATIVE SUMMARY OF OPERATING EXPERIENCE

During surveillance testing, drywell high pressure (RPS) switch PS 4315B, was found to have an out of tolerance setpoint.

RO Report Pending

1-18 A power increase was begun.

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- 1-19 The plant was operating at 533 MWe at 1500 hours. A RHRSW system snubber and the valve yoke on CV 2037 were found broken. An evaluation was begun.
- 1-21 The LPCI system was declared inoperable pending an engineering evaluation as a result of a stress analysis (which was conducted after snubber GBC-1-SS-56 was found broken) which concluded a section of RHR piping may have been overstressed.

RO Report Pending

- 1-22 A control rod sequence exchange was completed. A power increase was begun.
- 1-24 Engineering evaluations, hydrostatic testing, NDT, and operability testing were completed on the RHRSW and LPCI systems and these systems were declared operable.
- 1-25 The plant was operating at 526 MWe at 1440 hours.
- 1-30 Power was reduced to allow repair work to be completed on the "A" main condenser air ejector. A power increase was begun.
- 1-31 The plant was operating at 537 MWe at 2230 hours.